

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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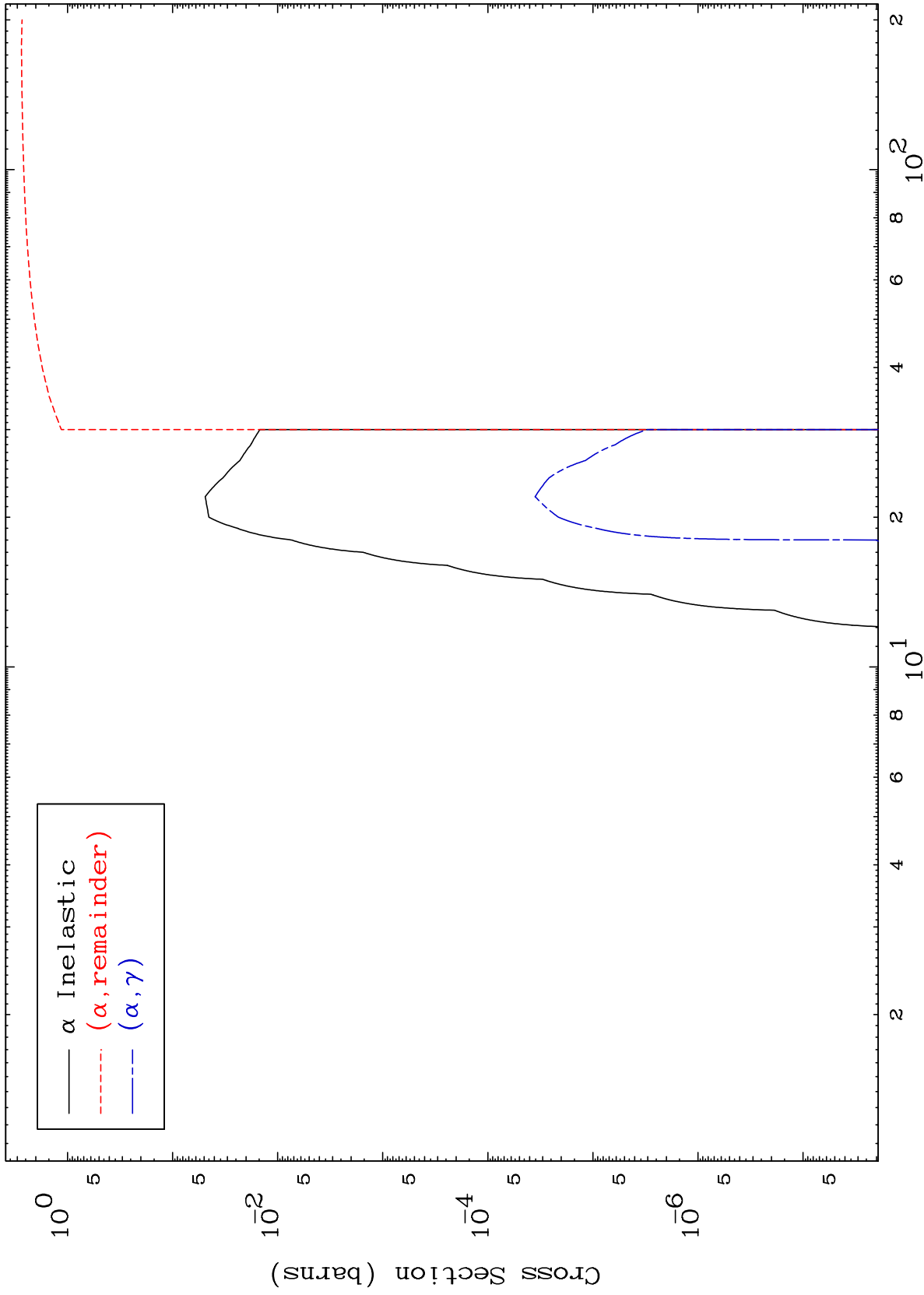
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7924

