

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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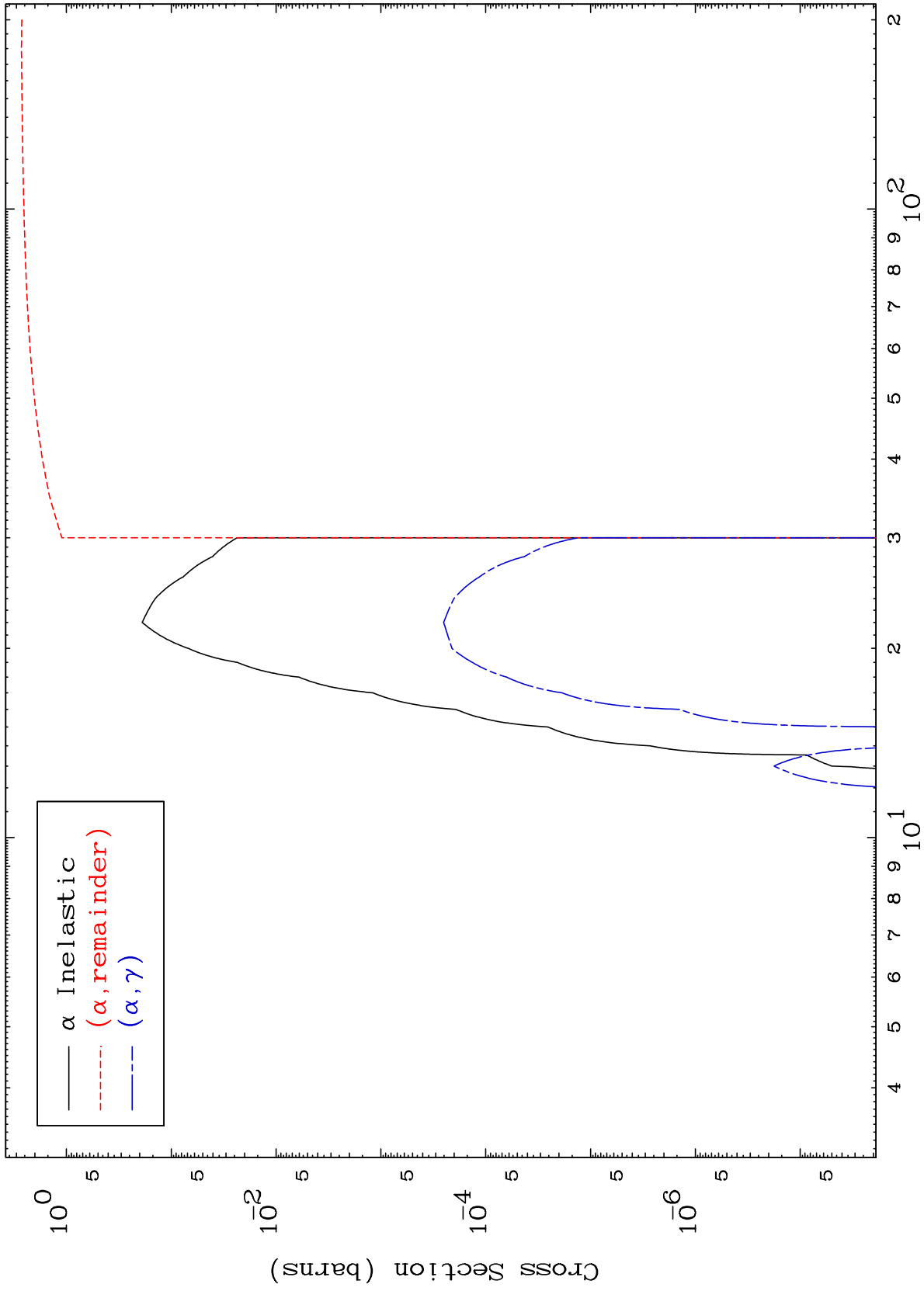
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7907

0 Kelvin  $\alpha$  Major

79-Au-191



1

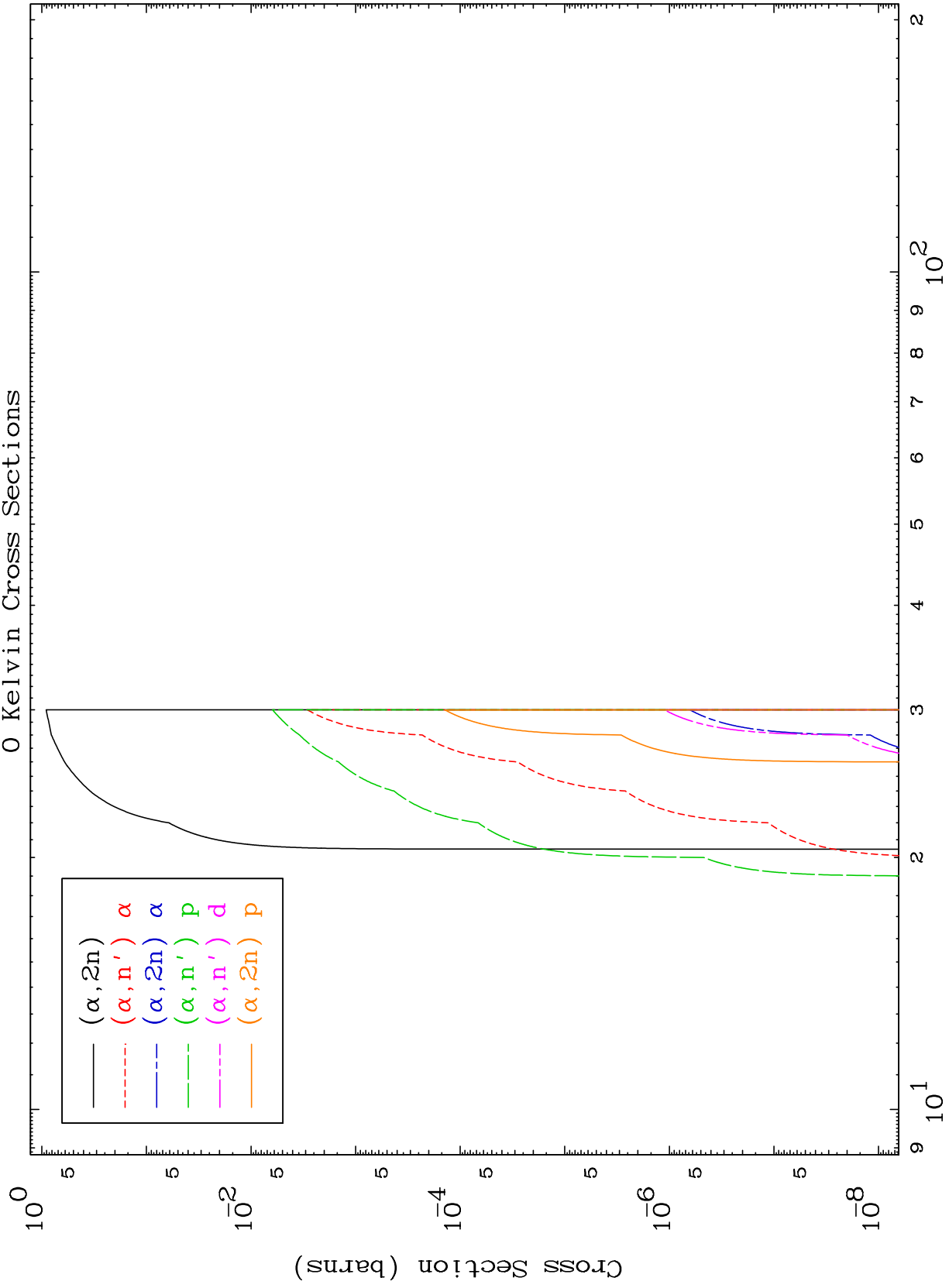
Incident Energy (MeV)

79-Au-191

MAT 7907

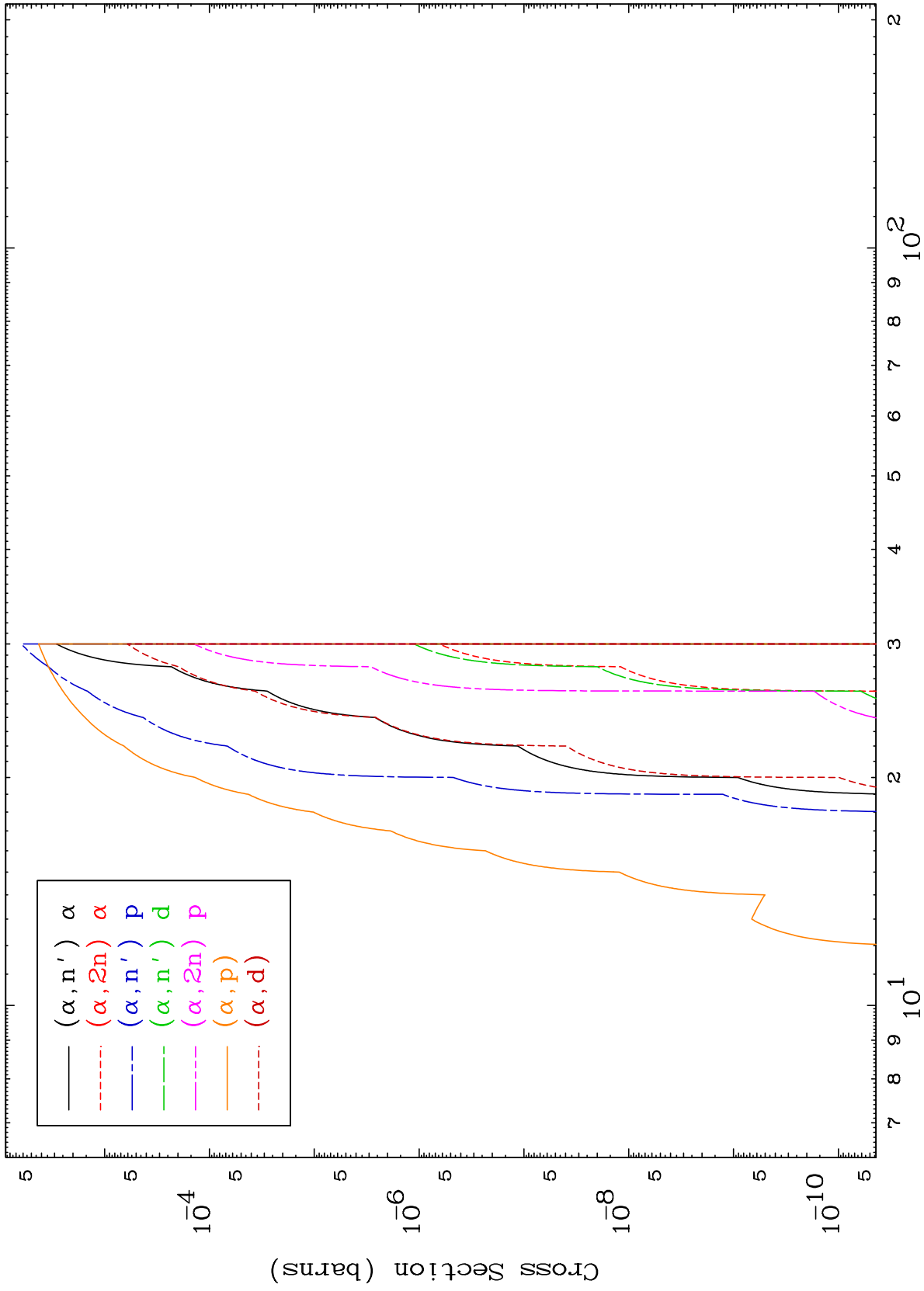
$\alpha$  Neutron Production  
0 Kelvin Cross Sections

