

Program EVALPLOT
(Version 2018-1)

by

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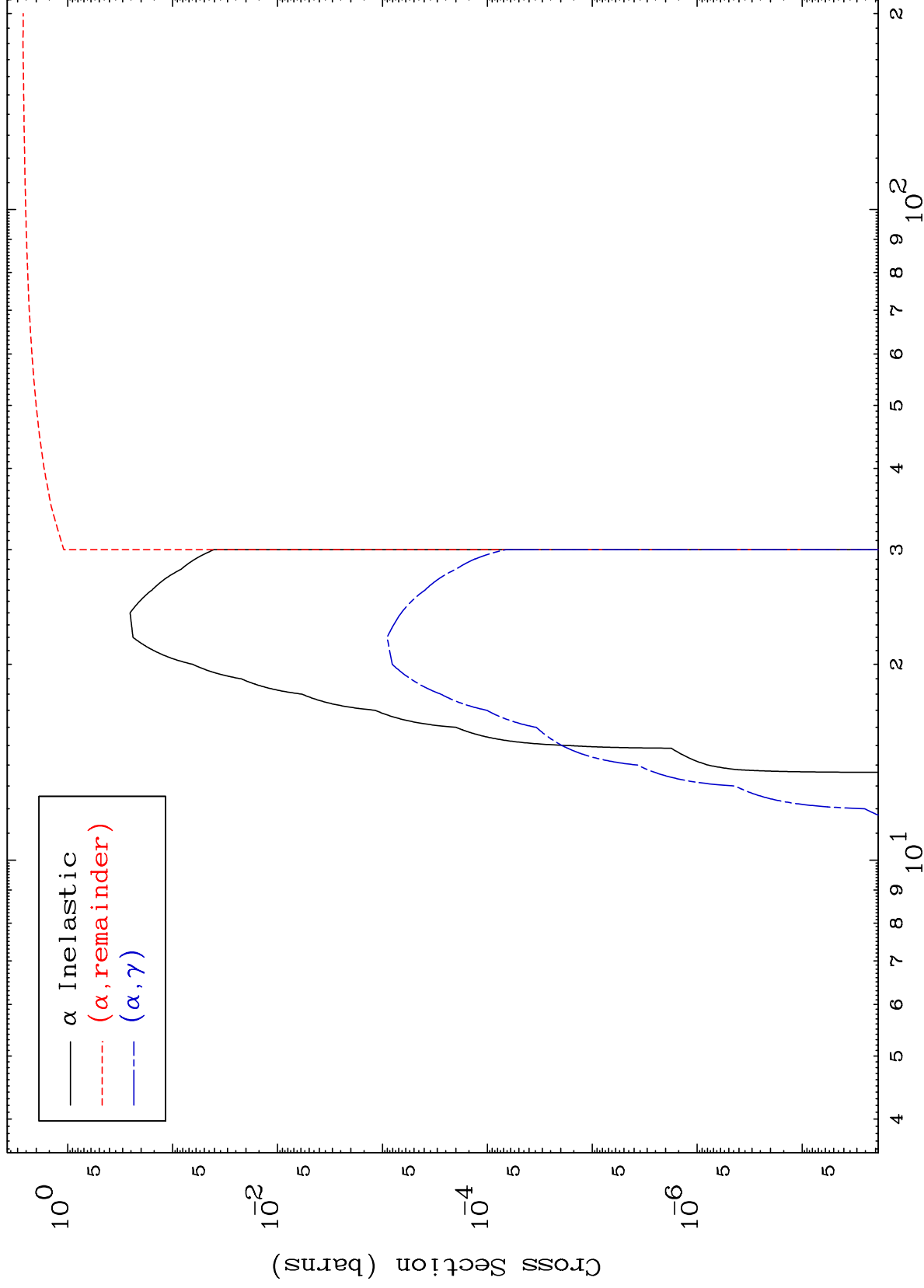
Press Mouse Button to Start

MAT 7901

α Major

79-Au-189

0 Kelvin Cross Sections



1

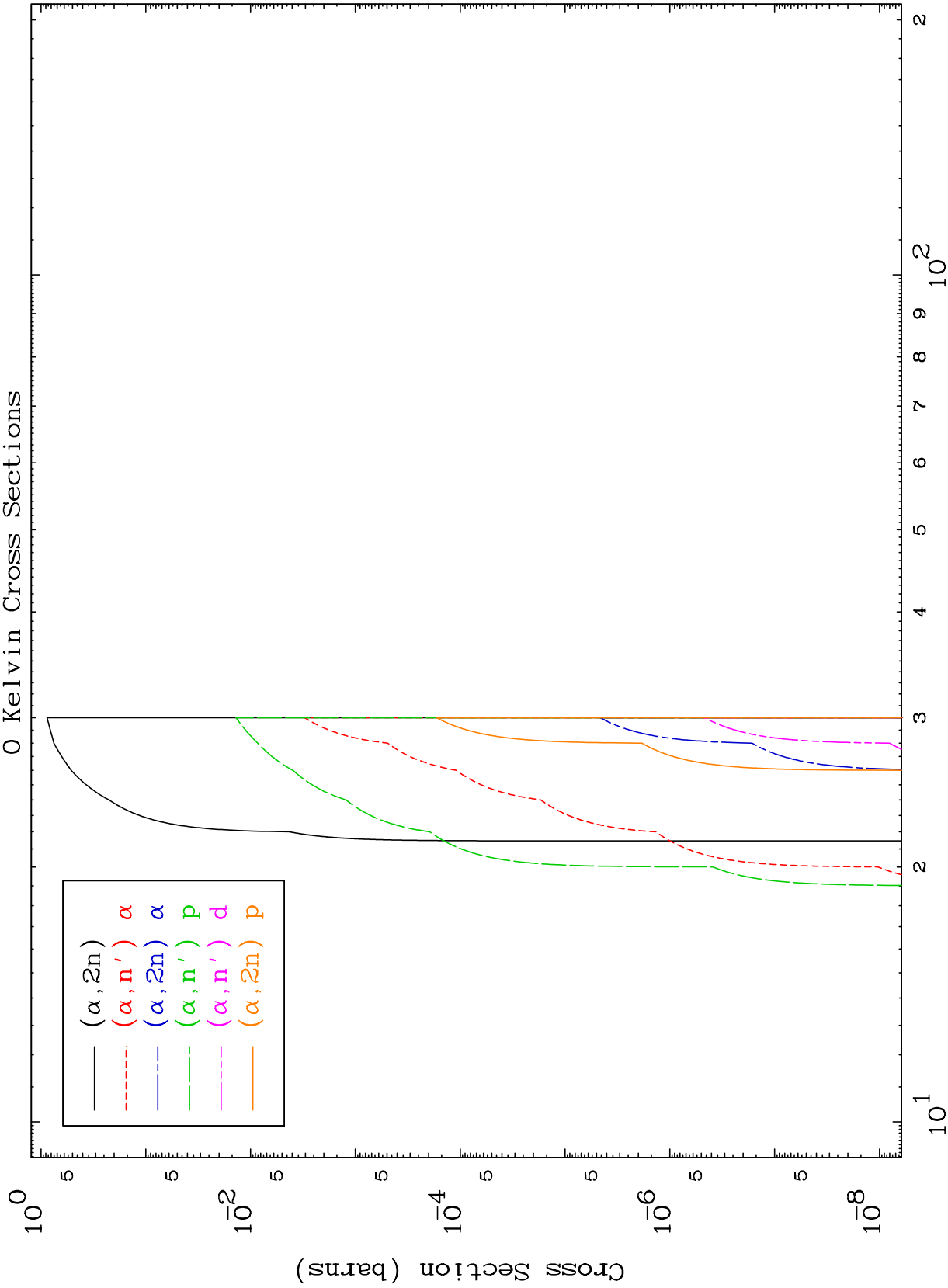
Incident Energy (MeV)

