

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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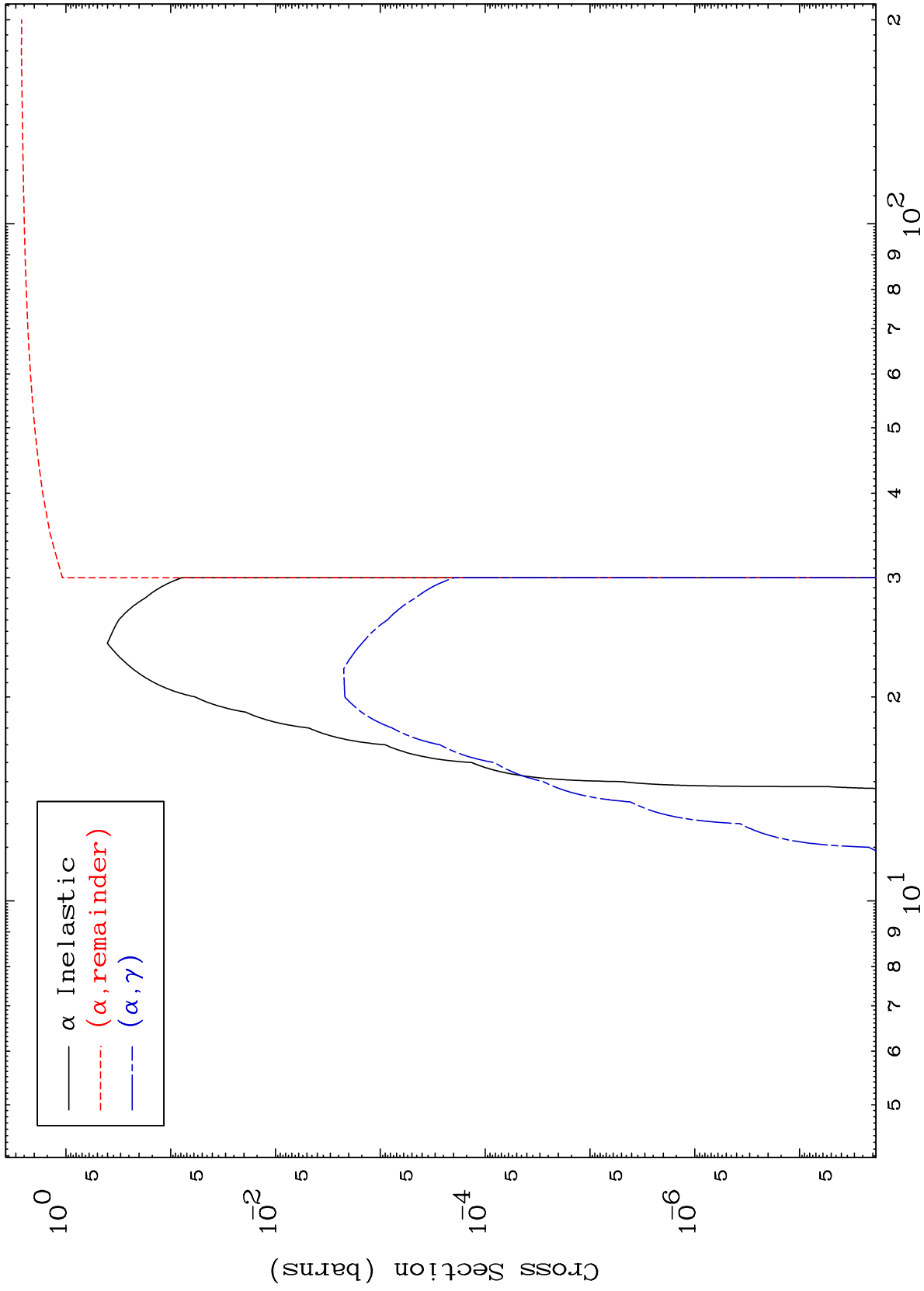
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7895