79-Au-191



79-Au-191

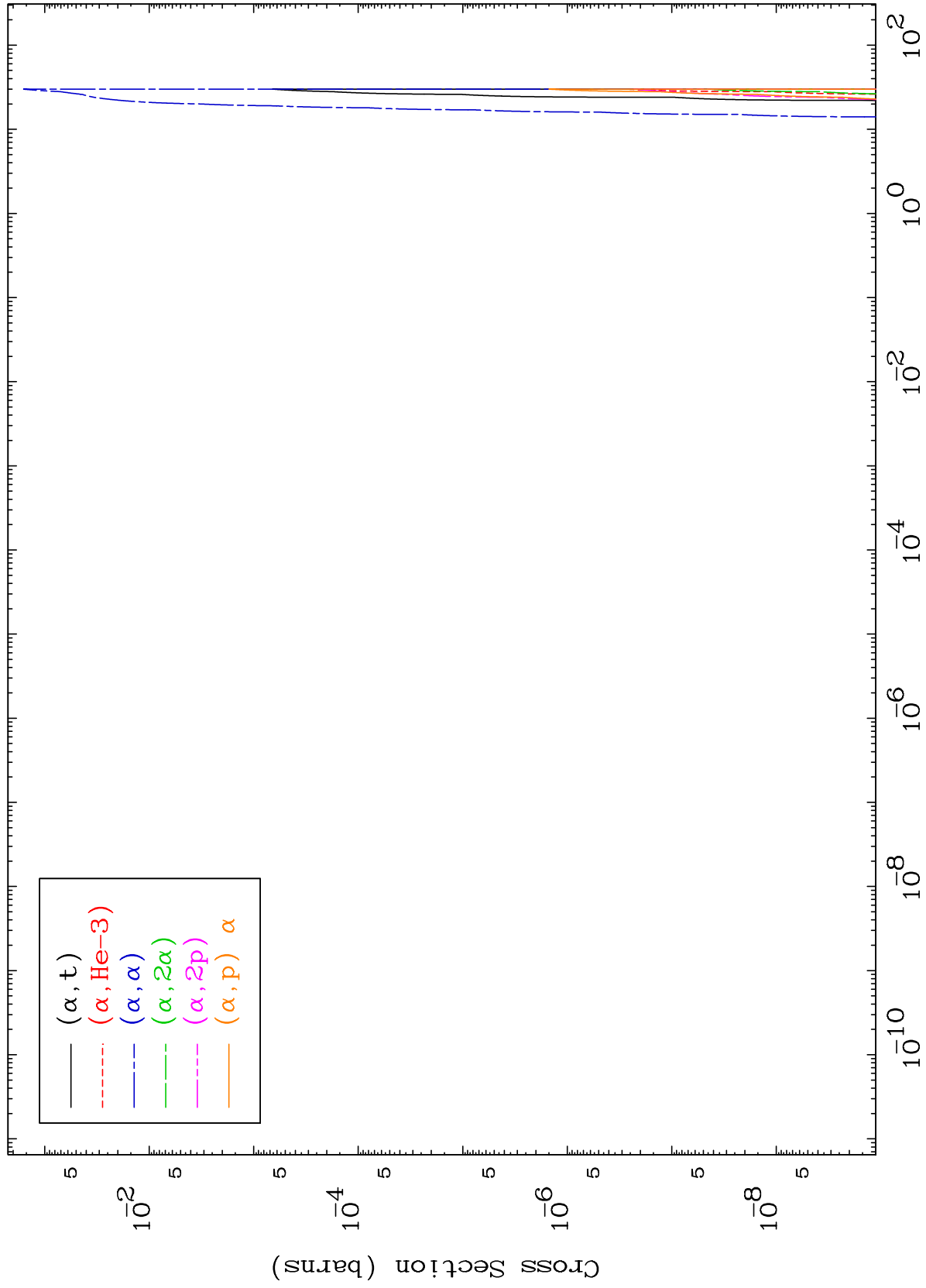
Incident Energy (MeV)



