

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

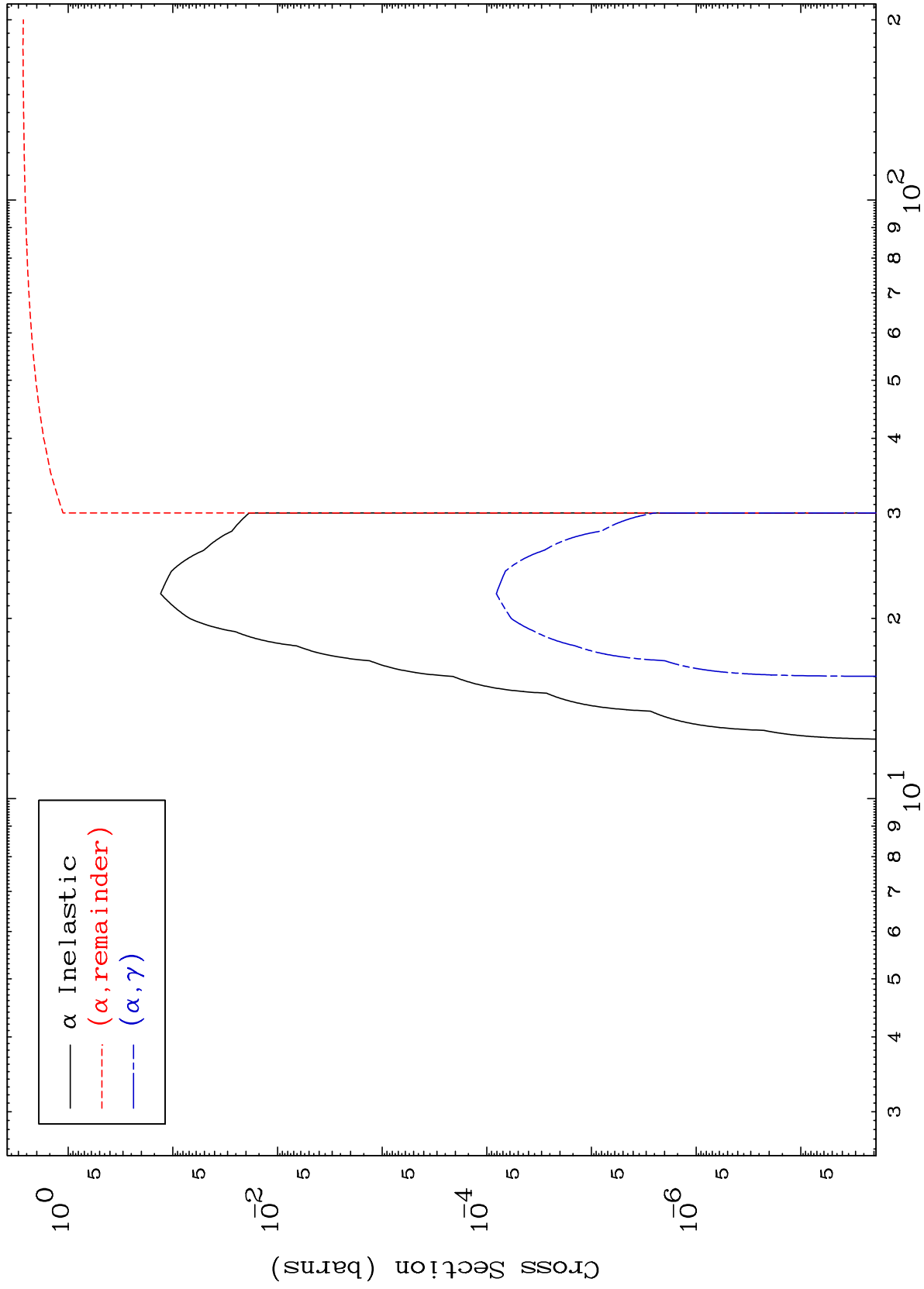
Press Mouse Button to Start

MAT 7913

$\alpha$  Major

79-Au-193

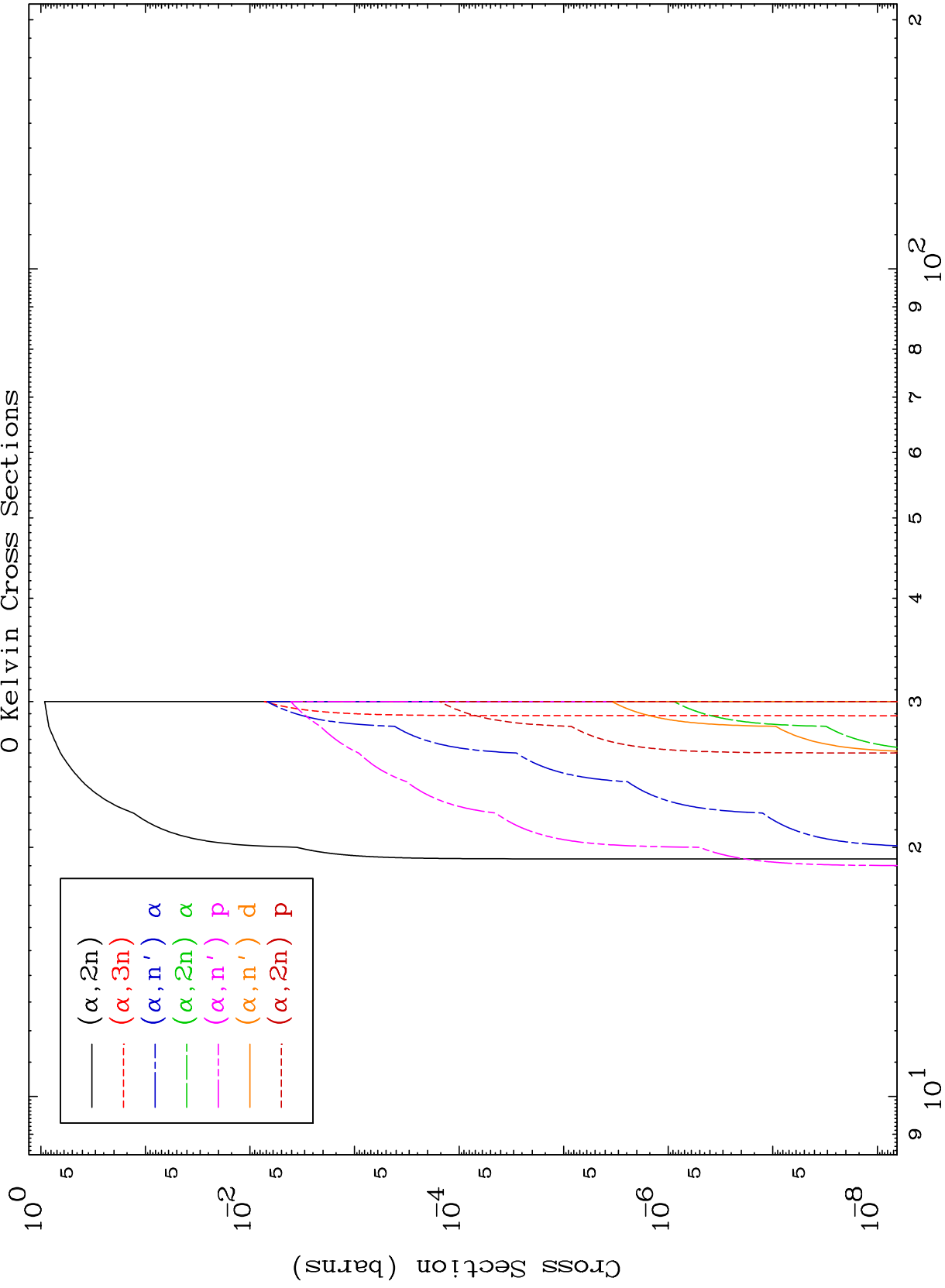
0 Kelvin Cross Sections



MAT 7913

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

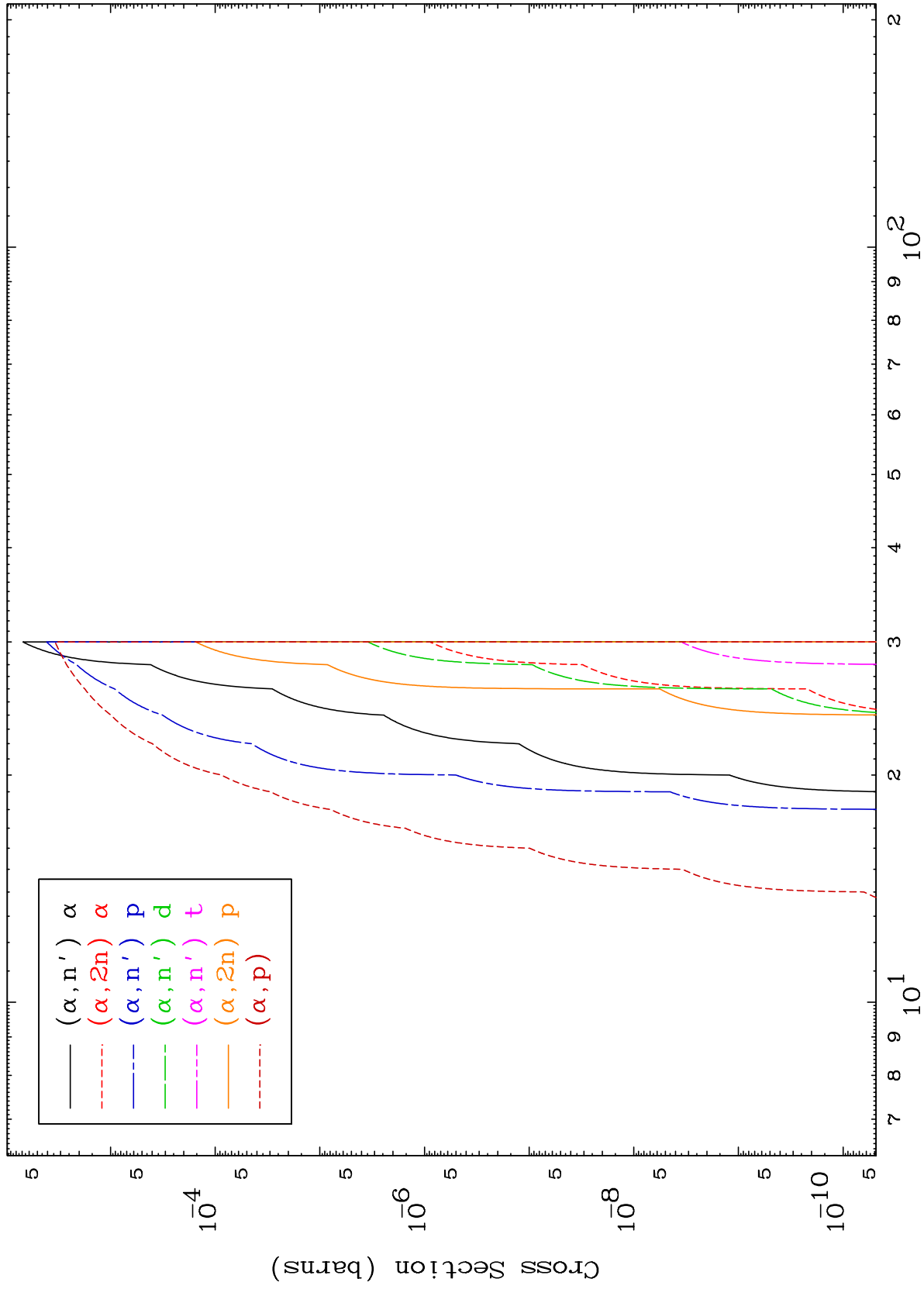
79-Au-193



2

Incident Energy (MeV)

