

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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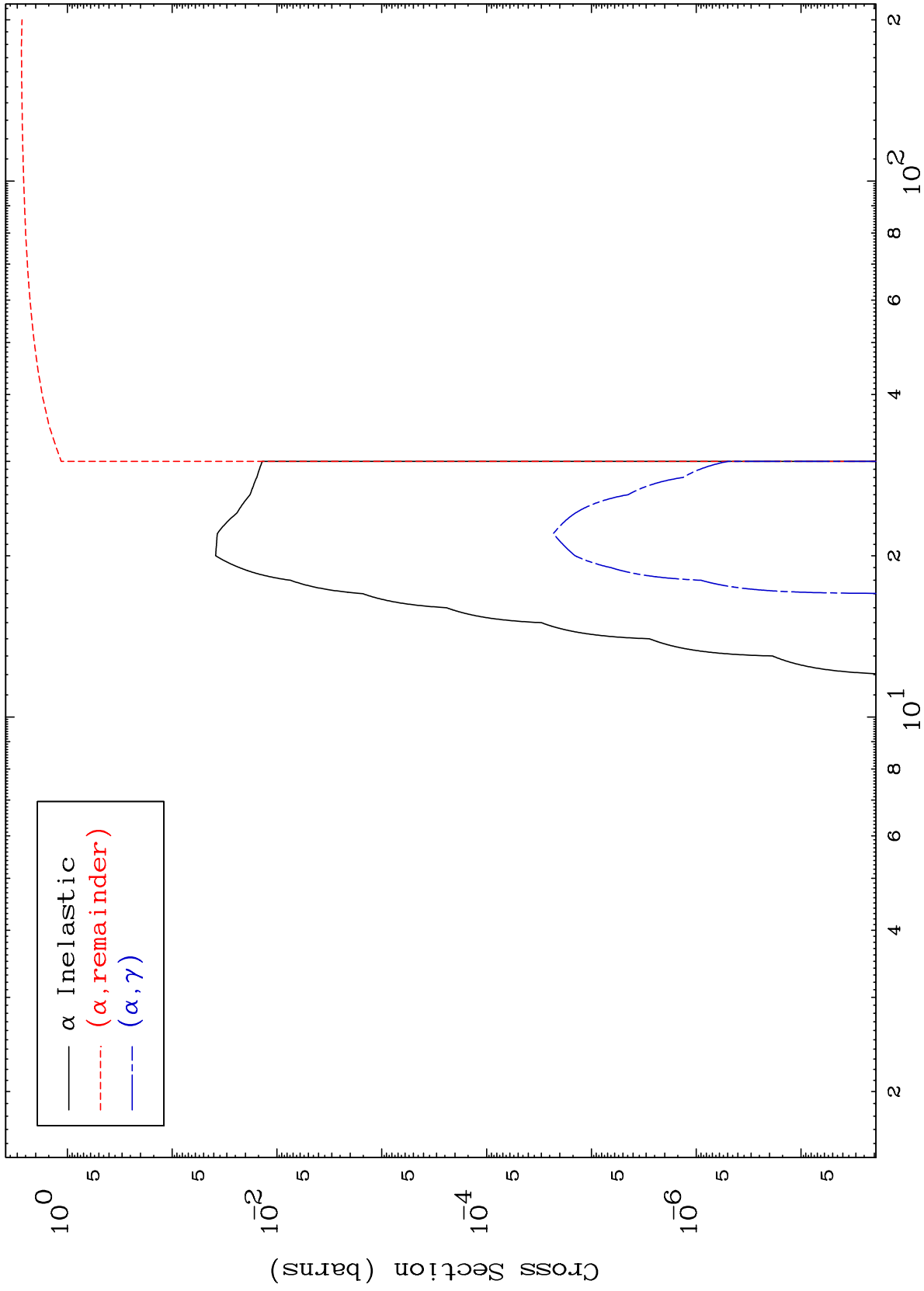
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7923

0 Kelvin  $\alpha$  Major  
Cross Sections

79-Au-196



1

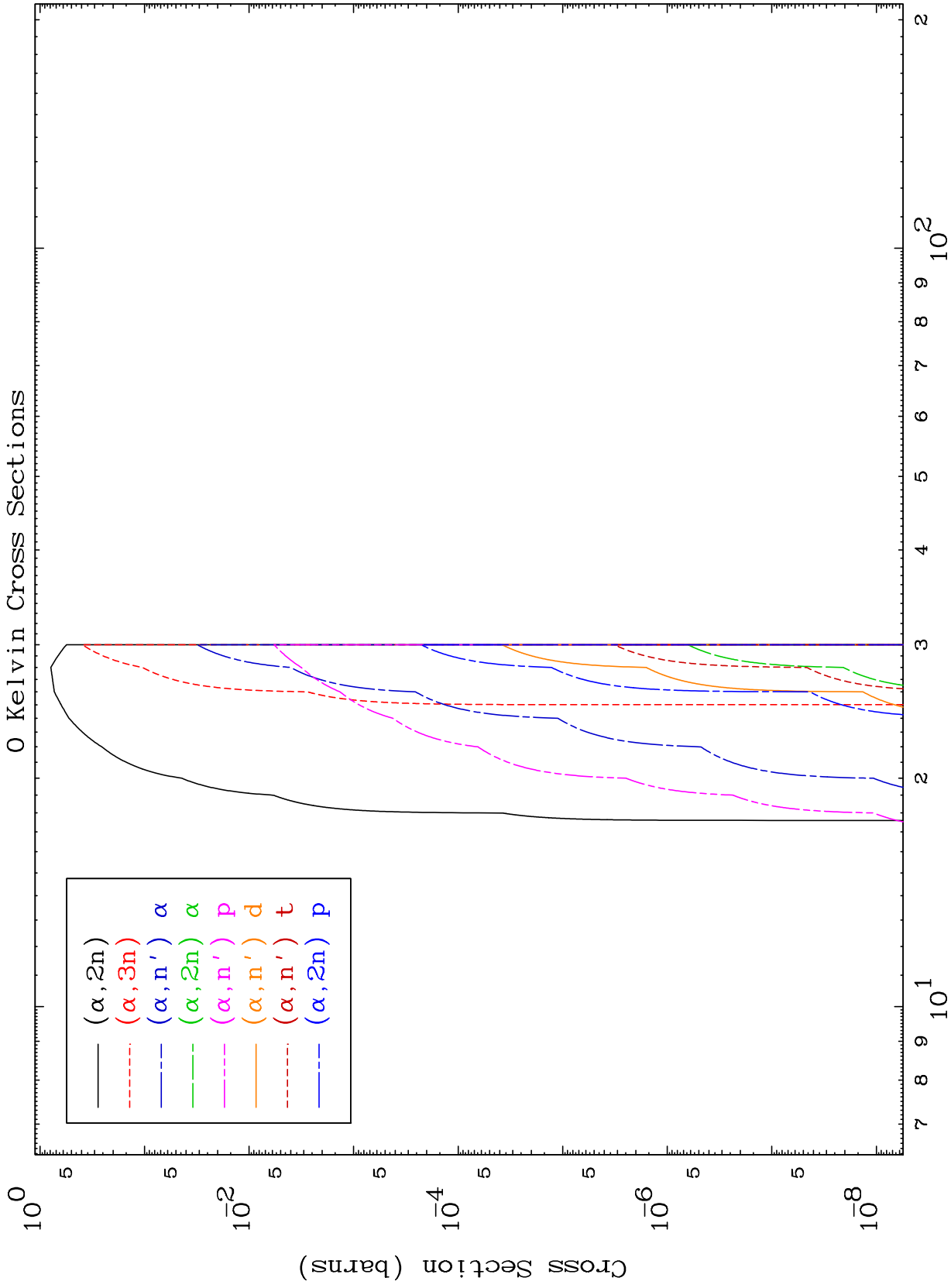
Incident Energy (MeV)

79-Au-196

MAT 7923

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

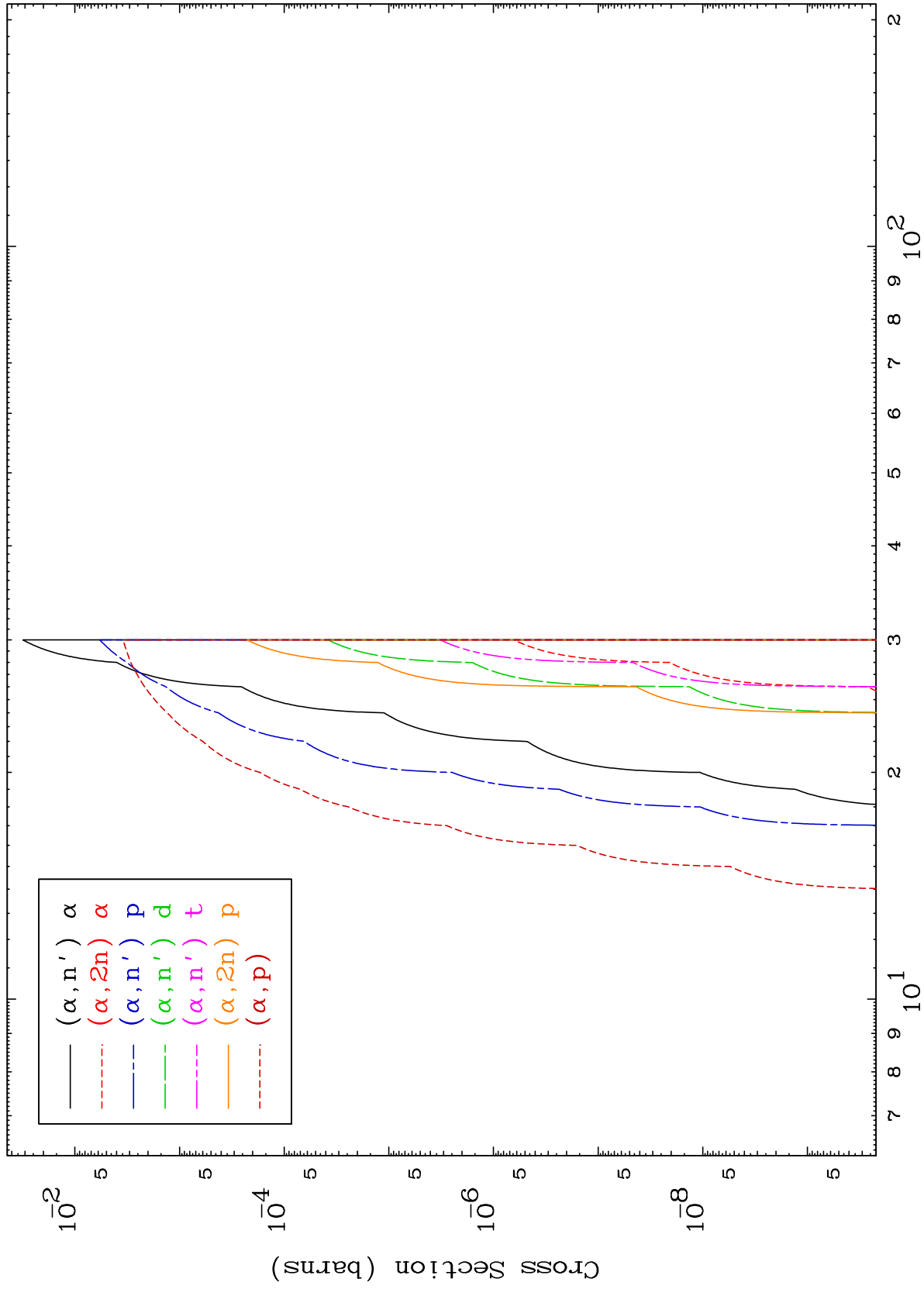
79-Au-196



2

Incident Energy (MeV)