0 Kelvin  $\alpha$  Major  
Cross Sections

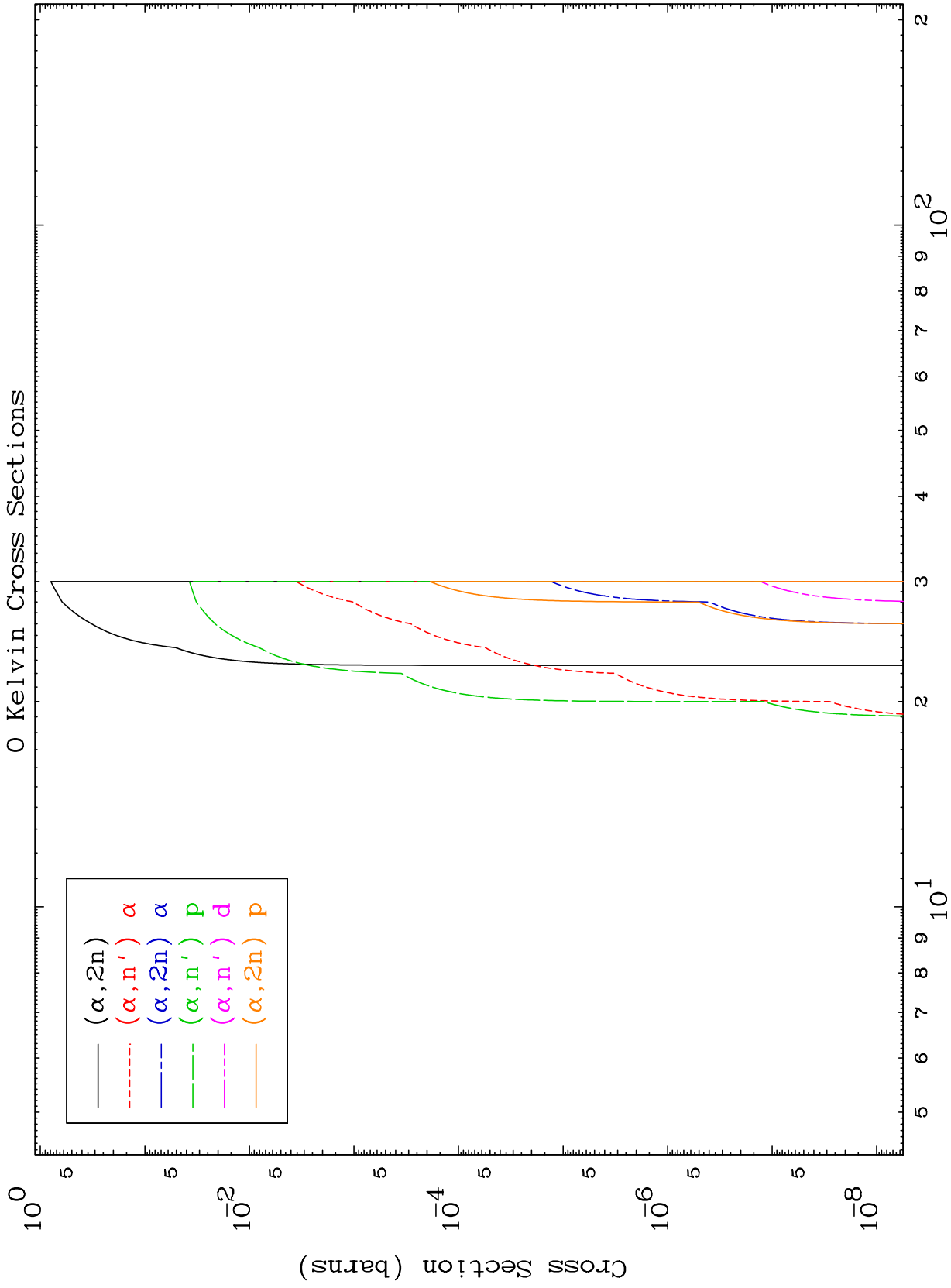
79-Au-187



MAT 7895

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

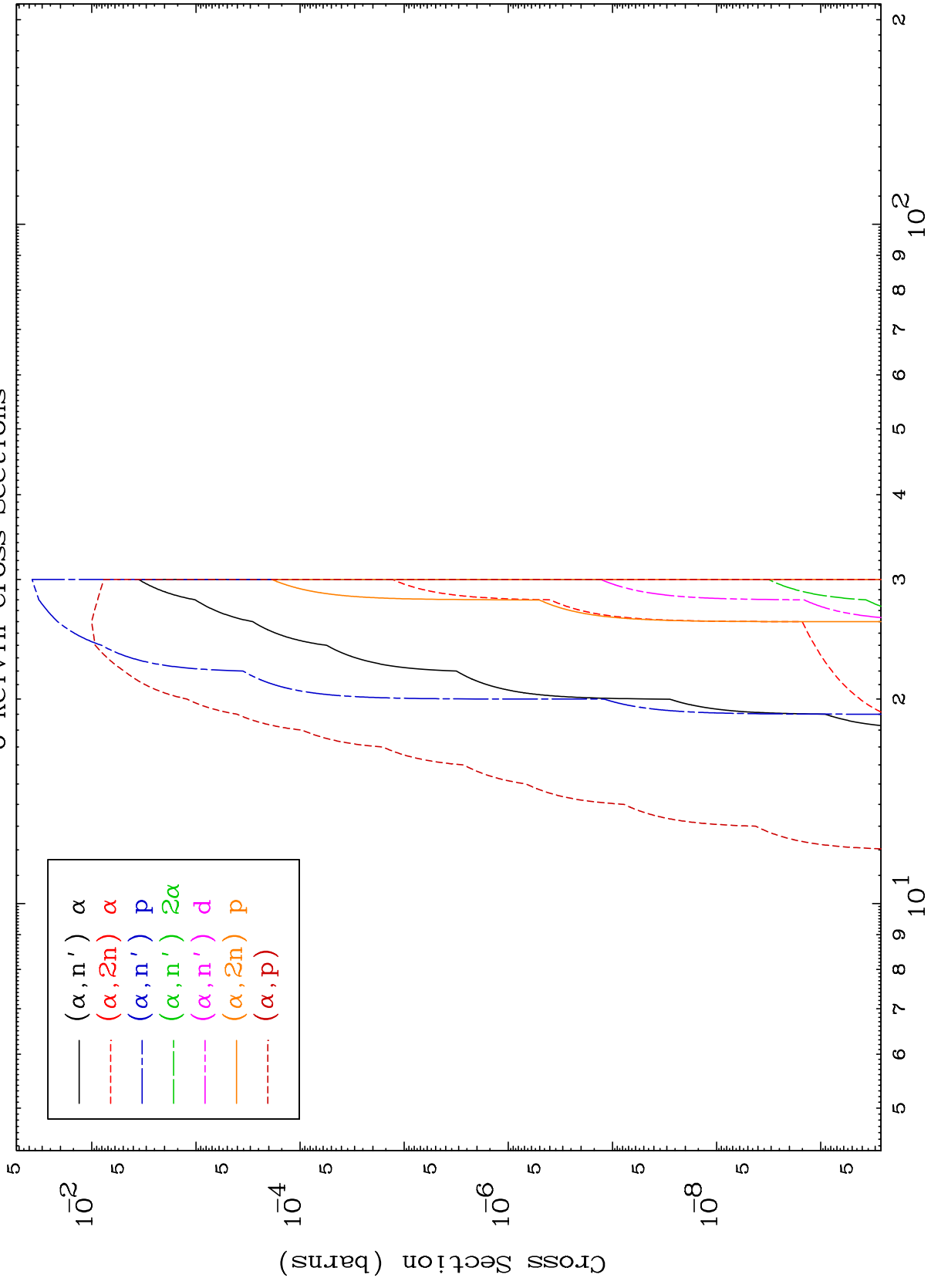
79-Au-187



2

Incident Energy (MeV)