MAT 7907

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

79-Au-191

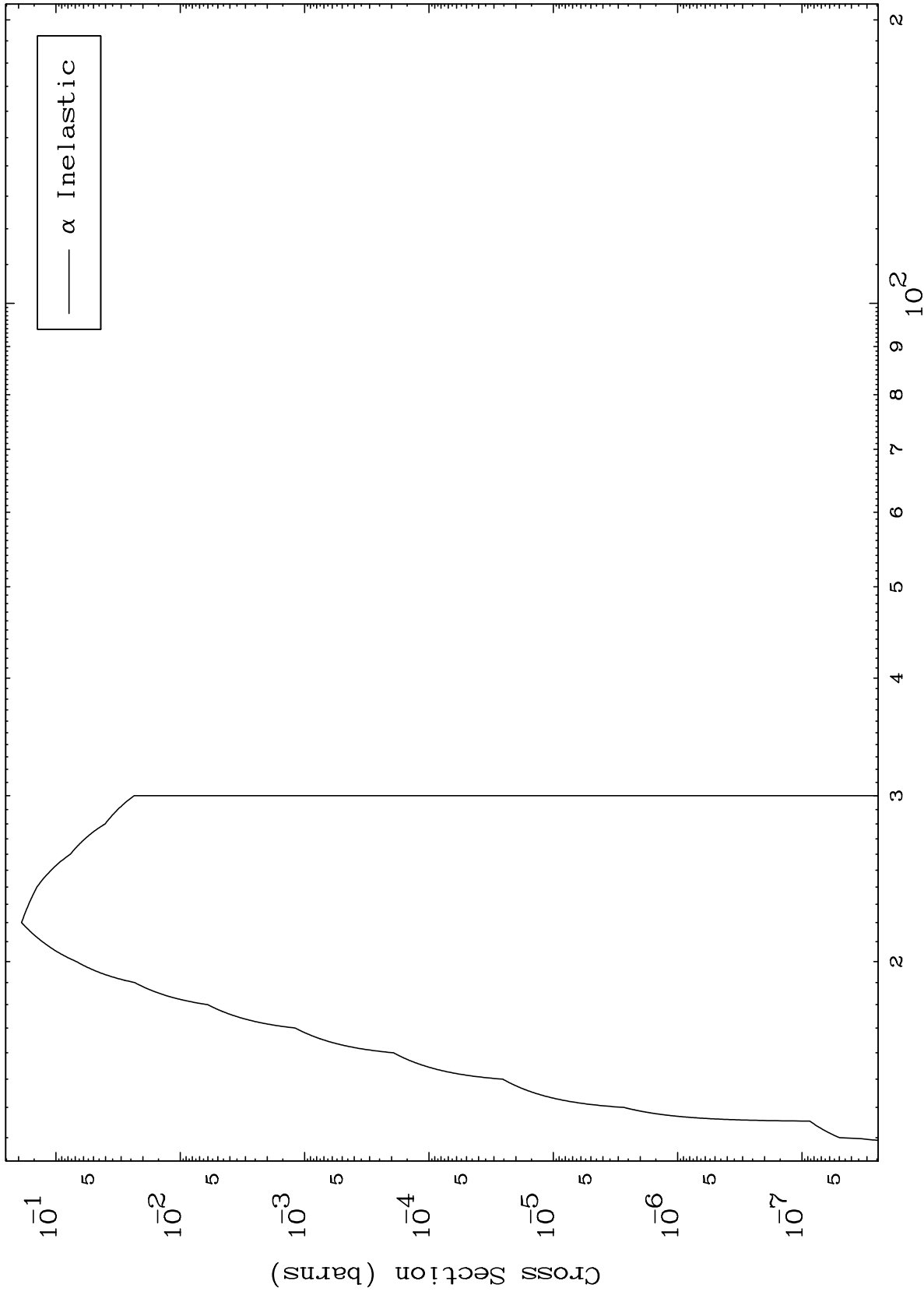


79-Au-191

MAT 7907

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

79-Au-191



5

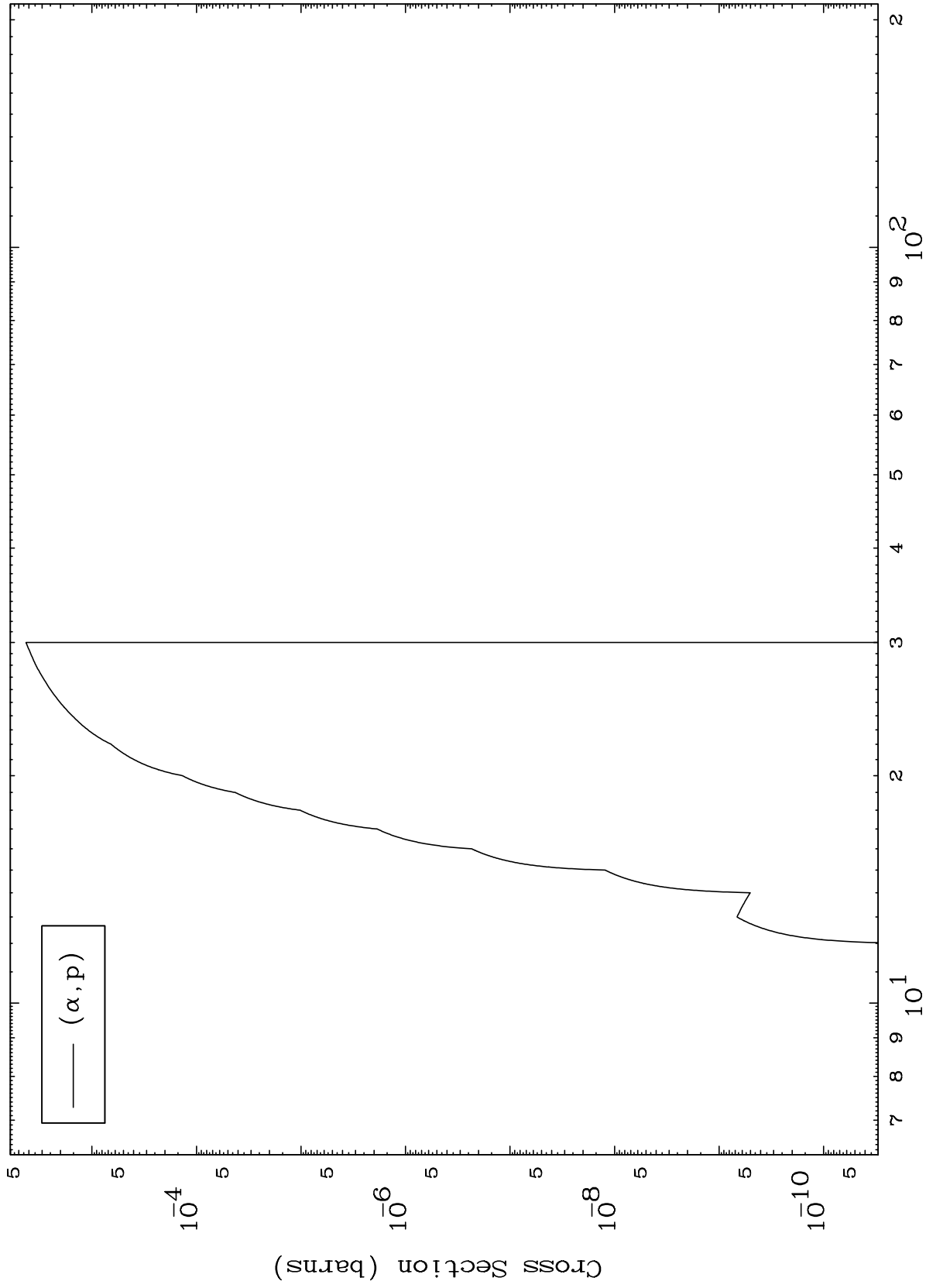
Incident Energy (MeV)

79-Au-191

MAT 7907

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

79-Au-191



6

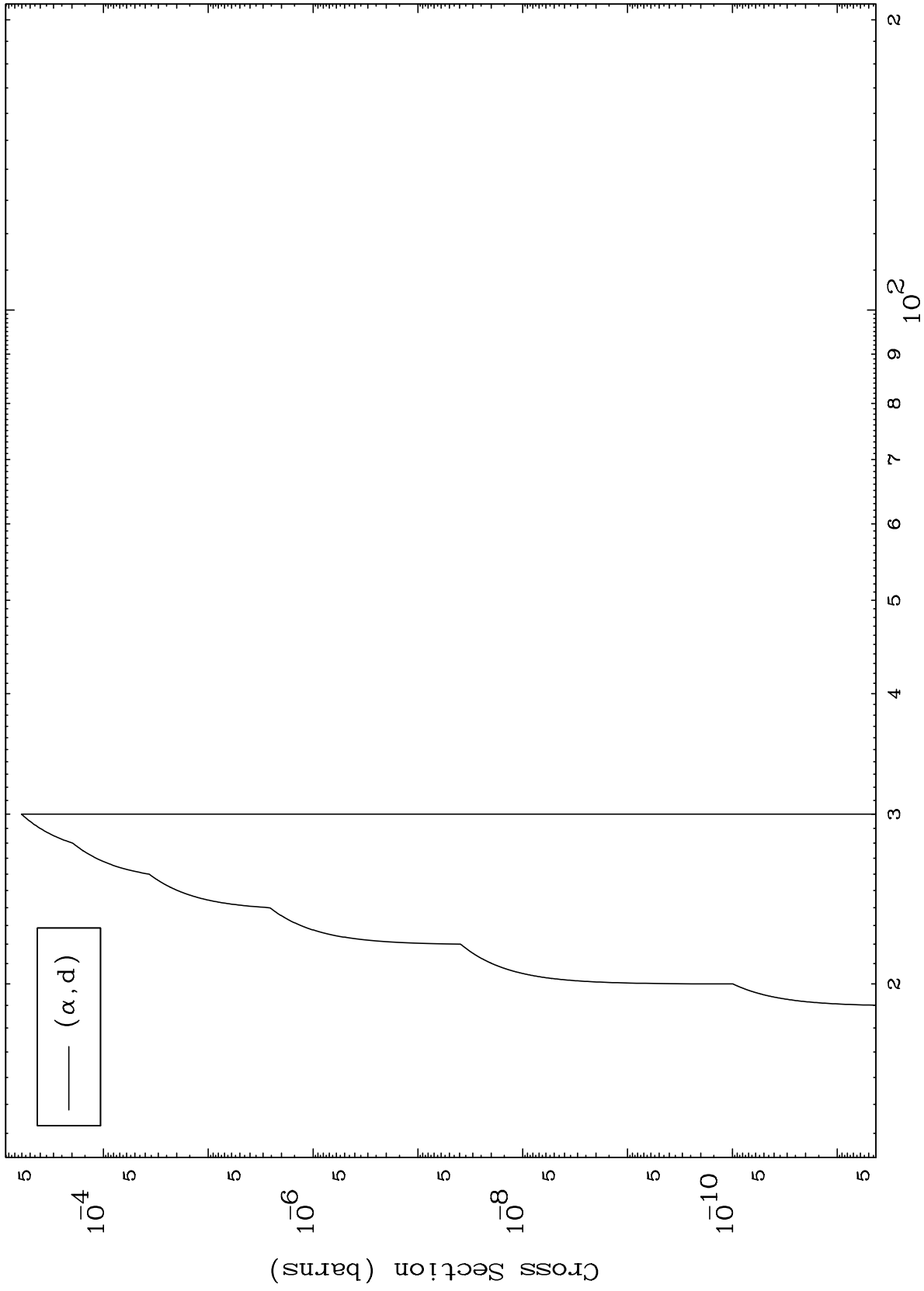
Incident Energy (MeV)

79-Au-191

MAT 7907

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

79-Au-191



7

Incident Energy (MeV)