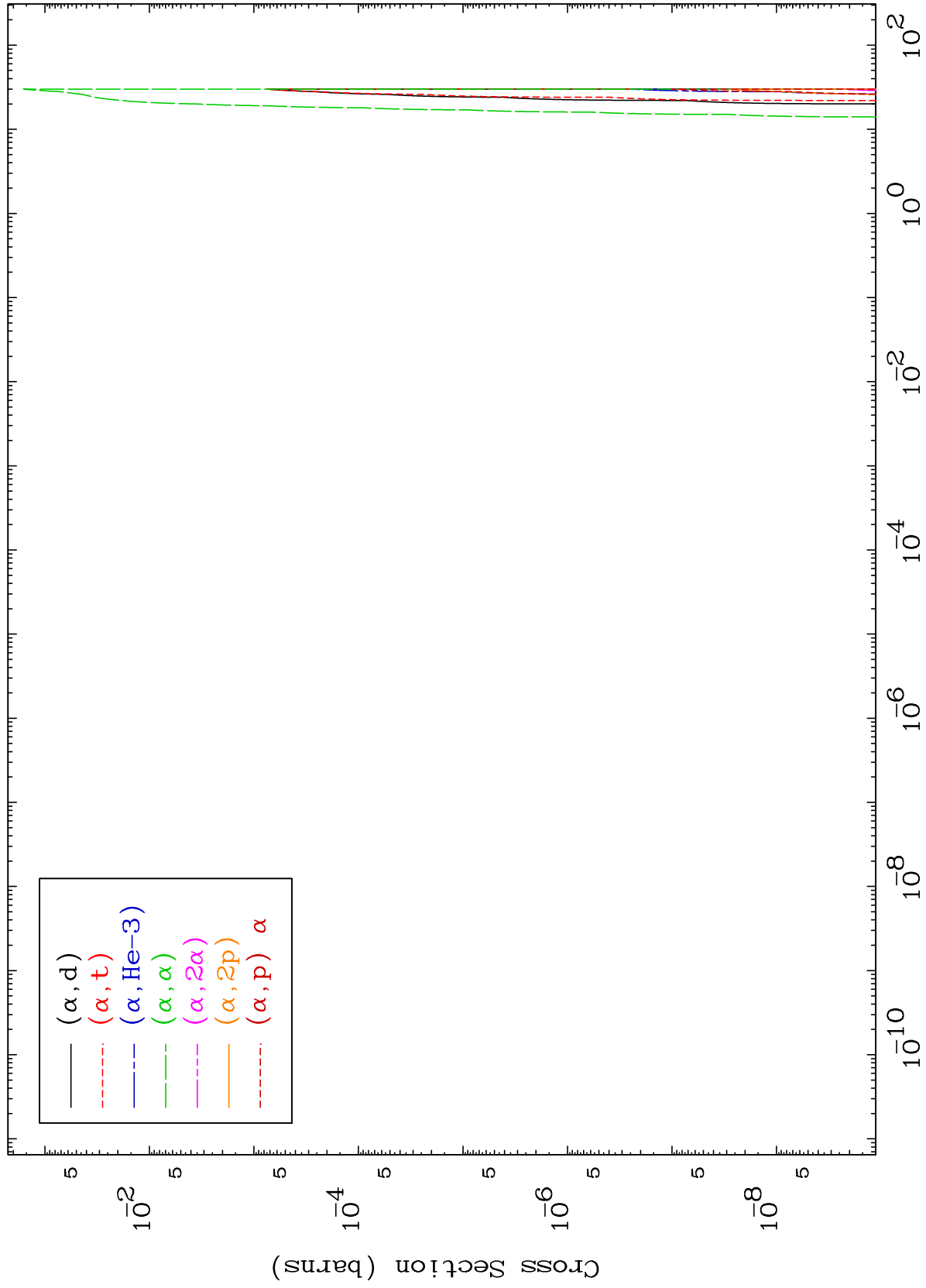
79-Au-193



MAT 7913

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

79-Au-193



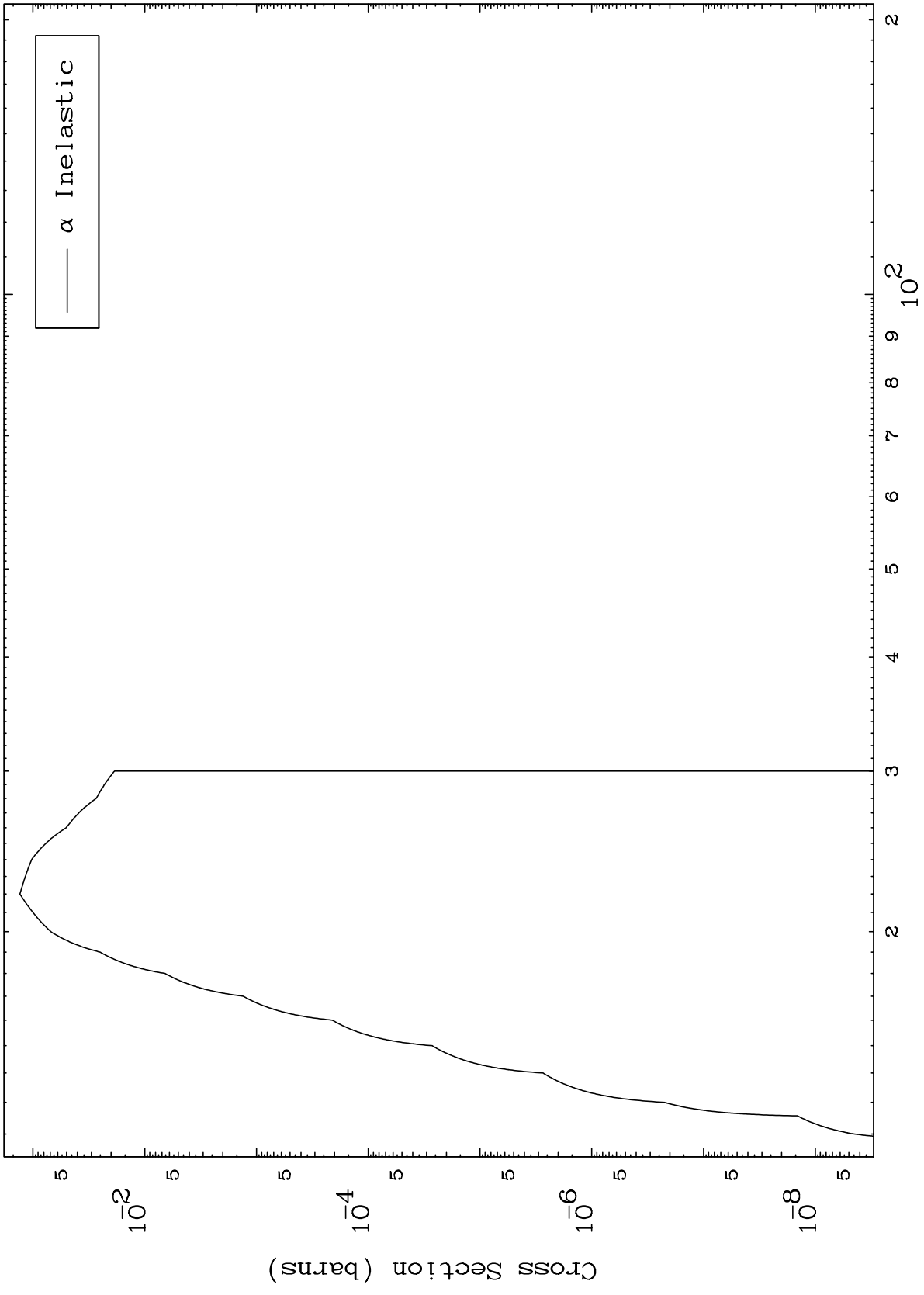
79-Au-193

Incident Energy (MeV)

MAT 7913

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

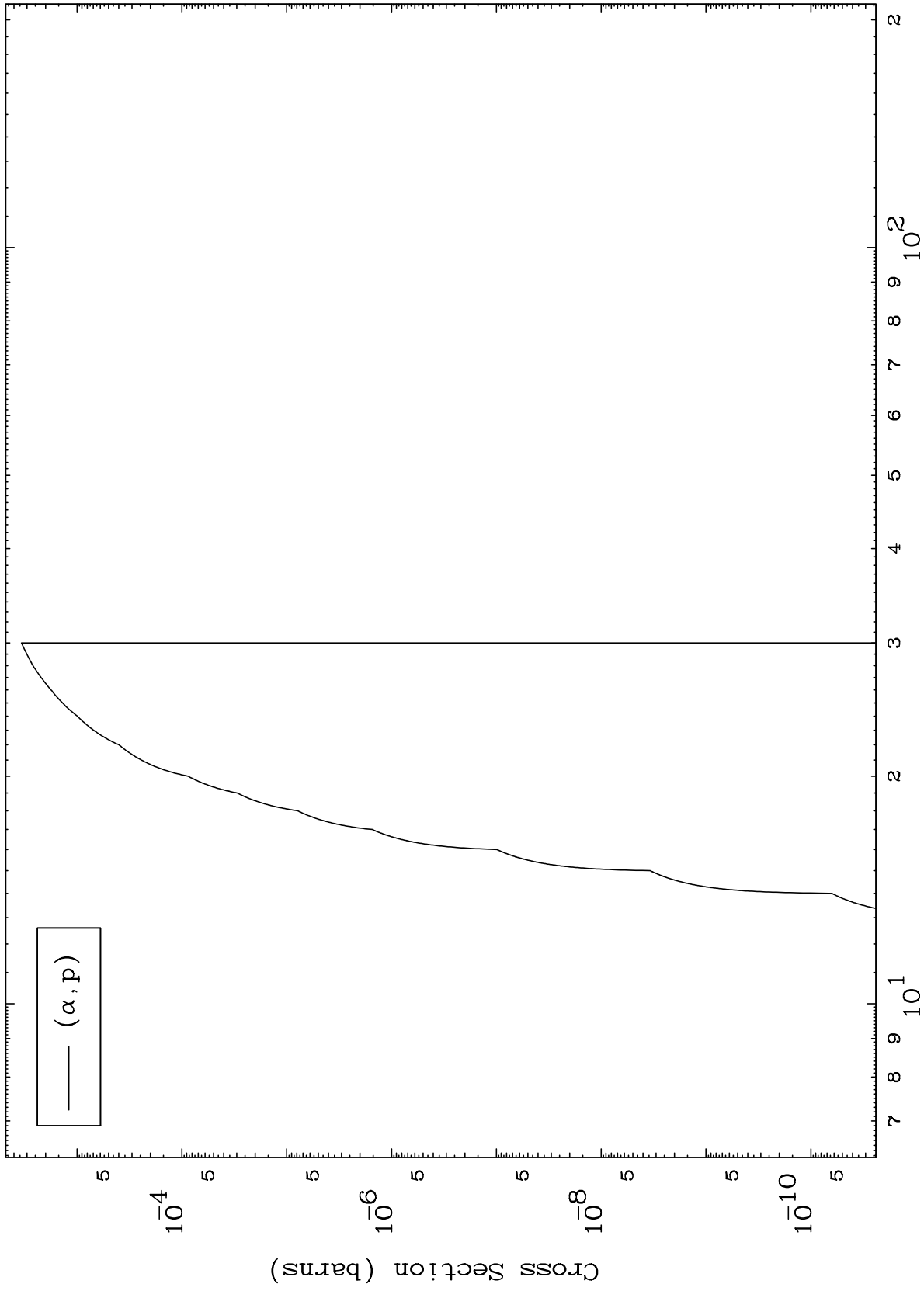
79-Au-193



5

Incident Energy (MeV)