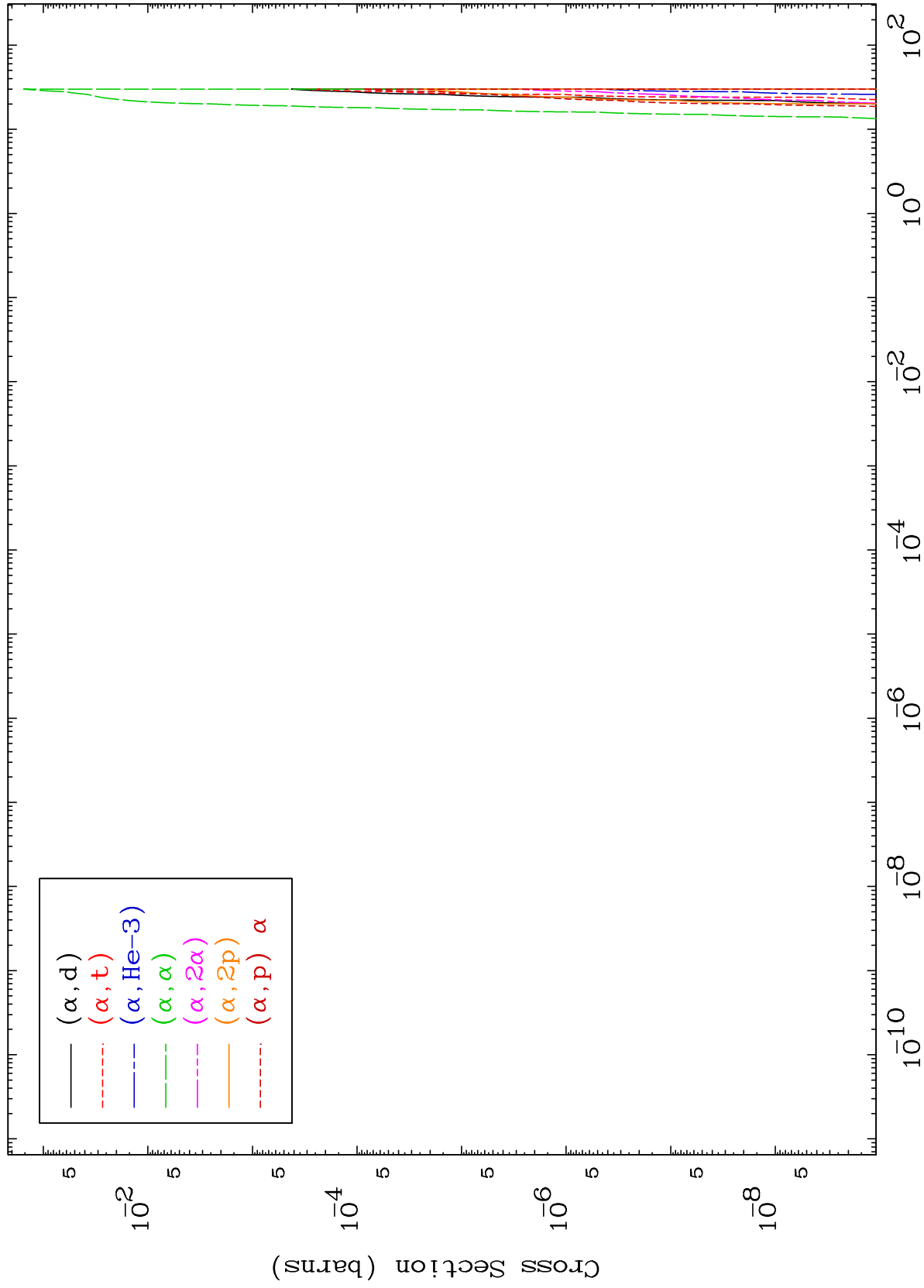
79-Au-187



MAT 7895

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

79-Au-187

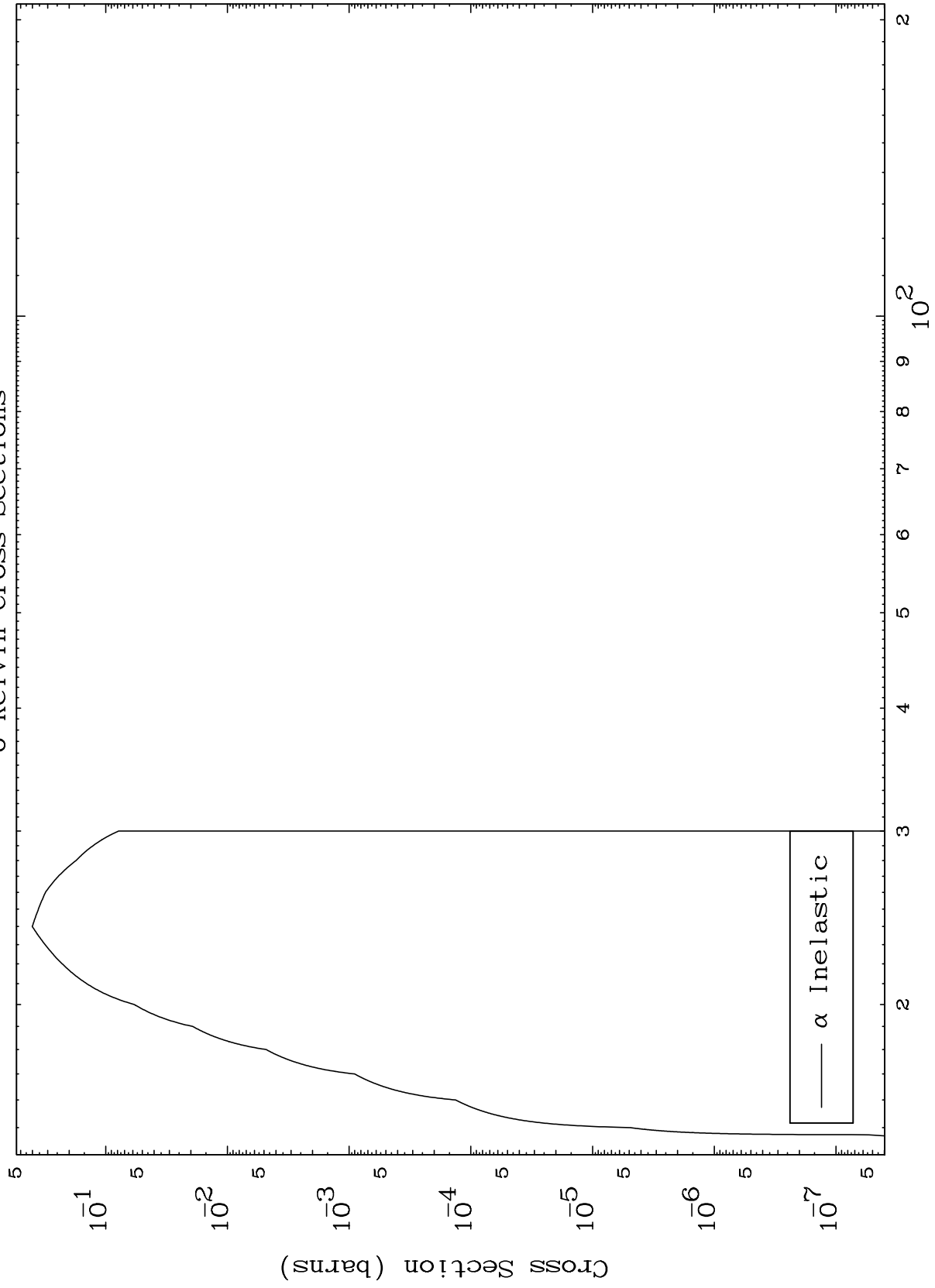


79-Au-187

MAT 7895

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

79-Au-187



5

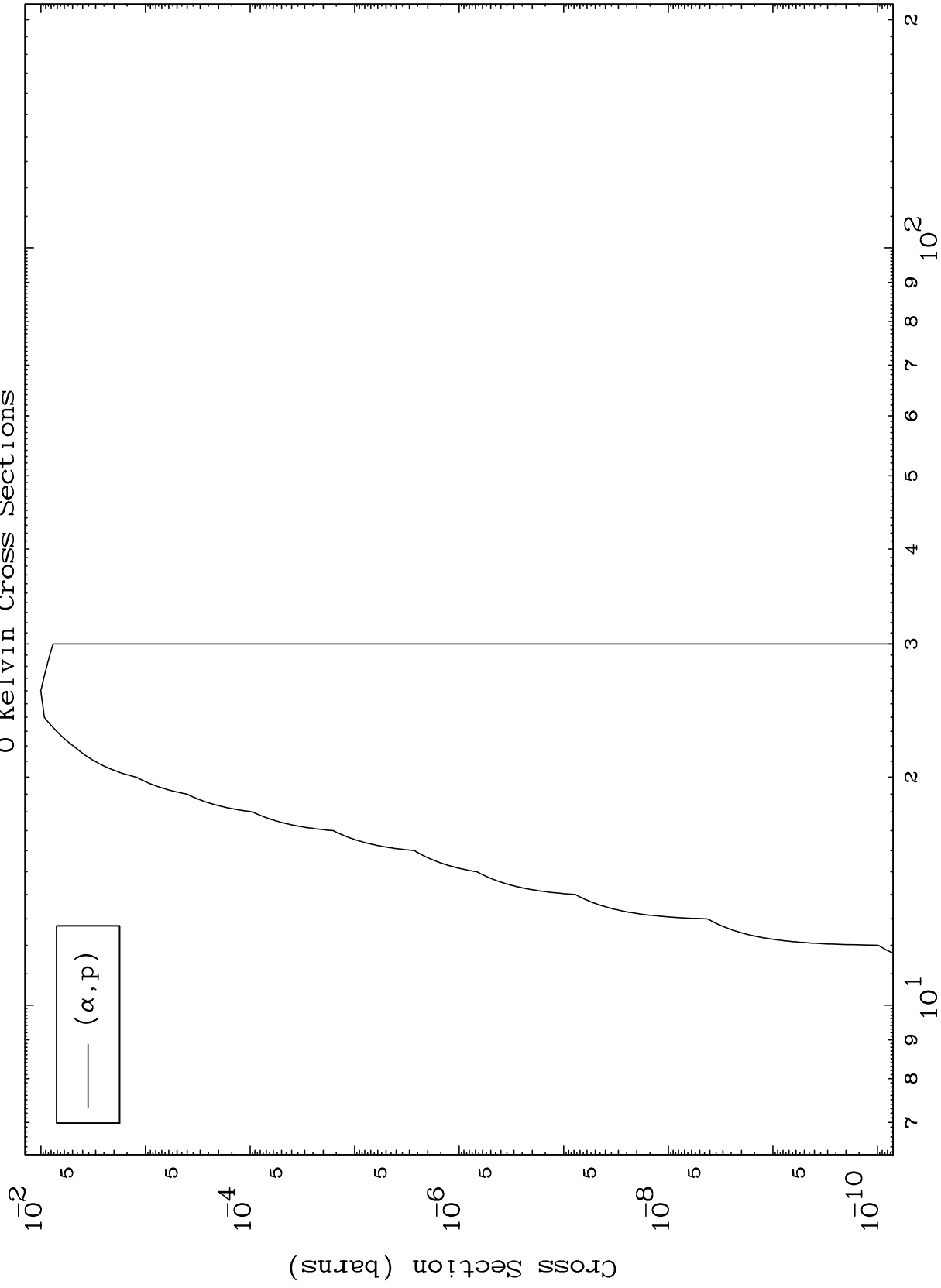
Incident Energy (MeV)

79-Au-187

MAT 7895

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

79-Au-187



6

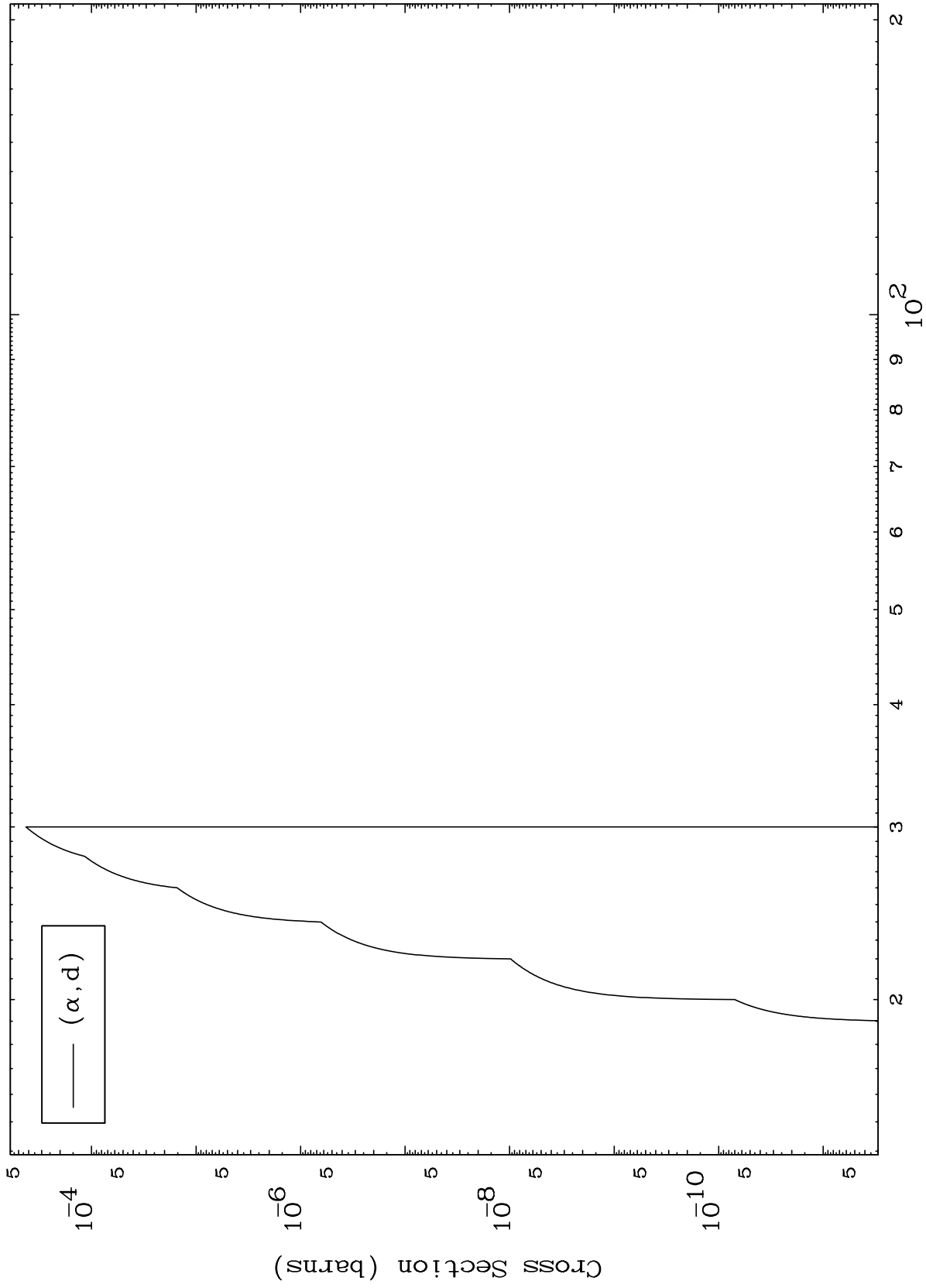
Incident Energy (MeV)

79-Au-187

MAT 7895

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

79-Au-187



7

Incident Energy (MeV)