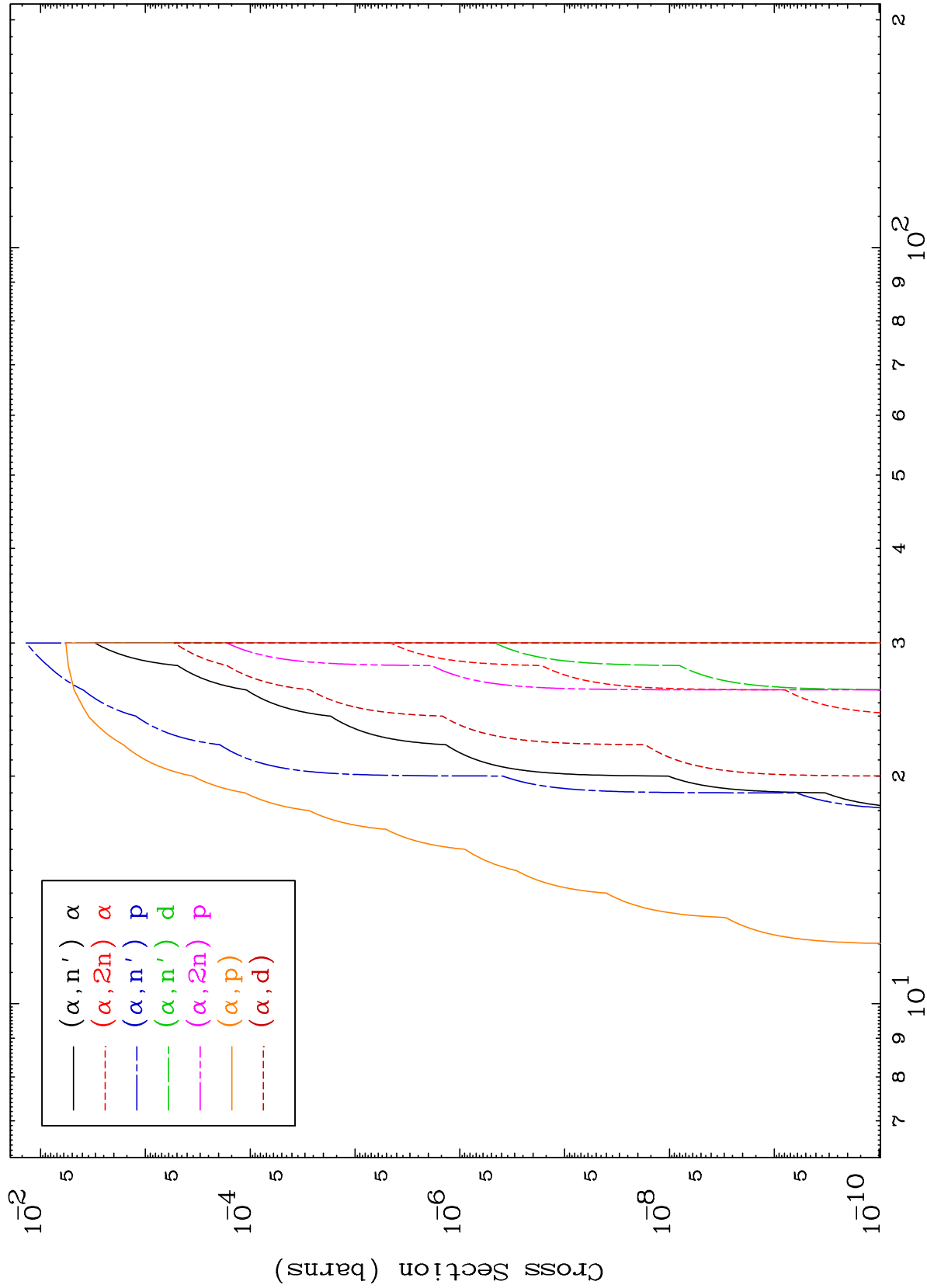
79-Au-189

MAT 7901

α Neutron Production
0 Kelvin Cross Sections

79-Au-189

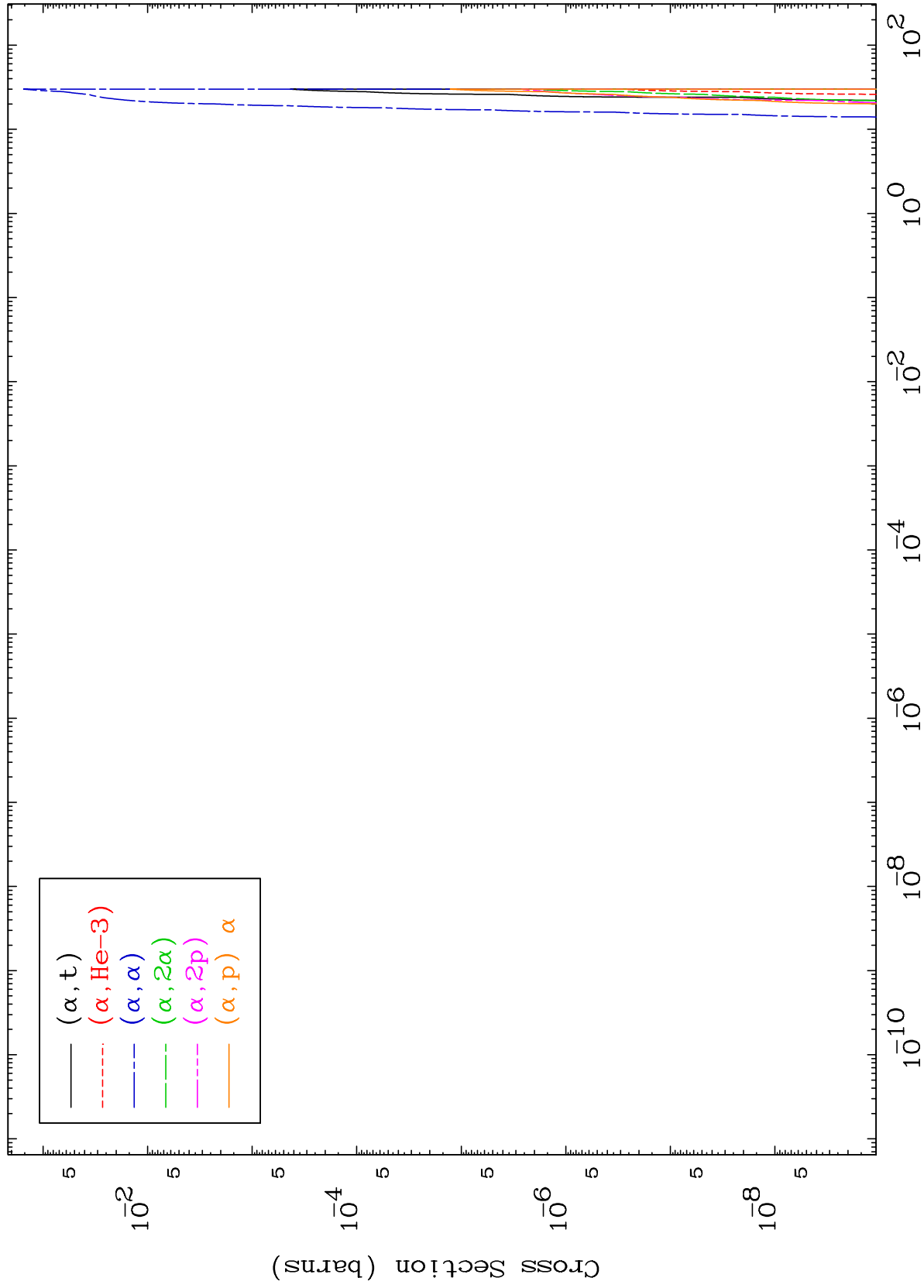




MAT 7901

α Charged Particle
0 Kelvin Cross Sections

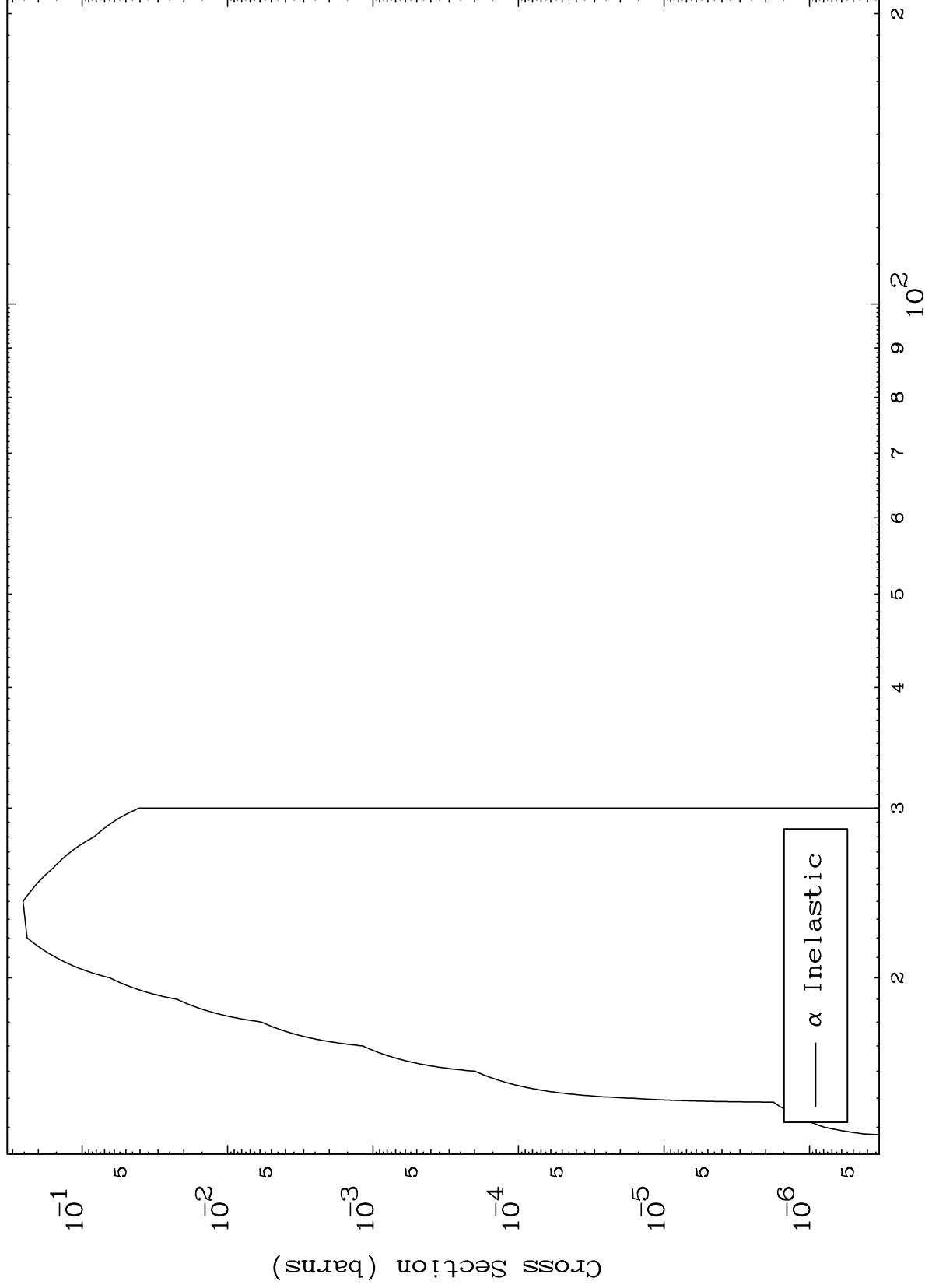
79-Au-189