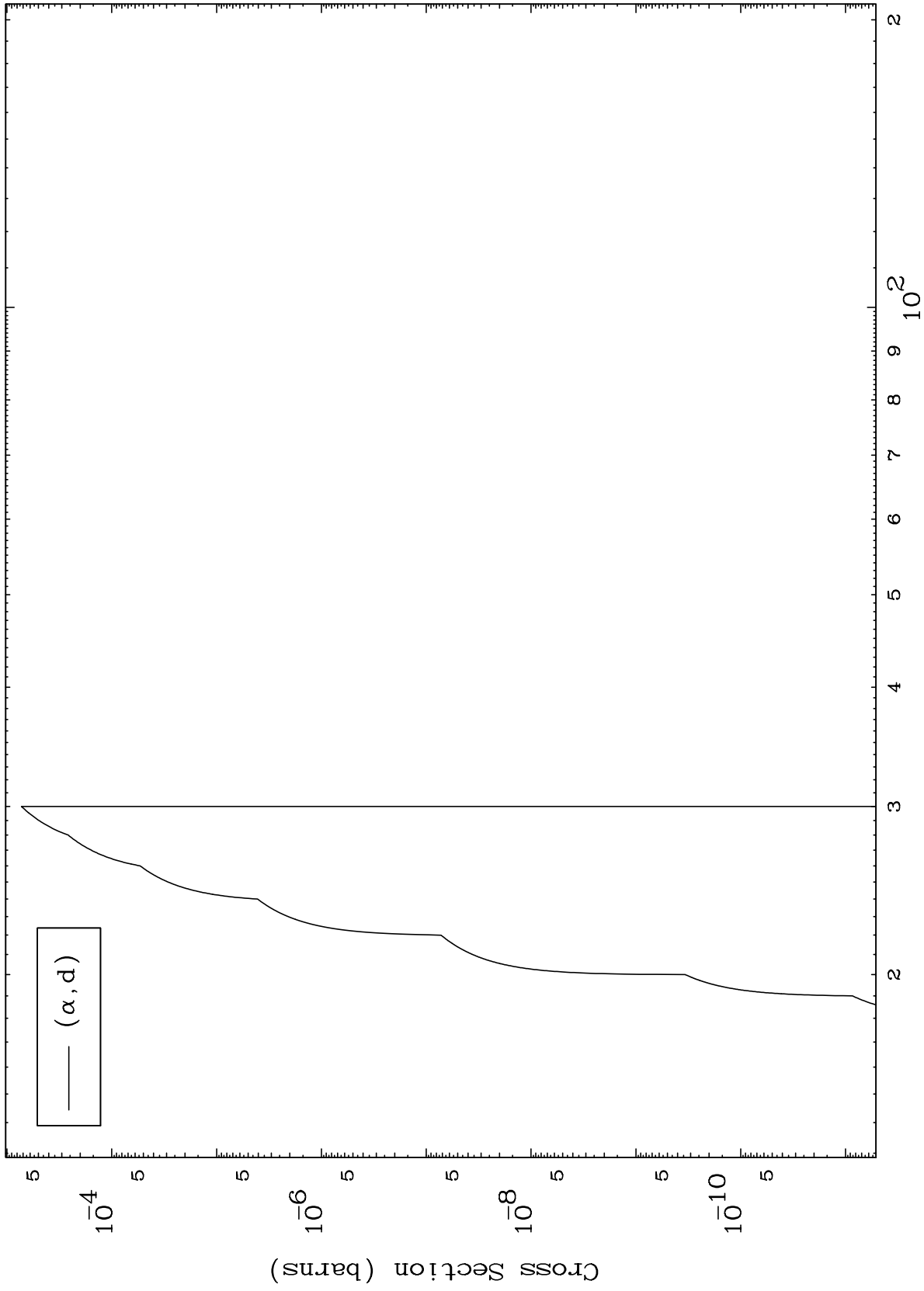
79-Au-193



MAT 7913

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

79-Au-193



7

Incident Energy (MeV)

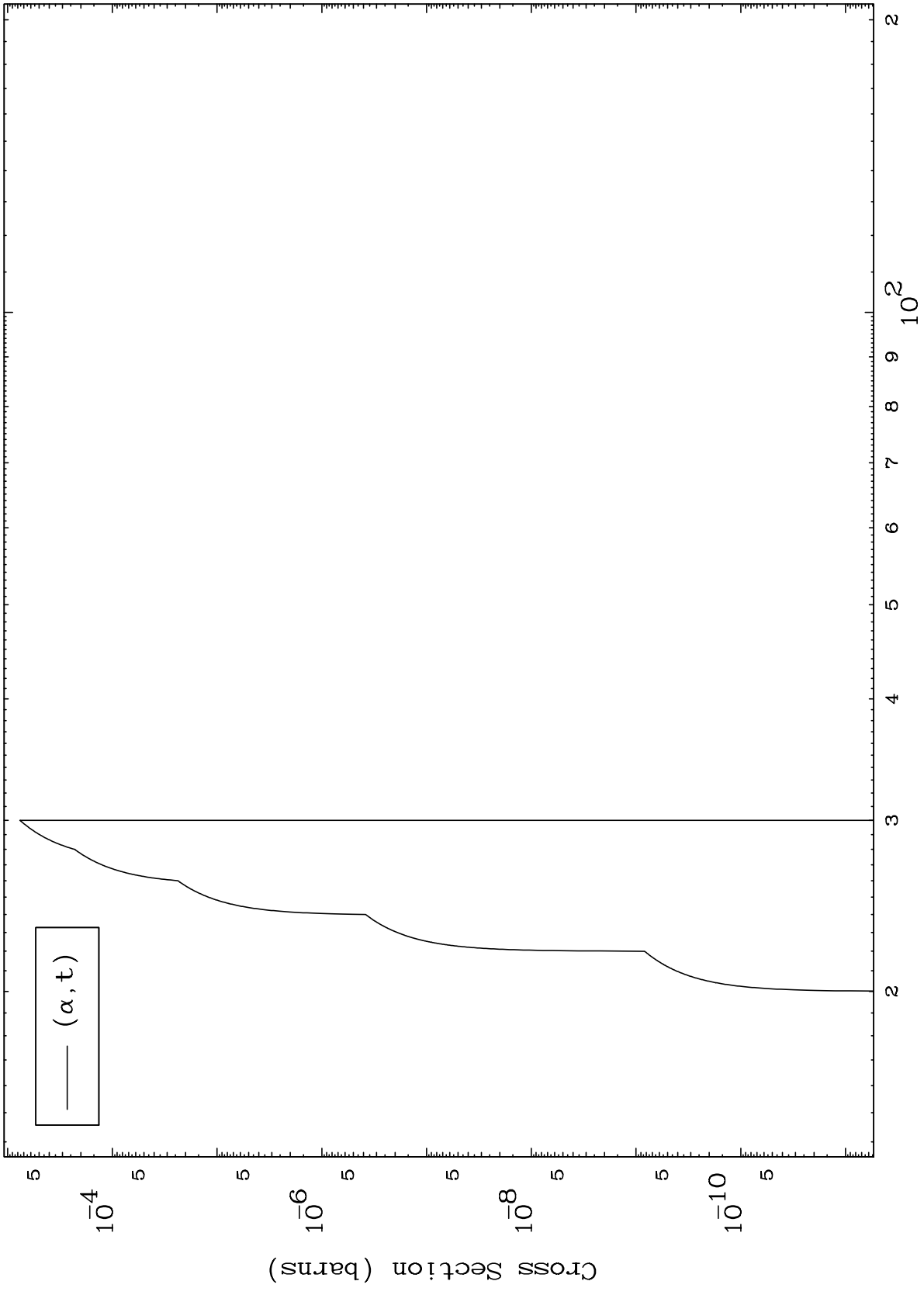
79-Au-193



MAT 7913

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

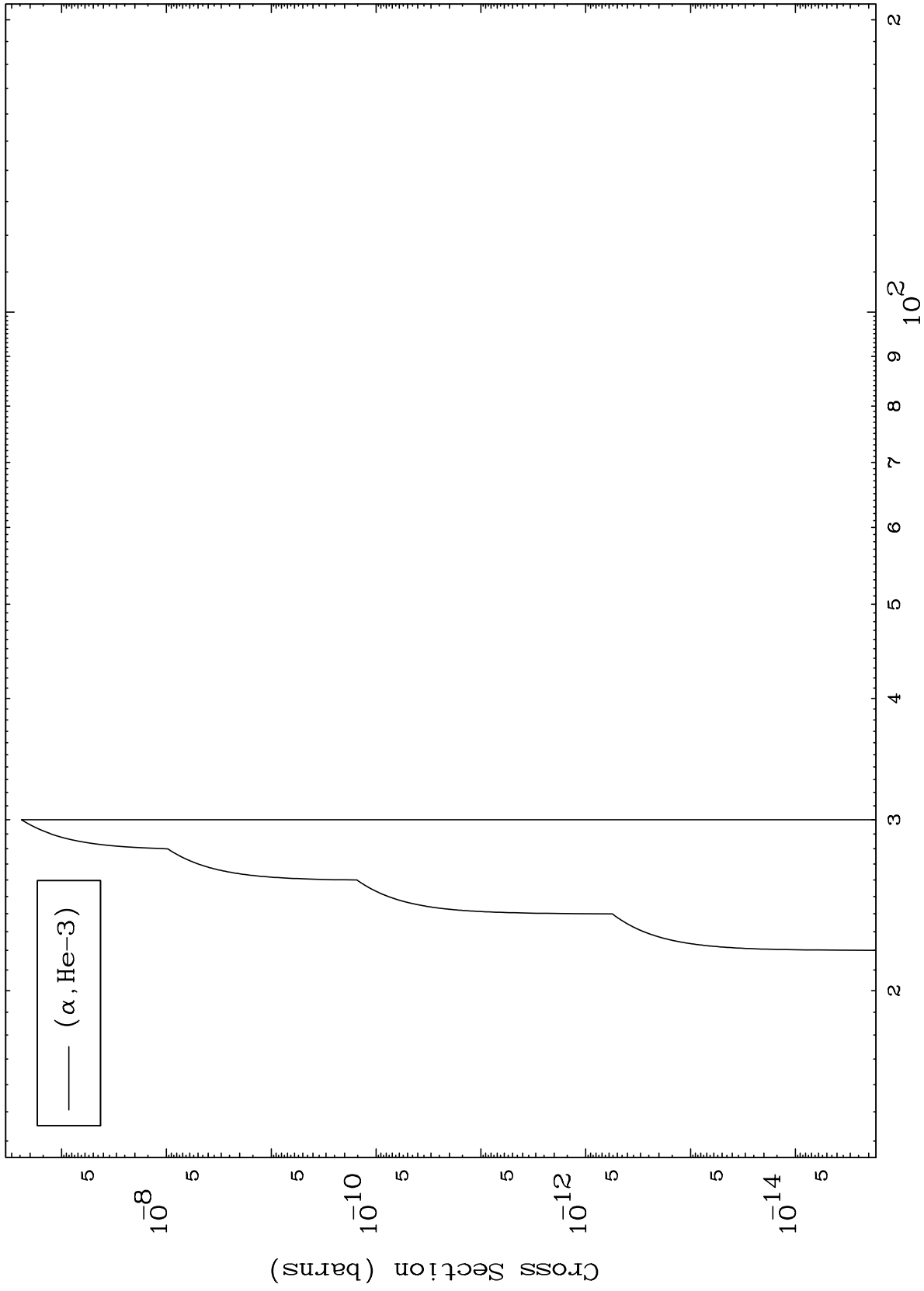
79-Au-193



8

Incident Energy (MeV)