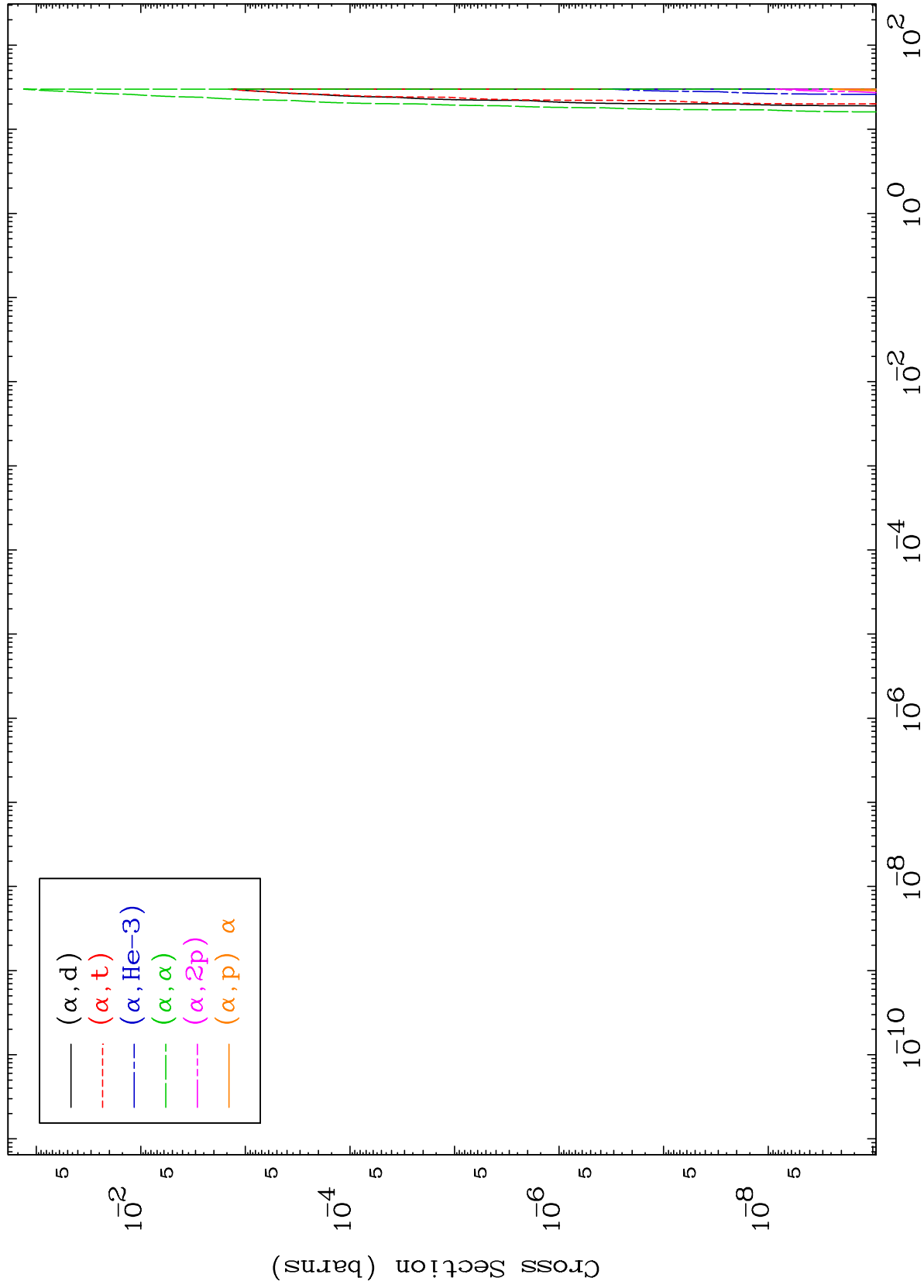
79-Au-196



MAT 7923

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

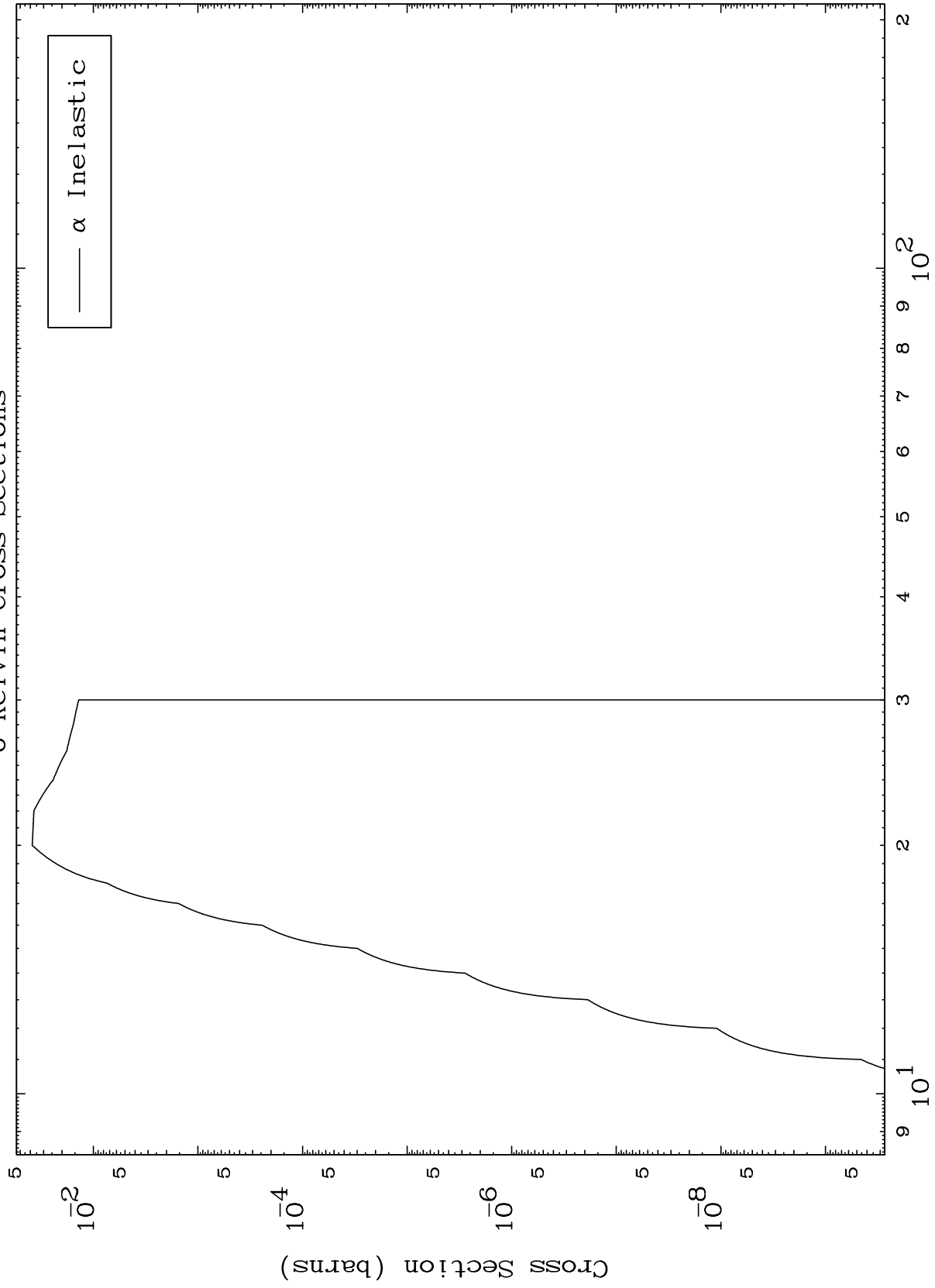
79-Au-196



MAT 7923

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

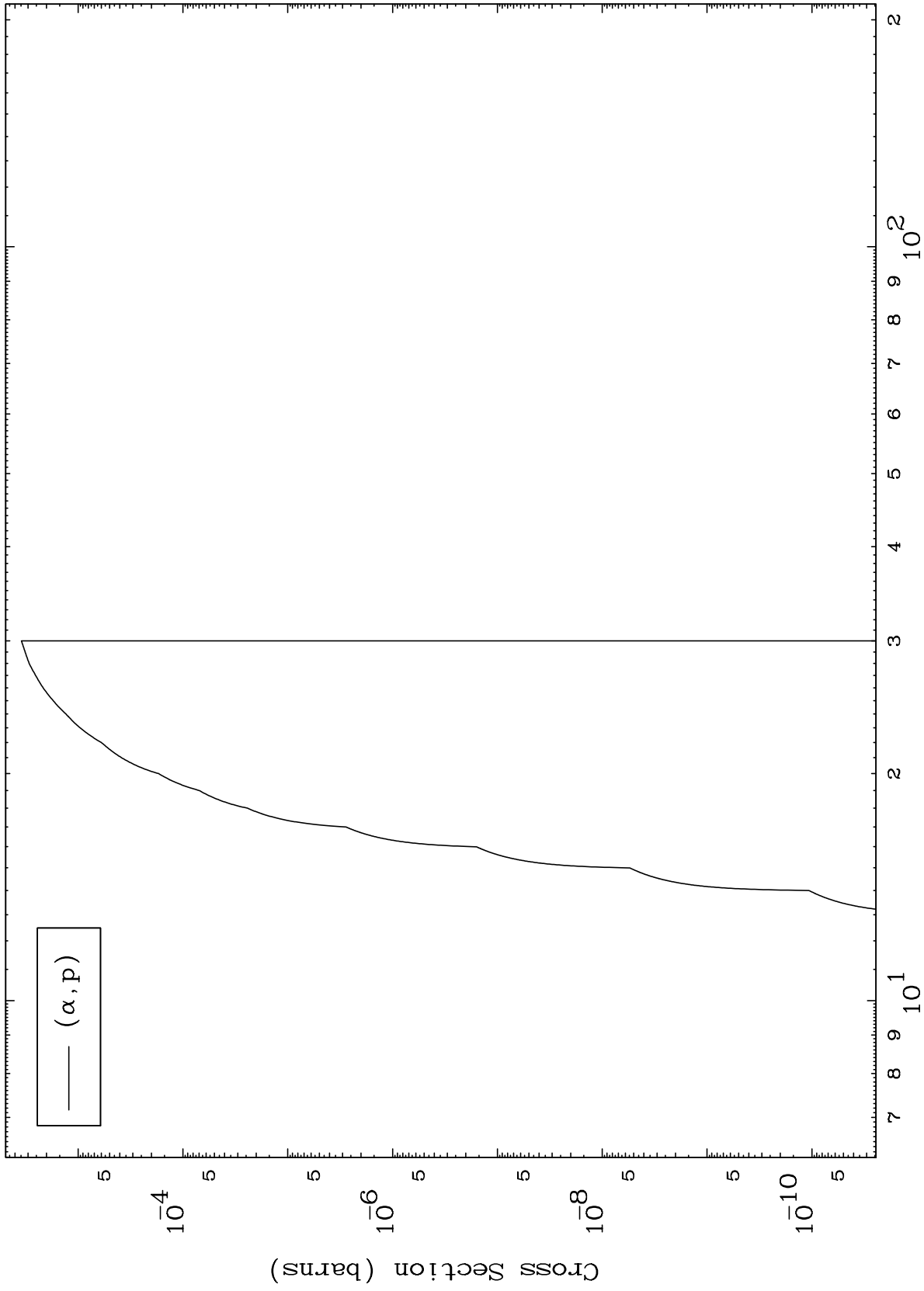
79-Au-196



5

Incident Energy (MeV)