MAT 7901

(α, n') Level
0 Kelvin Cross Sections

79-Au-189



5

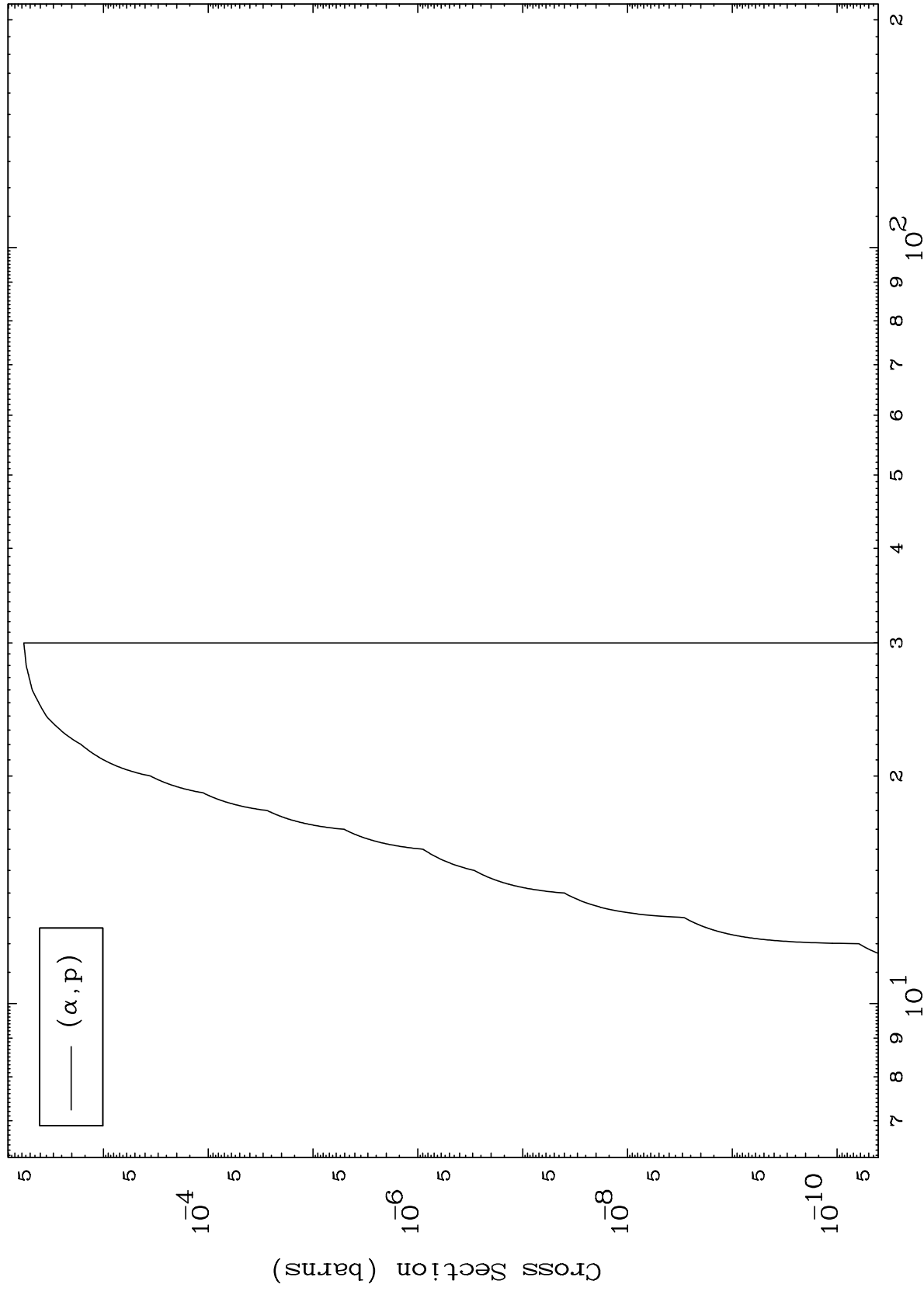
Incident Energy (MeV)

79-Au-189

MAT 7901

79-Au-189

(α ,p) Levels
0 Kelvin Cross Sections



6

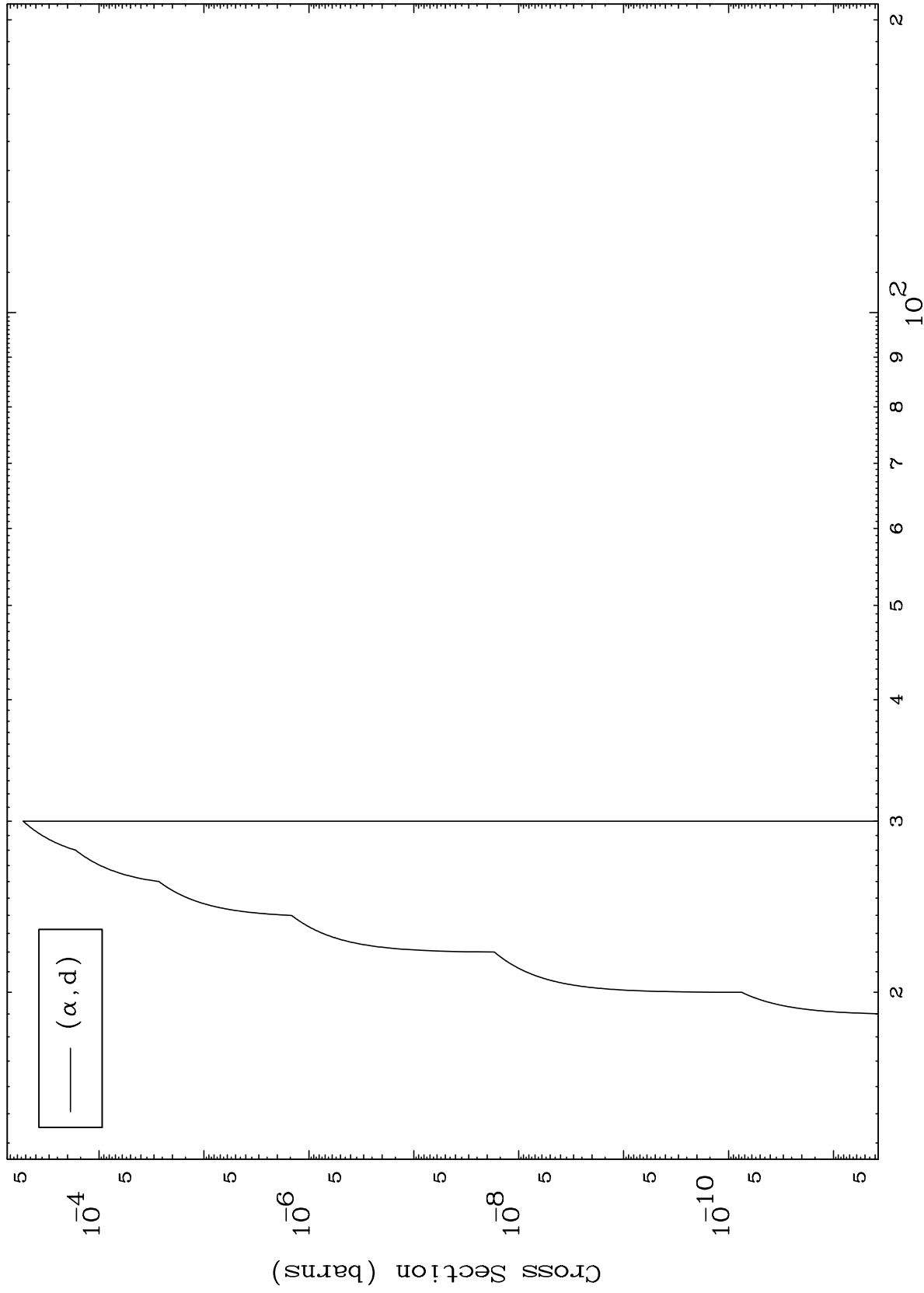
Incident Energy (MeV)