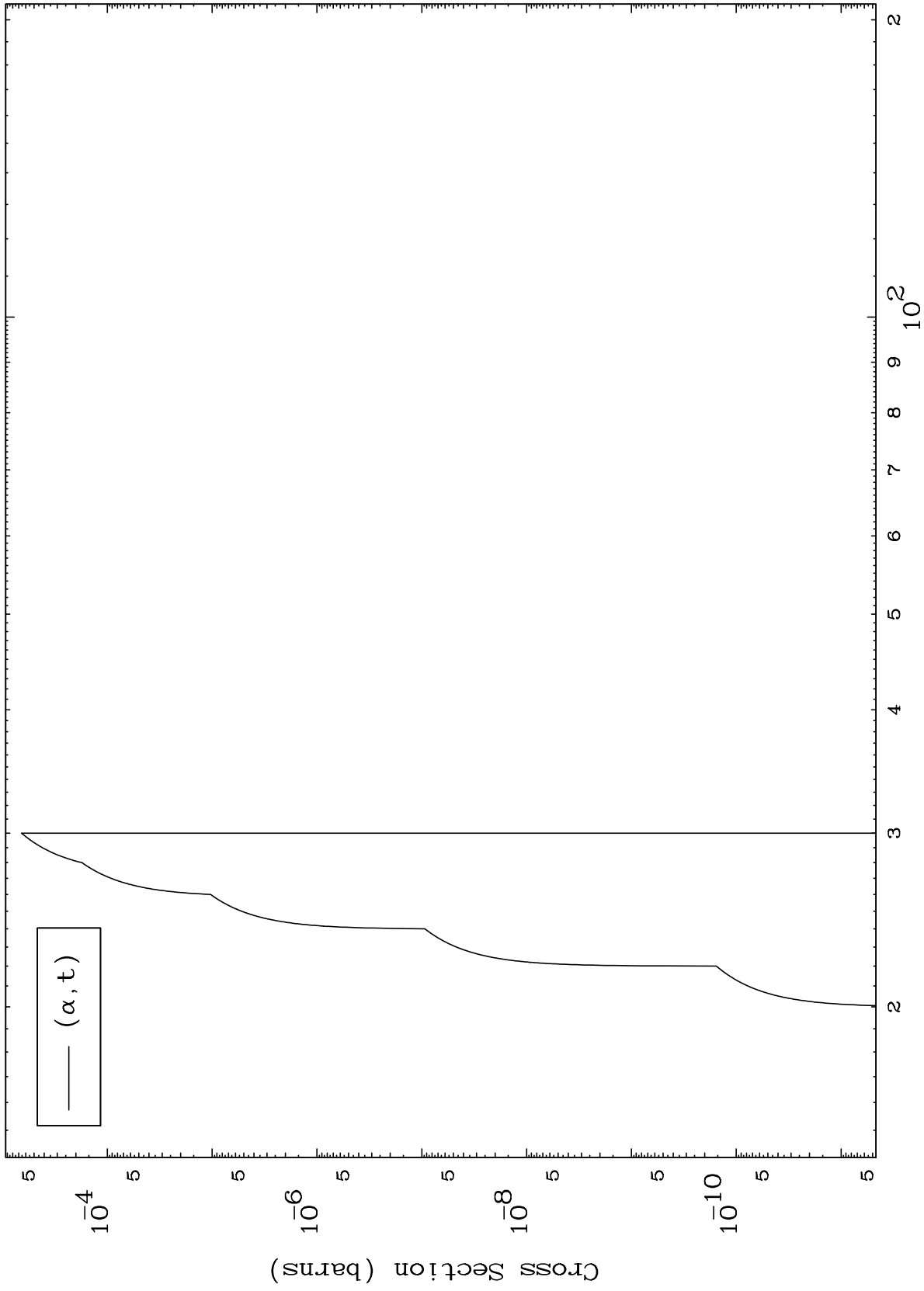
79-Au-191

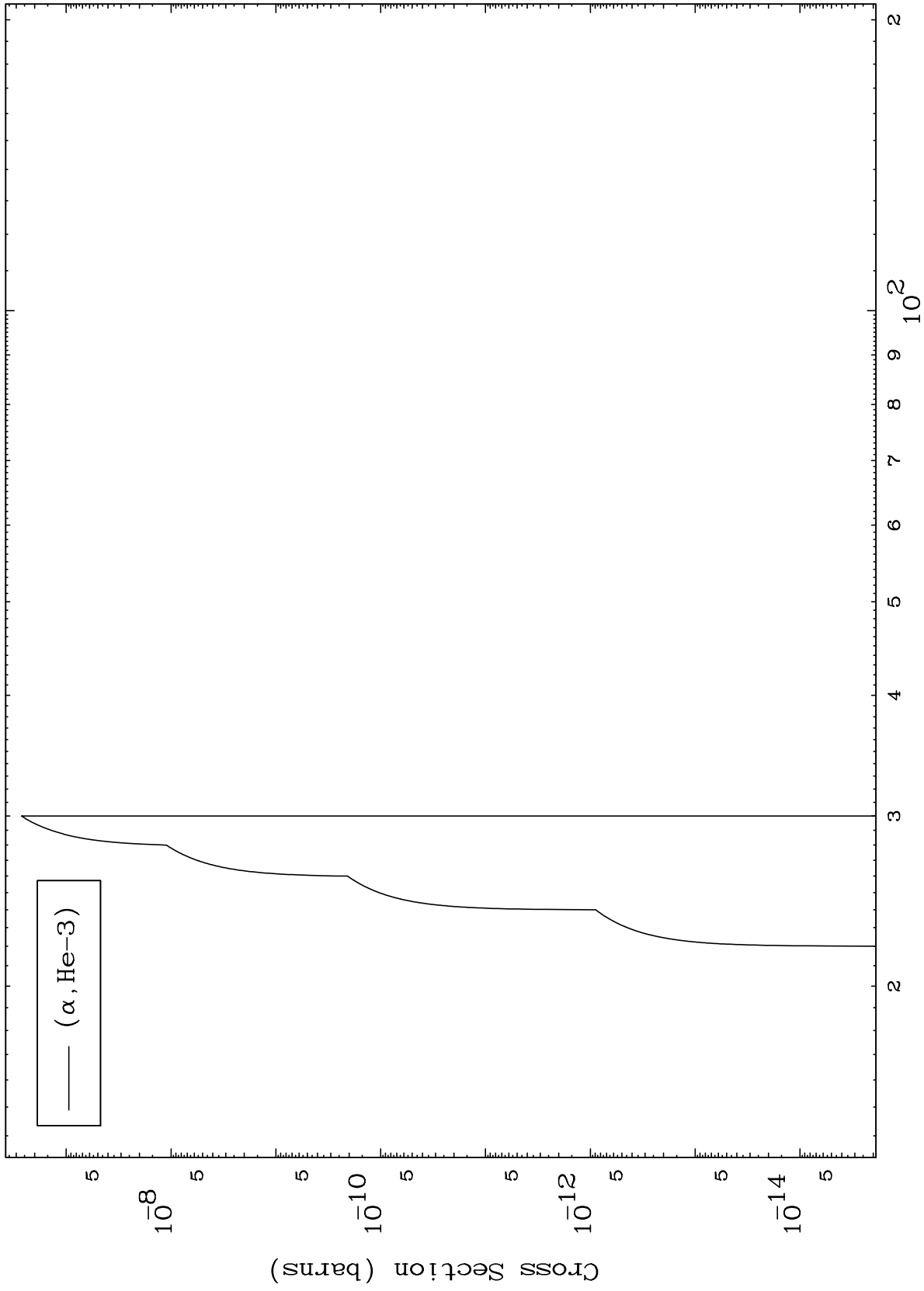


MAT 7907

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

79-Au-191

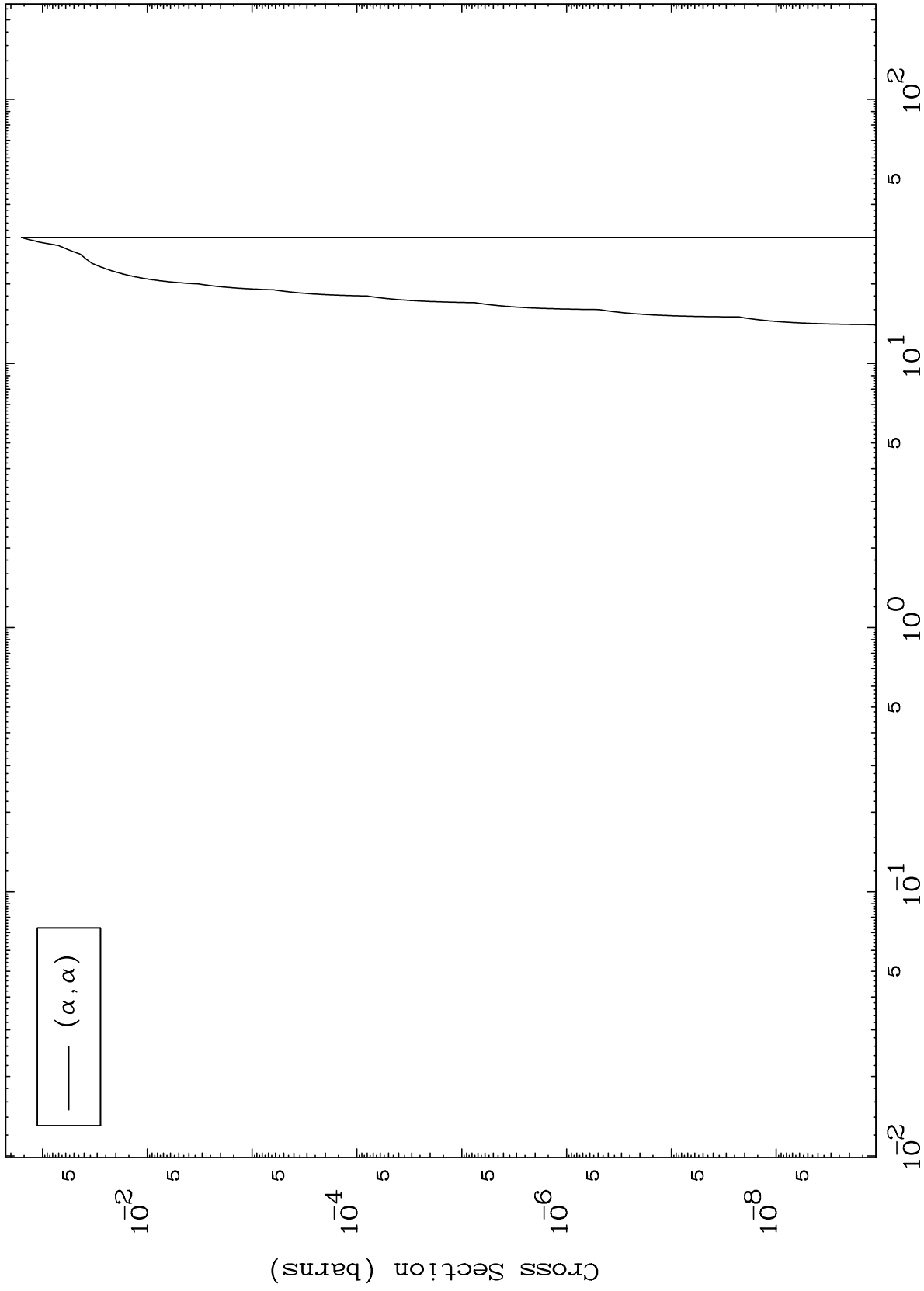




MAT 7907

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

79-Au-191



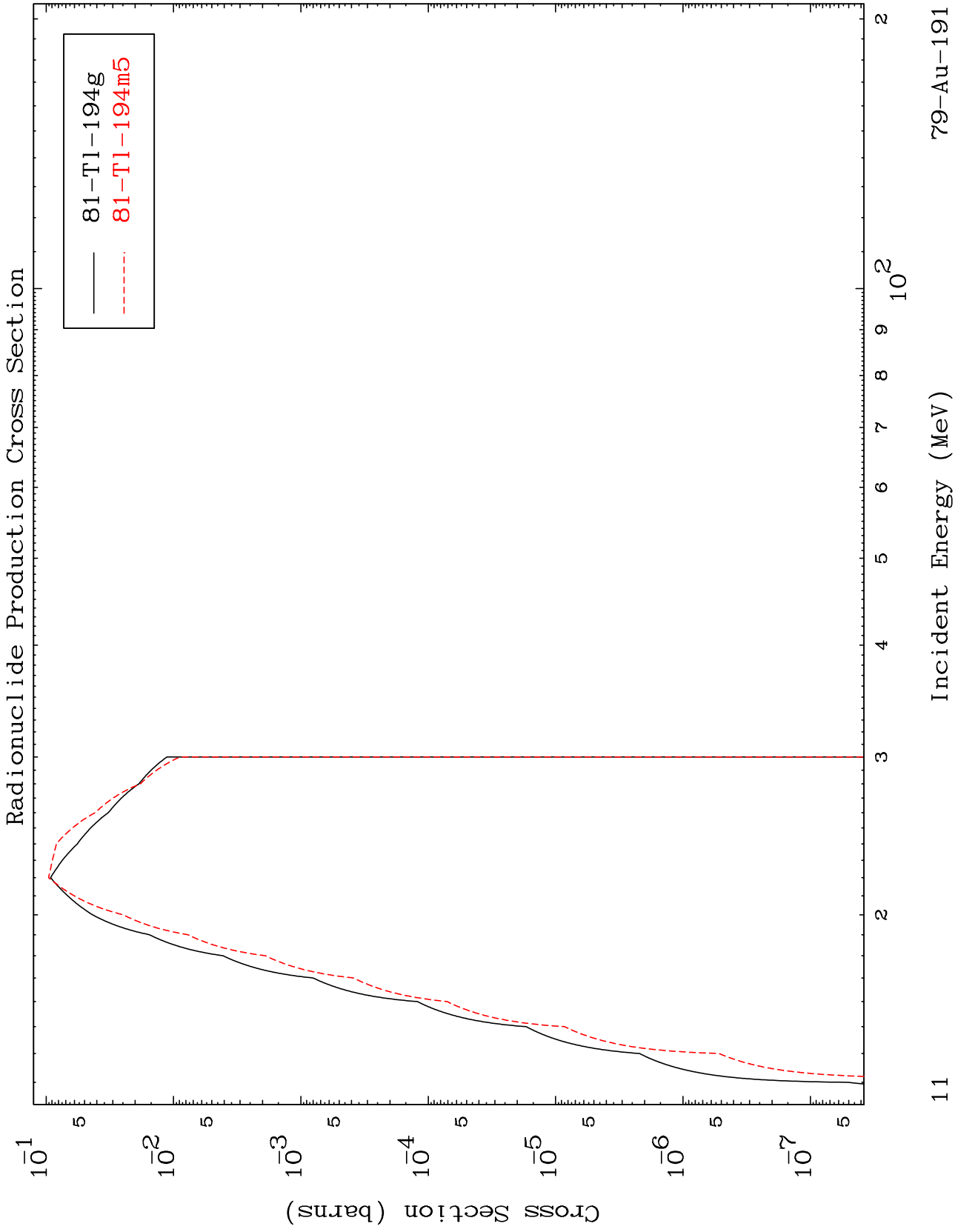
Incident Energy (MeV)