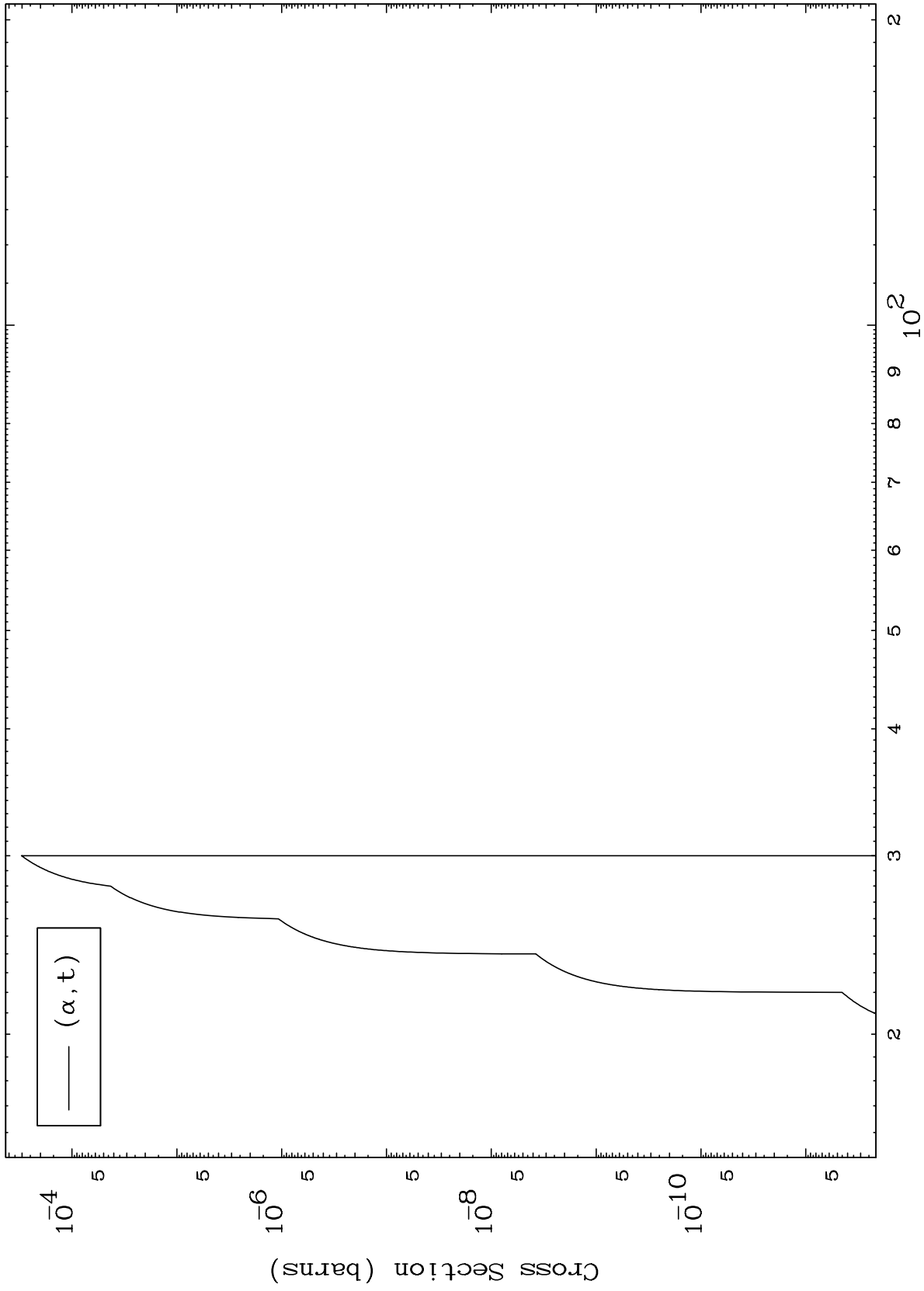
79-Au-187

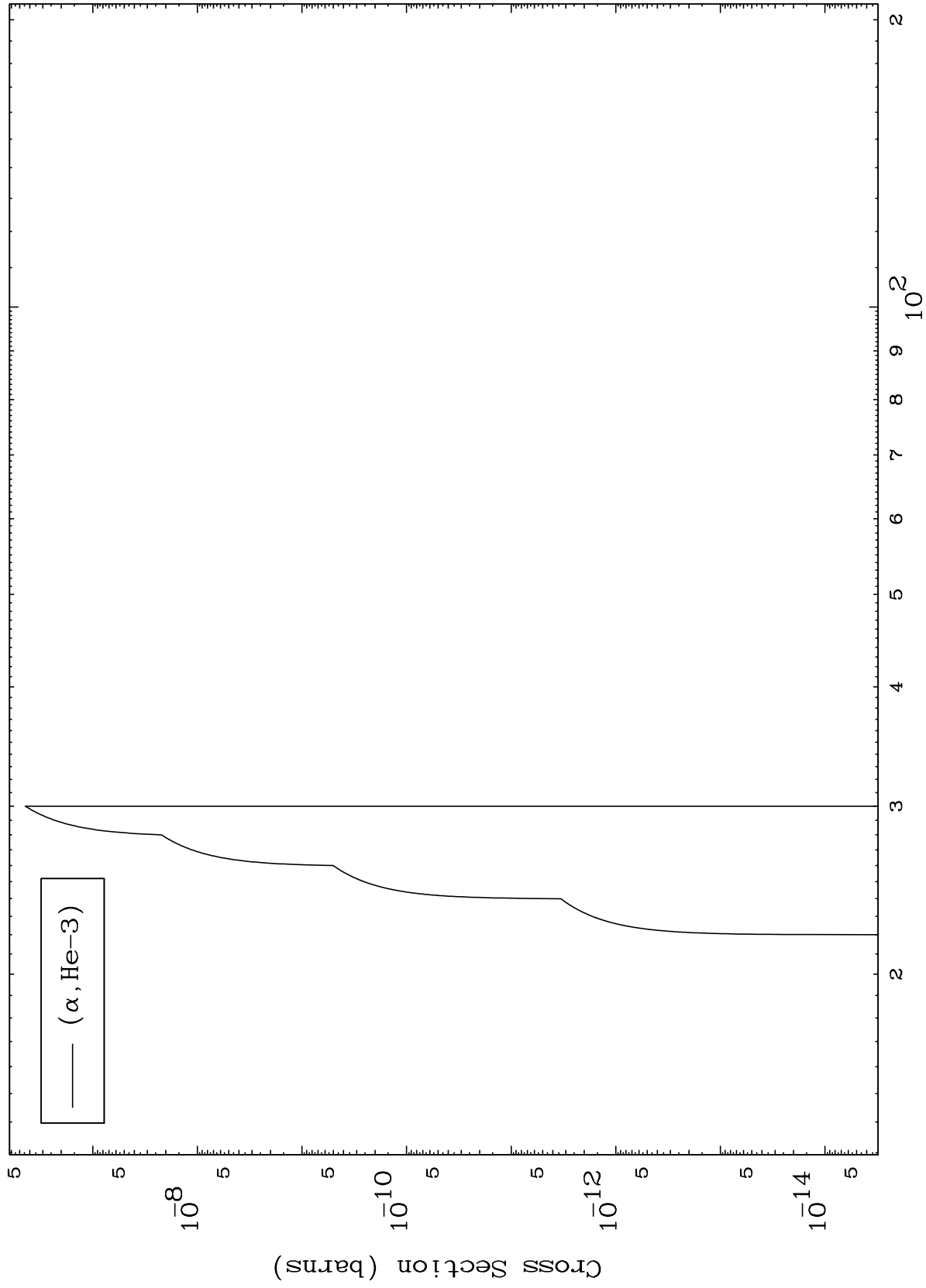


MAT 7895

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

79-Au-187

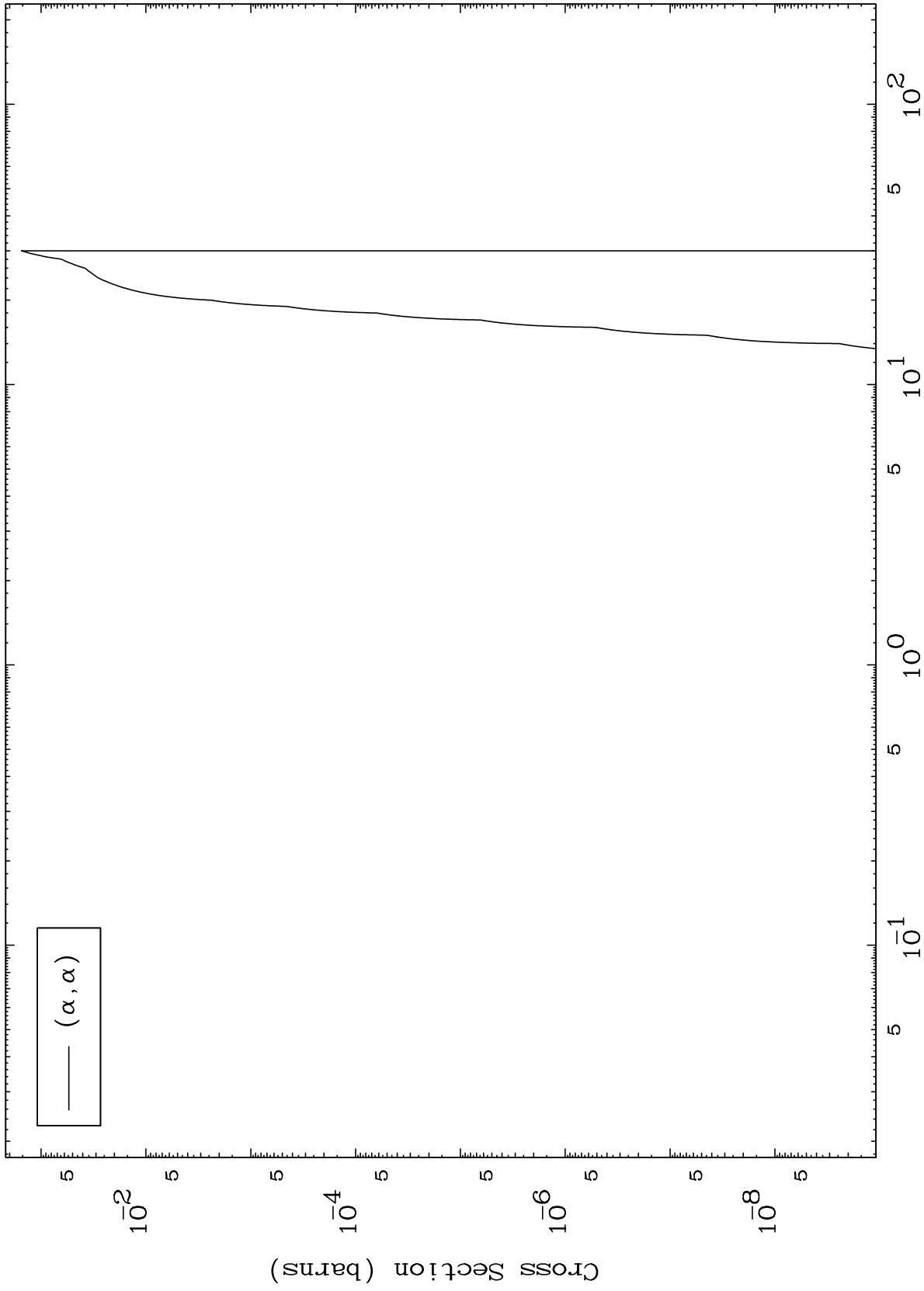




MAT 7895

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