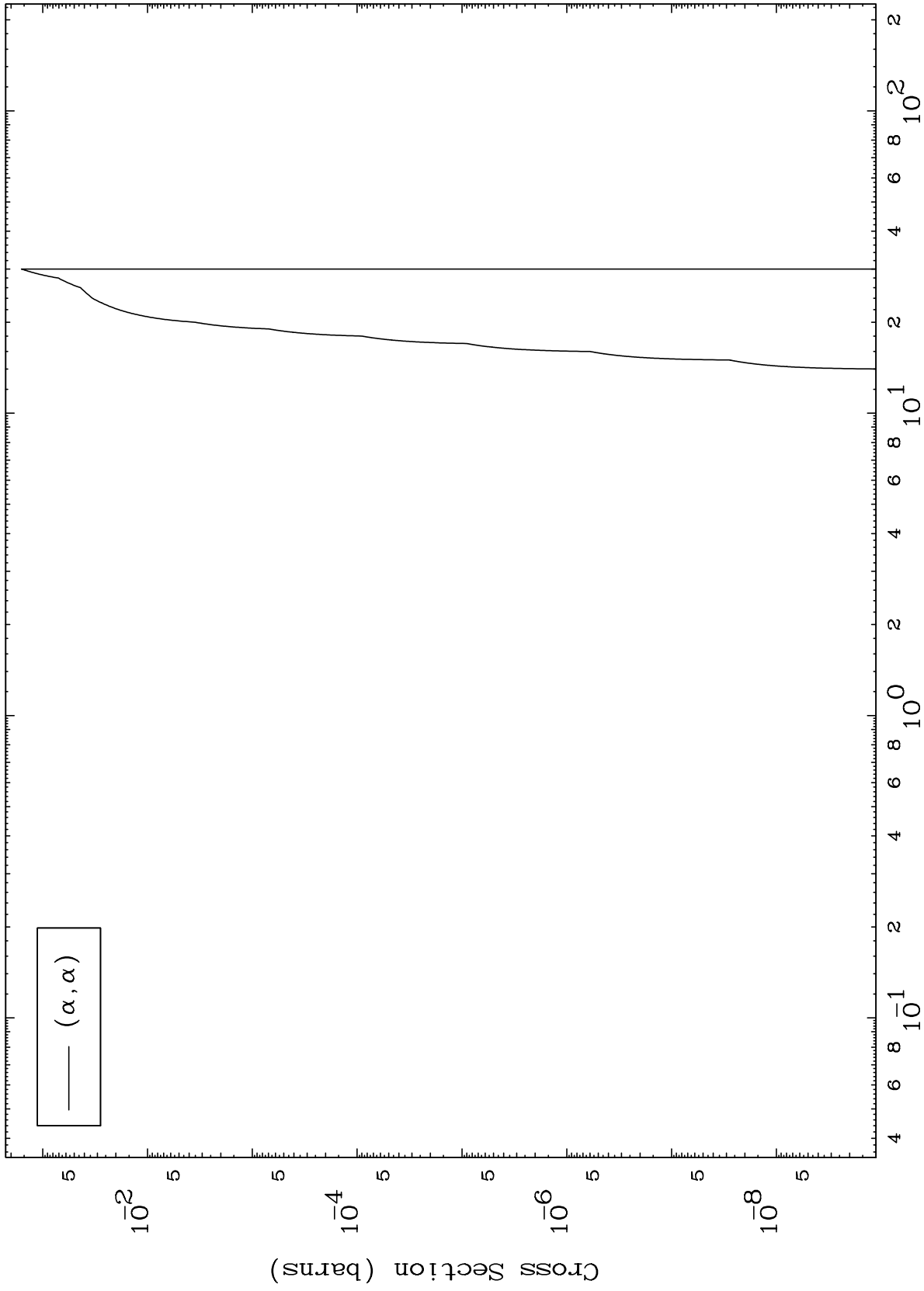
79-Au-193



MAT 7913

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

79-Au-193

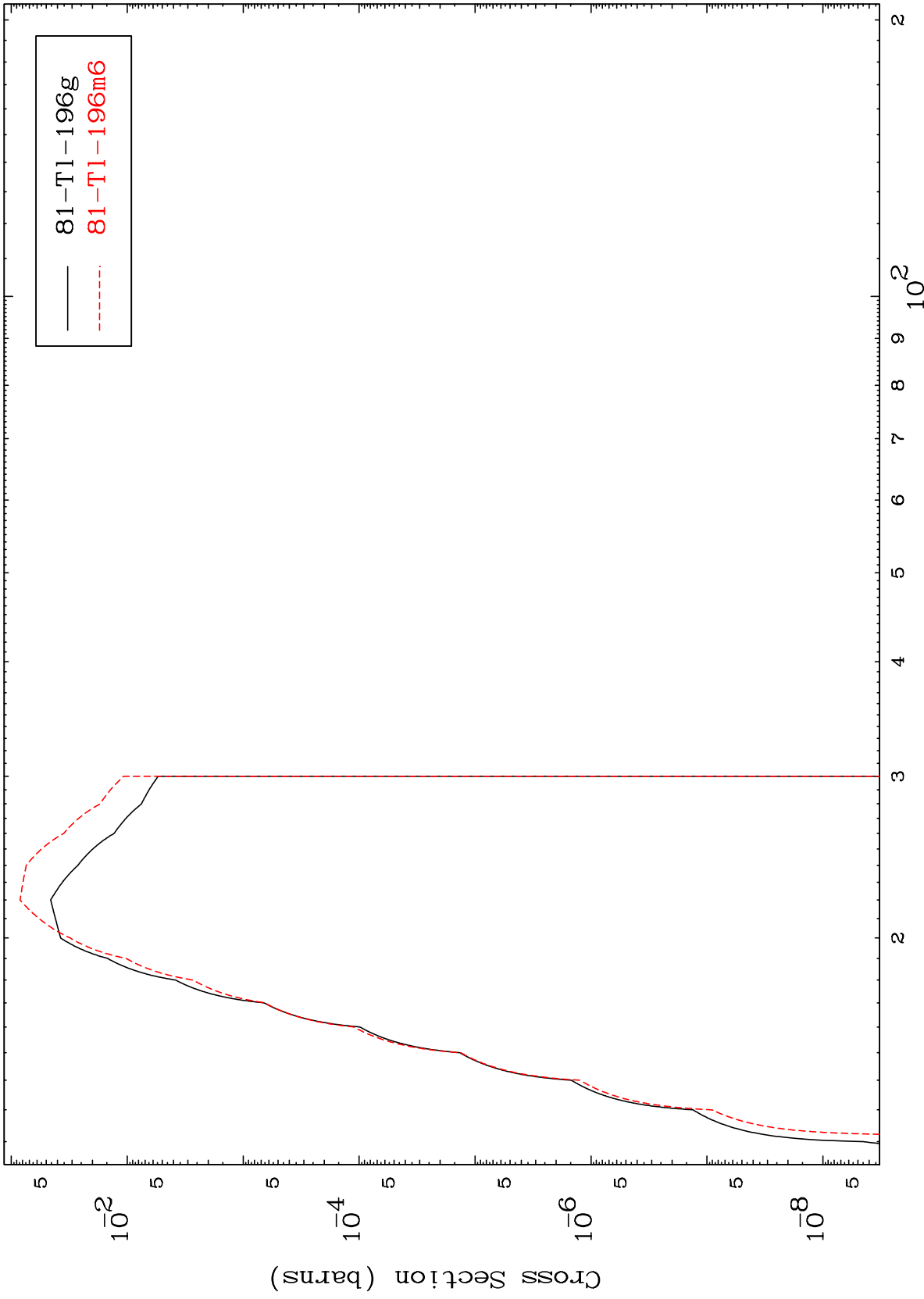


10

Incident Energy (MeV)