0 Kelvin  $\alpha$  Major  
Cross Sections

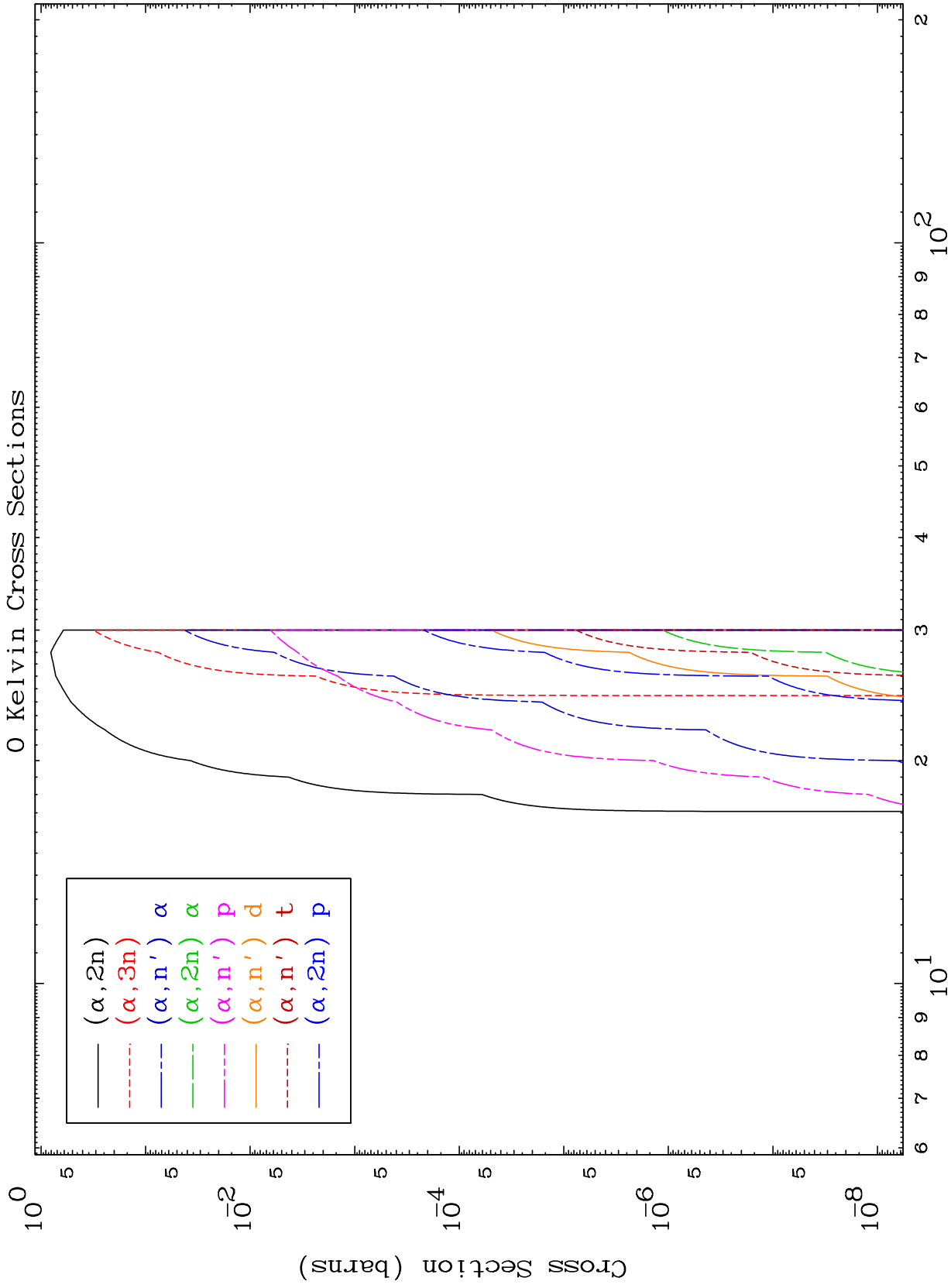
79-Au-196



MAT 7924

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

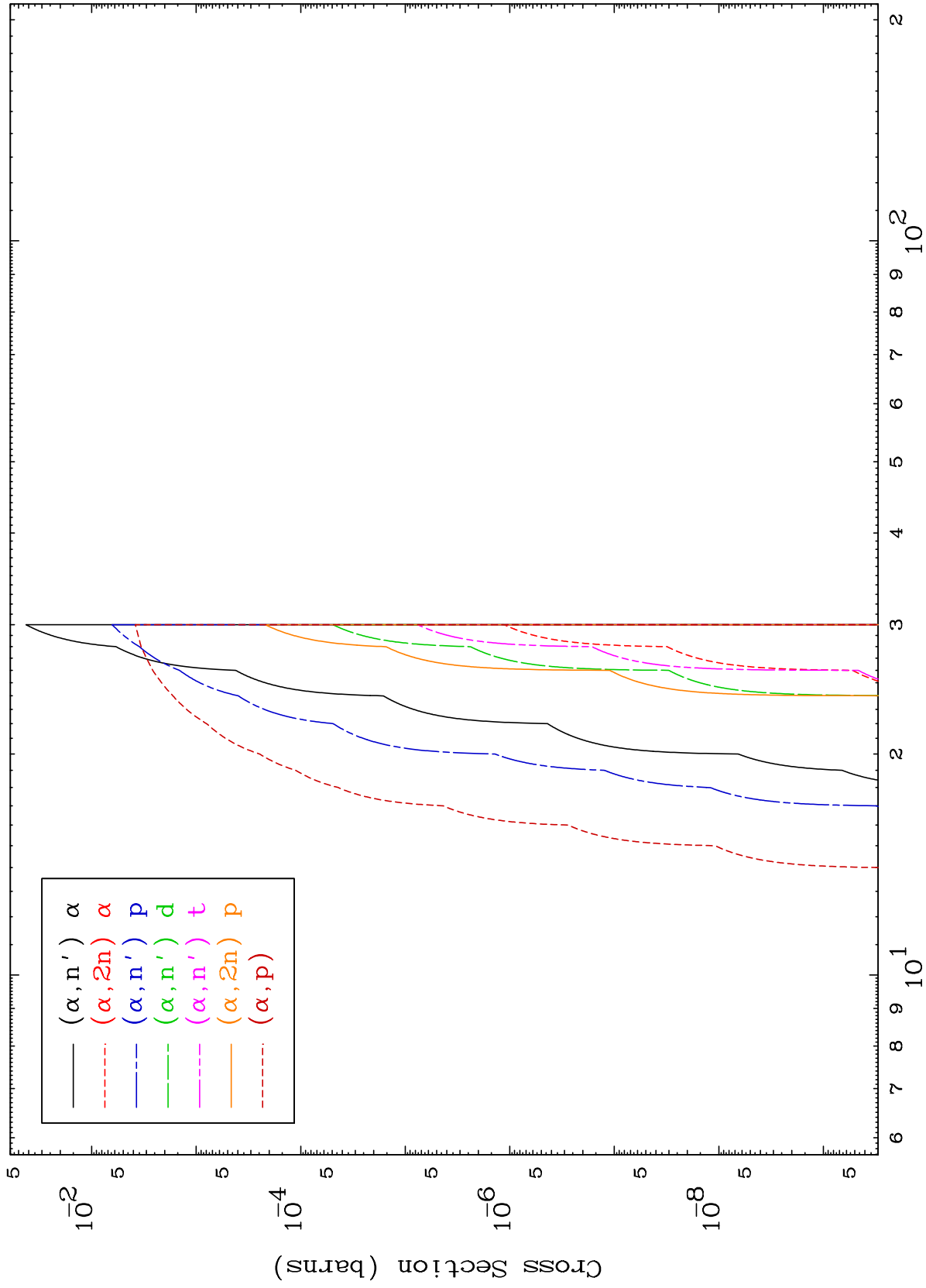
79-Au-196



2

Incident Energy (MeV)