79-Au-189

MAT 7901

79-Au-189

(α ,d) Levels
0 Kelvin Cross Sections



7

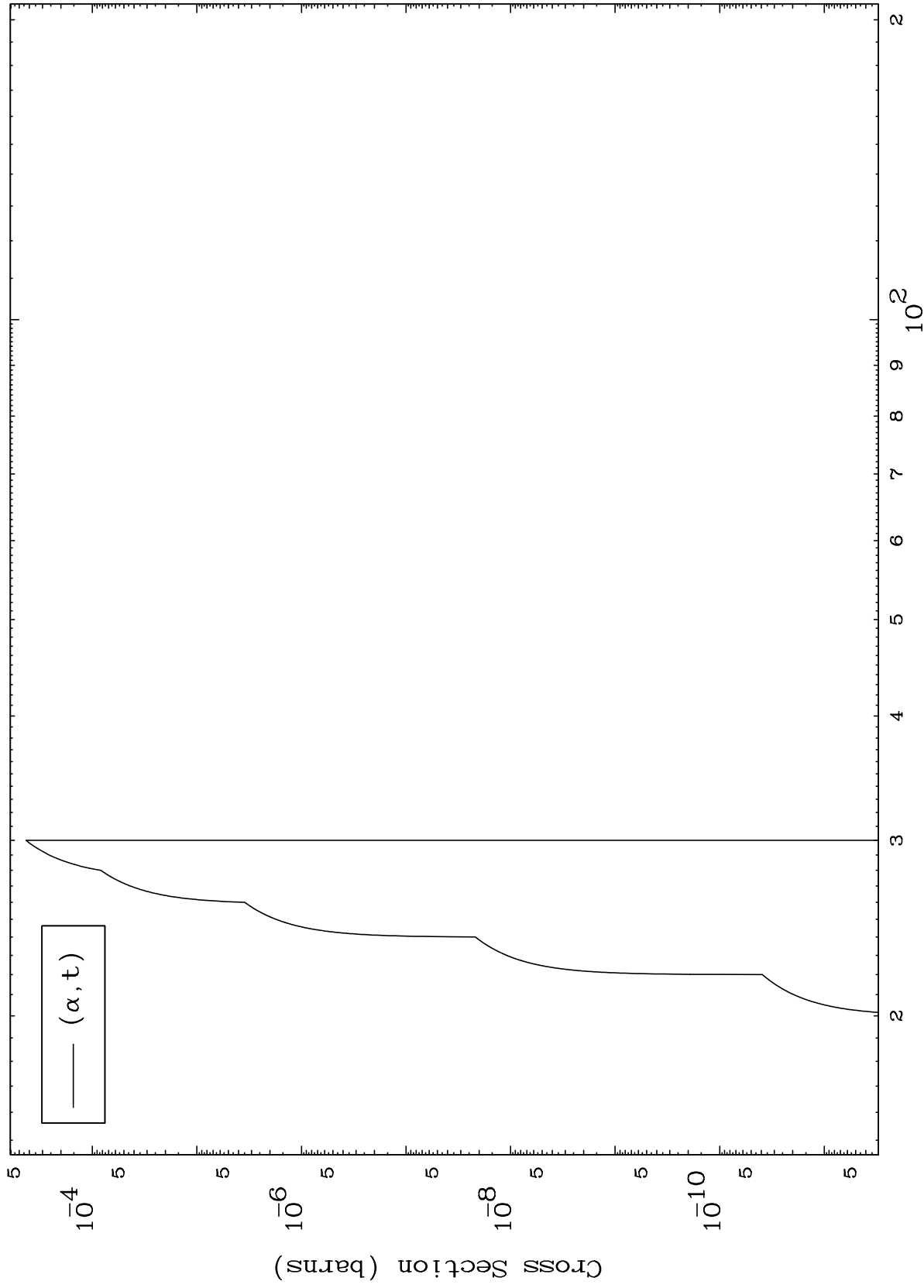
Incident Energy (MeV)