79-Au-193

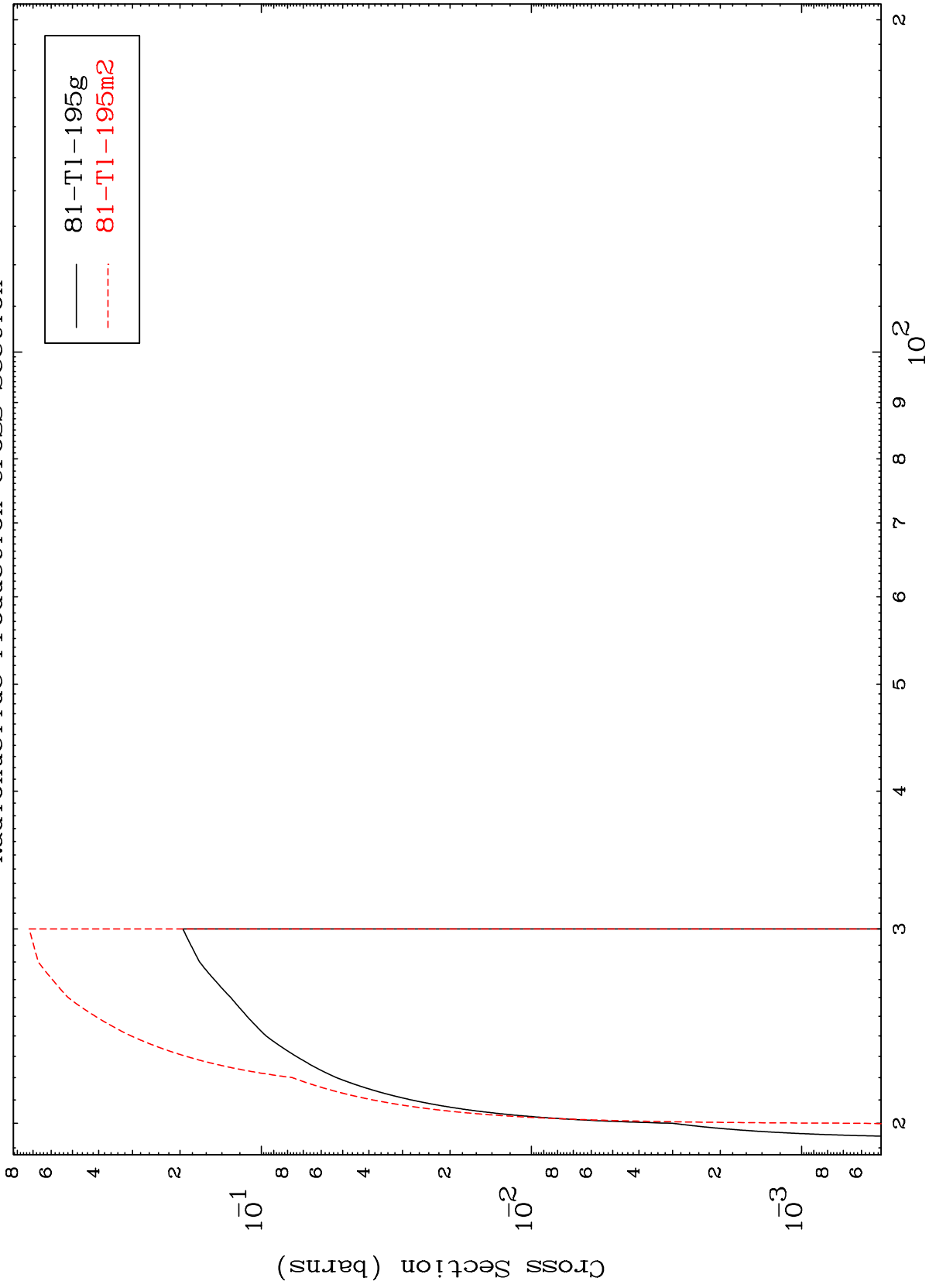
Radionuclide Production Cross Section



MAT 7913

79-Au-193

( $\alpha, 2n$ )  
Radionuclide Production Cross Section



12

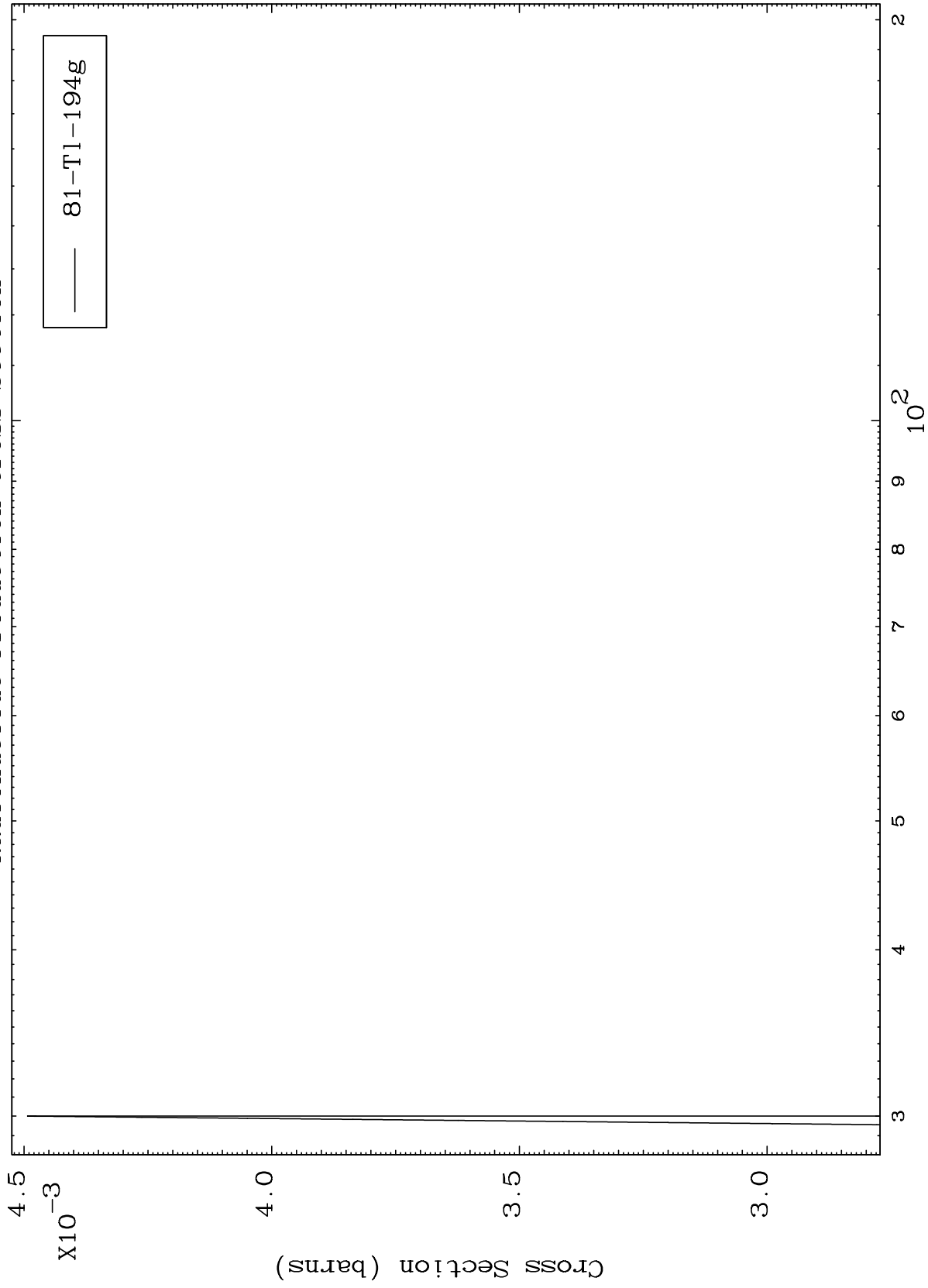
Incident Energy (MeV)

79-Au-193

MAT 7913

79-Au-193

( $\alpha, 3n$ )  
Radionuclide Production Cross Section



13

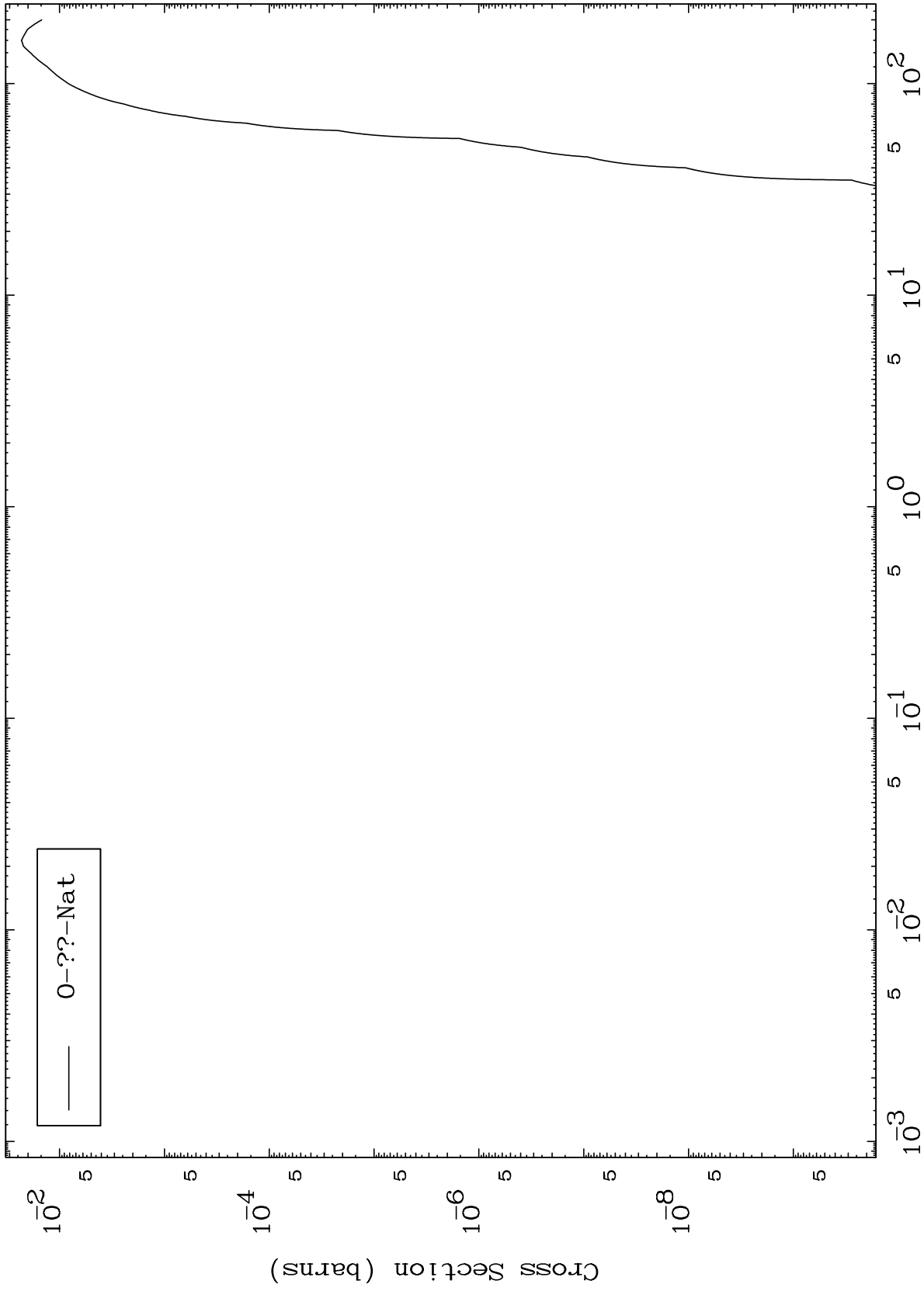
Incident Energy (MeV)

79-Au-193

MAT 7913

$\alpha$  Fission  
Radionuclide Production Cross Section

79-Au-193



14

Incident Energy (MeV)