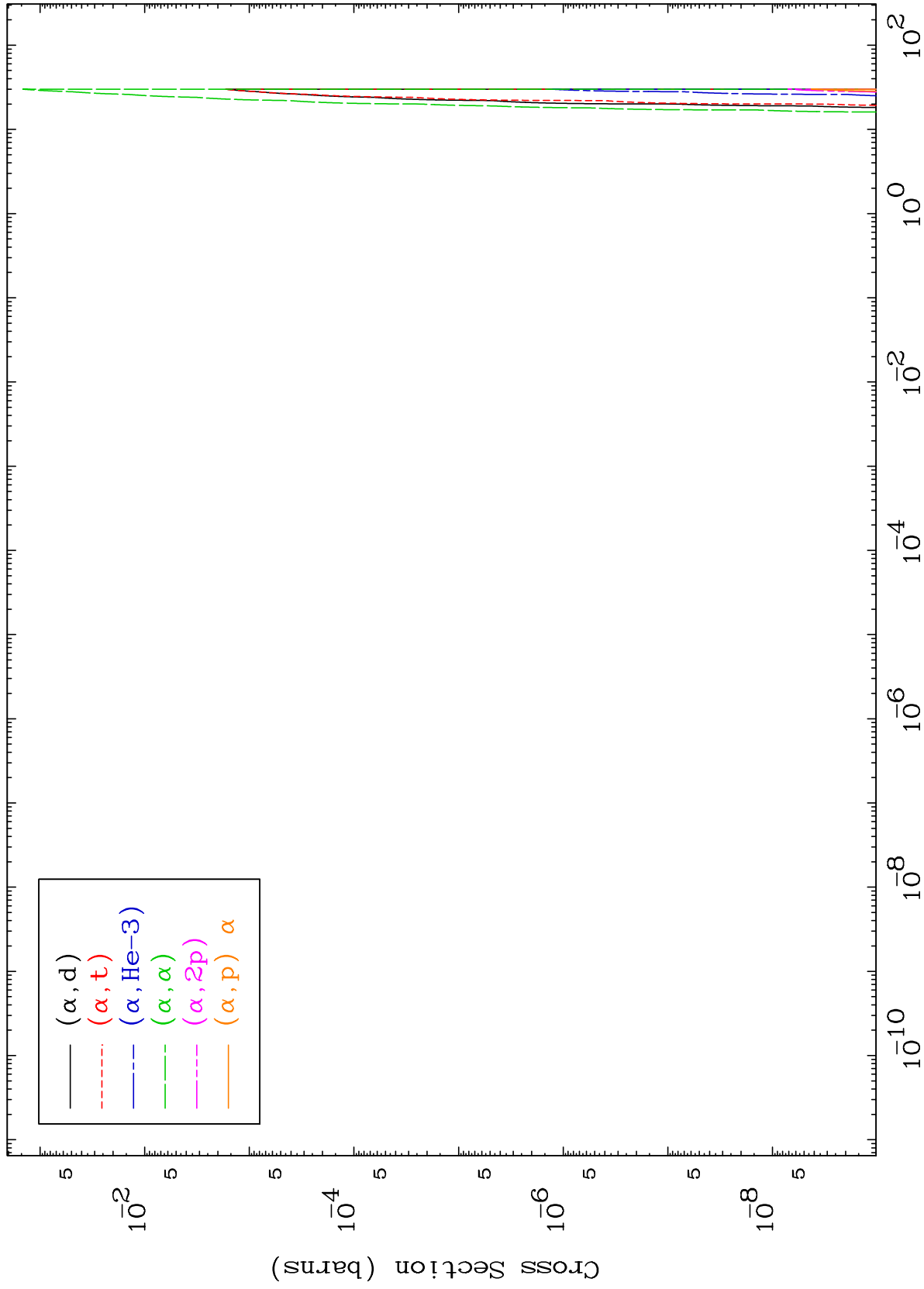
79-Au-196



MAT 7924

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

79-Au-196



4

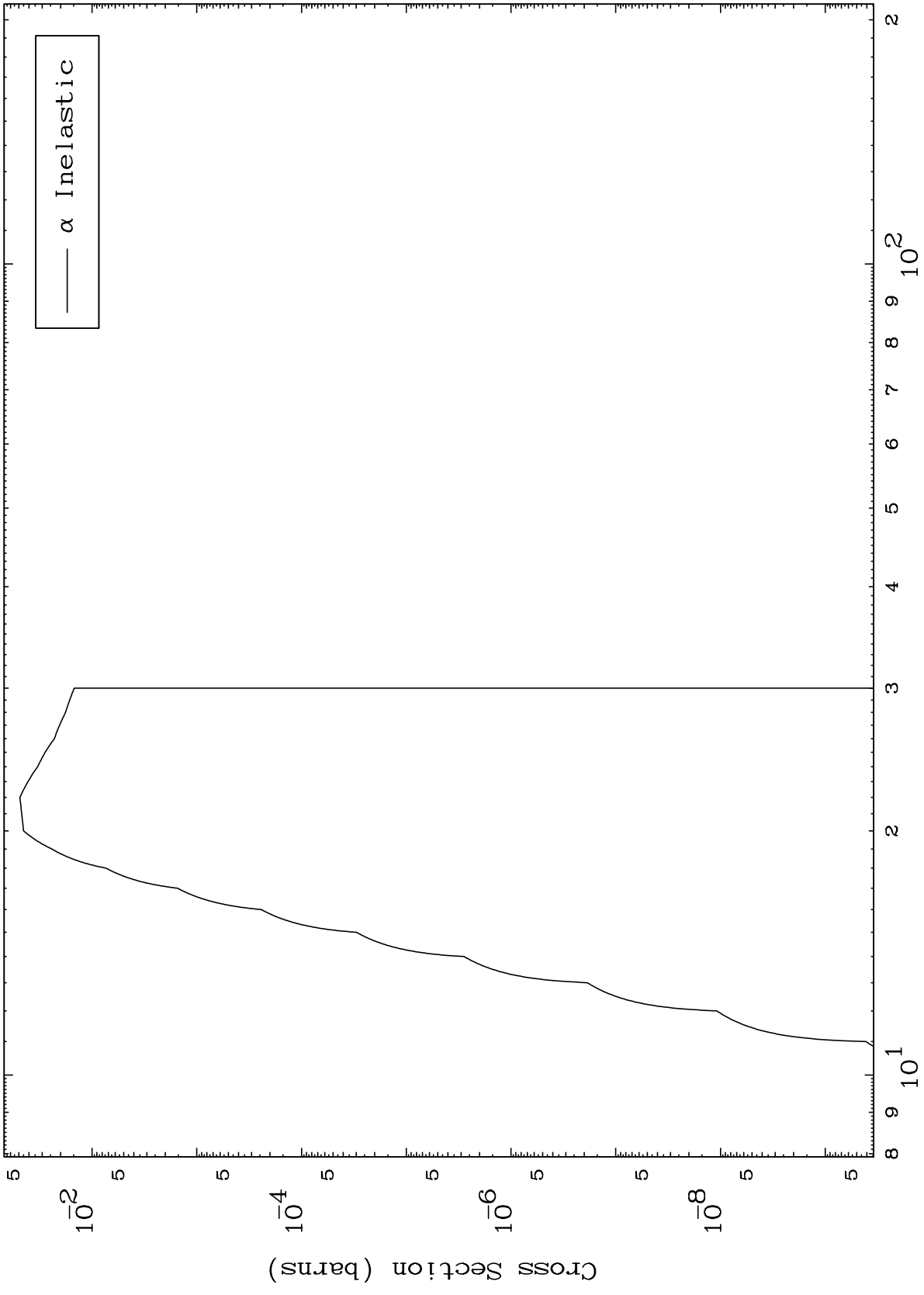
Incident Energy (MeV)