79-Au-187



10

Incident Energy (MeV)

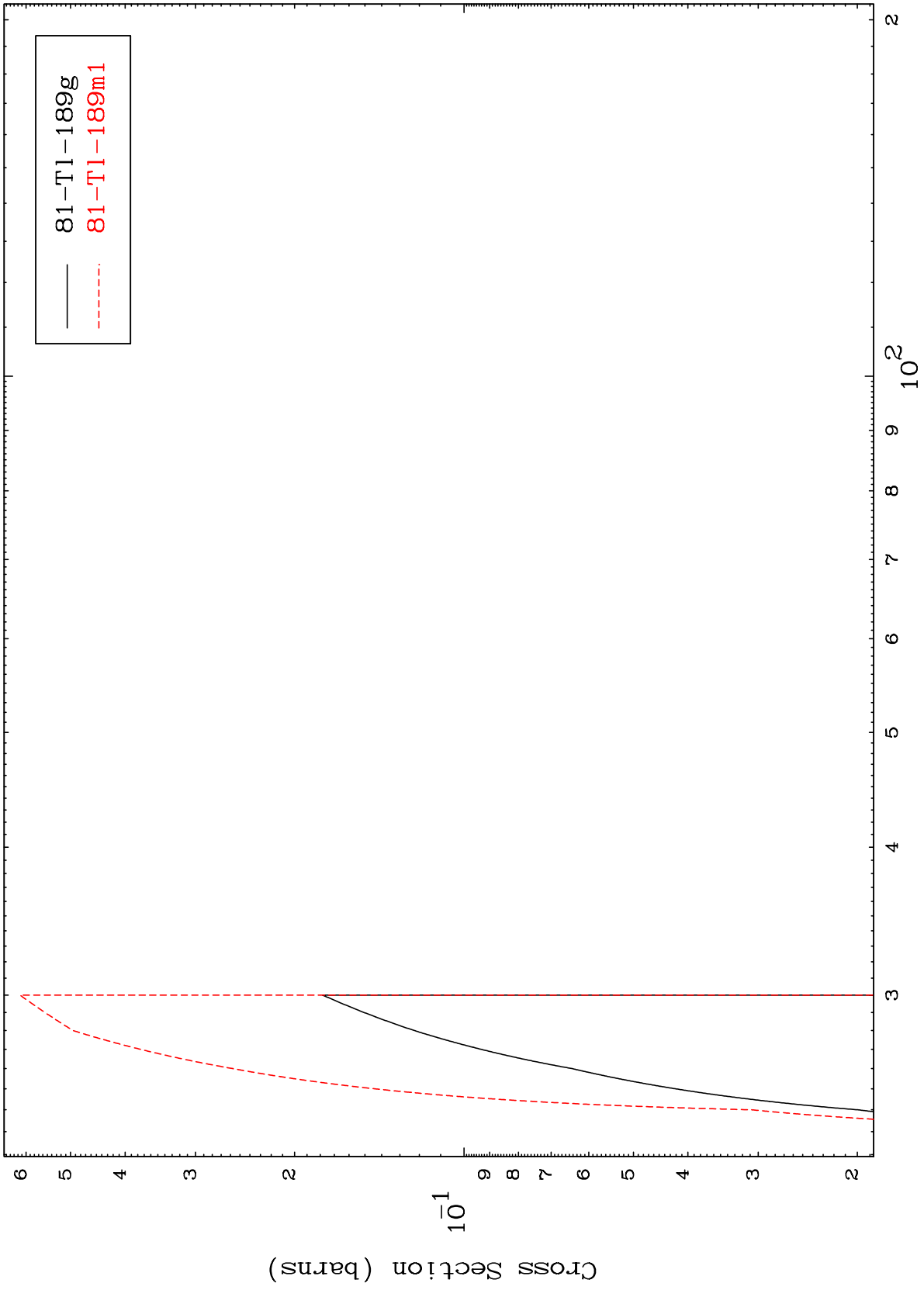
79-Au-187

MAT 7895

( $\alpha, 2n$ )

<sup>79</sup>Au-187

Radionuclide Production Cross Section



11

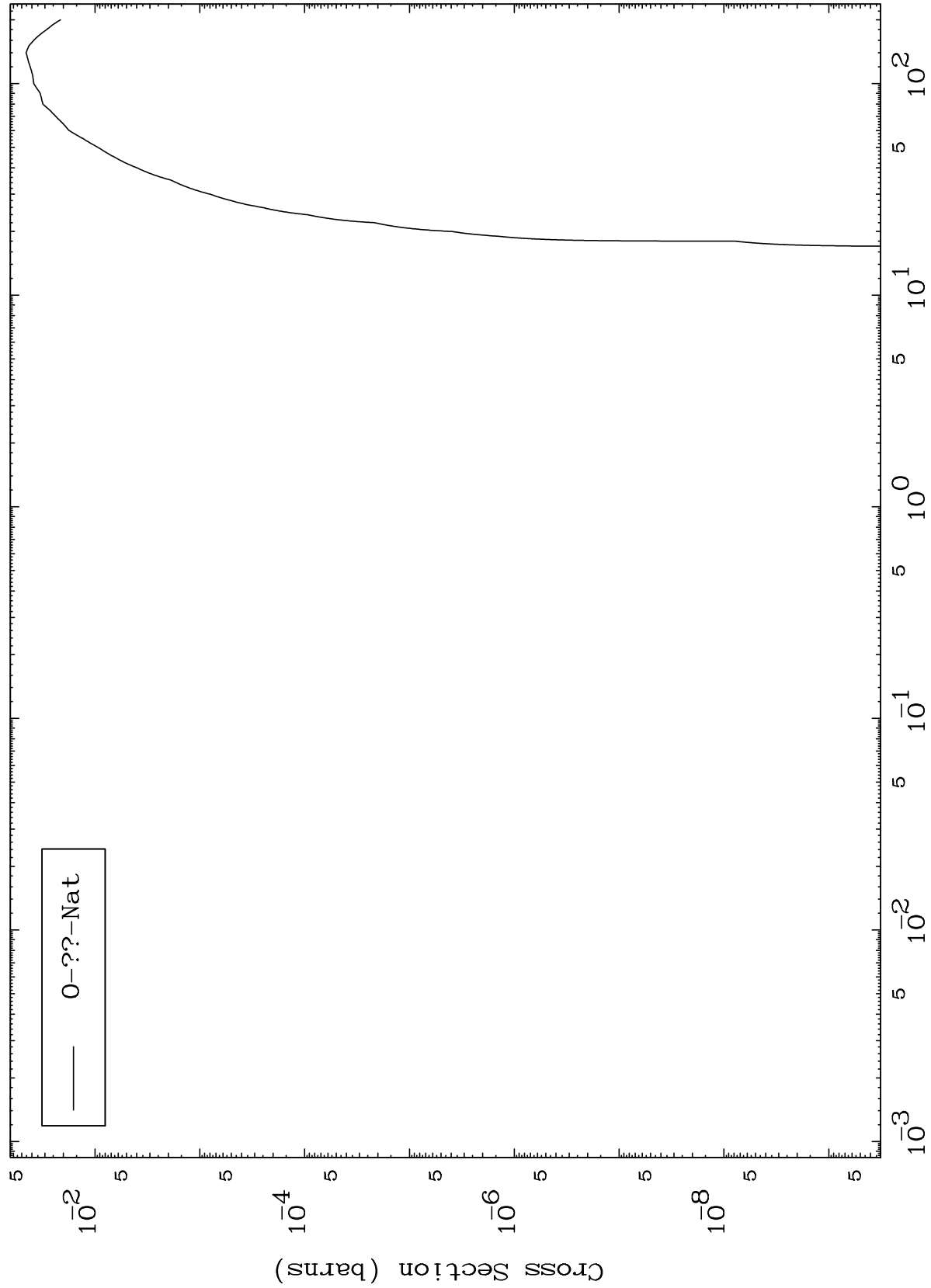
Incident Energy (MeV)