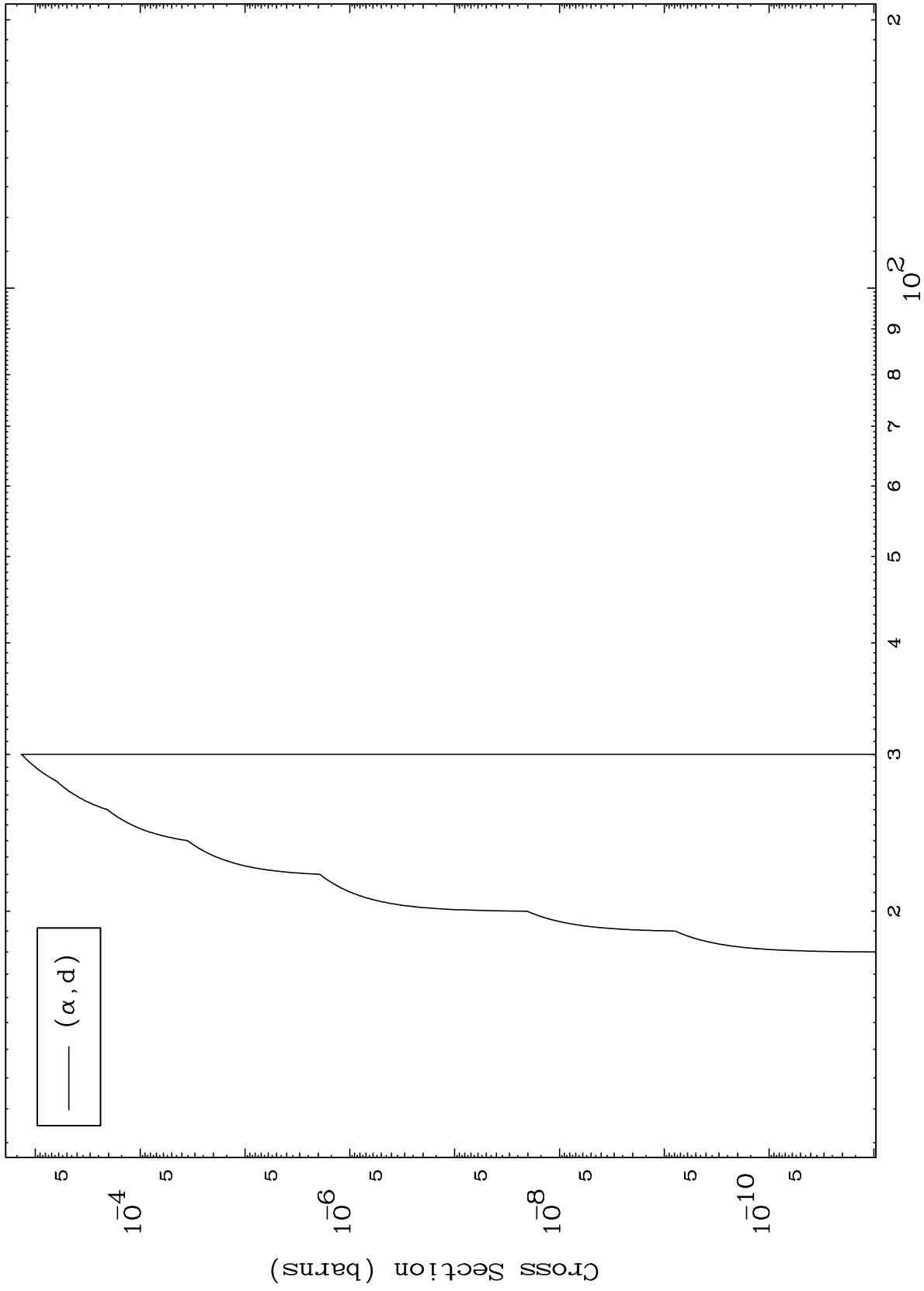
79-Au-196



MAT 7923

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

79-Au-196

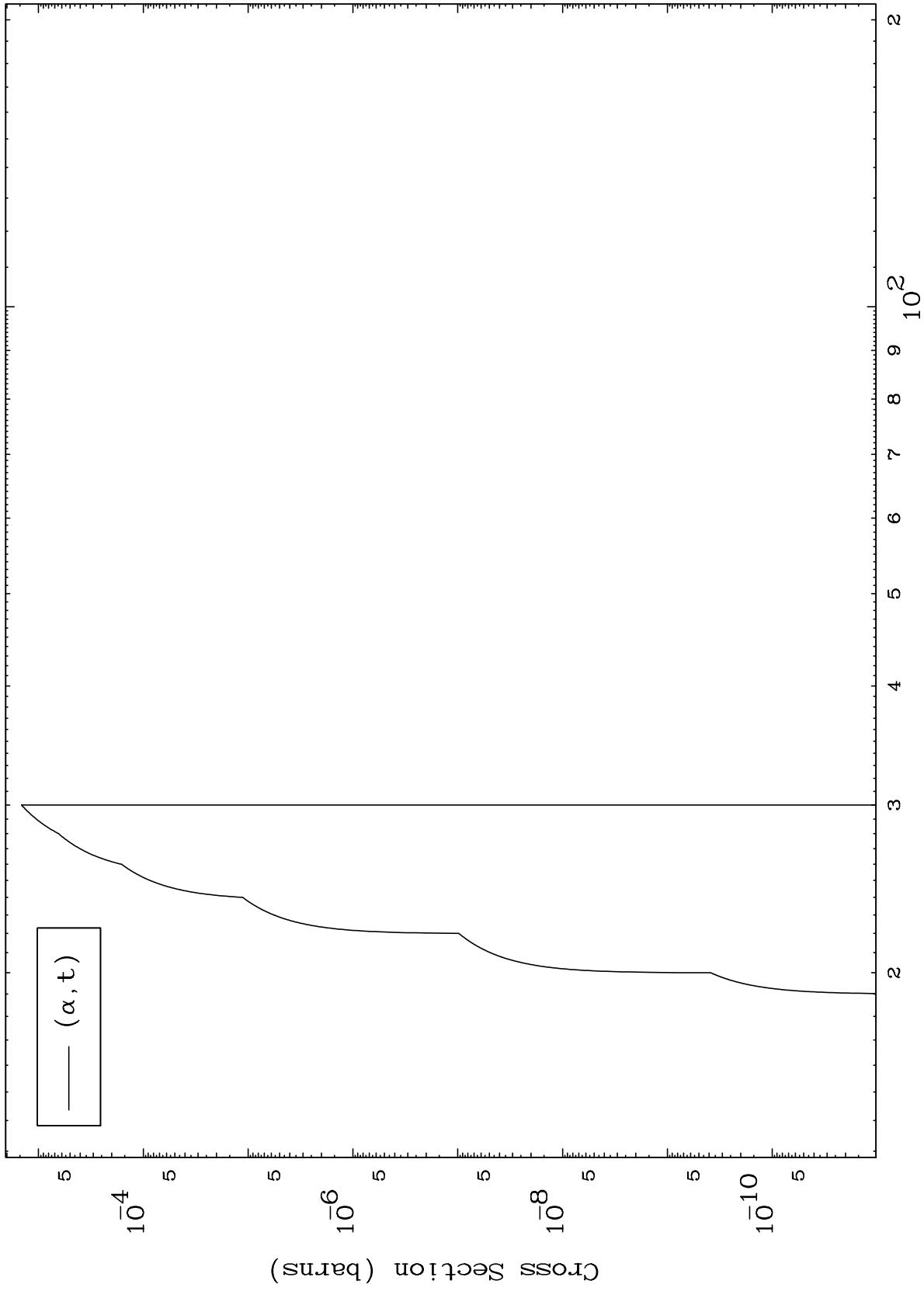


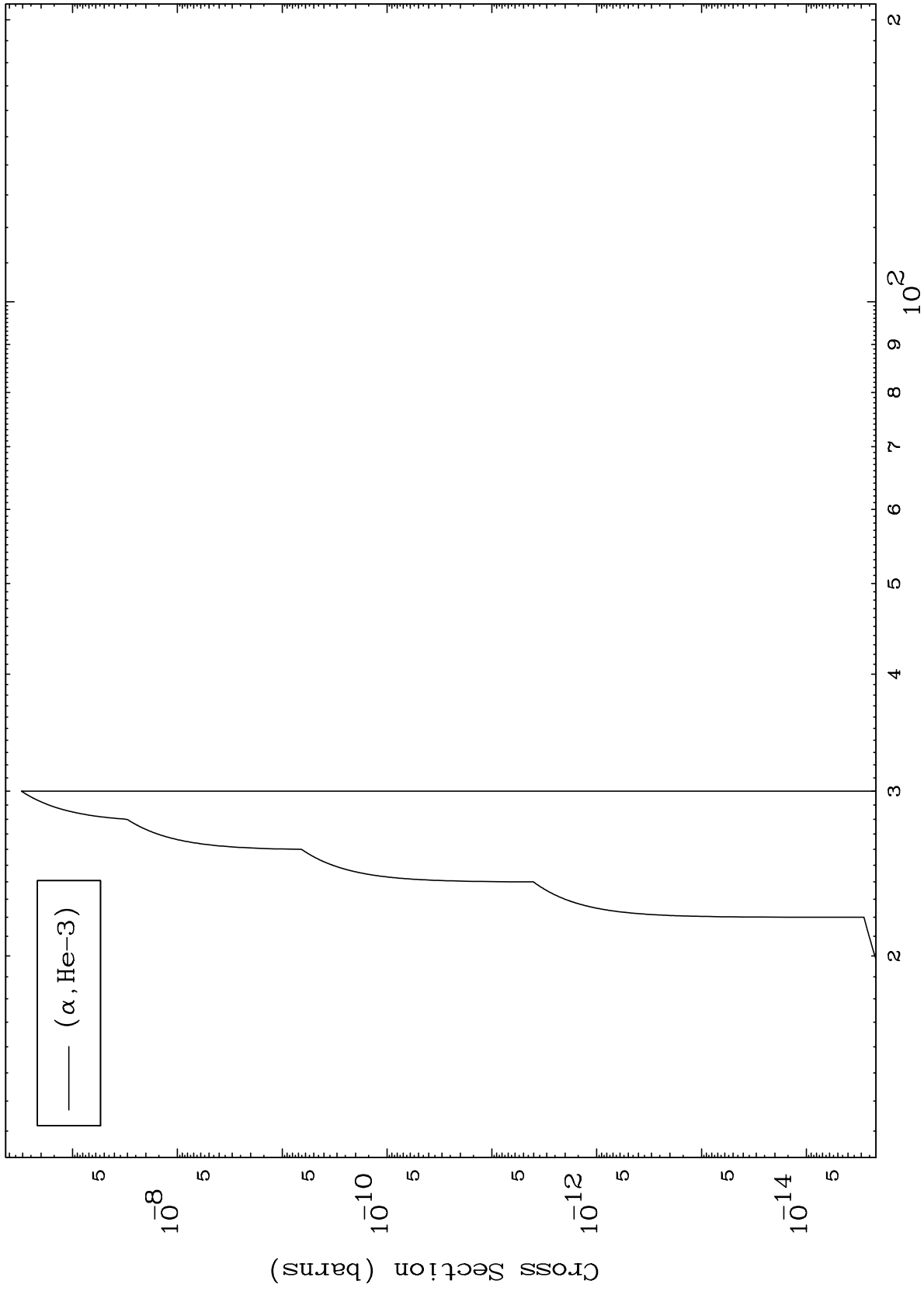
7

Incident Energy (MeV)