79-Au-196

MAT 7924

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

79-Au-196



5

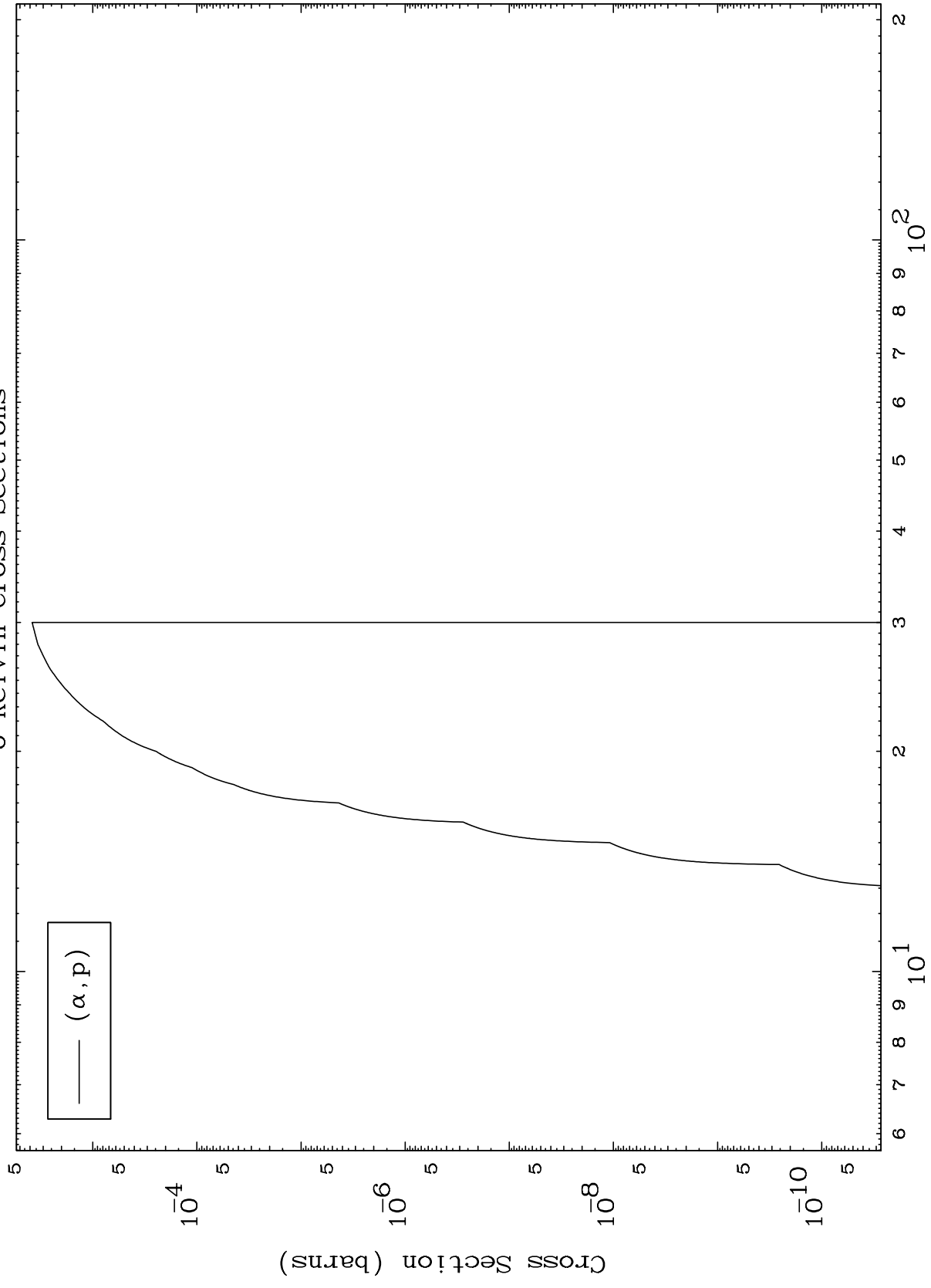
Incident Energy (MeV)