<sup>79</sup>Au-187

MAT 7895

$\alpha$  Fission  
Radionuclide Production Cross Section

79-Au-187



12

Incident Energy (MeV)

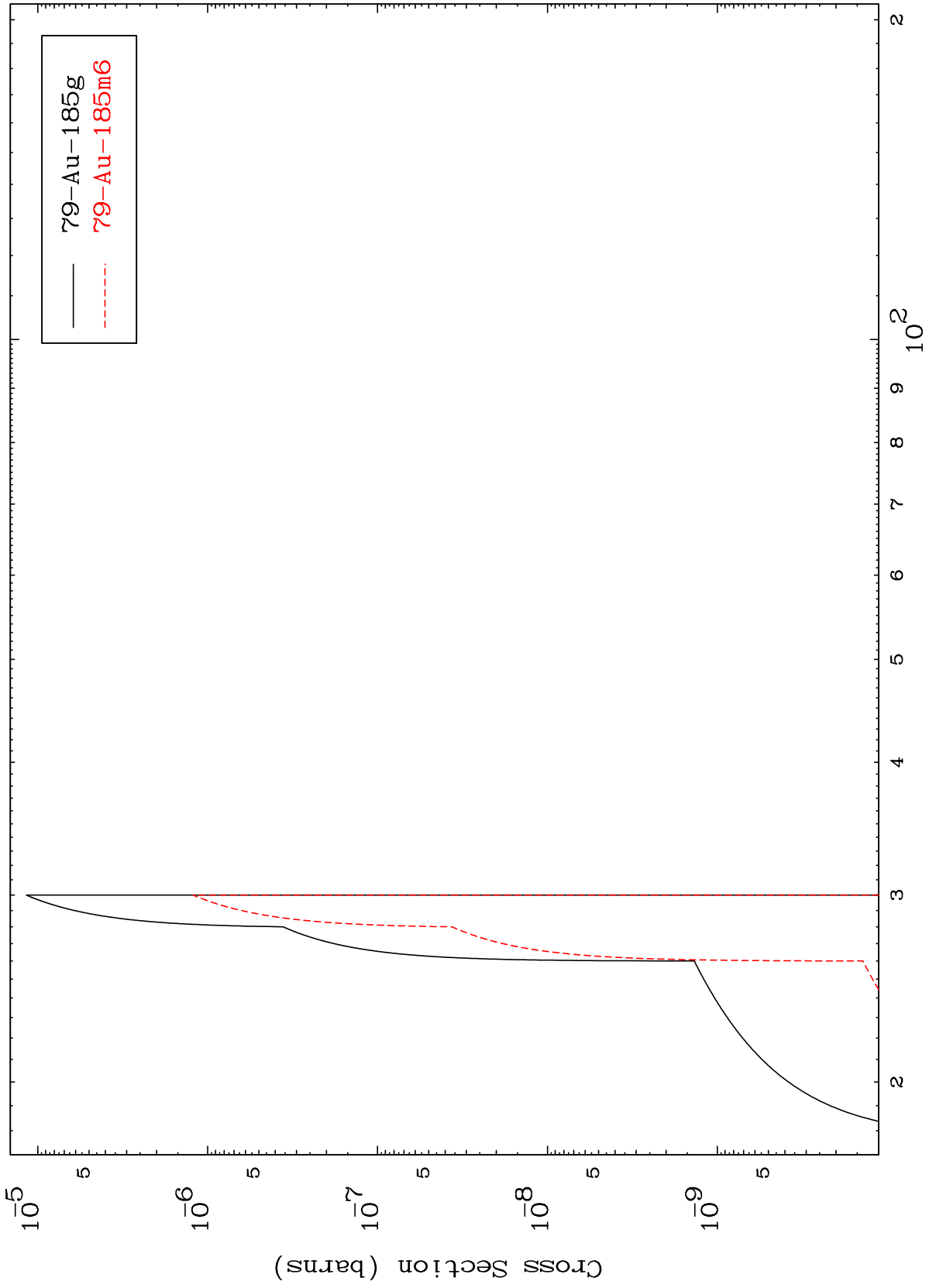
79-Au-187

MAT 7895

$(\alpha, 2n) \alpha$

$^{79}\text{Au-187}$

Radionuclide Production Cross Section



13

Incident Energy (MeV)

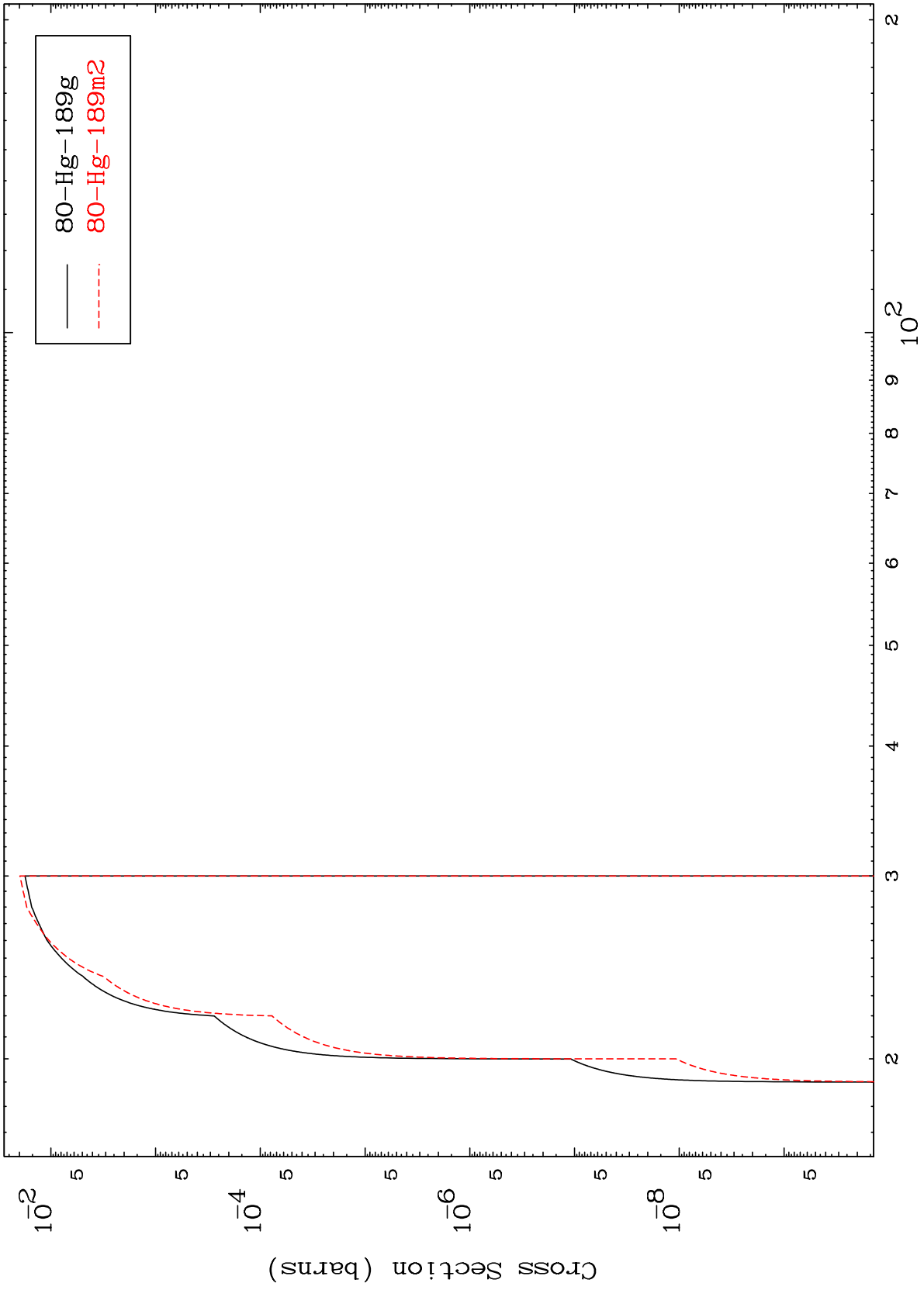
$^{79}\text{Au-187}$

MAT 7895

( $\alpha, n'$ ) p

79-Au-187

Radionuclide Production Cross Section



14

Incident Energy (MeV)