79-Au-191

MAT 7907

$\alpha$  Inelastic

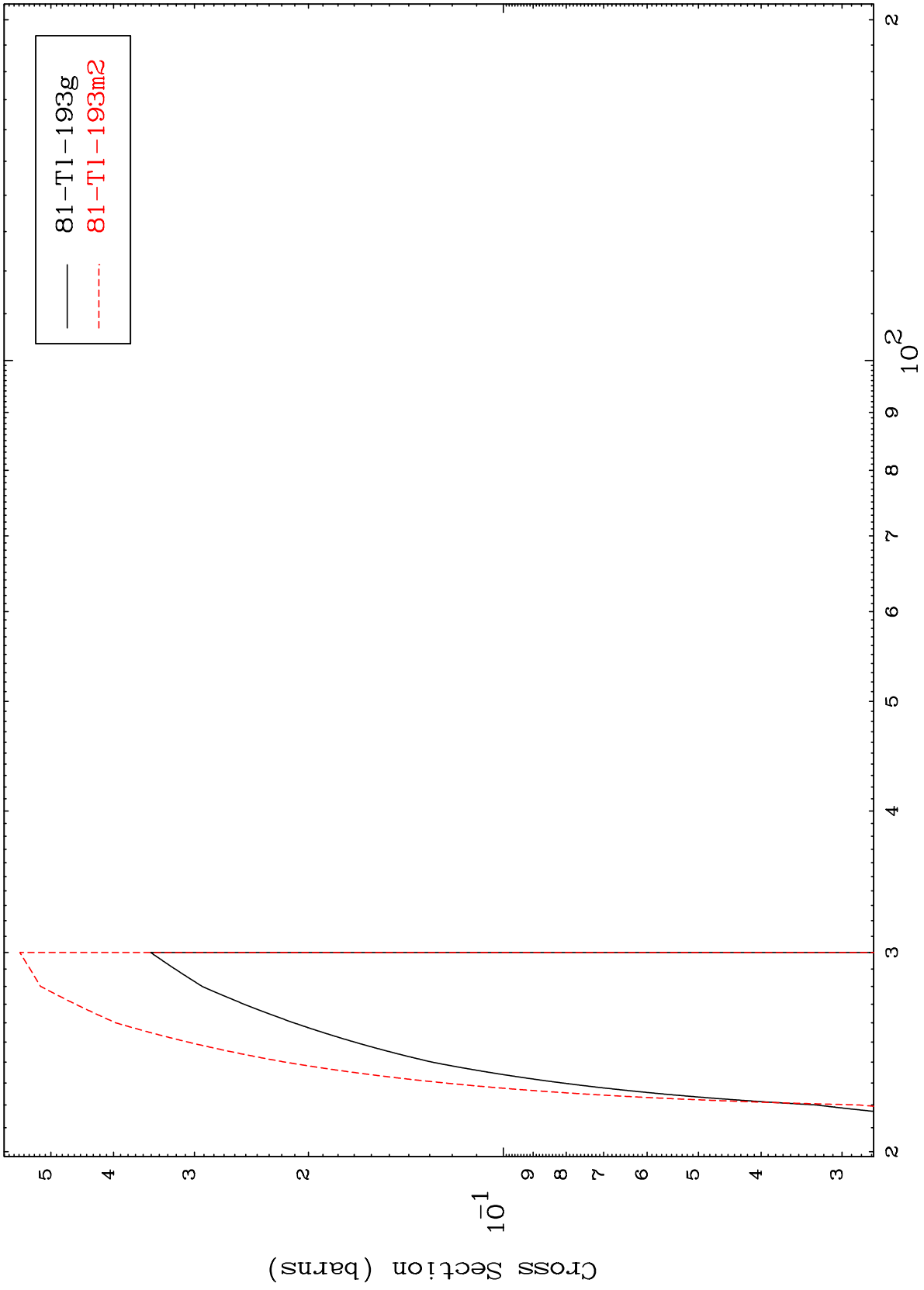
79-Au-191



MAT 7907

79-Au-191

( $\alpha, 2n$ )  
Radionuclide Production Cross Section



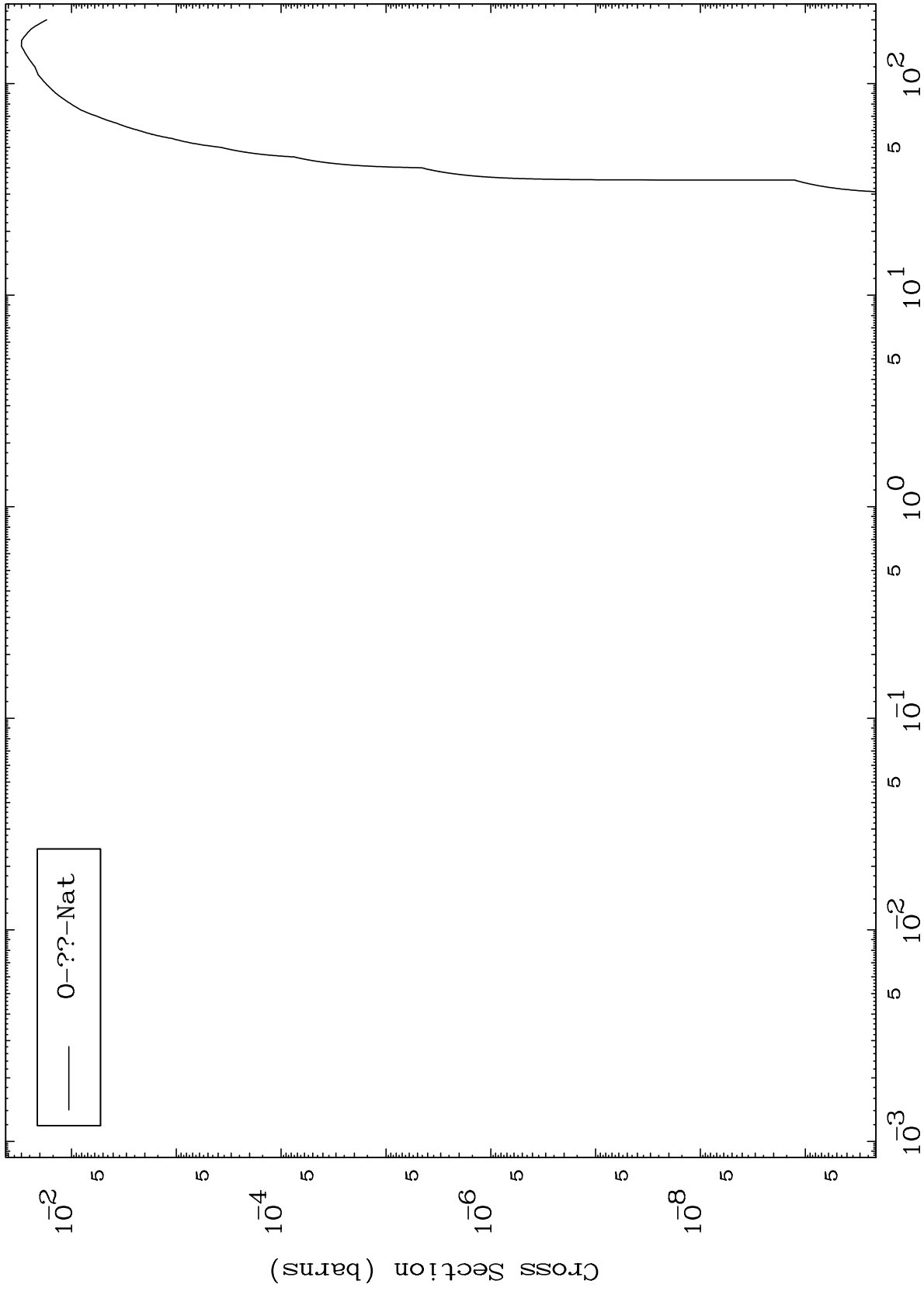
12

79-Au-191

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$\alpha$  Fission  
Radionuclide Production Cross Section

79-Au-191



0-?-?-Nat

13