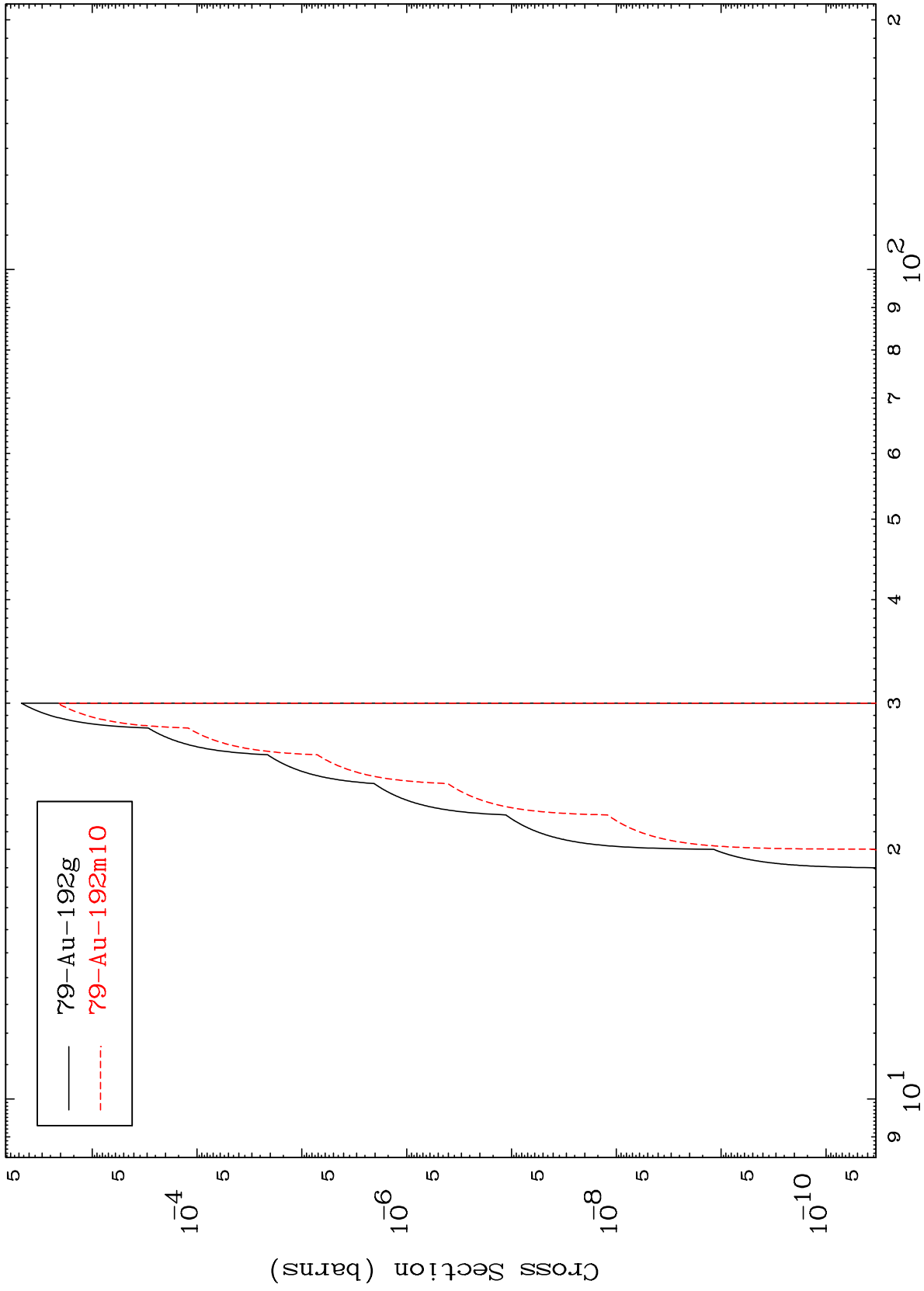
79-Au-193

MAT 7913

$(\alpha, n')$   $\alpha$

79-Au-193

Radionuclide Production Cross Section



15

Incident Energy (MeV)

79-Au-193

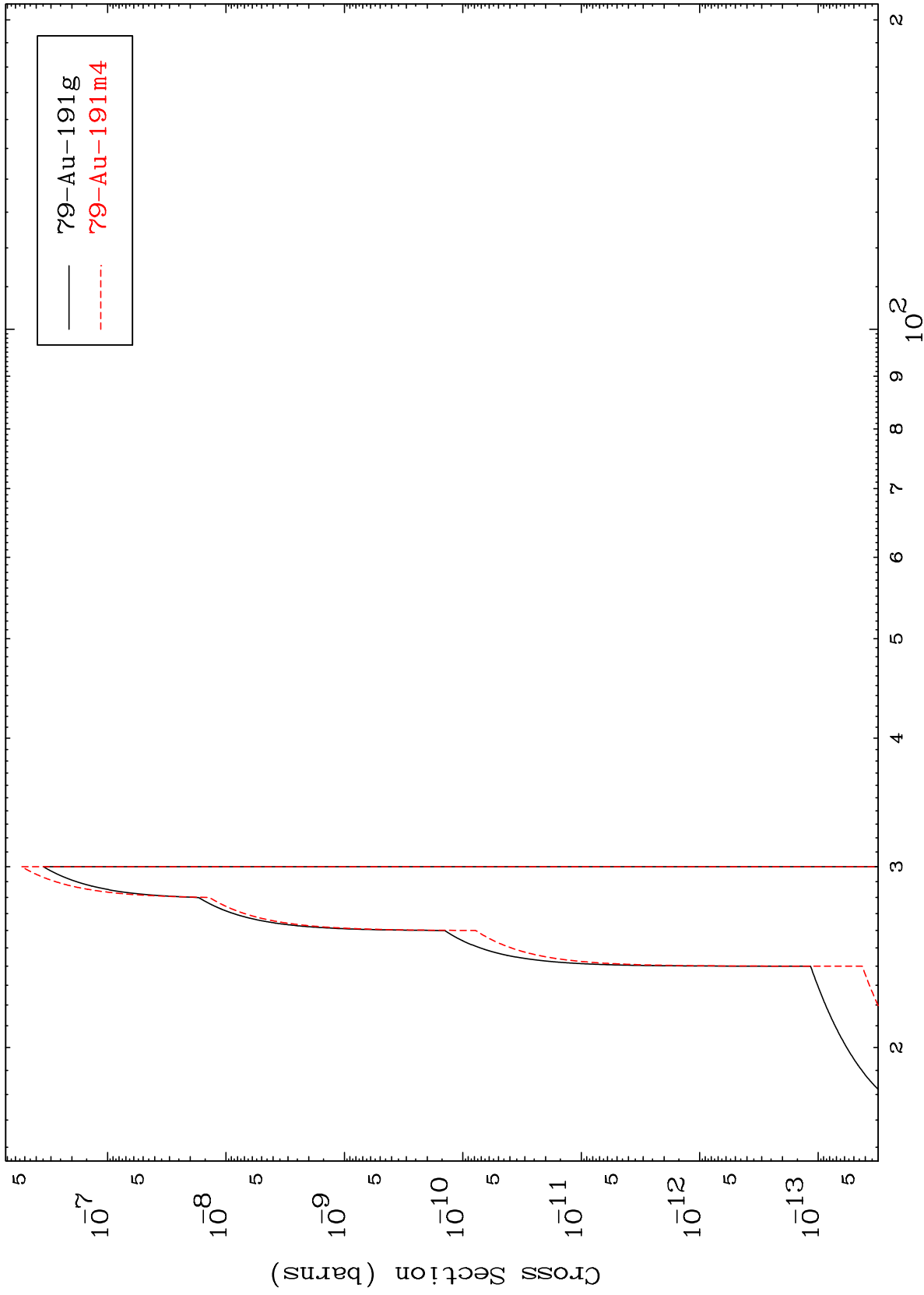


MAT 7913

$(\alpha, 2n) \alpha$

$^{79}\text{Au-193}$

Radionuclide Production Cross Section



16

Incident Energy (MeV)

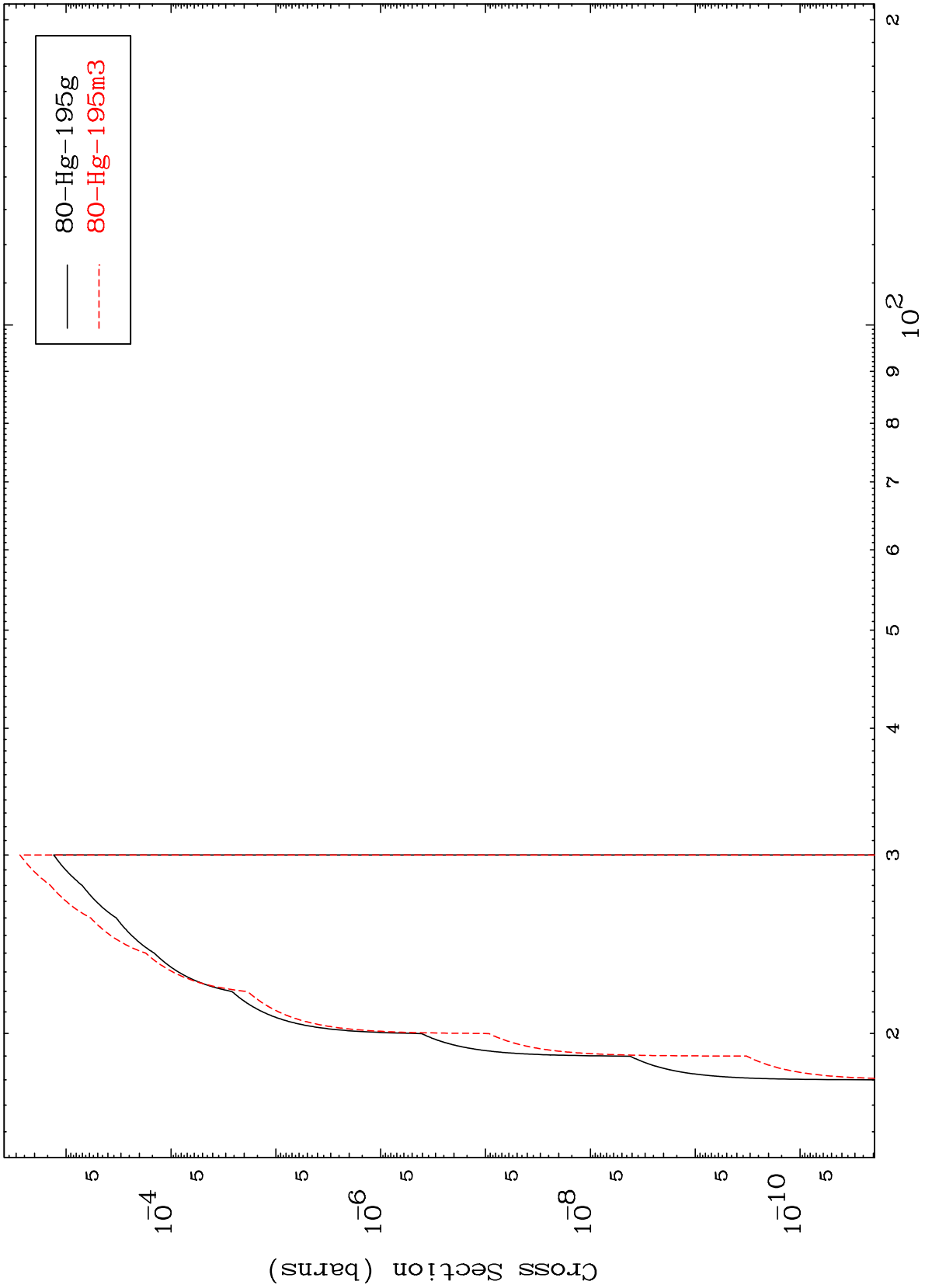
$^{79}\text{Au-193}$

MAT 7913

( $\alpha, n'$ ) p

79-Au-193

Radionuclide Production Cross Section



17

Incident Energy (MeV)