79-Au-189

MAT 7901

79-Au-189

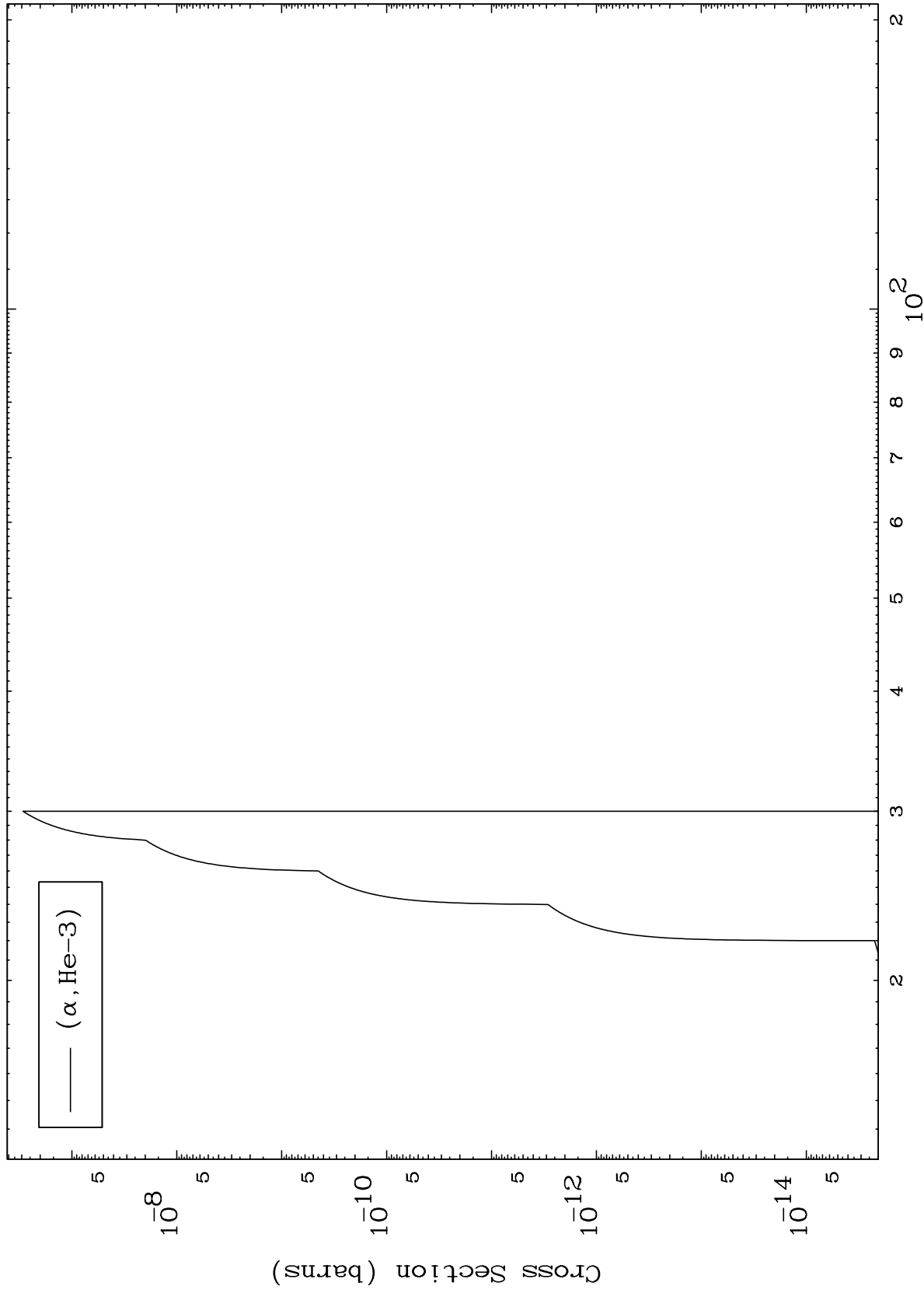
(α, t) Levels
0 Kelvin Cross Sections



8

79-Au-189

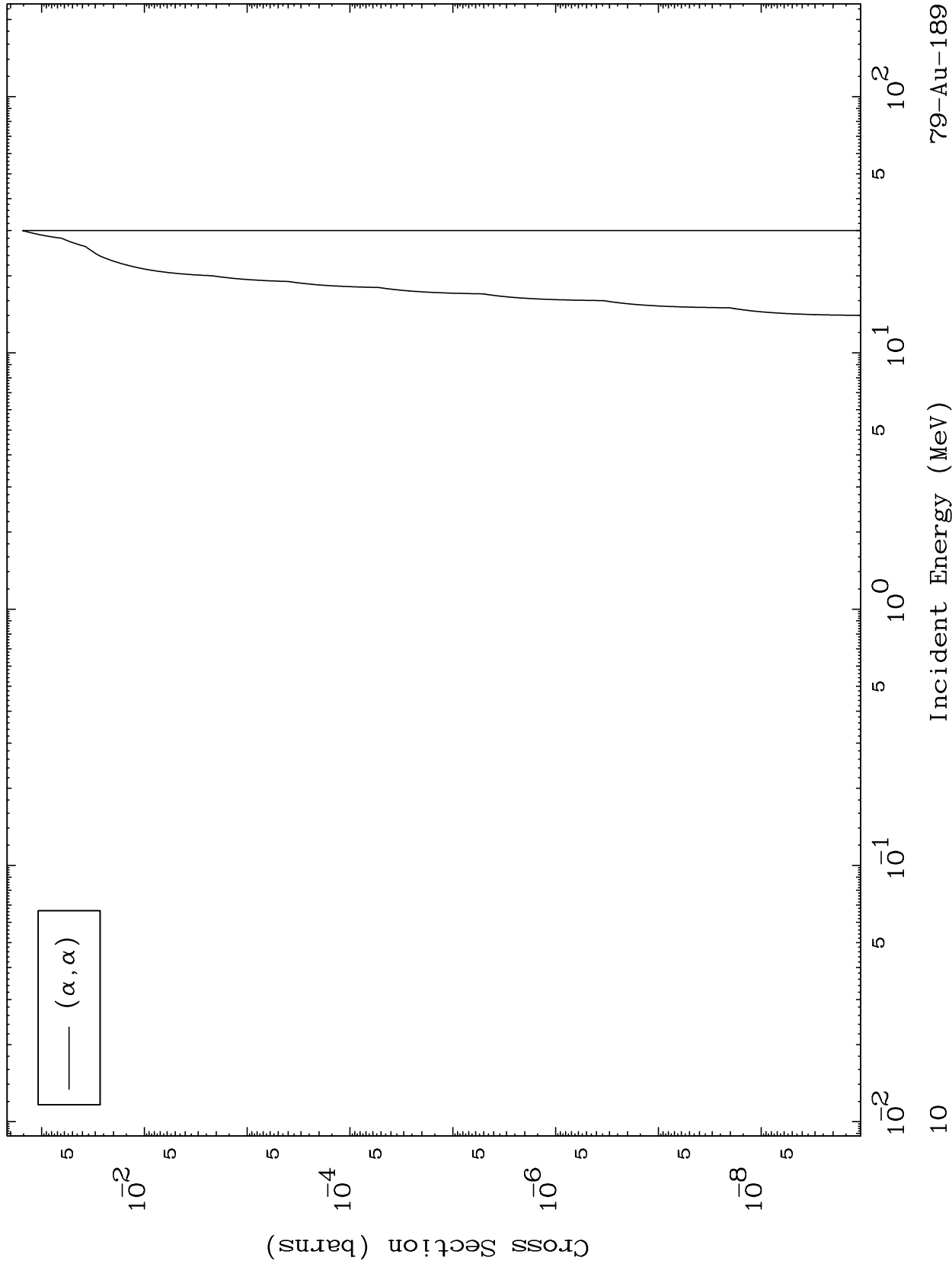
(α ,He3) Levels
0 Kelvin Cross Sections

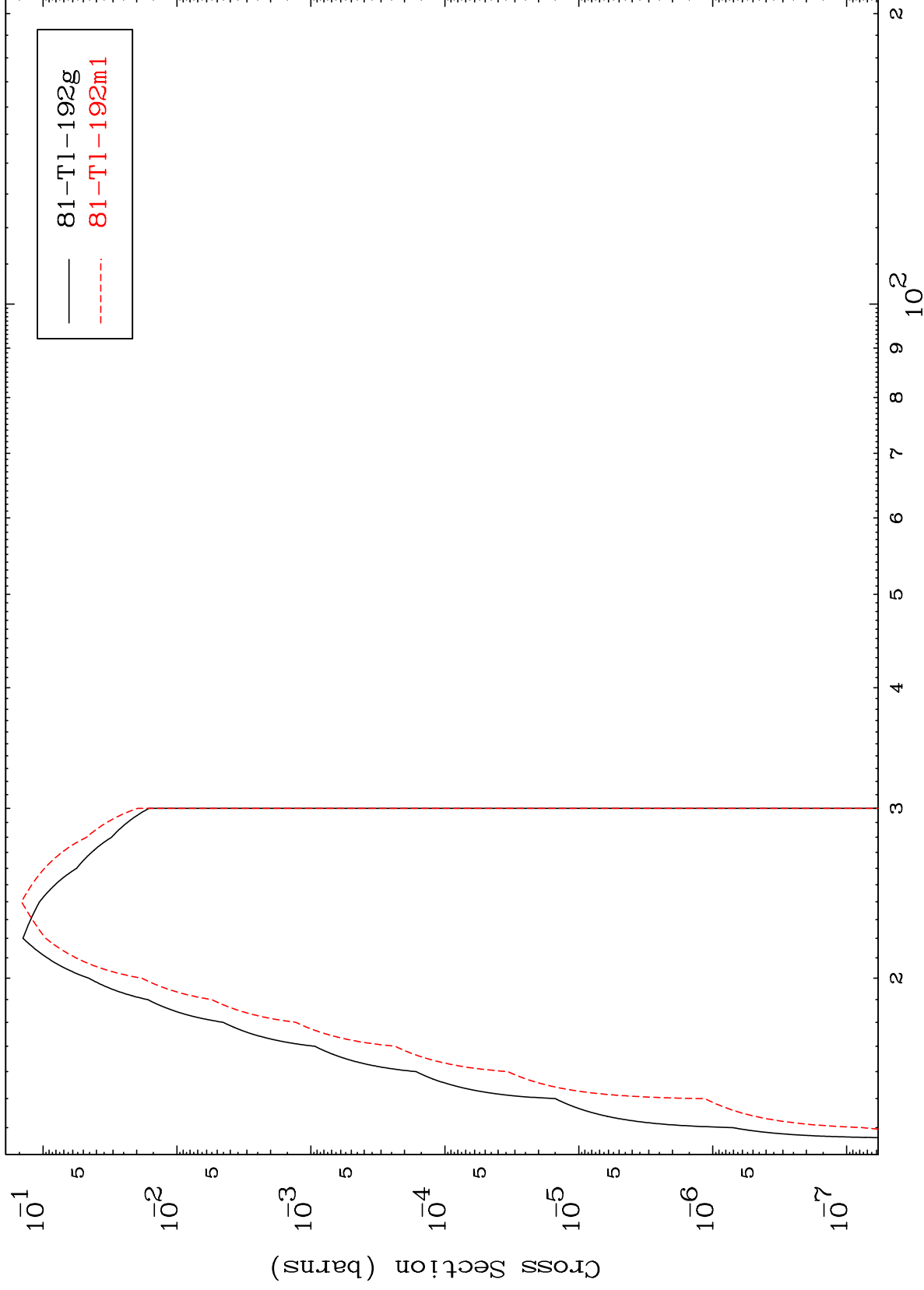


MAT 7901

(α, α) Levels