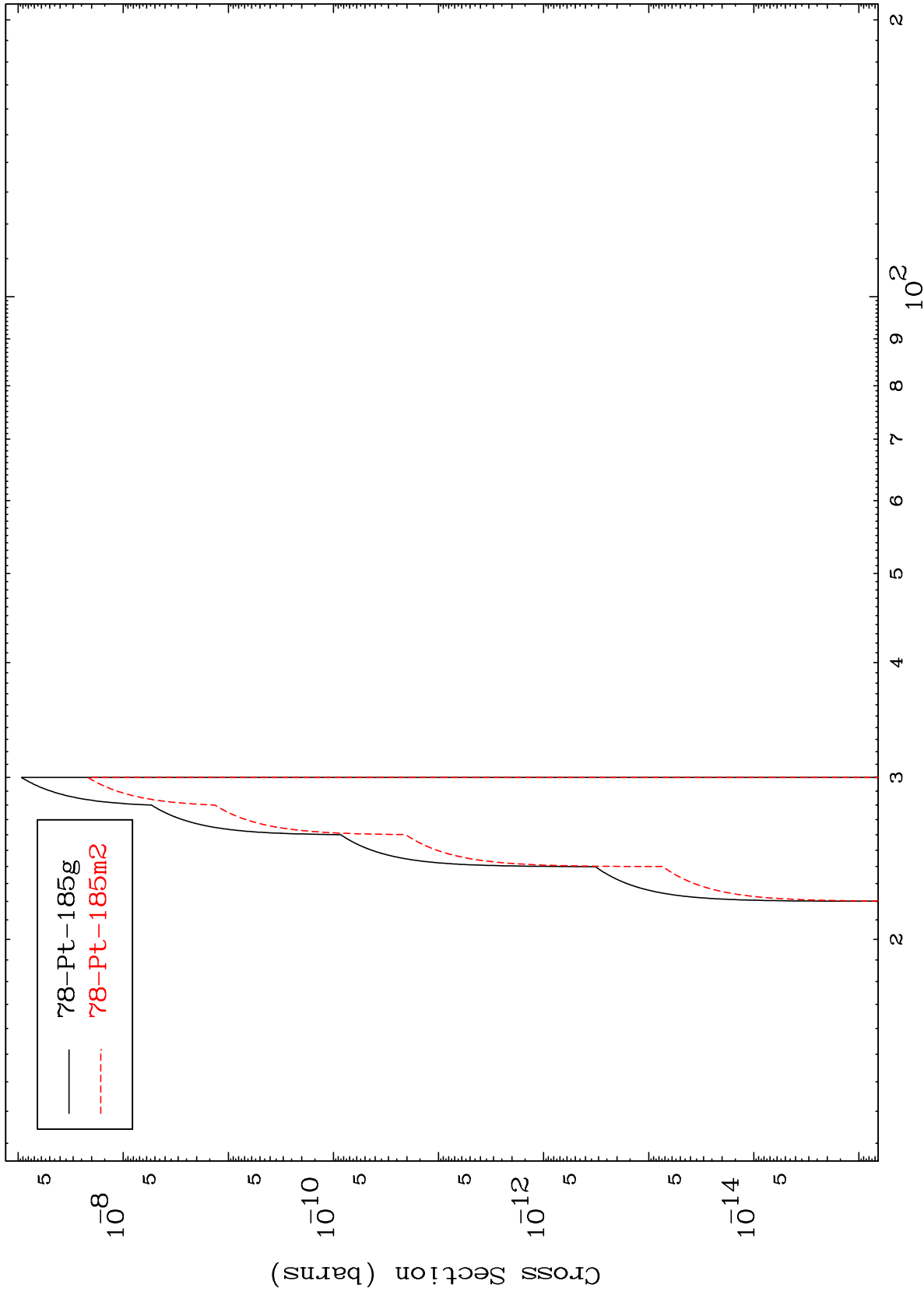
79-Au-187

MAT 7895

( $\alpha, n'$ ) p  $\alpha$

79-Au-187

Radionuclide Production Cross Section



15

Incident Energy (MeV)

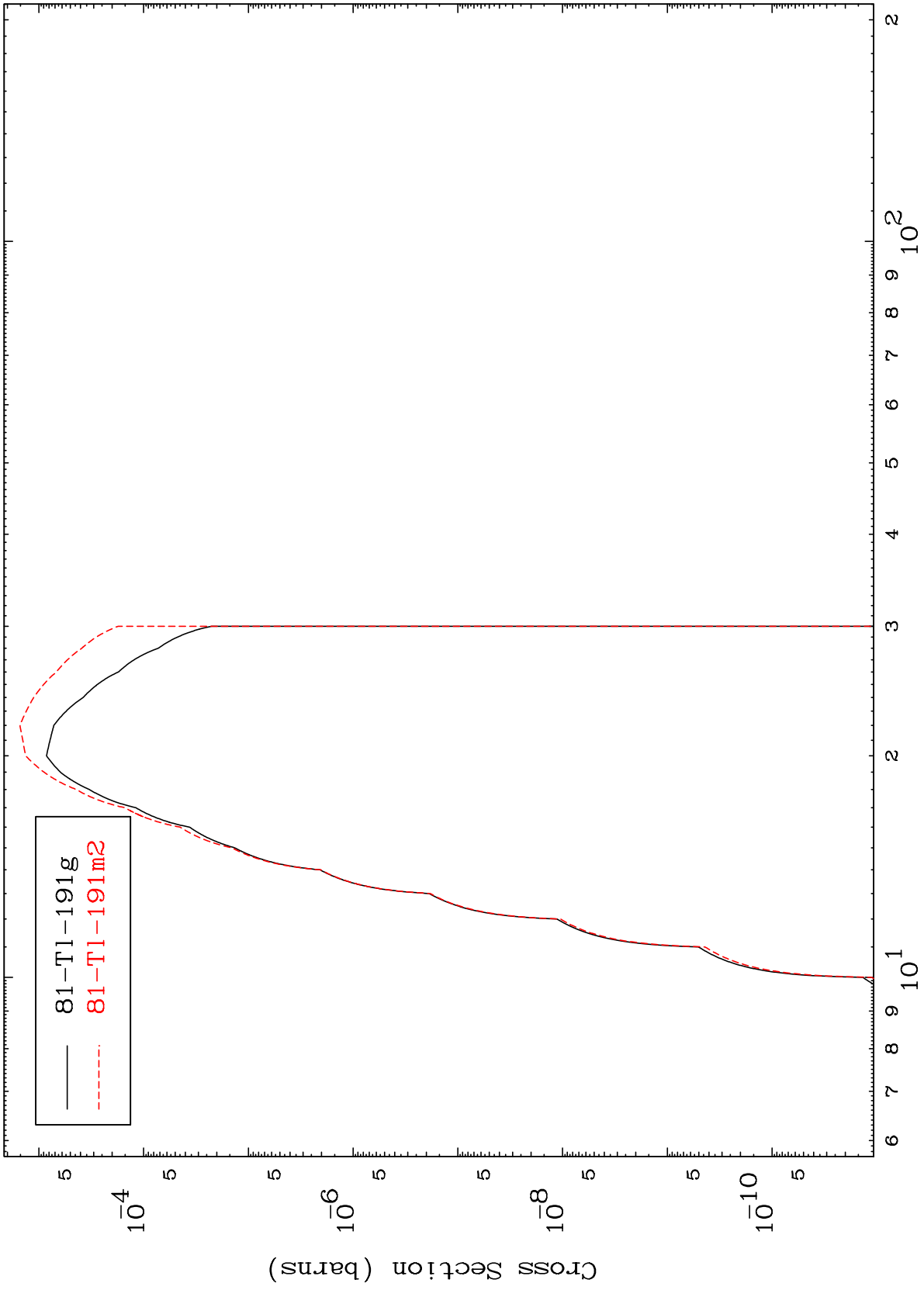
79-Au-187



MAT 7895

79-Au-187

Radionuclide Production Cross Section  
( $\alpha, \gamma$ )



16

Incident Energy (MeV)

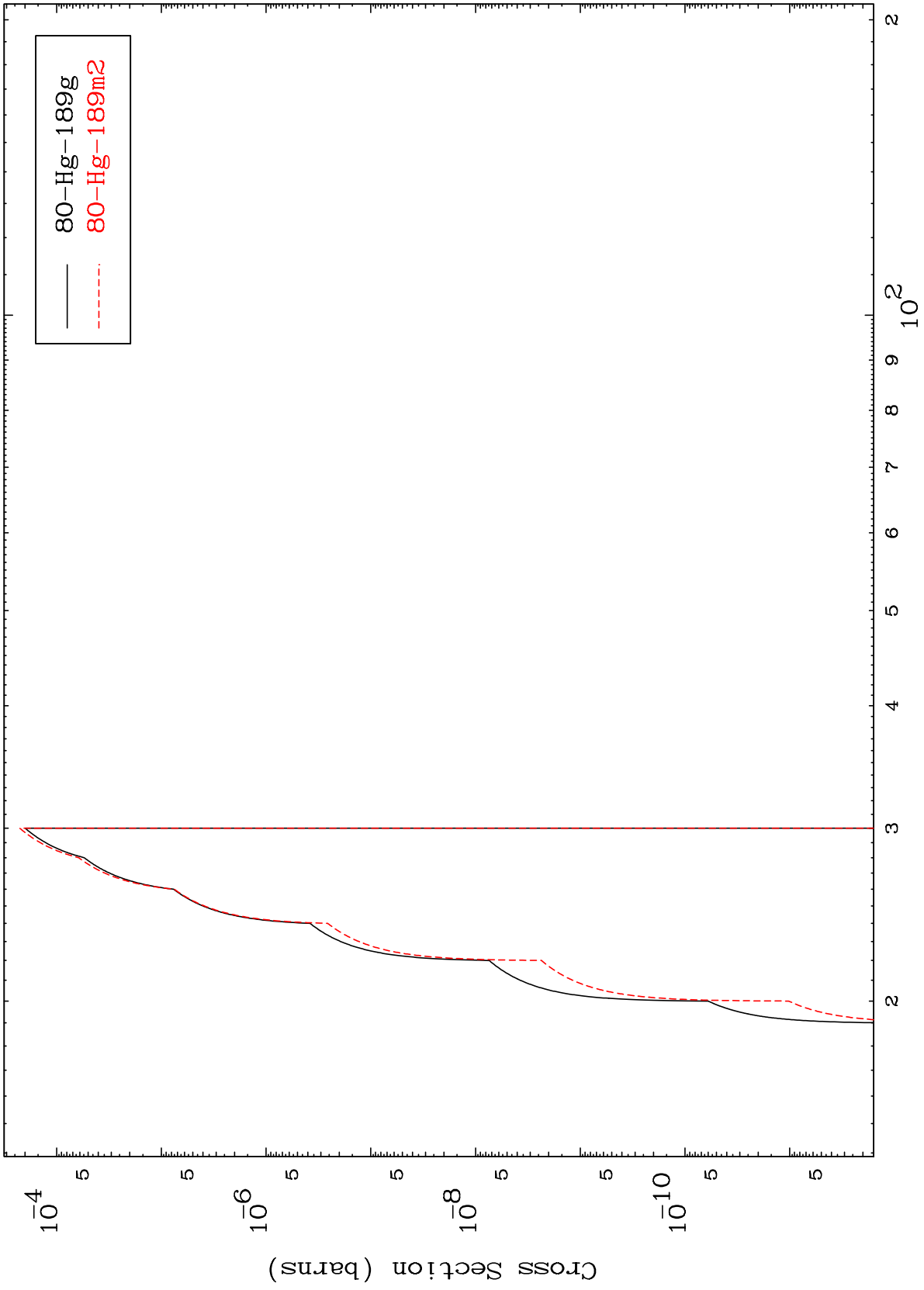
79-Au-187

MAT 7895

( $\alpha, d$ )