Incident Energy (MeV)

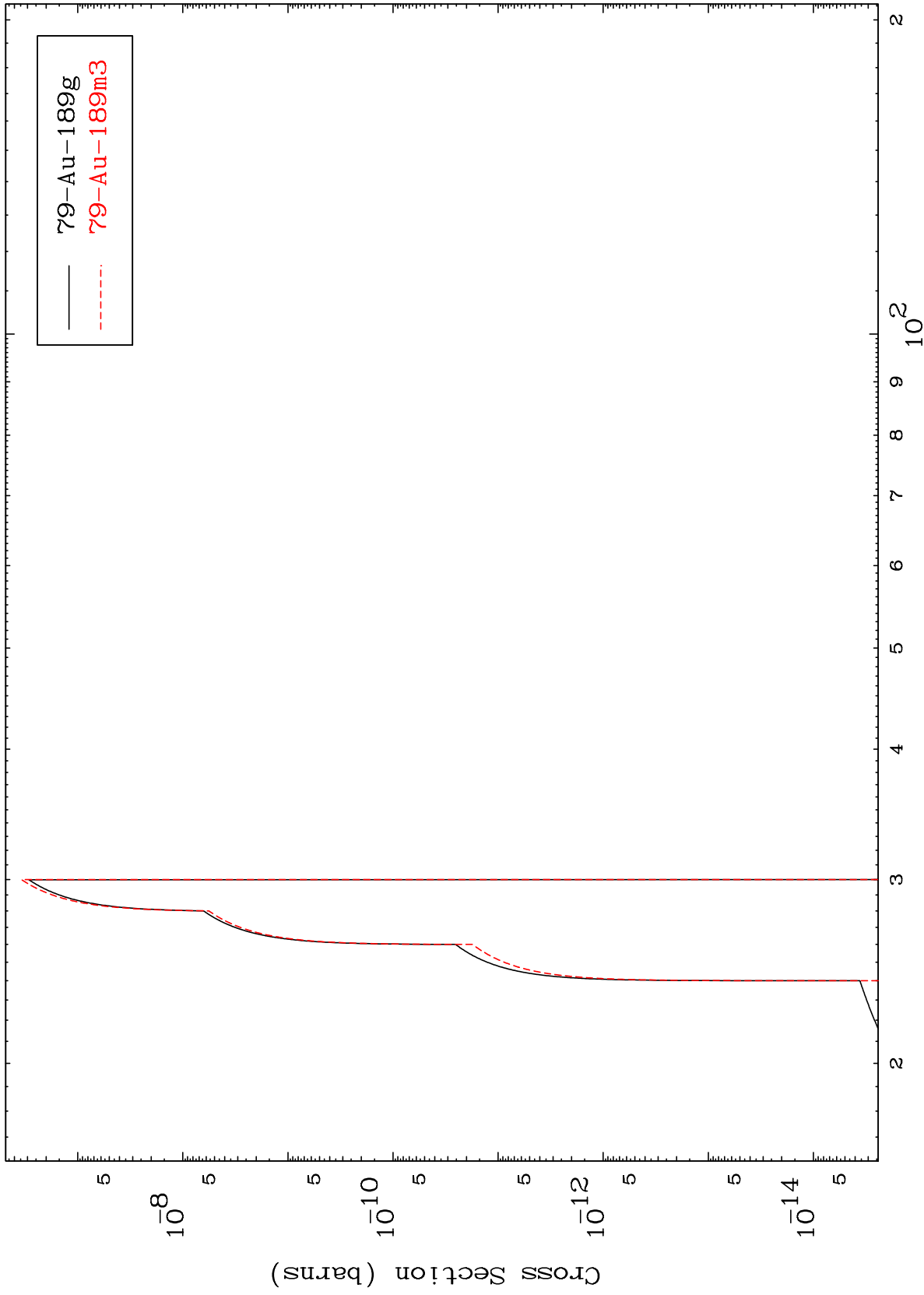
79-Au-191

MAT 7907

$(\alpha, 2n) \alpha$

$^{79}\text{Au-191}$

Radionuclide Production Cross Section



14

Incident Energy (MeV)

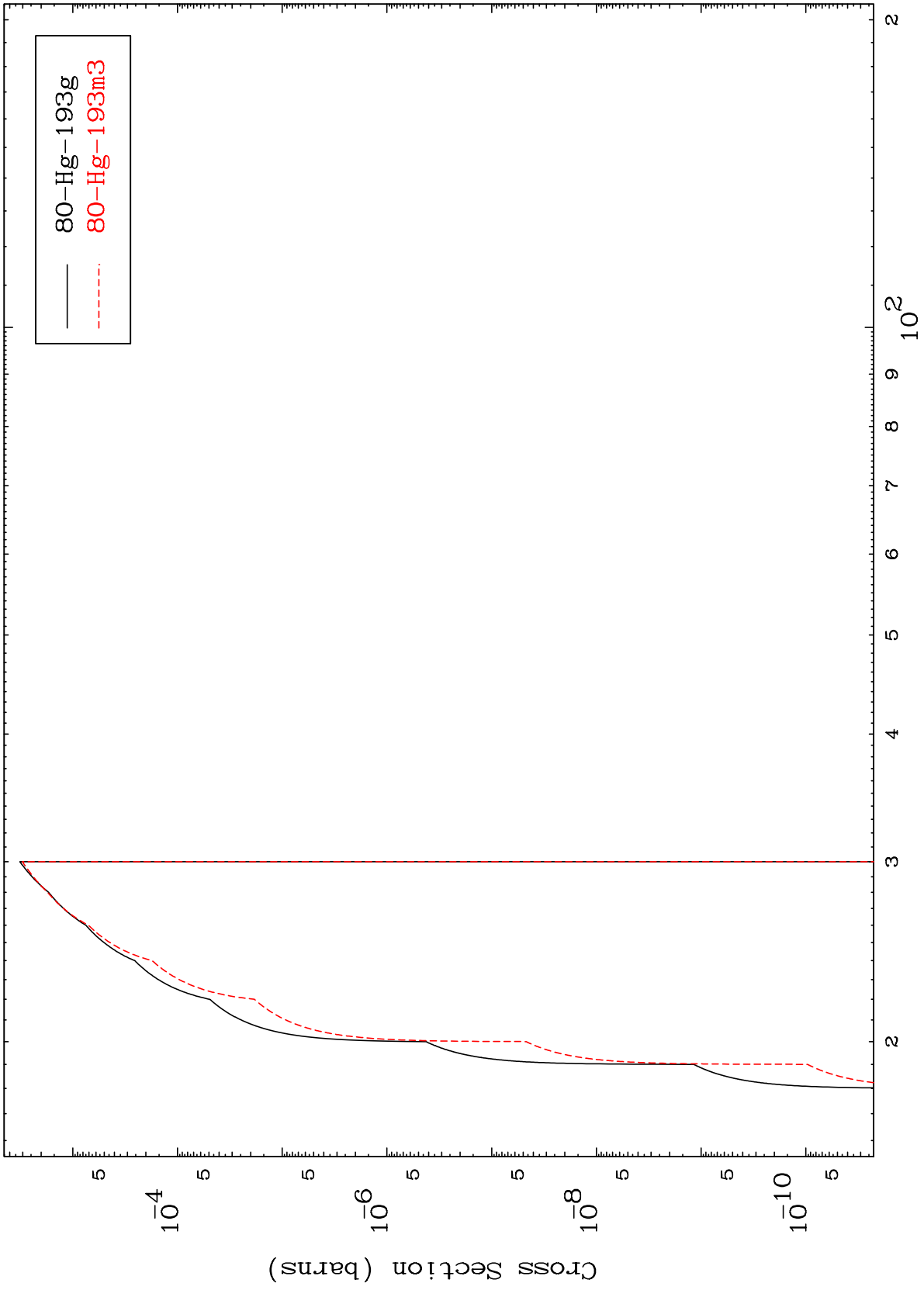
$^{79}\text{Au-191}$

MAT 7907

( $\alpha, n'$ ) p

79-Au-191

Radionuclide Production Cross Section



15

Incident Energy (MeV)