0 Kelvin Cross Sections

79-Au-189

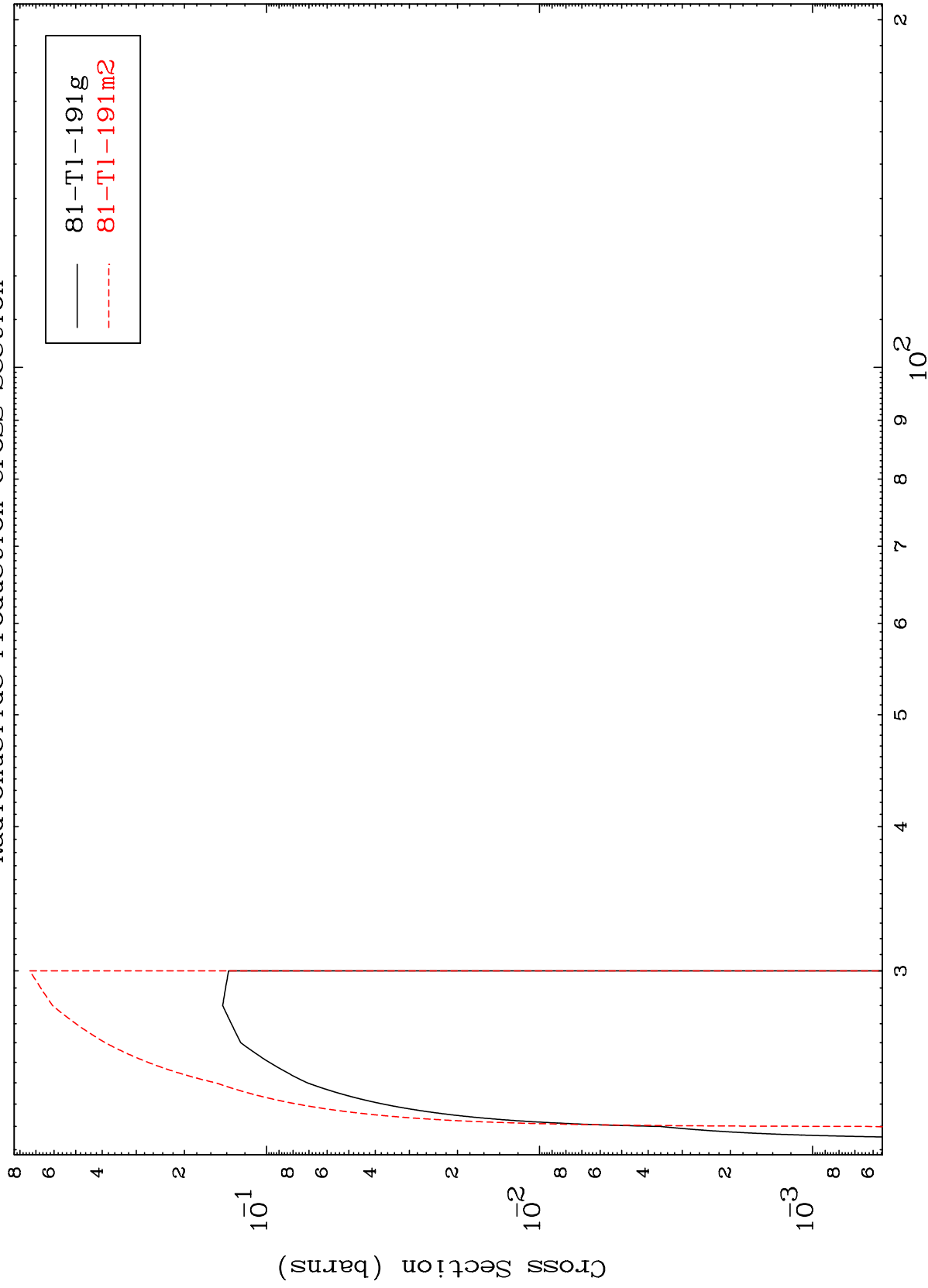


α Inelastic
Radionuclide Production Cross Section

MAT 7901

79-Au-189

$(\alpha, 2n)$
Radionuclide Production Cross Section



12

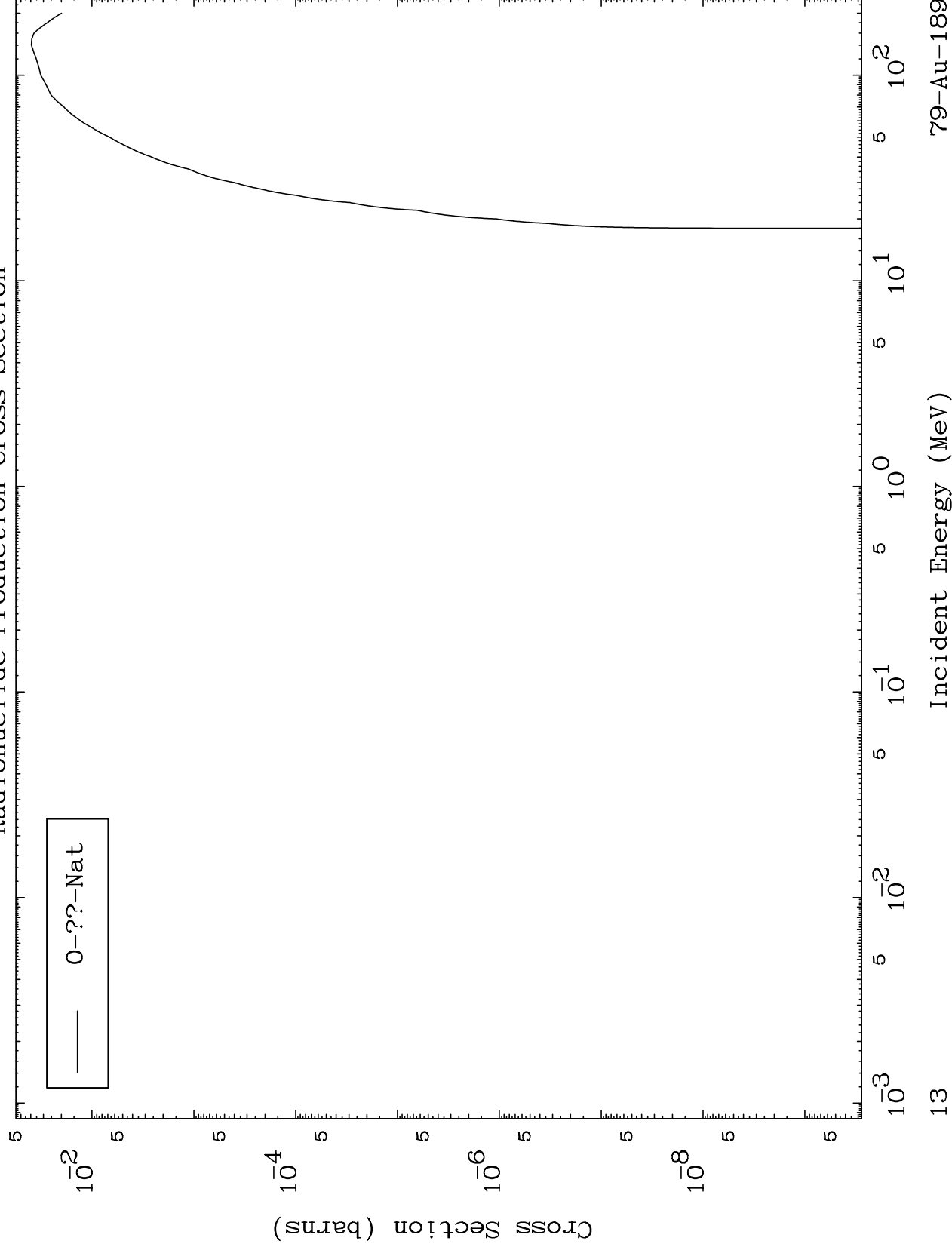
Incident Energy (MeV)