79-Au-196

MAT 7924

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

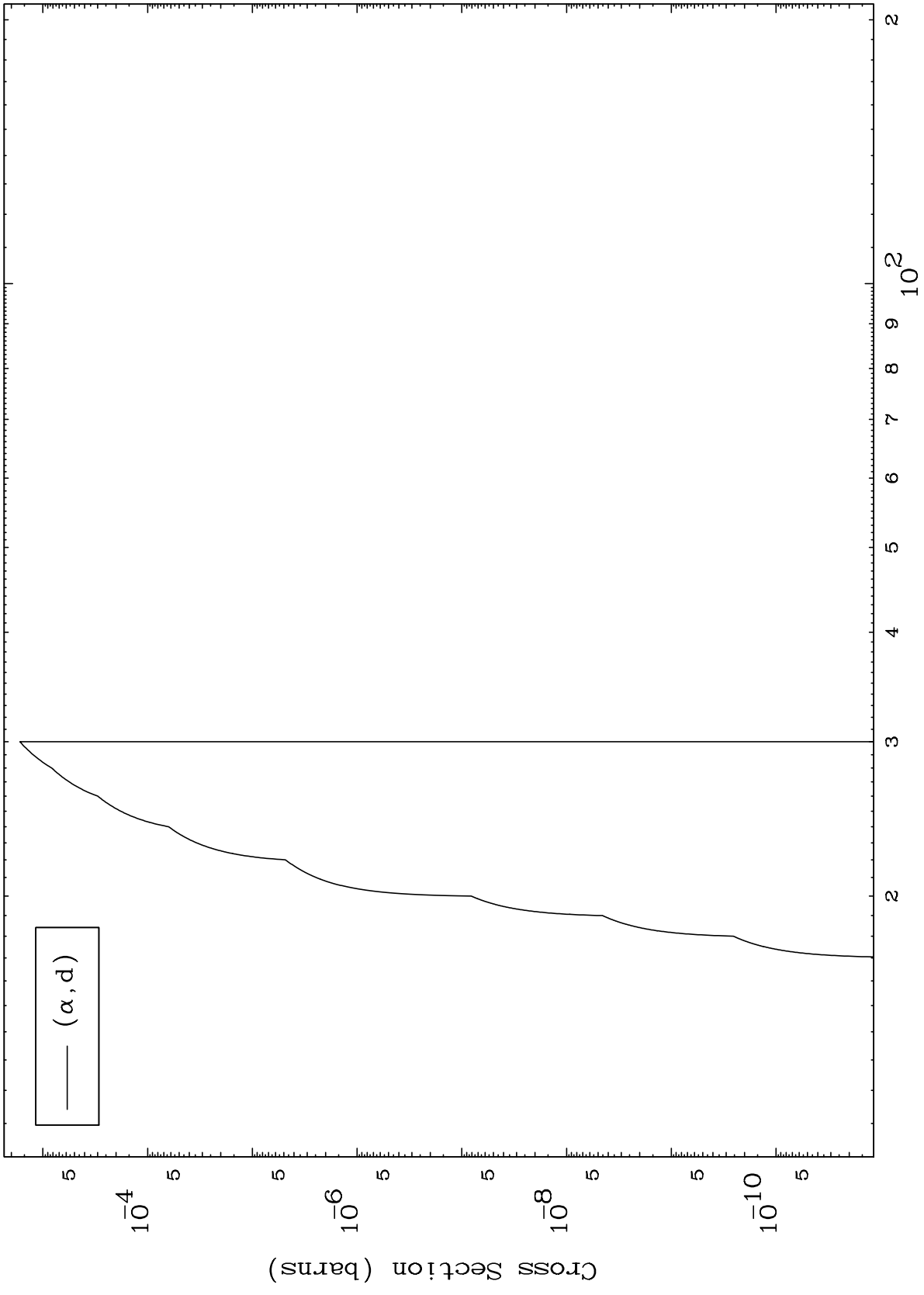
79-Au-196



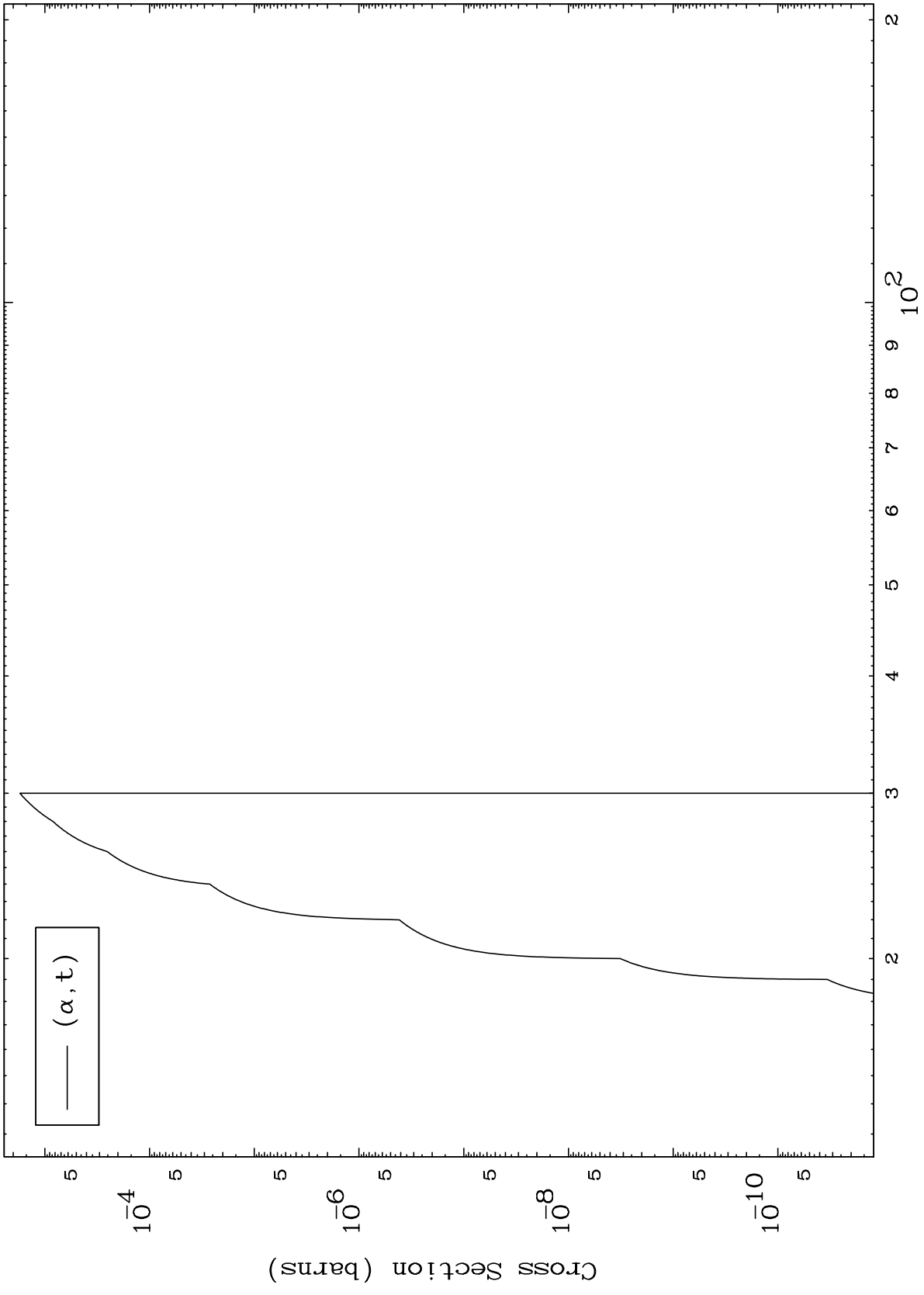
6

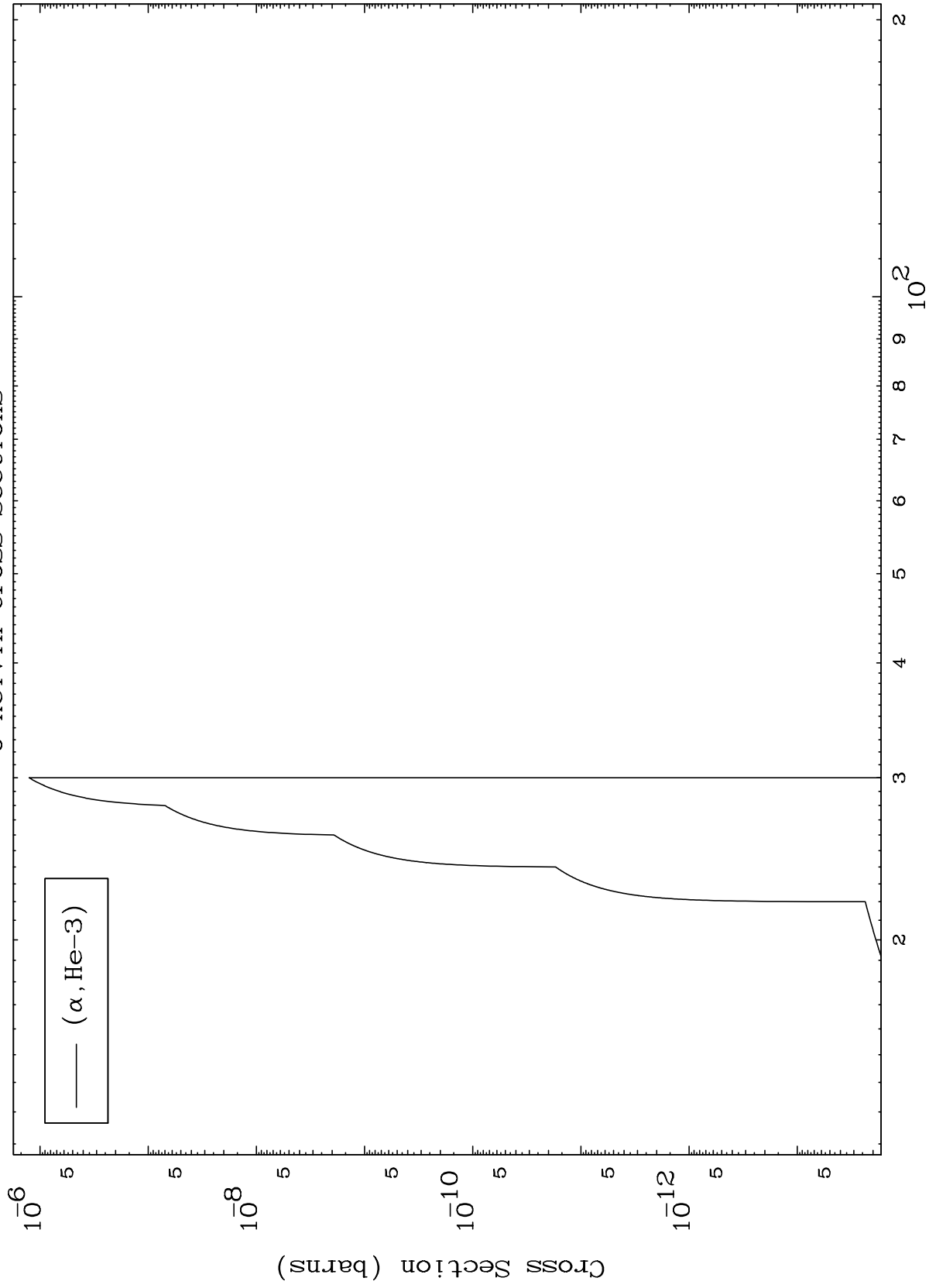