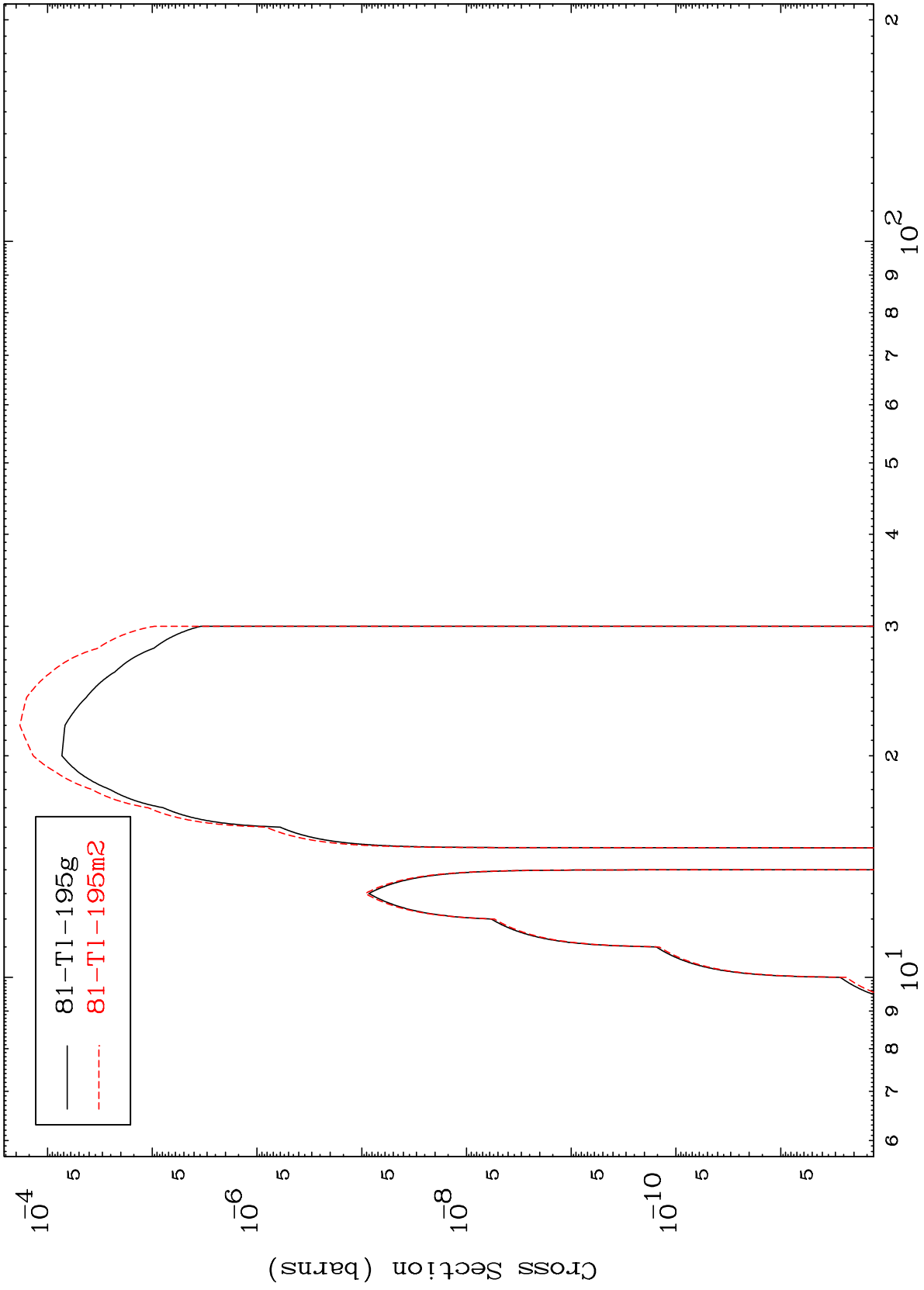
79-Au-191



MAT 7907

79-Au-191

$(\alpha, \gamma)$   
Radionuclide Production Cross Section



16

Incident Energy (MeV)

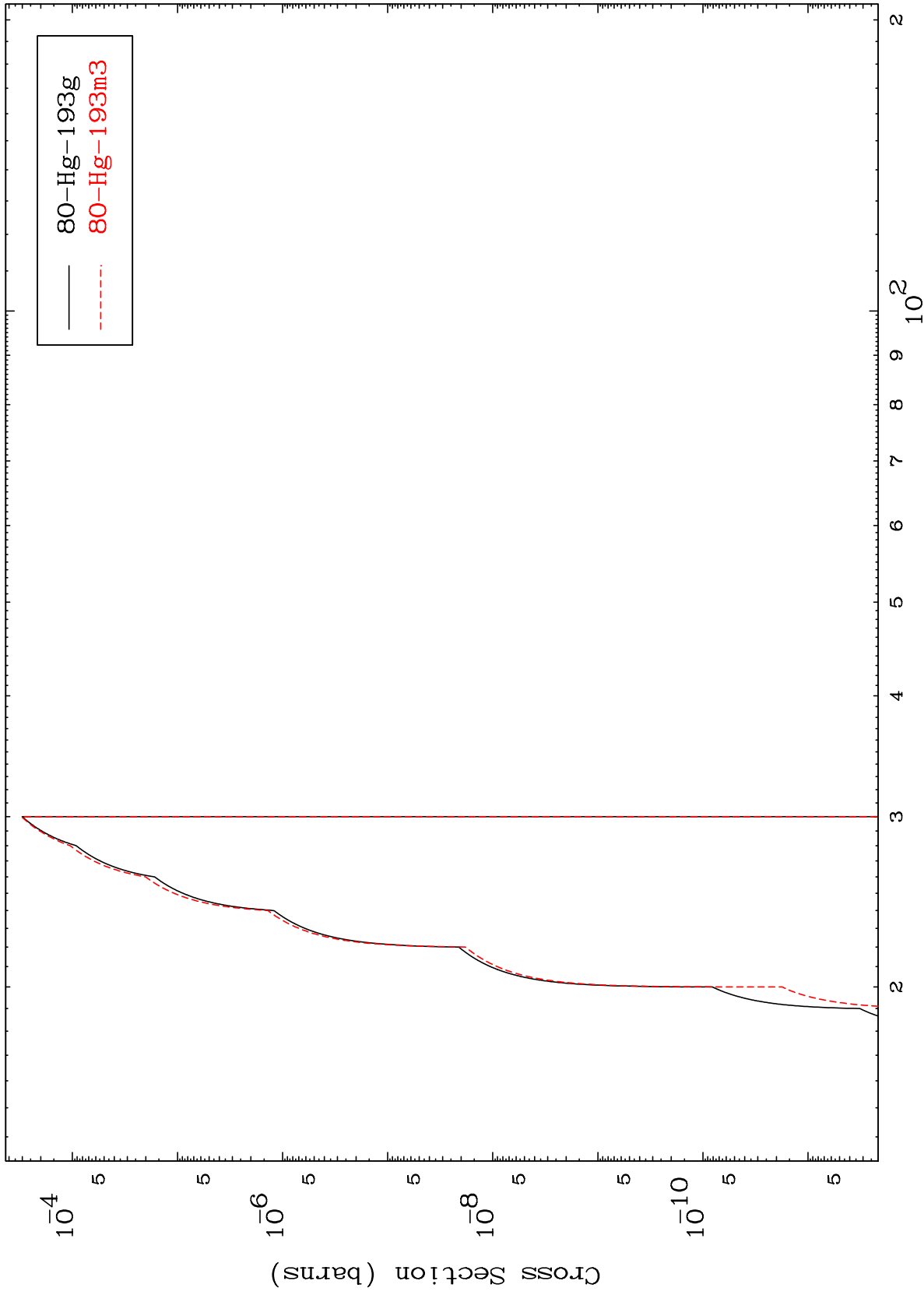
79-Au-191

MAT 7907

( $\alpha, d$ )

79-Au-191

Radionuclide Production Cross Section



17

Incident Energy (MeV)