79-Au-189

MAT 7901

α Fission
Radionuclide Production Cross Section

79-Au-189

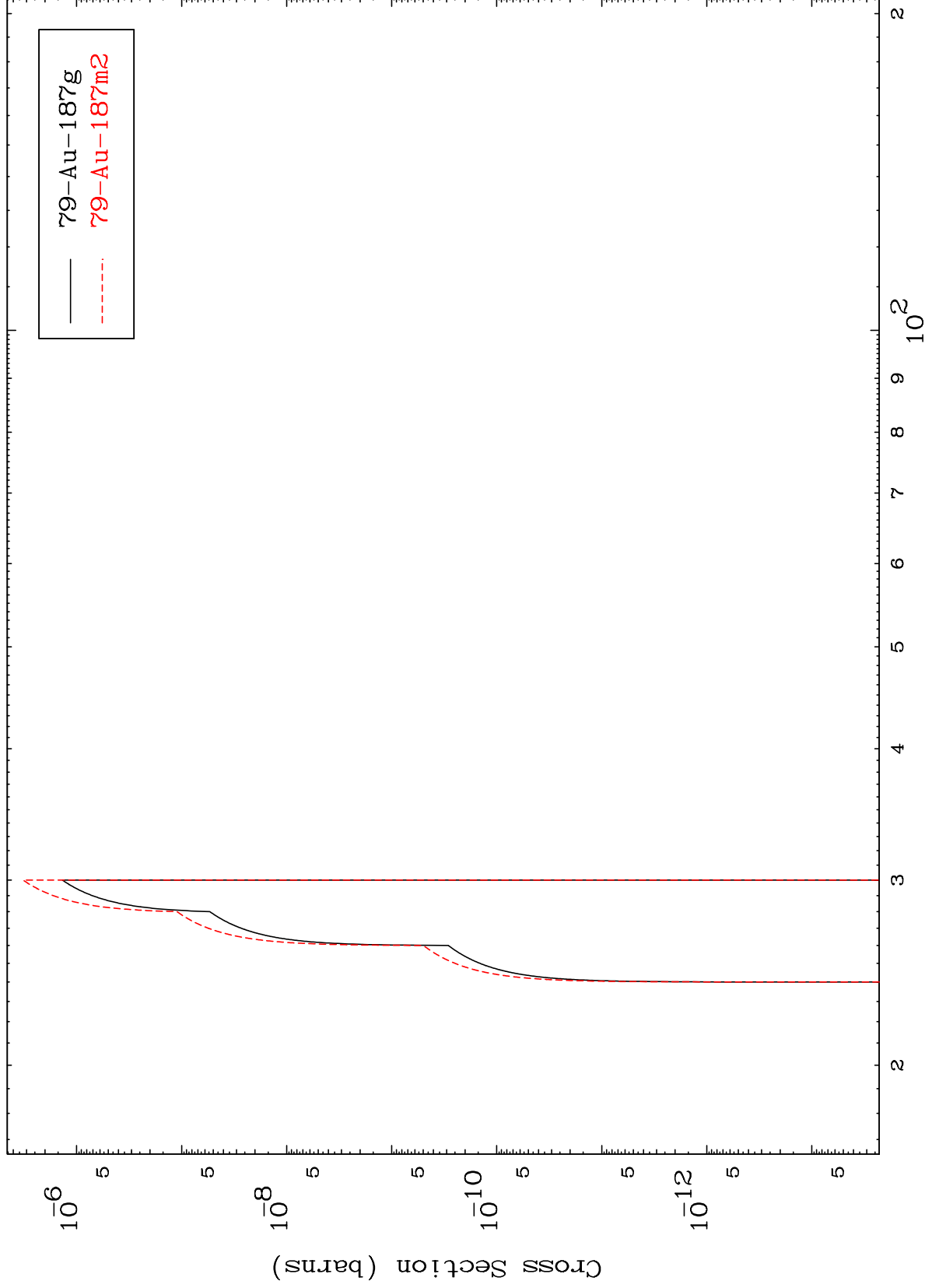


MAT 7901

$(\alpha, 2n) \alpha$

$^{79}\text{Au-189}$

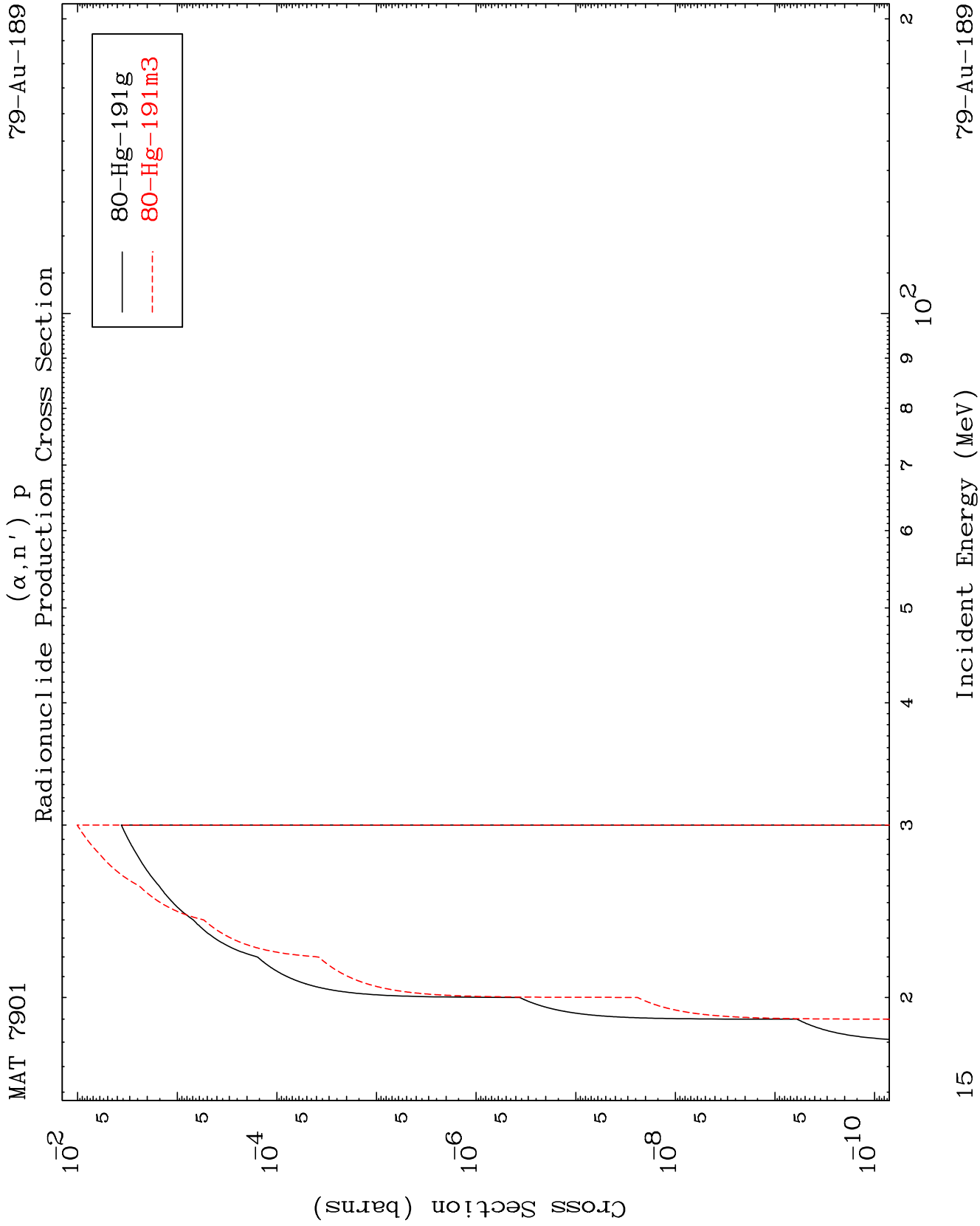
Radionuclide Production Cross Section



14

Incident Energy (MeV)