Incident Energy (MeV)

79-Au-196





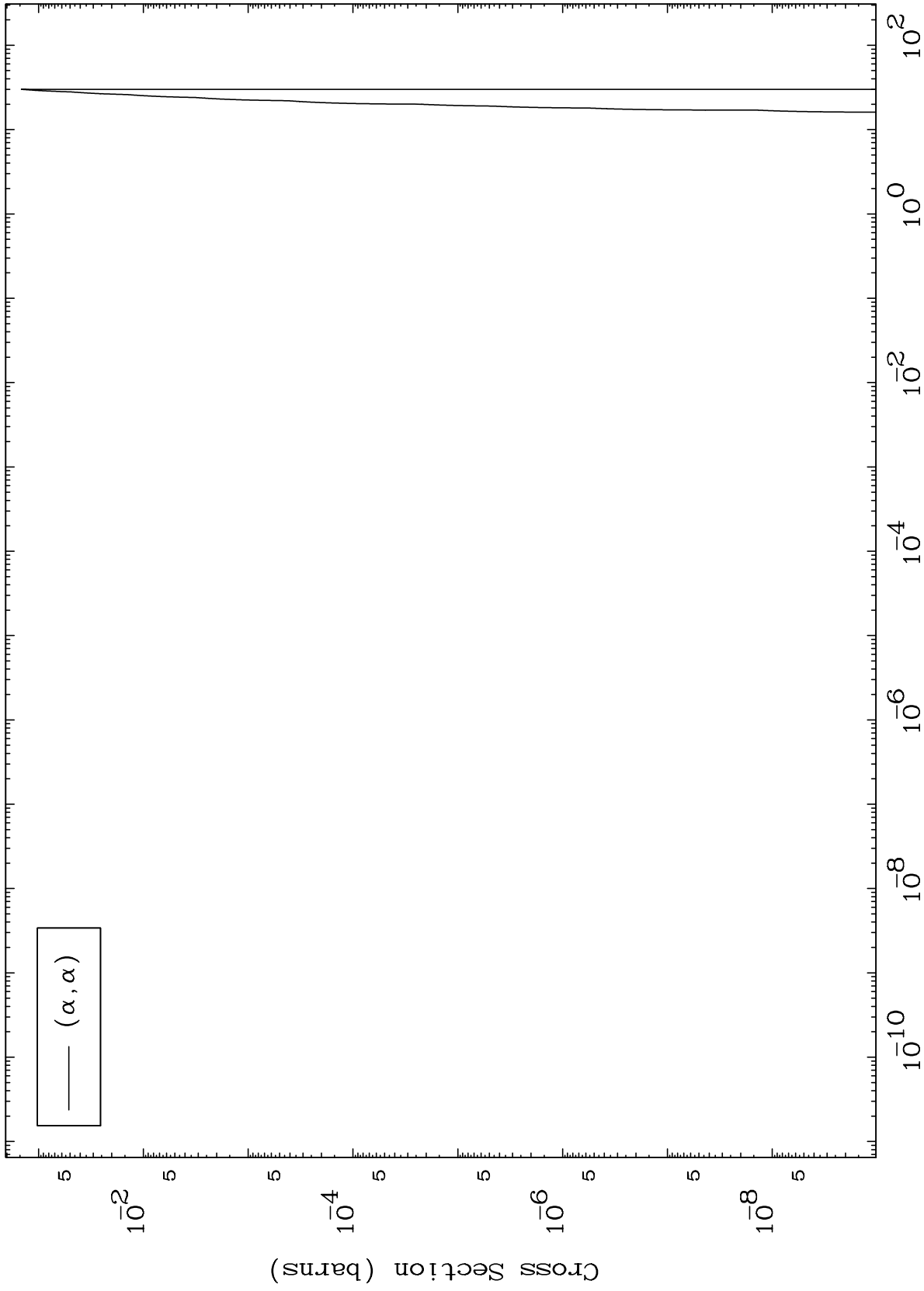




MAT 7924

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

79-Au-196



10

Incident Energy (MeV)