79-Au-187

Radionuclide Production Cross Section



17

Incident Energy (MeV)

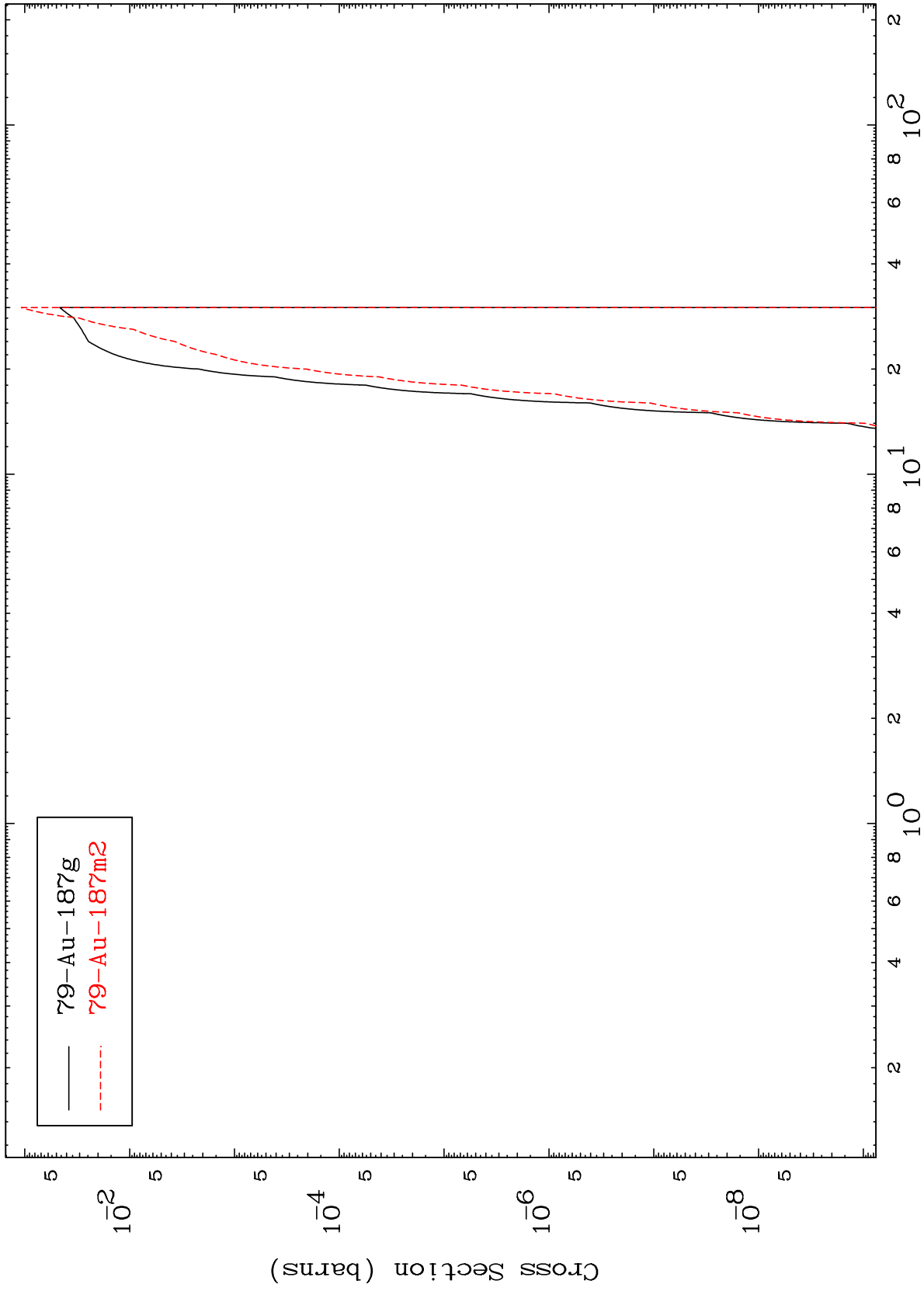
79-Au-187

MAT 7895

( $\alpha, \alpha$ )

<sup>79</sup>Au-187

Radionuclide Production Cross Section



18

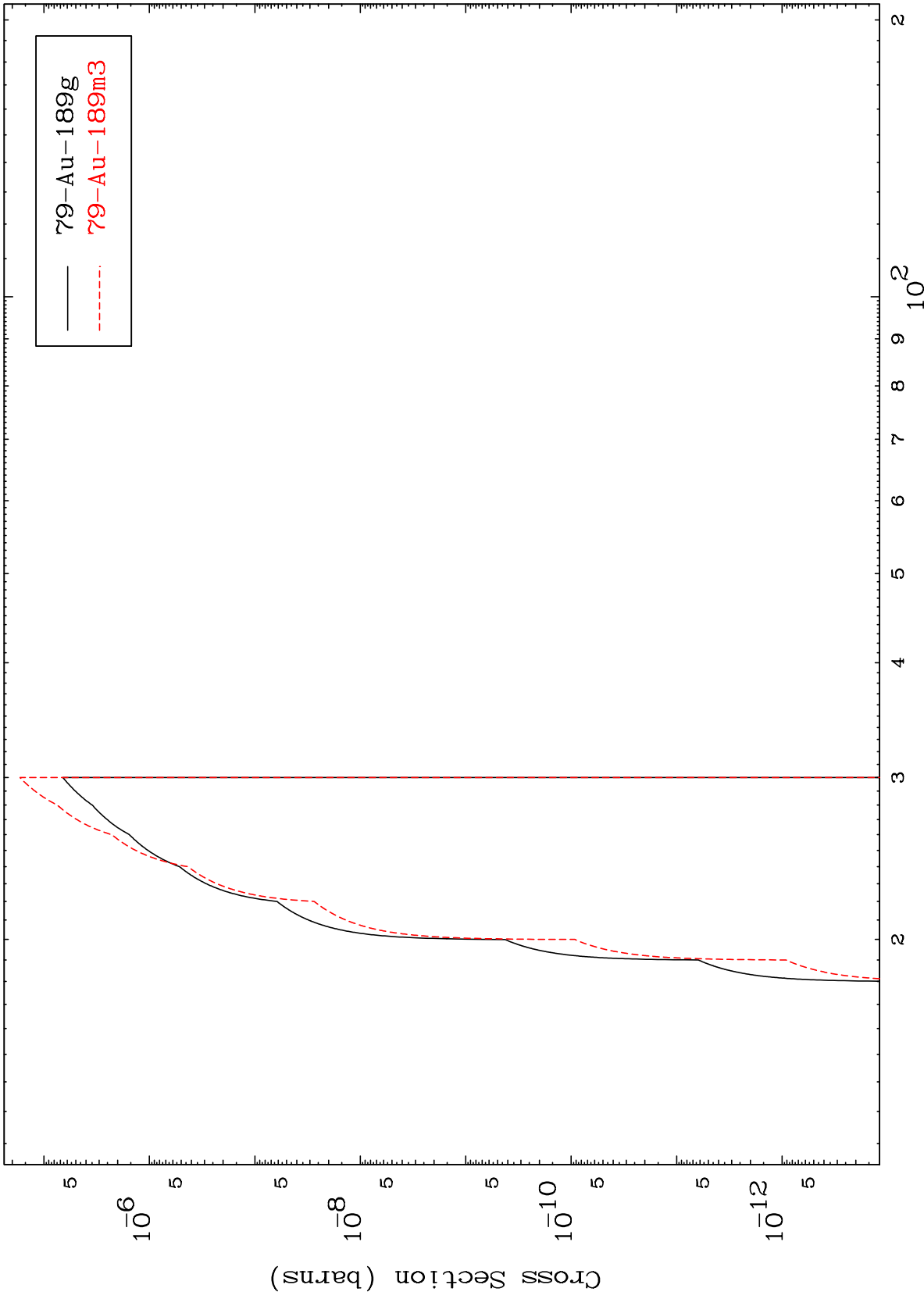
Incident Energy (MeV)

<sup>79</sup>Au-187

MAT 7895

<sup>79</sup>Au-187

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



19

Incident Energy (MeV)

<sup>79</sup>Au-187