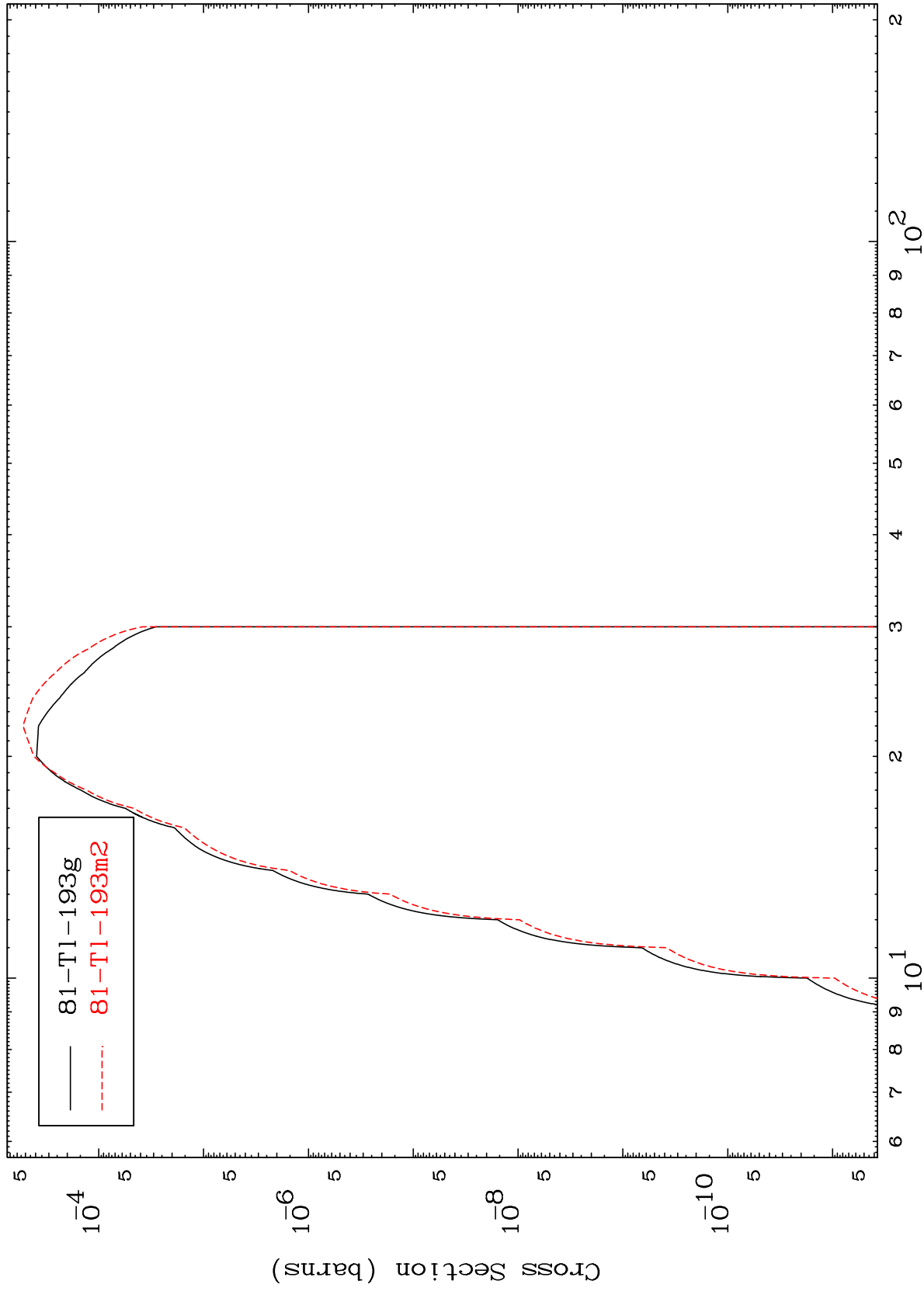
$^{79}\text{Au-189}$



MAT 7901

79-Au-189

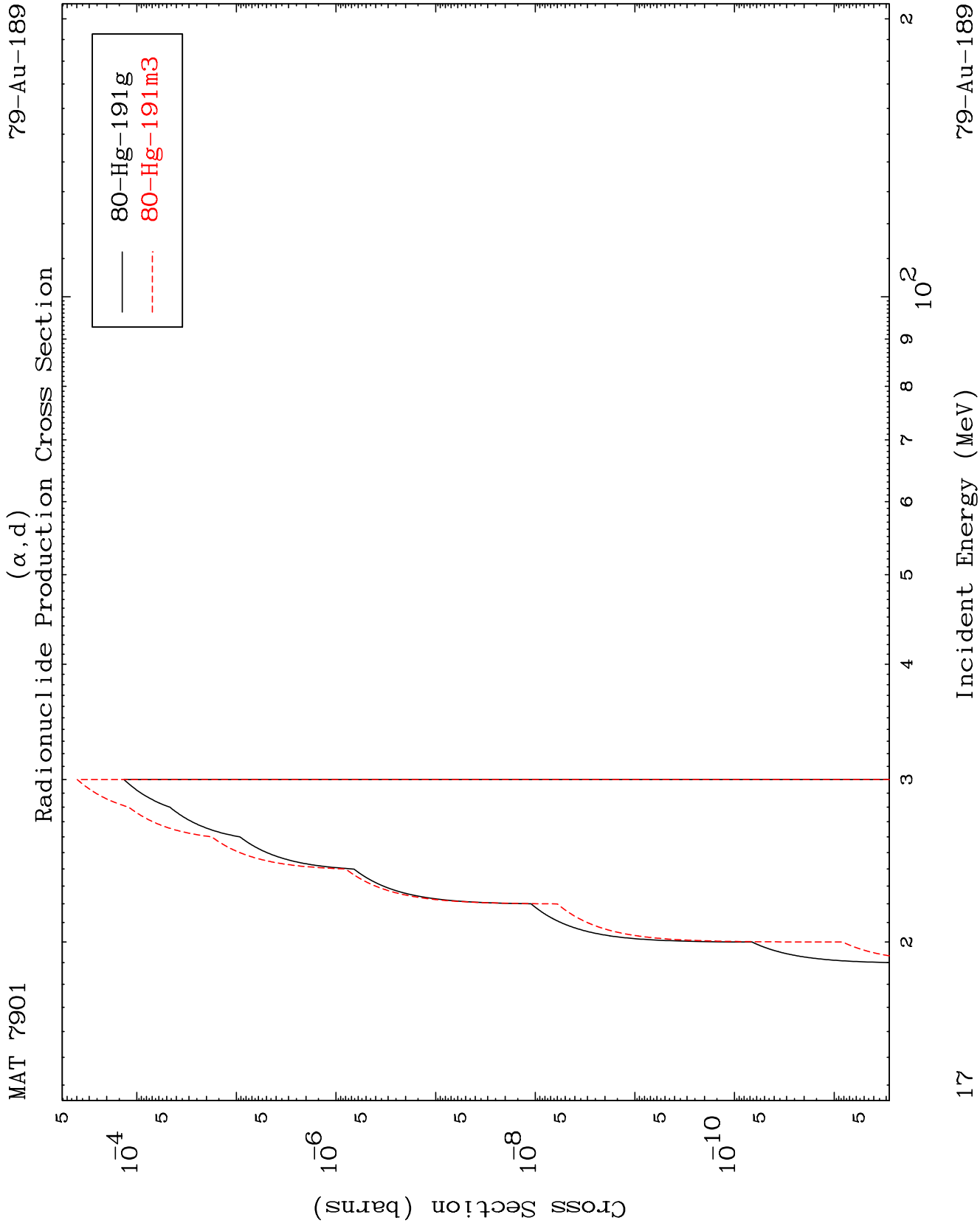
(α, γ)
Radionuclide Production Cross Section



16

Incident Energy (MeV)

79-Au-189

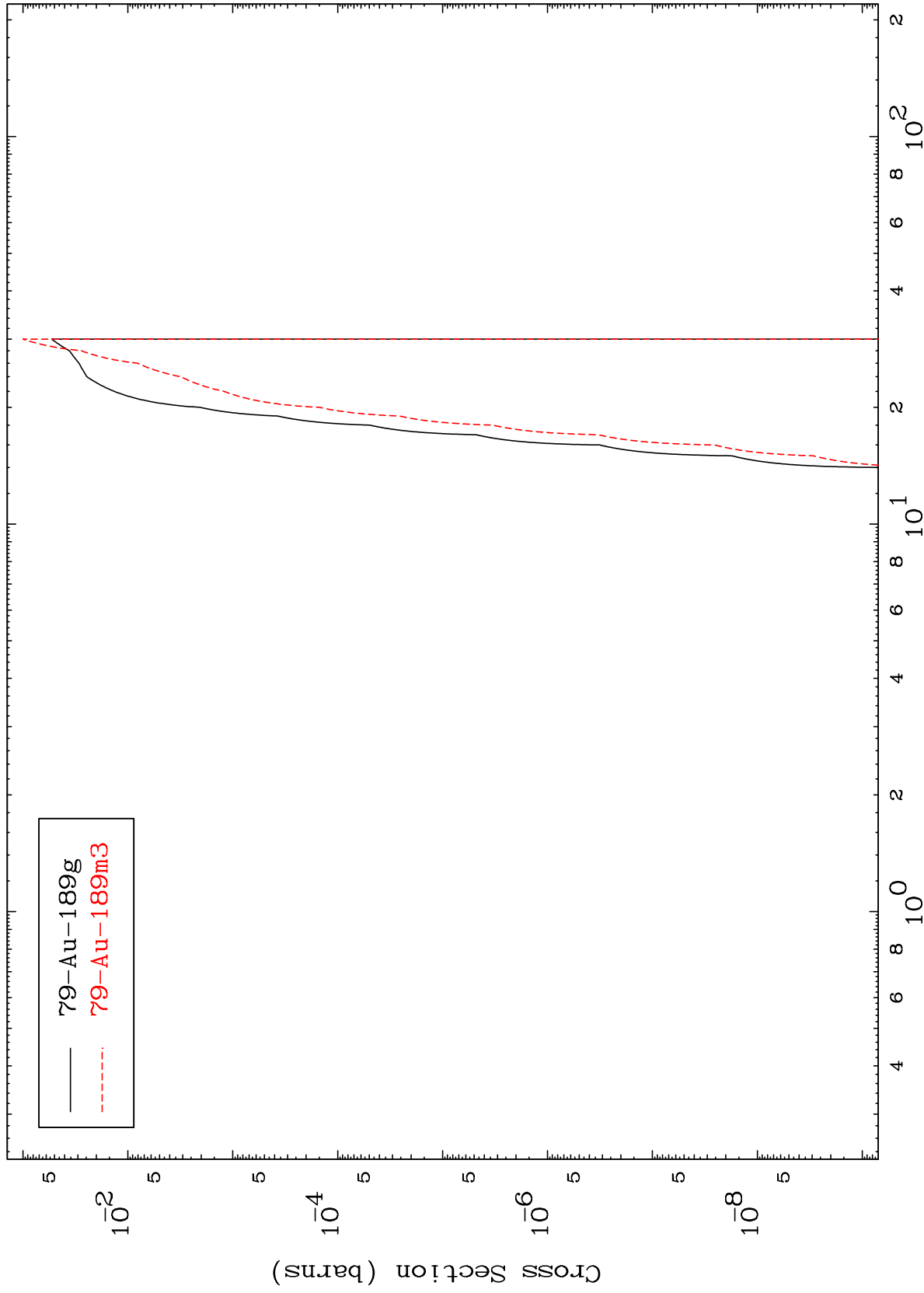


MAT 7901

(α, α)

$^{79}\text{Au-189}$

Radionuclide Production Cross Section



18

Incident Energy (MeV)

$^{79}\text{Au-189}$

($\alpha, 2p$)
Radionuclide Production Cross Section