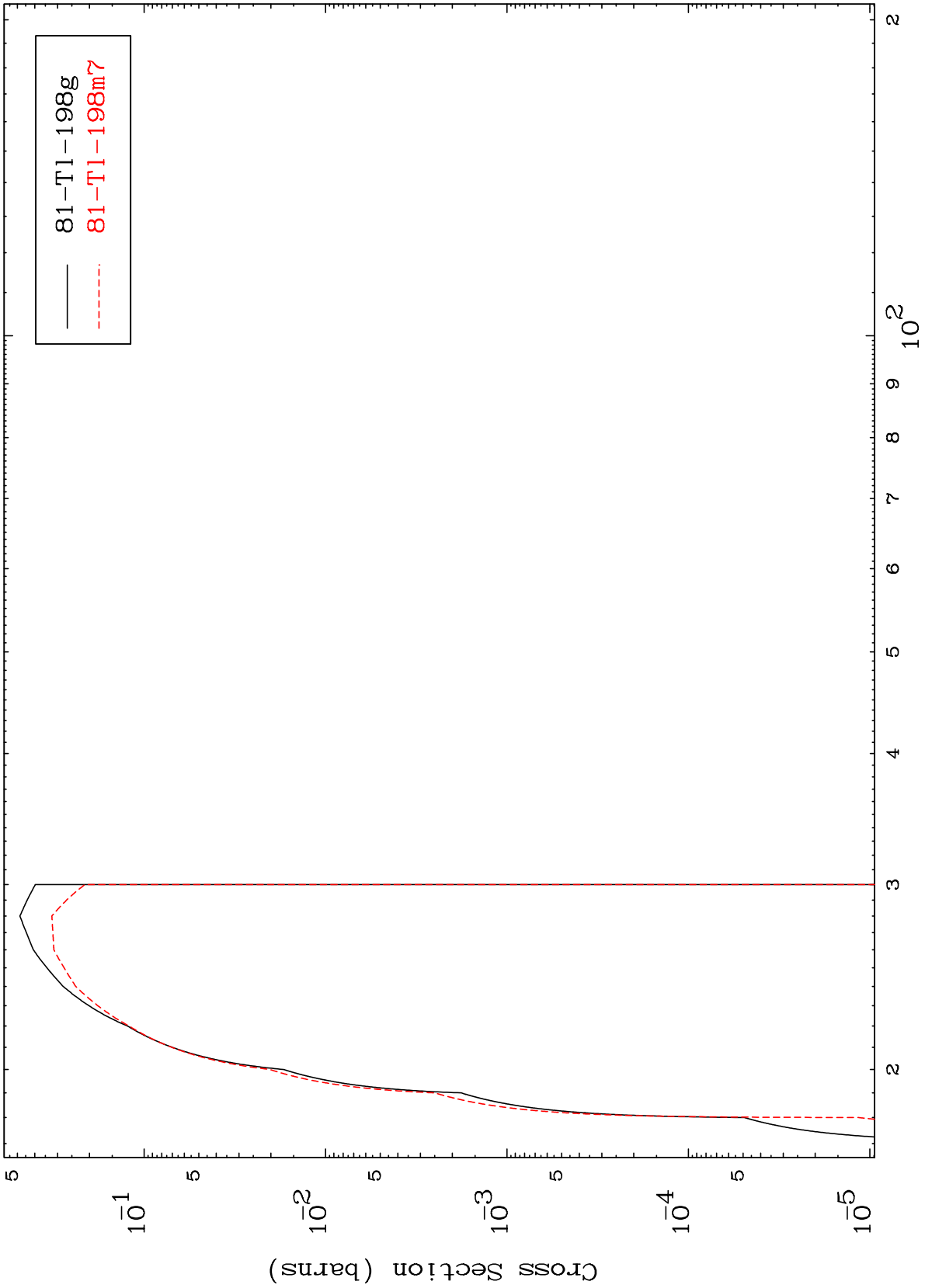
79-Au-196

MAT 7924

( $\alpha, 2n$ )

79-Au-196

Radionuclide Production Cross Section



11

Incident Energy (MeV)