79-Au-196



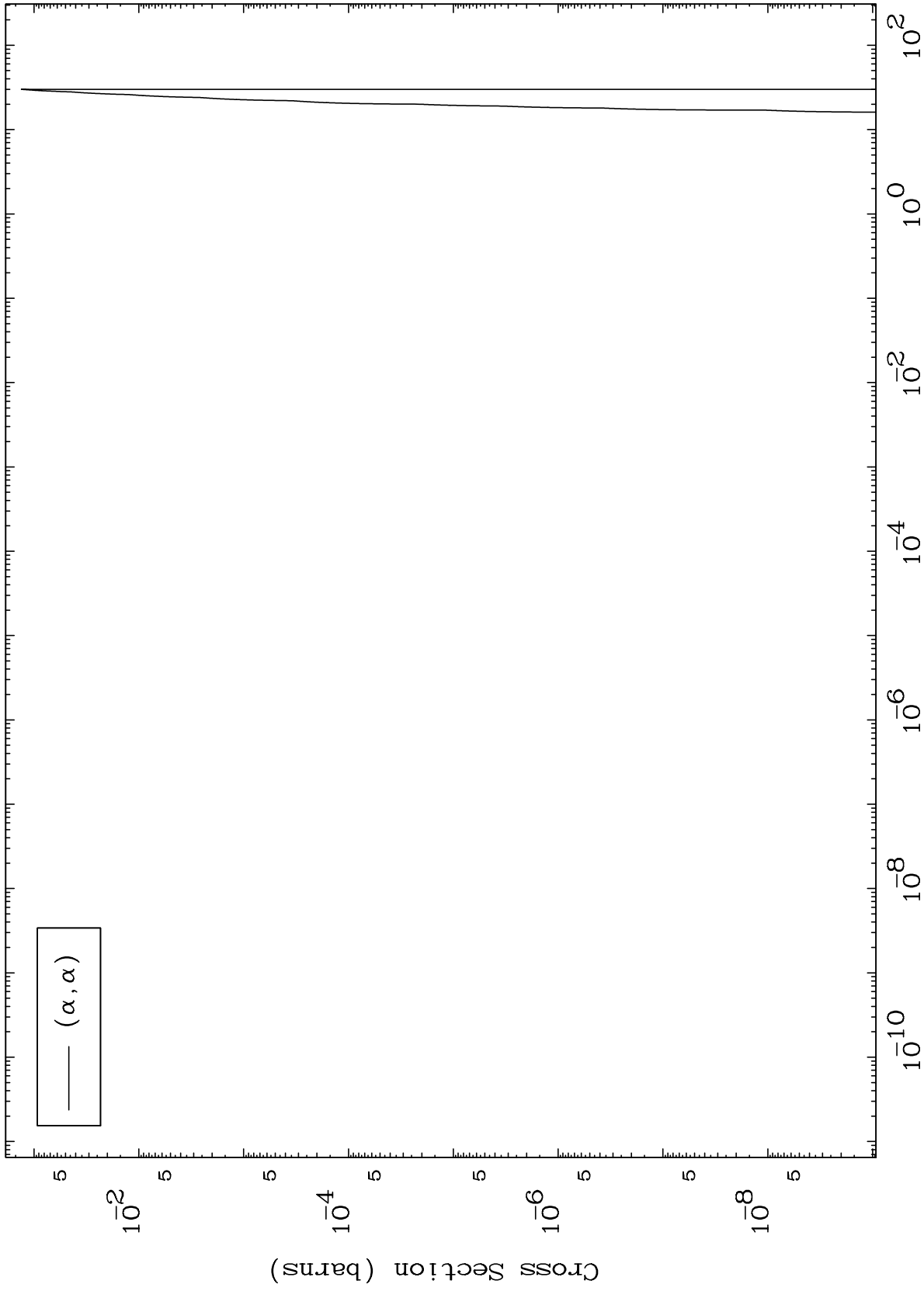




MAT 7923

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

79-Au-196



10

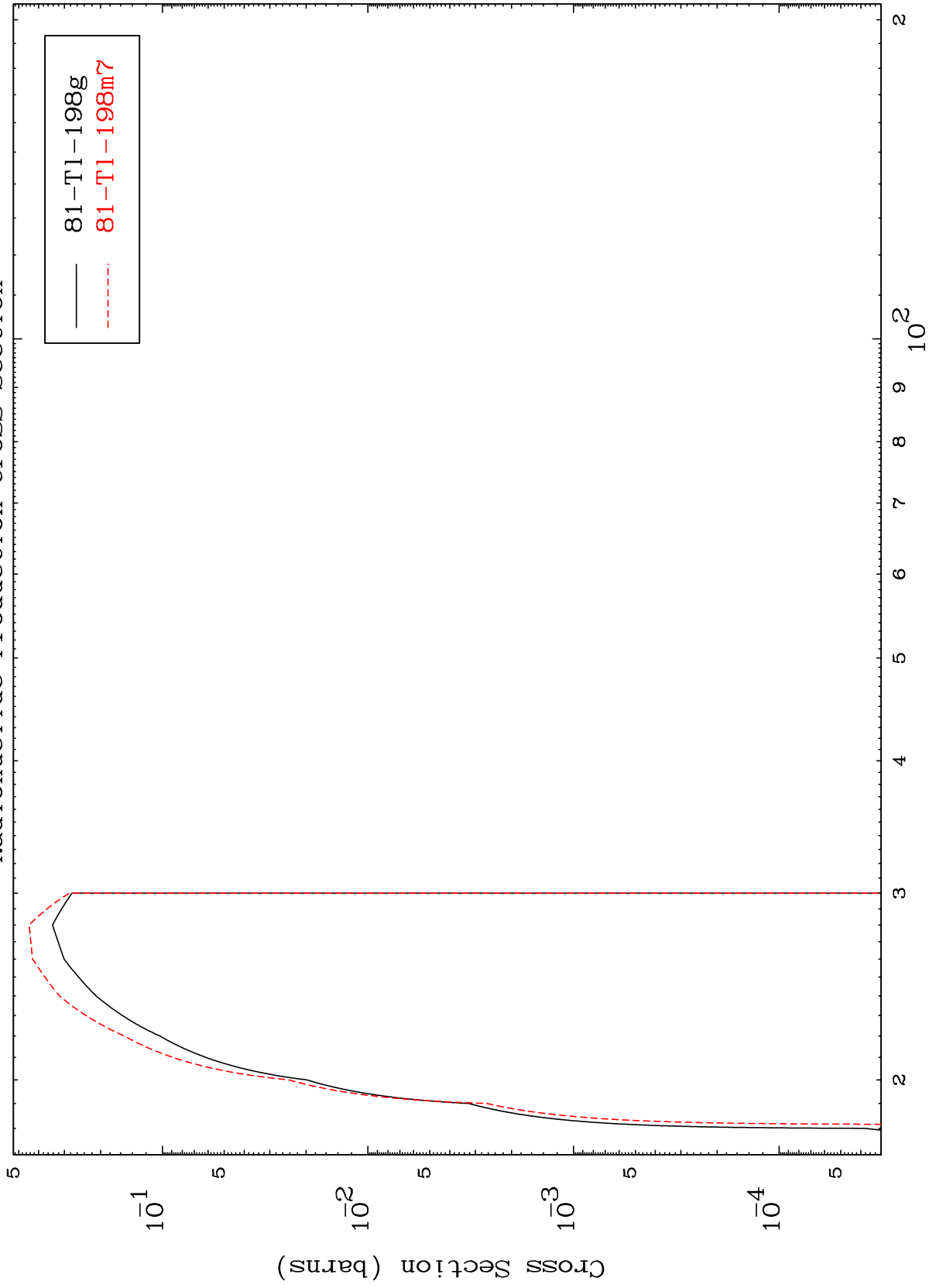
Incident Energy (MeV)

79-Au-196

MAT 7923

79-Au-196

( $\alpha, 2n$ )  
Radionuclide Production Cross Section



11

Incident Energy (MeV)

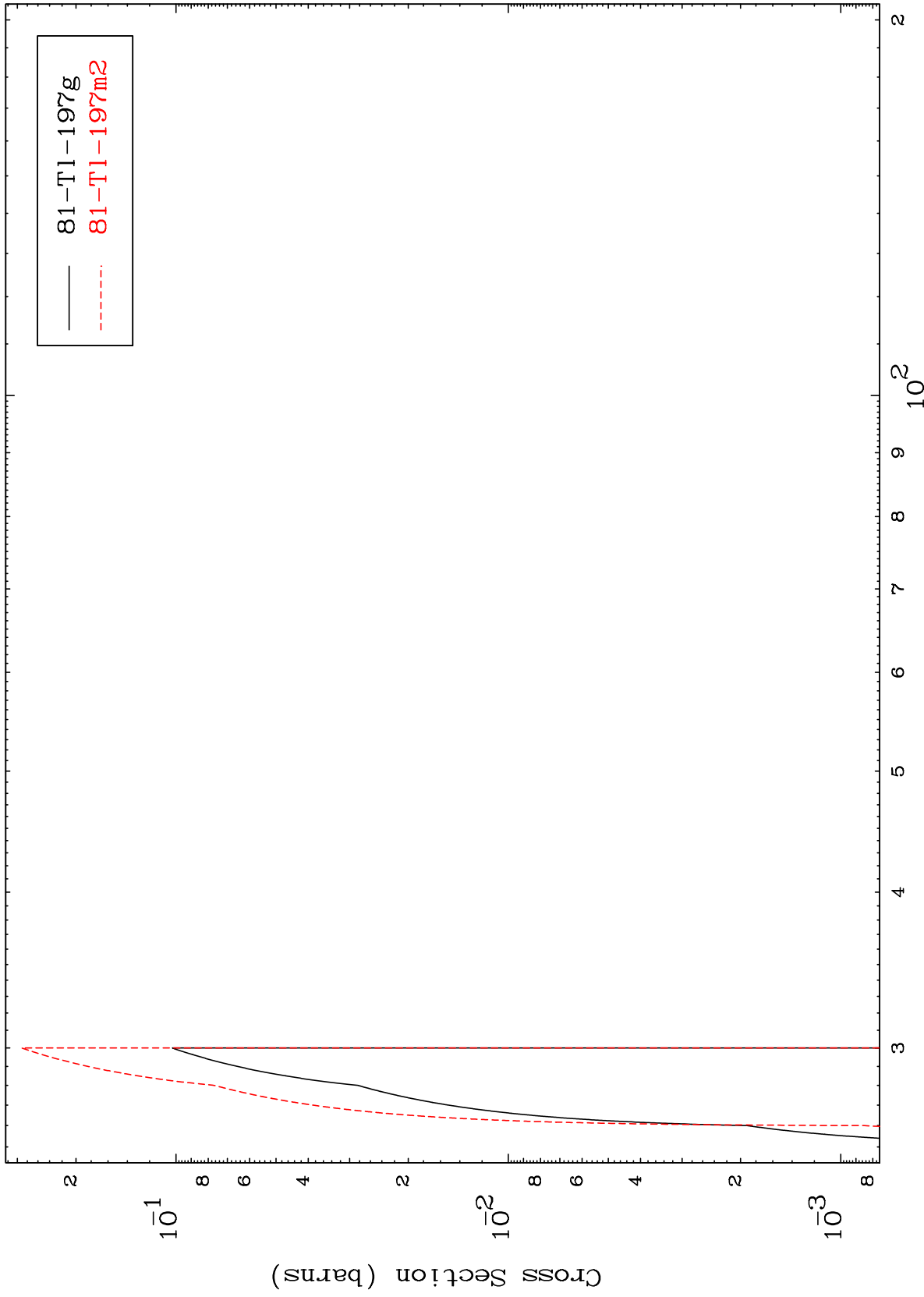
79-Au-196

MAT 7923

( $\alpha, 3n$ )