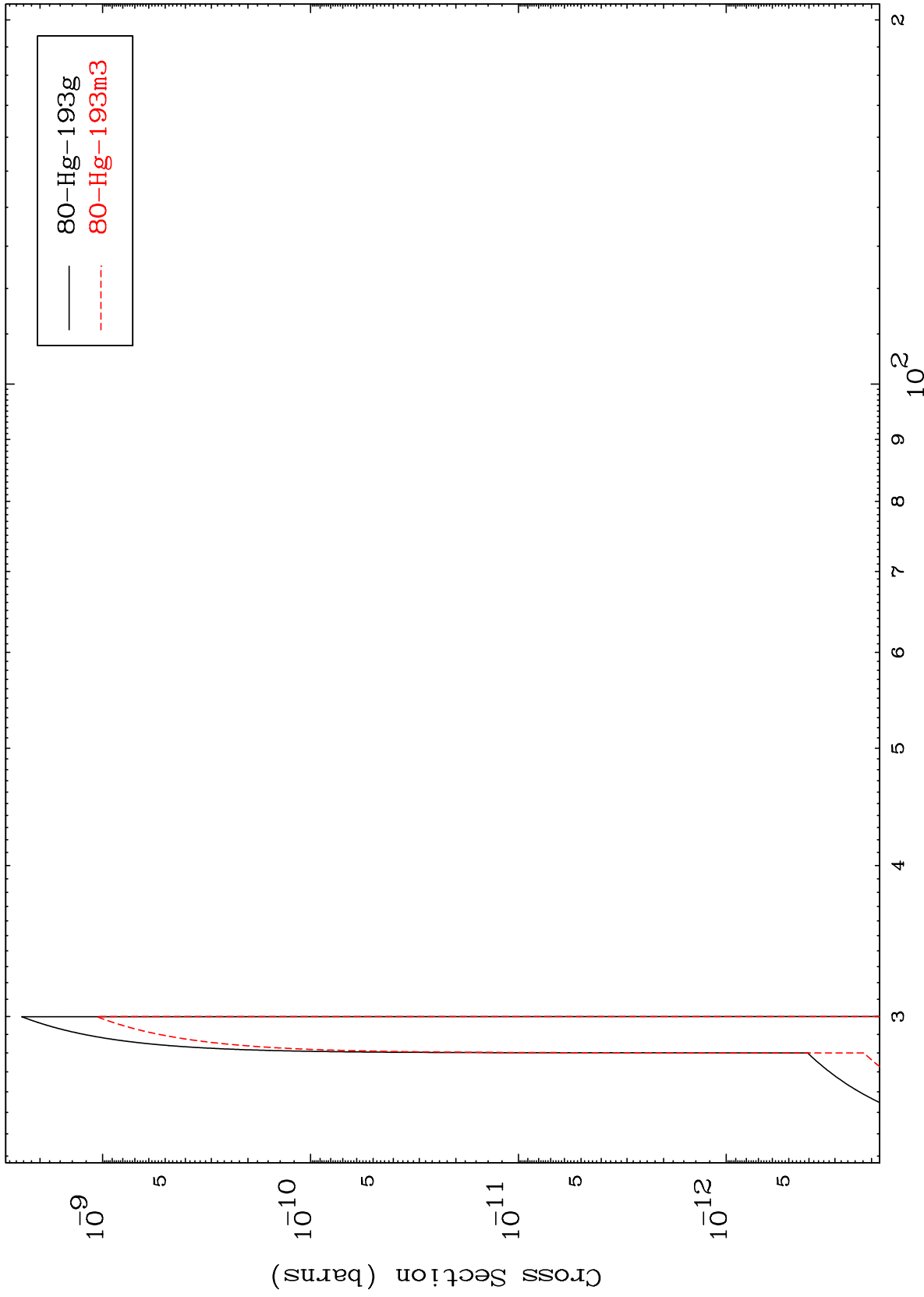
79-Au-193

MAT 7913

( $\alpha, n'$ ) t

79-Au-193

Radionuclide Production Cross Section



18

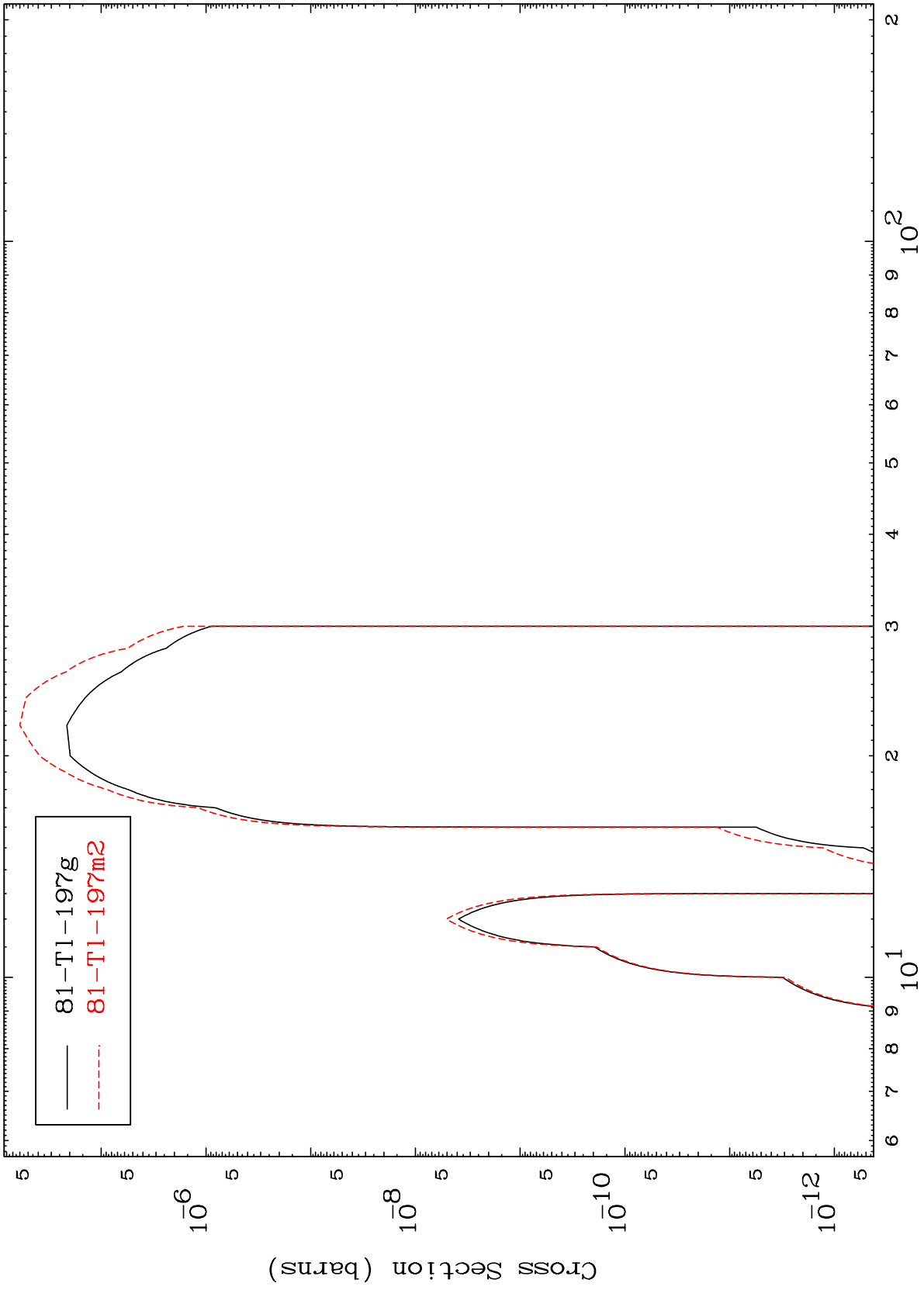
Incident Energy (MeV)

79-Au-193

MAT 7913

79-Au-193

$(\alpha, \gamma)$   
Radionuclide Production Cross Section



19

Incident Energy (MeV)

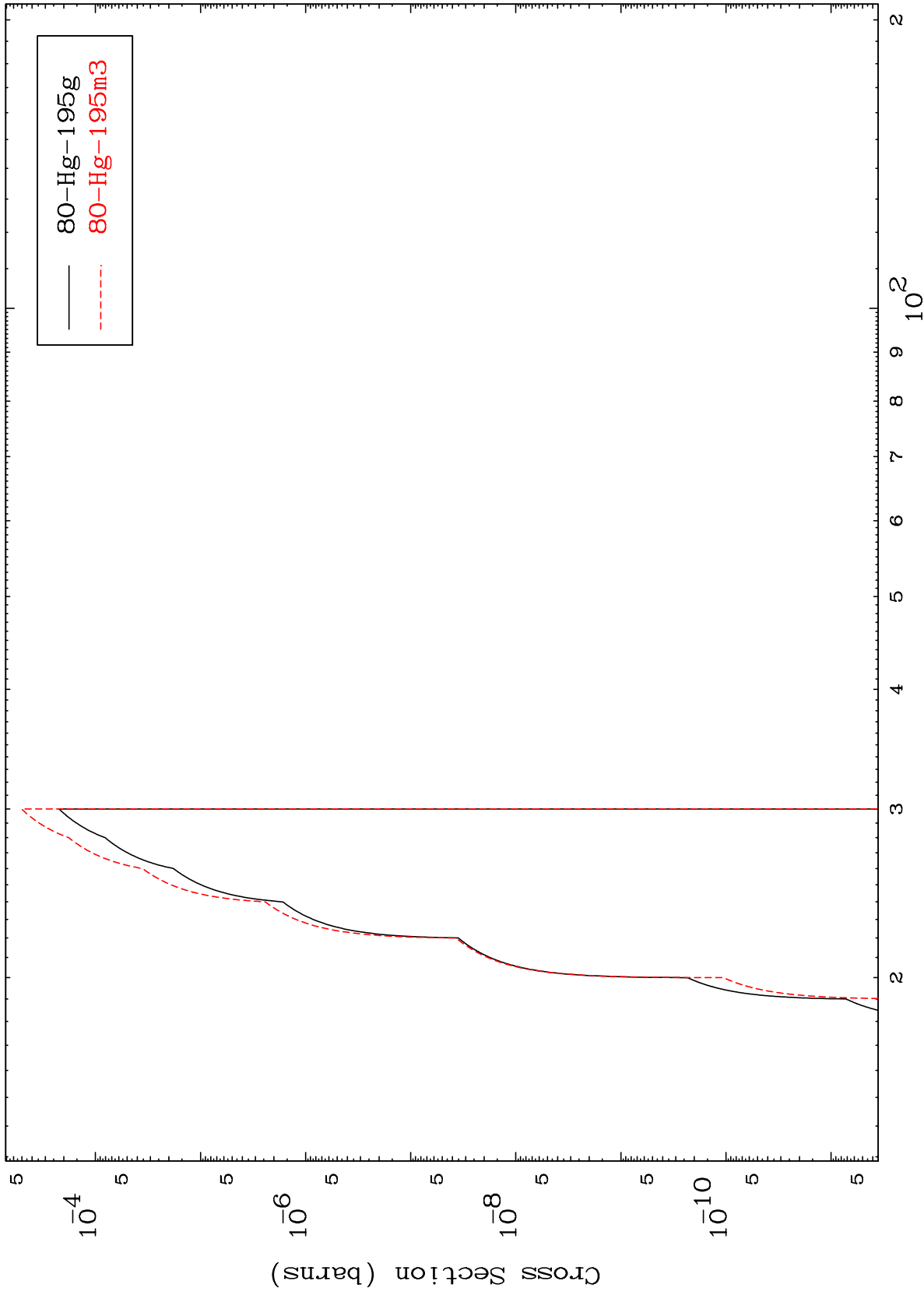
79-Au-193

MAT 7913

( $\alpha, d$ )

79-Au-193

Radionuclide Production Cross Section



20

Incident Energy (MeV)