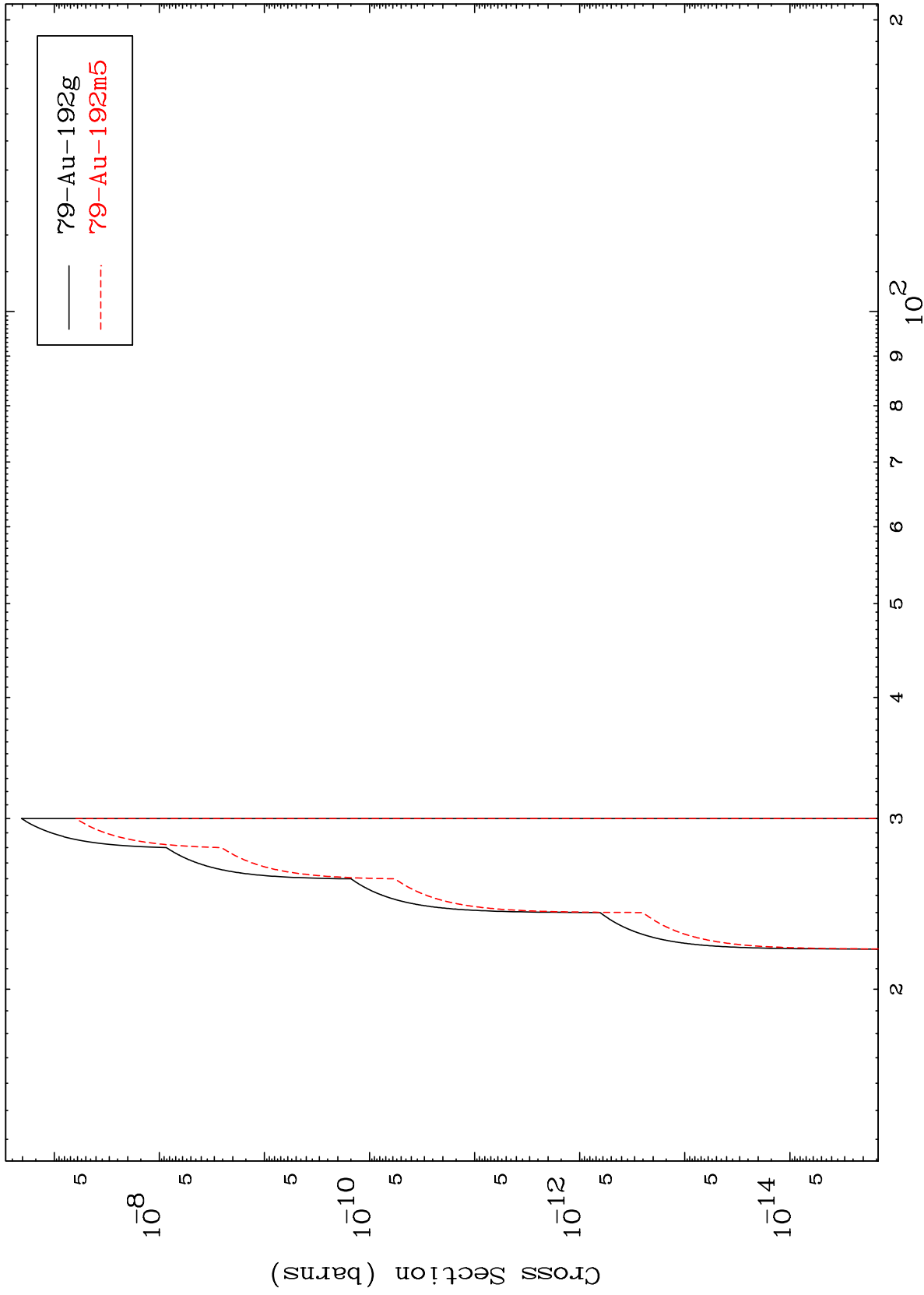
79-Au-191

MAT 7907

( $\alpha, \text{He-3}$ )

$^{79}\text{Au-191}$

Radionuclide Production Cross Section



18

Incident Energy (MeV)

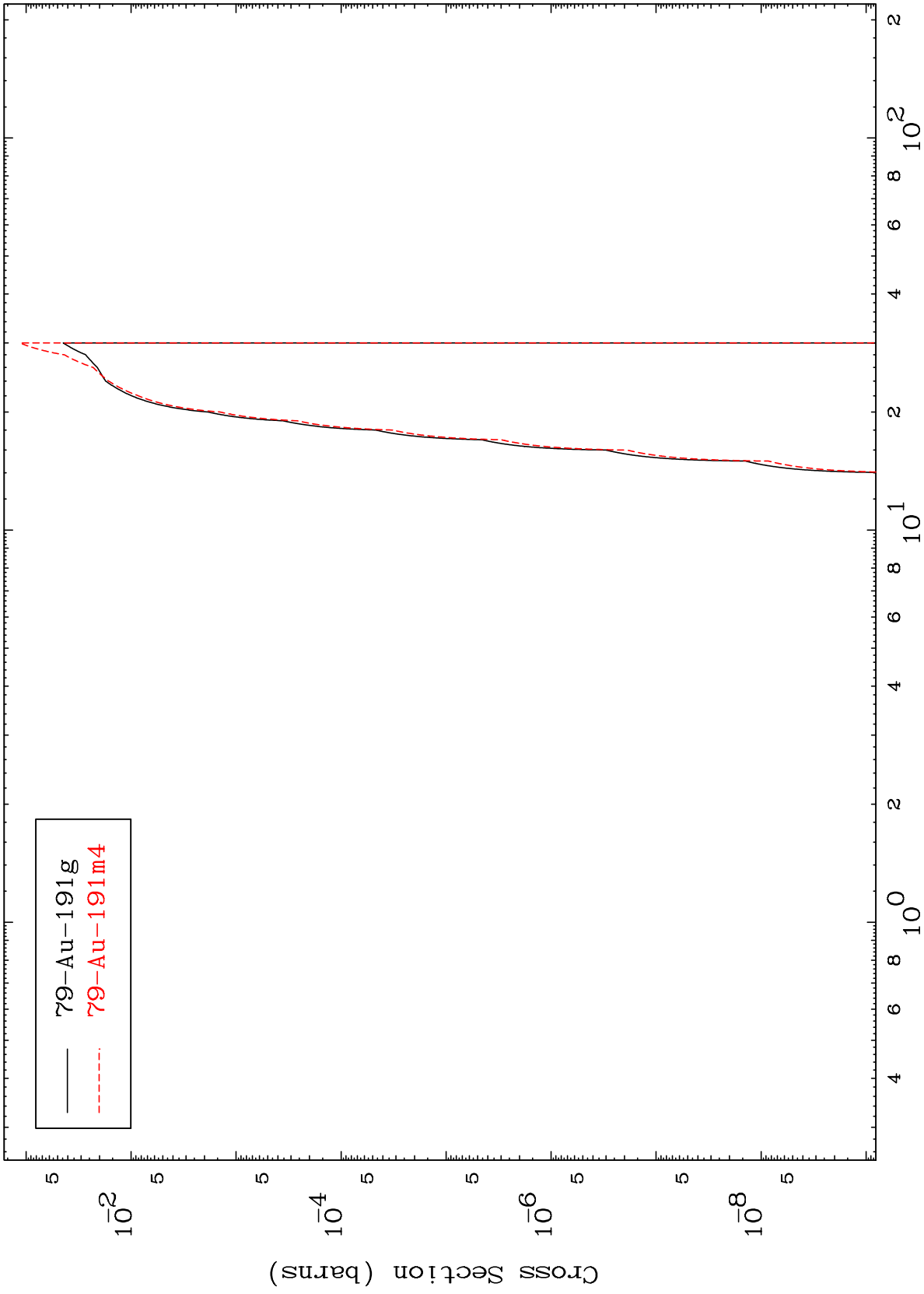
$^{79}\text{Au-191}$

MAT 7907

( $\alpha, \alpha$ )

<sup>79</sup>Au-191

Radionuclide Production Cross Section



19

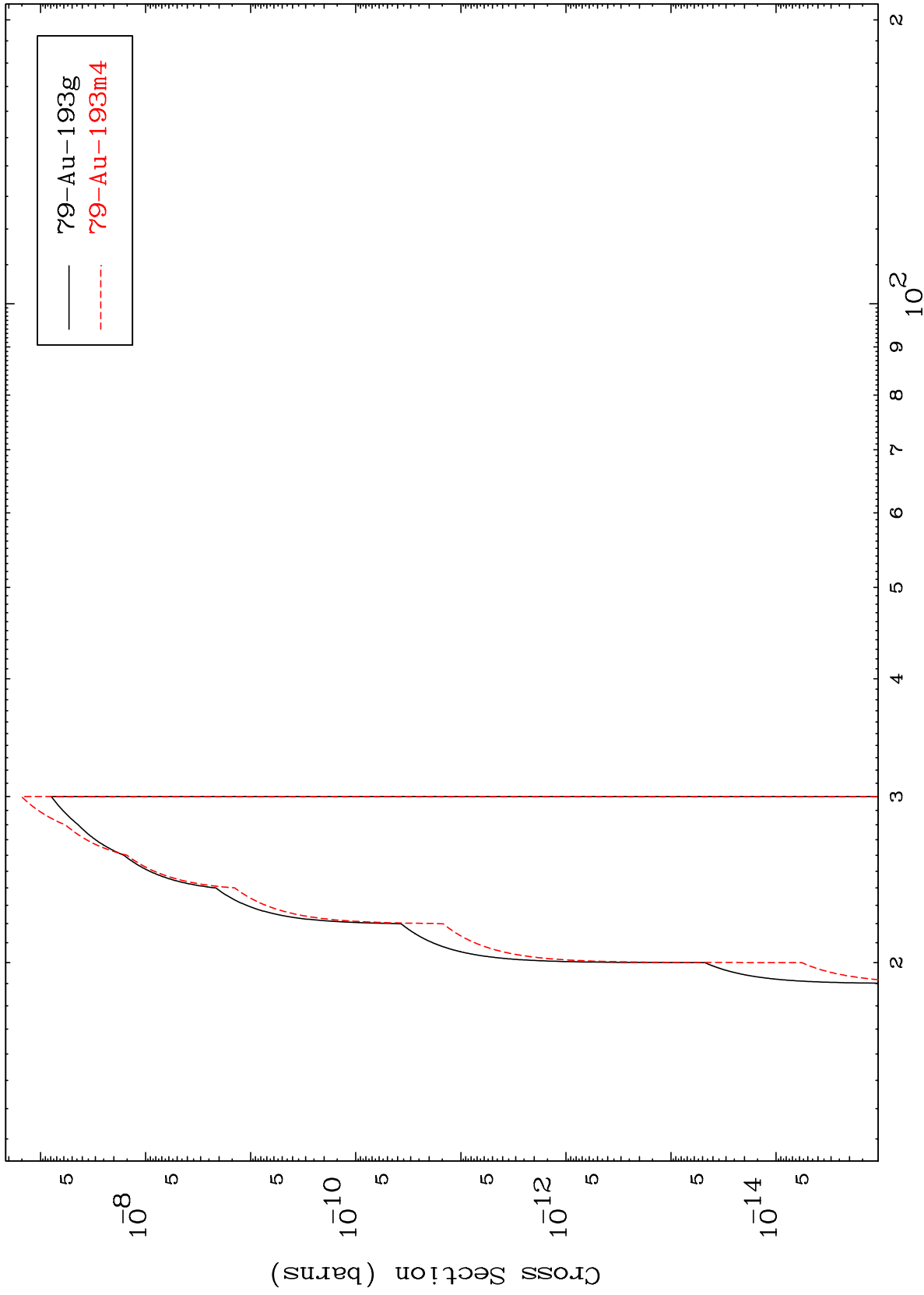
Incident Energy (MeV)

<sup>79</sup>Au-191

MAT 7907

79-Au-191

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



20

79-Au-191