79-Au-196

Radionuclide Production Cross Section



12

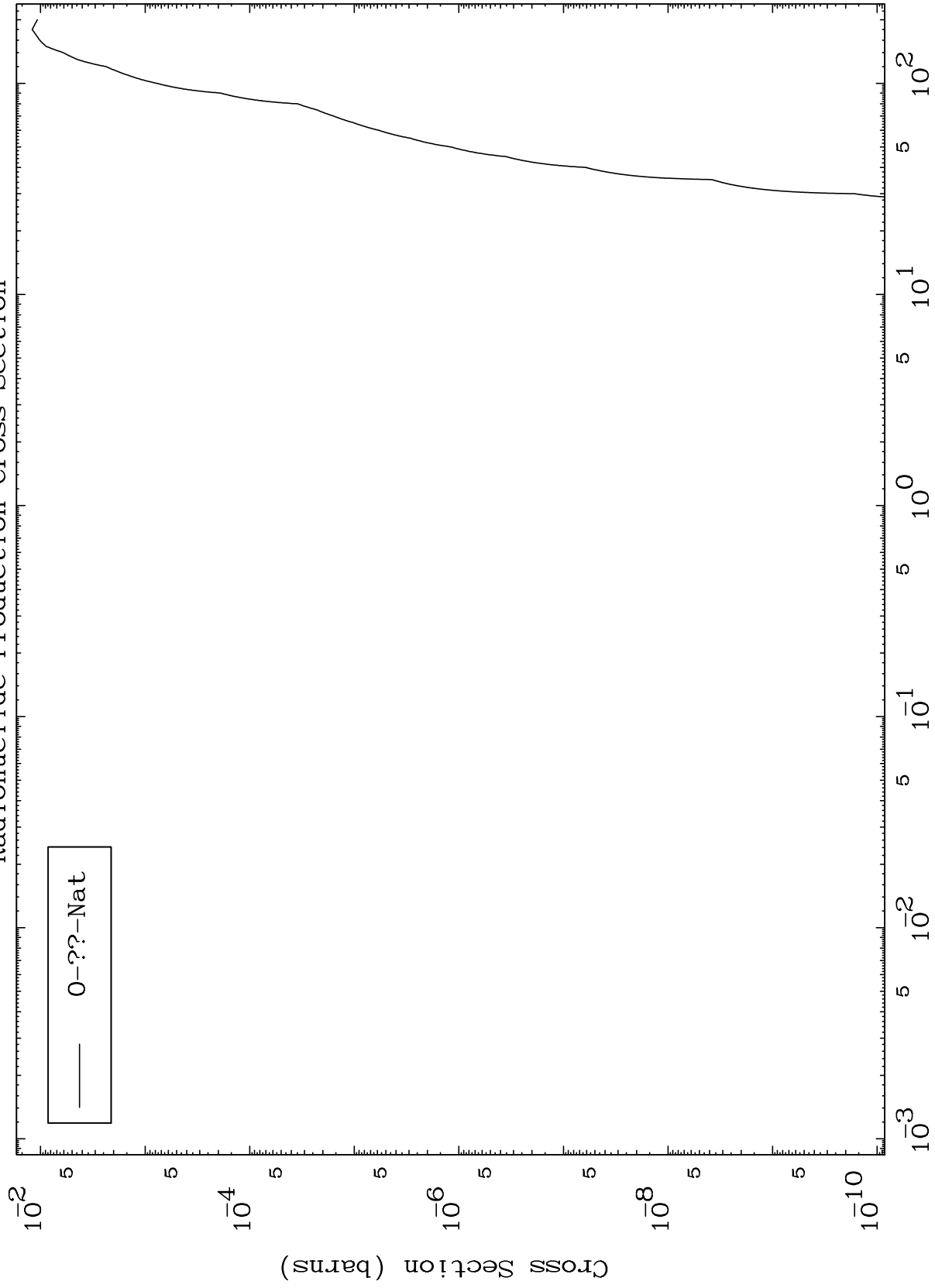
Incident Energy (MeV)

79-Au-196

MAT 7923

$\alpha$  Fission  
Radionuclide Production Cross Section

79-Au-196



13

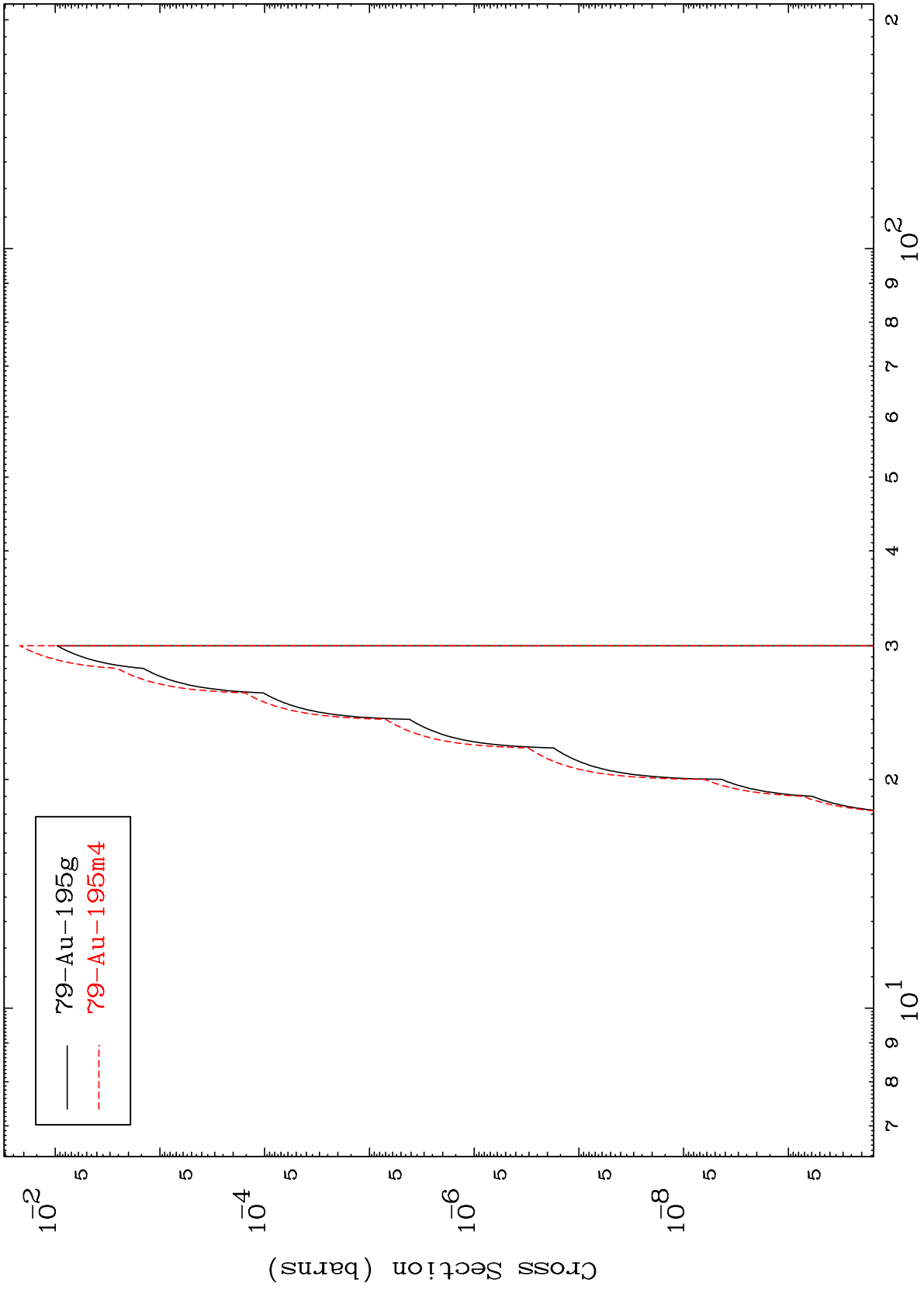
Incident Energy (MeV)