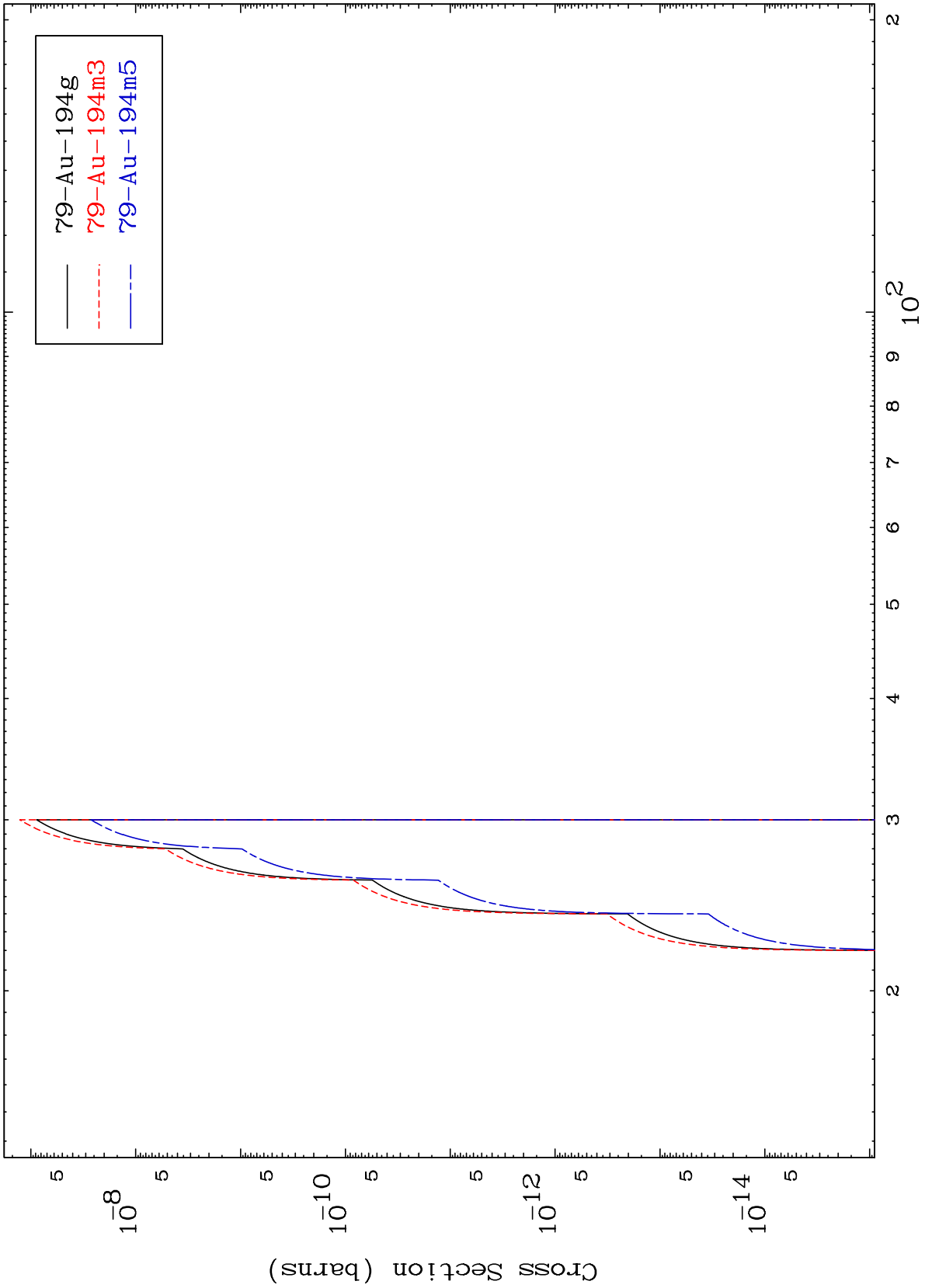
79-Au-193

MAT 7913

( $\alpha, \text{He-3}$ )

$^{79}\text{Au-193}$

Radionuclide Production Cross Section



21

Incident Energy (MeV)

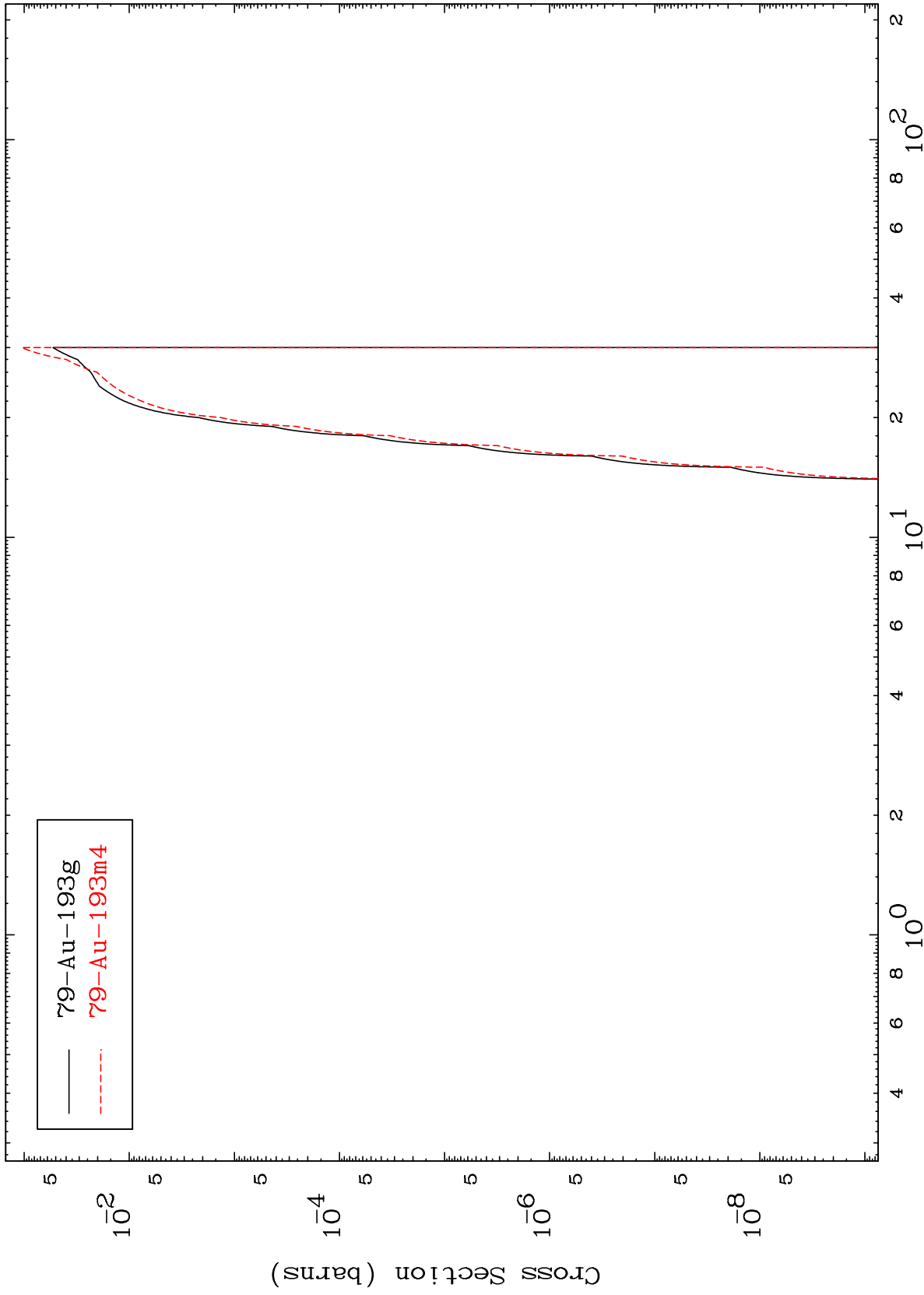
$^{79}\text{Au-193}$

MAT 7913

( $\alpha, \alpha$ )

<sup>79</sup>Au-193

Radionuclide Production Cross Section



22

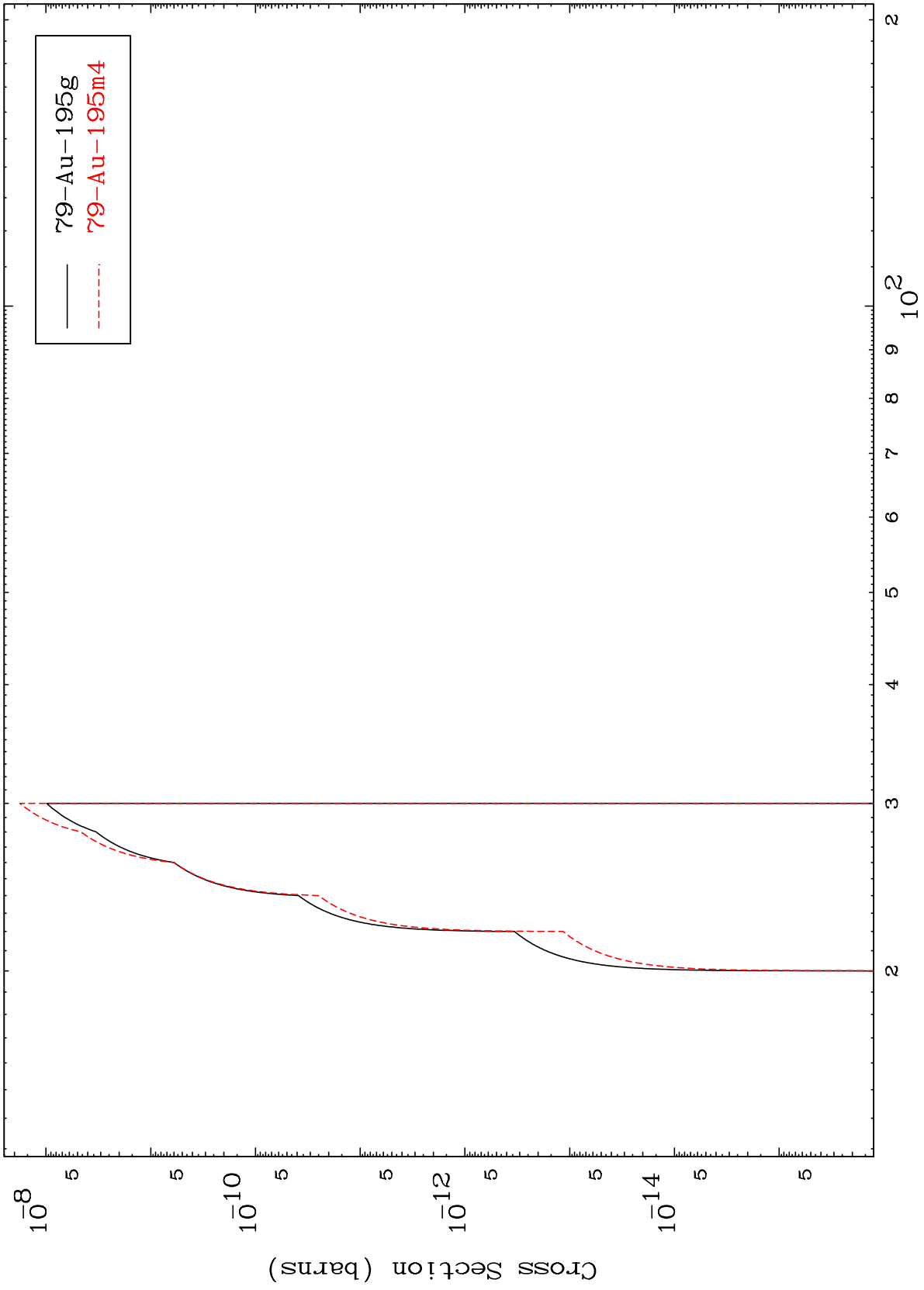
Incident Energy (MeV)

<sup>79</sup>Au-193

MAT 7913

<sup>79</sup>Au-193

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



23

<sup>79</sup>Au-193