79-Au-196

MAT 7923

<sup>79</sup>Au-196

( $\alpha, n'$ )  $\alpha$   
Radionuclide Production Cross Section



14

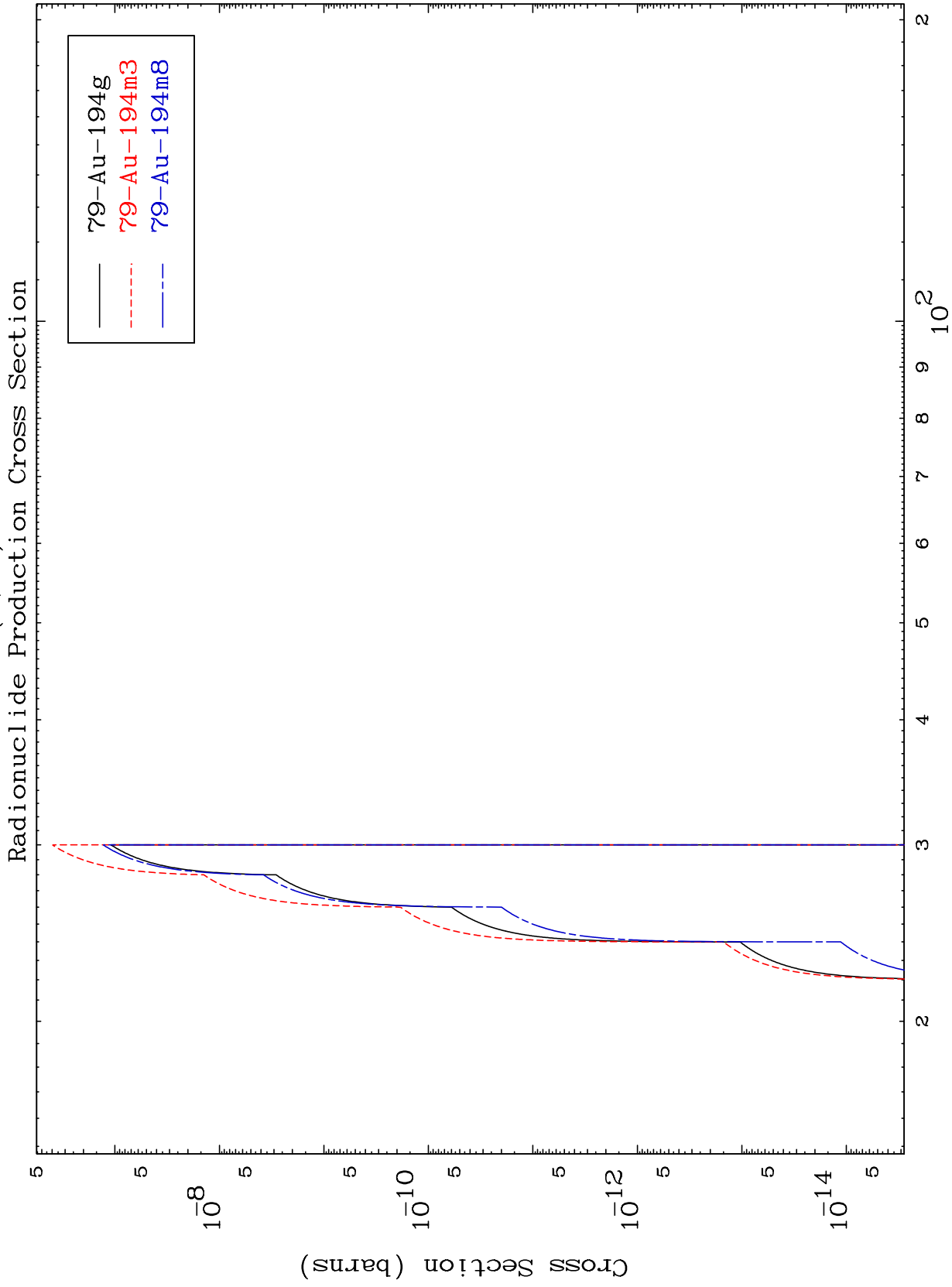
Incident Energy (MeV)

<sup>79</sup>Au-196

MAT 7923

$(\alpha, 2n) \alpha$

$^{79}\text{Au-196}$



15

Incident Energy (MeV)

$^{79}\text{Au-196}$

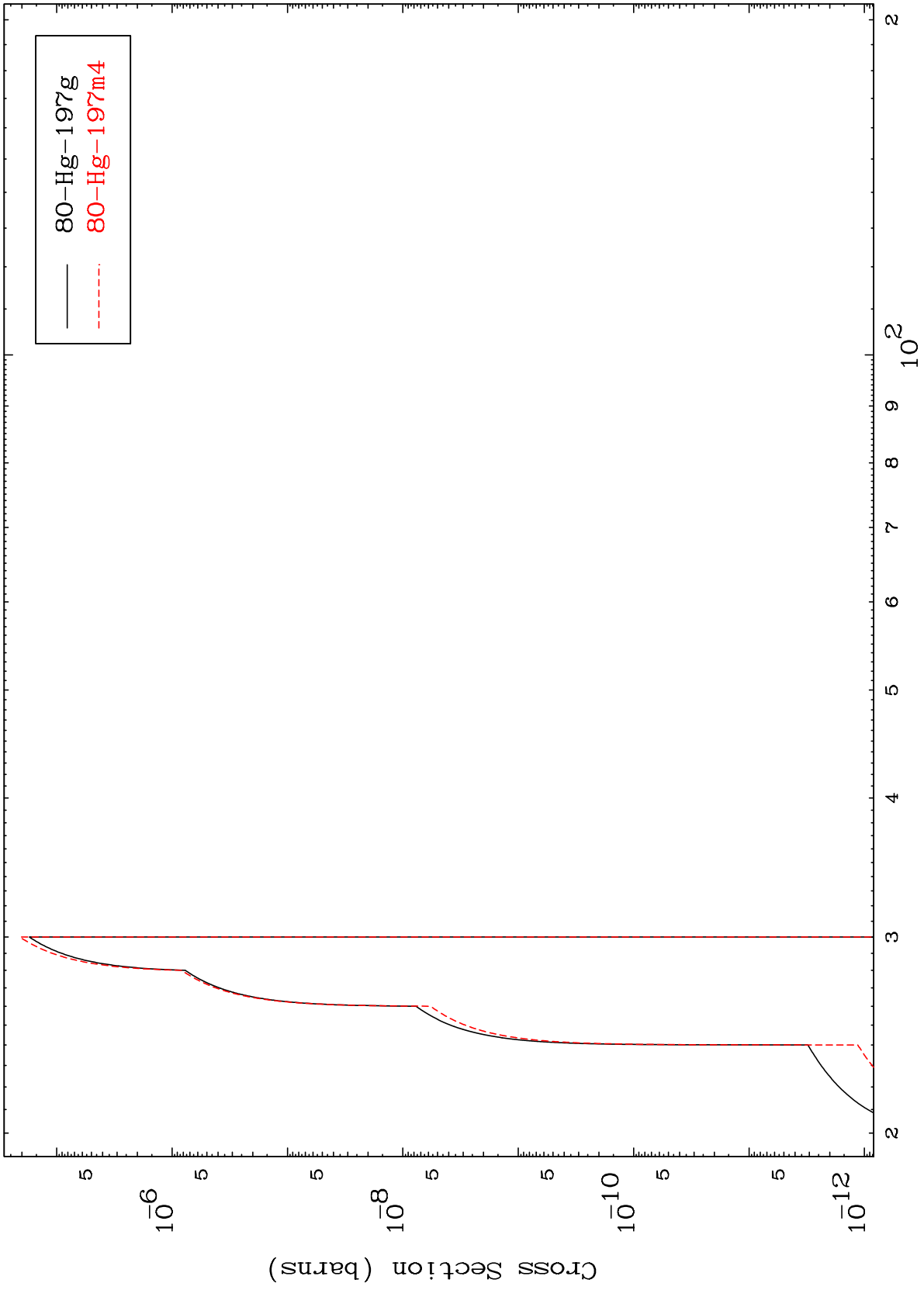


MAT 7923

( $\alpha, n'$ ) d

79-Au-196

Radionuclide Production Cross Section



16

Incident Energy (MeV)

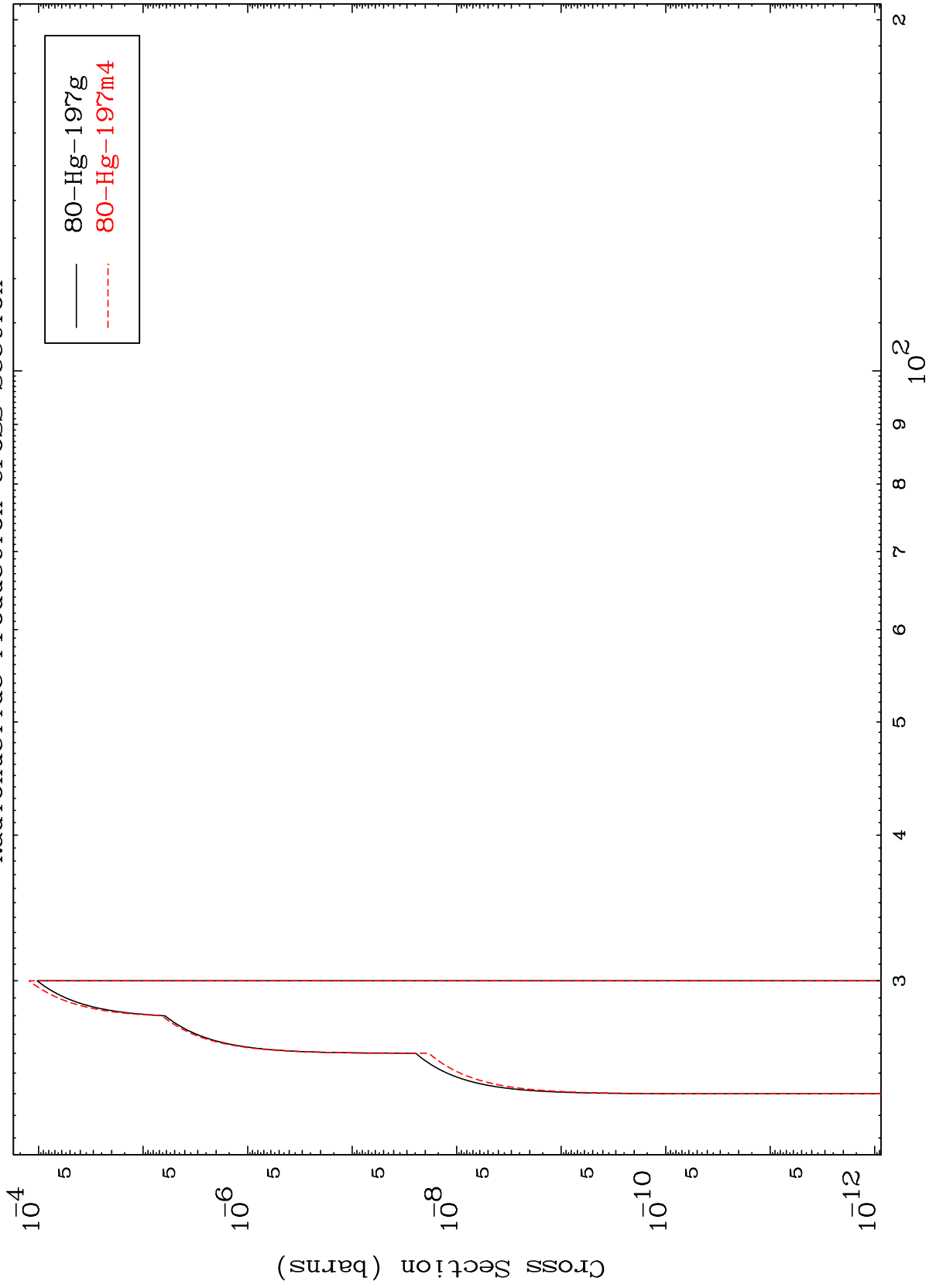
79-Au-196

MAT 7923

( $\alpha, 2n$ ) p

79-Au-196

Radionuclide Production Cross Section



17

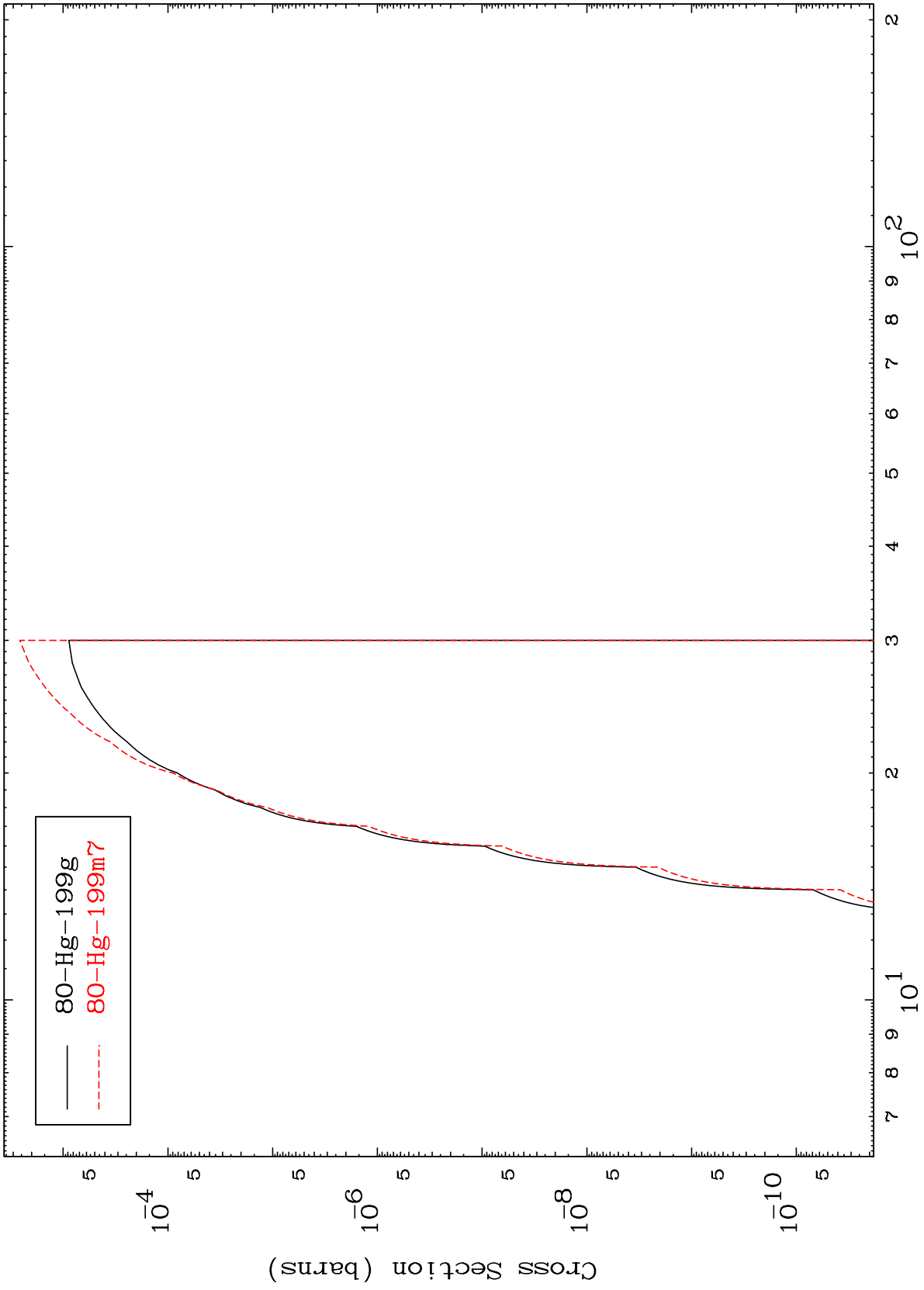
Incident Energy (MeV)

79-Au-196

MAT 7923

79-Au-196

( $\alpha, p$ )  
Radionuclide Production Cross Section



18

Incident Energy (MeV)

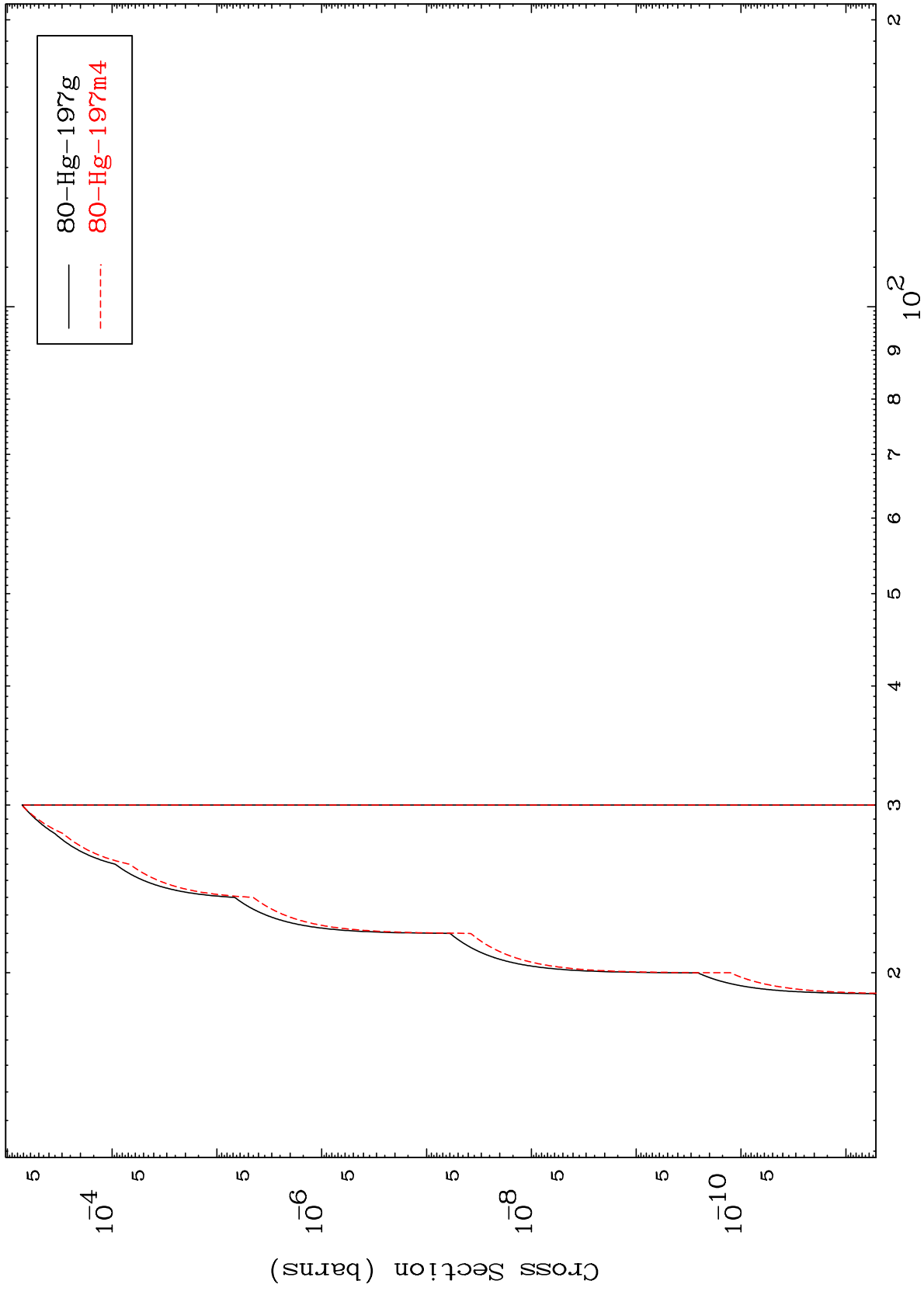
79-Au-196

MAT 7923

( $\alpha, t$ )

79-Au-196

Radionuclide Production Cross Section



19

Incident Energy (MeV)

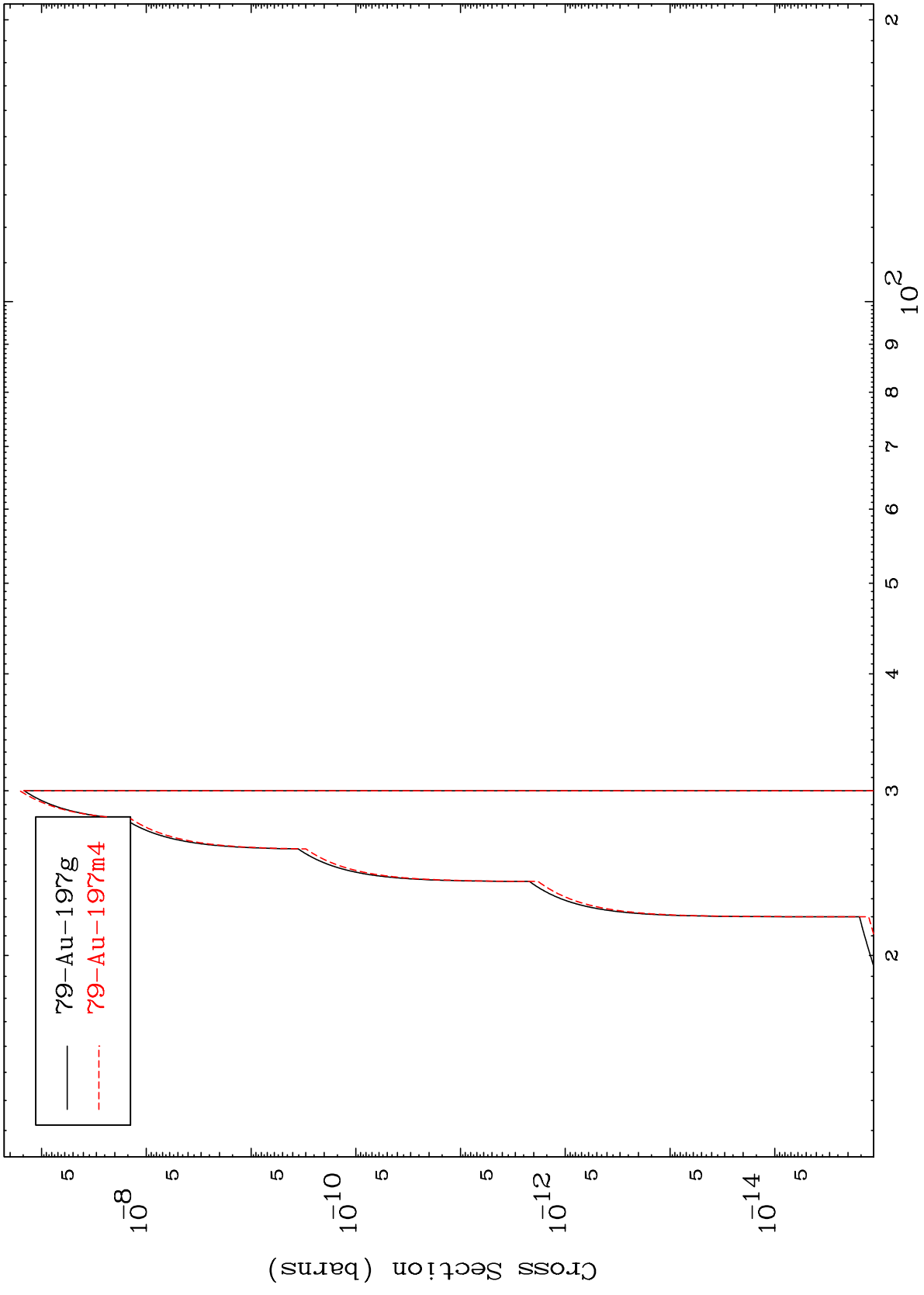
79-Au-196

MAT 7923

( $\alpha$ , He-3)

79-Au-196

Radionuclide Production Cross Section



20

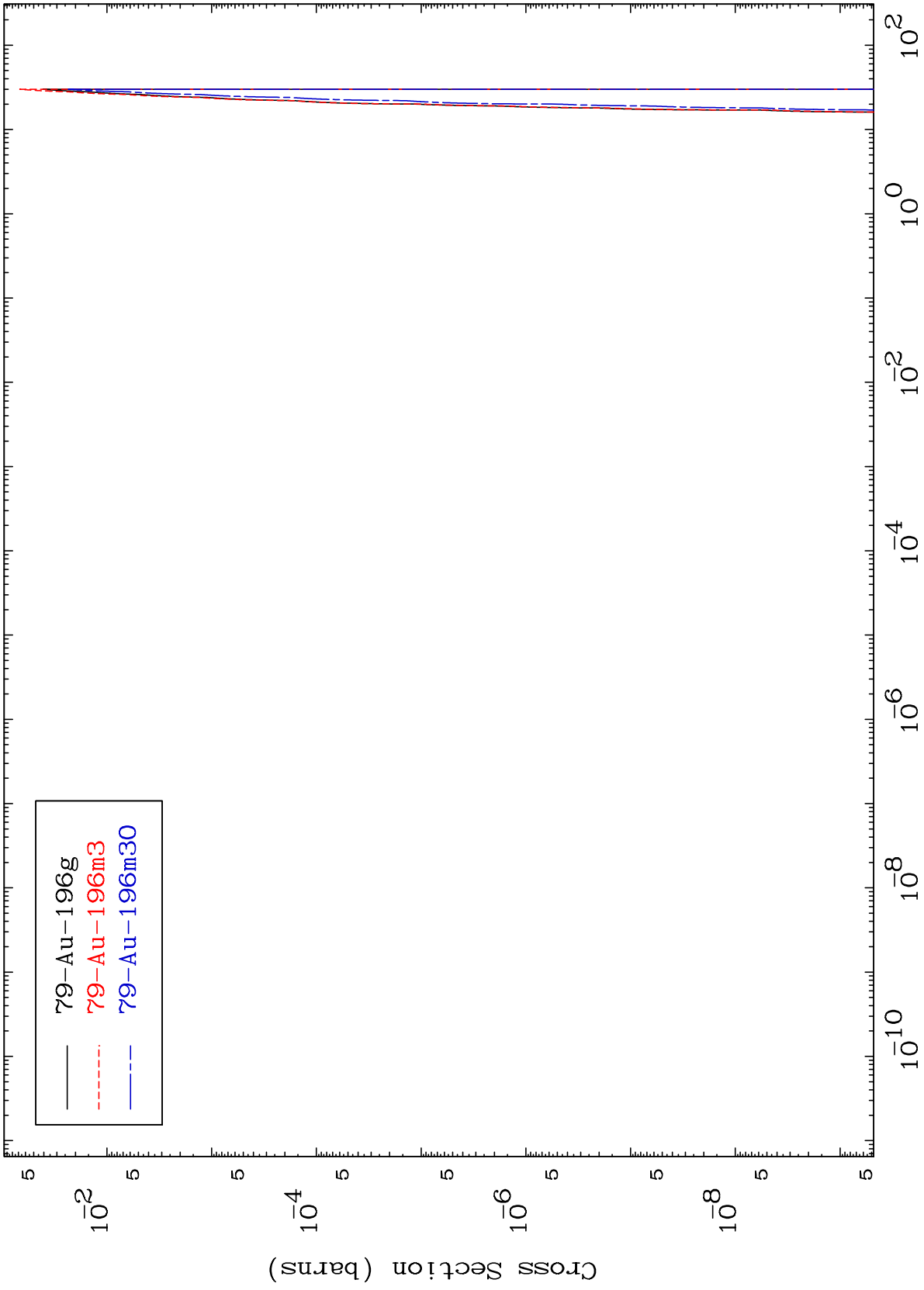
Incident Energy (MeV)

79-Au-196

MAT 7923

<sup>79</sup>Au-196

( $\alpha, \alpha$ )  
Radionuclide Production Cross Section



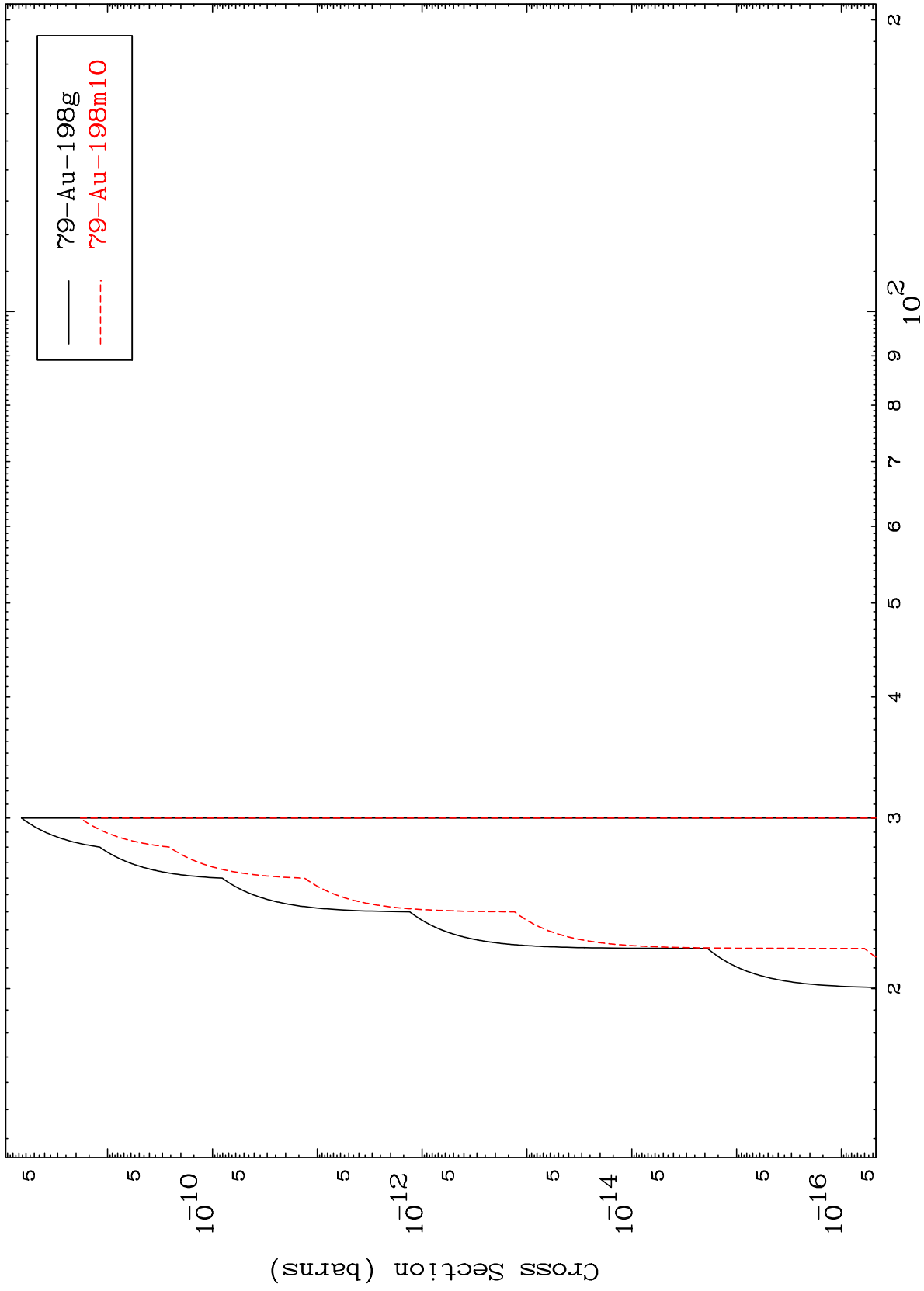
21

<sup>79</sup>Au-196

MAT 7923

79-Au-196

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



22

79-Au-196

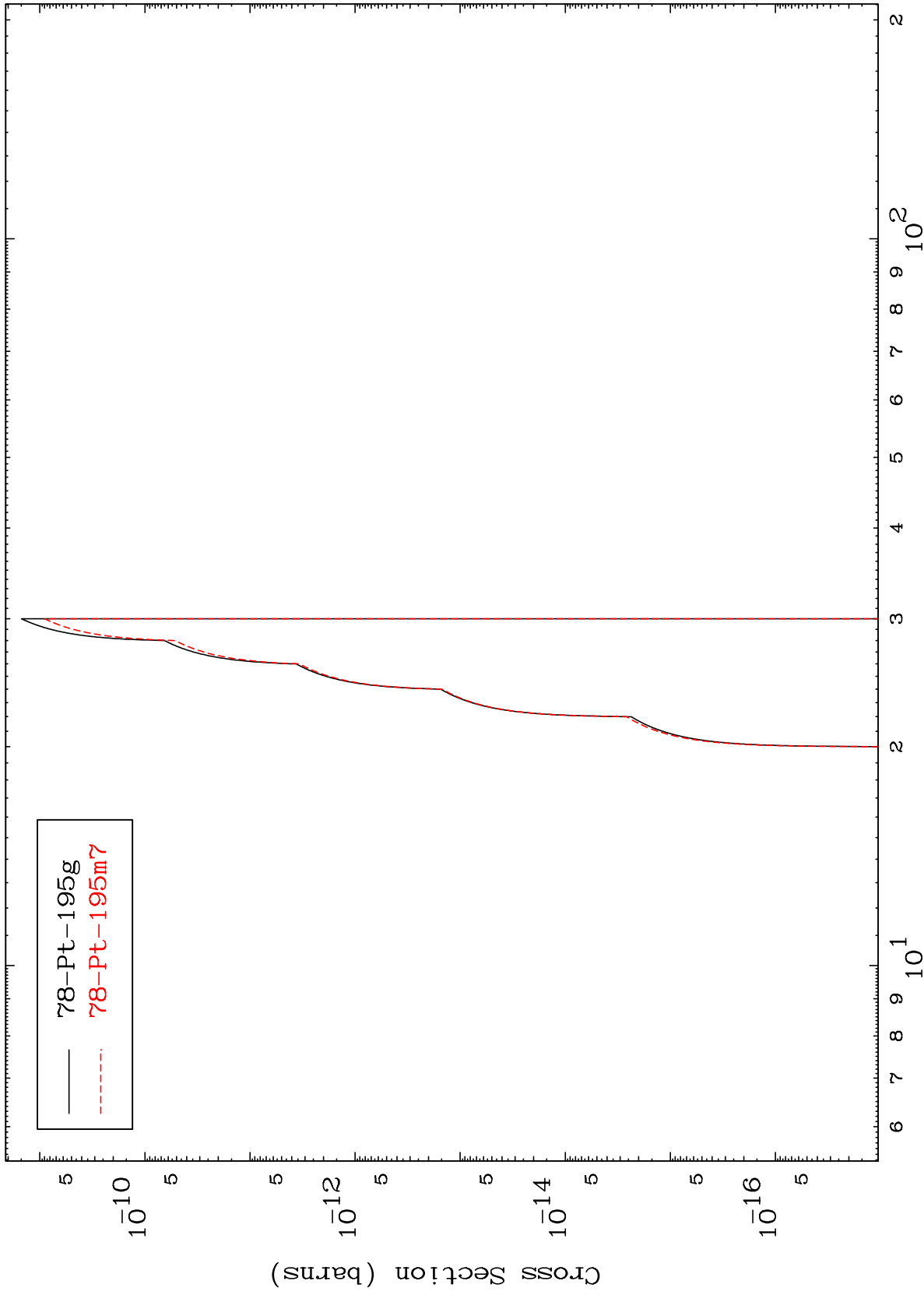
Incident Energy (MeV)

MAT 7923

( $\alpha, p$ )  $\alpha$

<sup>79</sup>Au-196

Radionuclide Production Cross Section



23

Incident Energy (MeV)

<sup>79</sup>Au-196