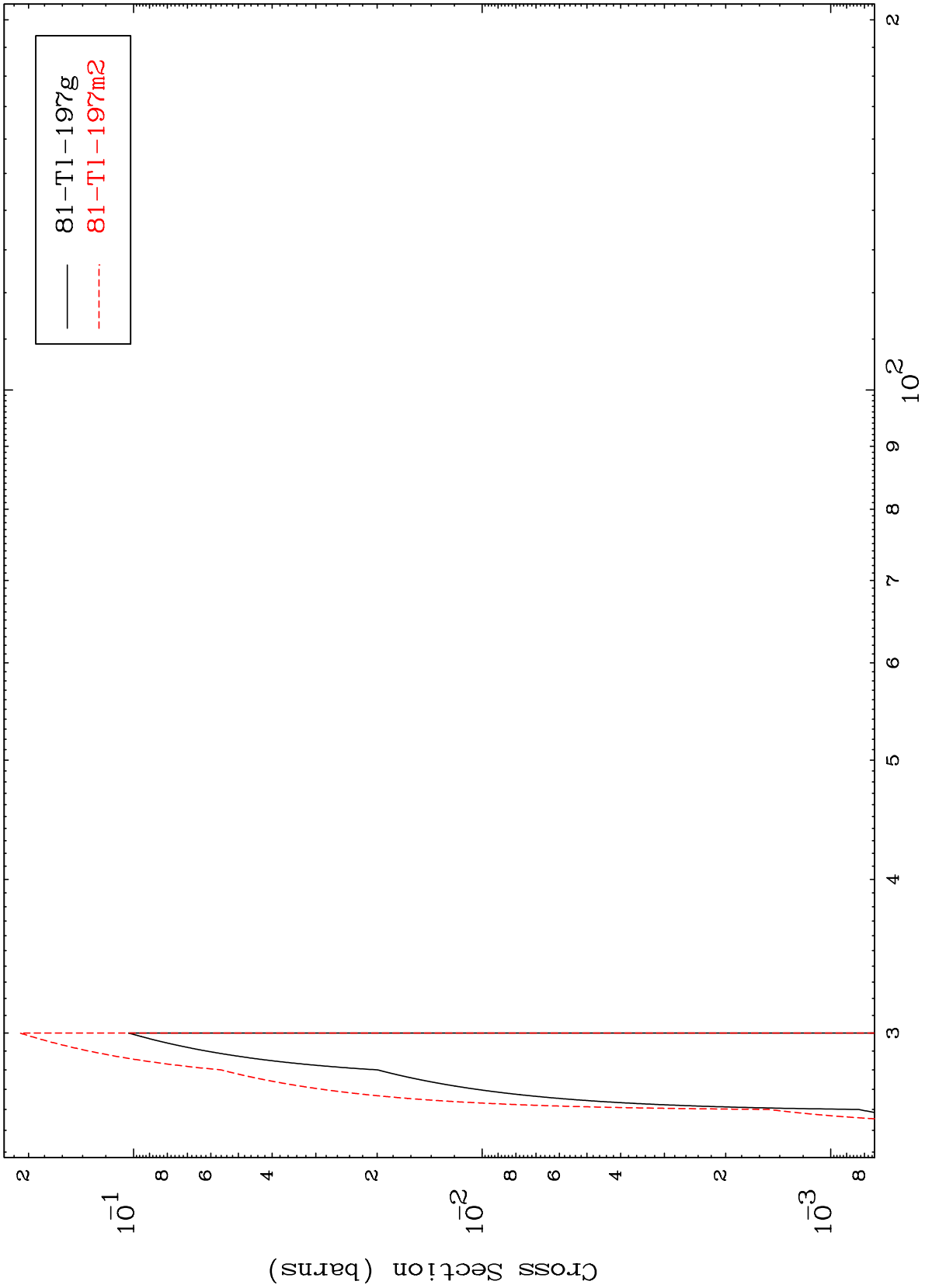
79-Au-196

MAT 7924

( $\alpha, 3n$ )

79-Au-196

Radionuclide Production Cross Section



12

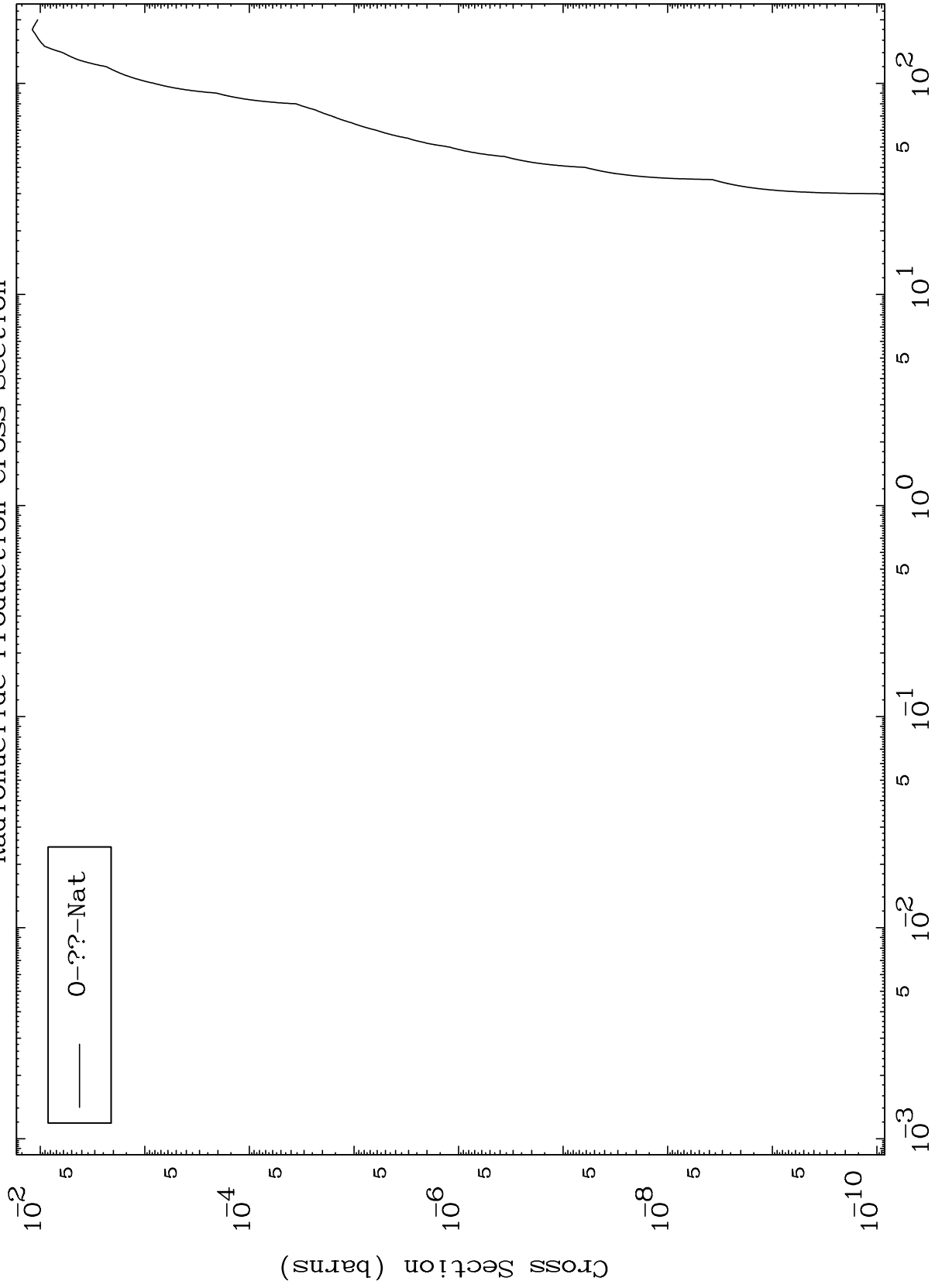
Incident Energy (MeV)

79-Au-196

MAT 7924

$\alpha$  Fission  
Radionuclide Production Cross Section

79-Au-196



13

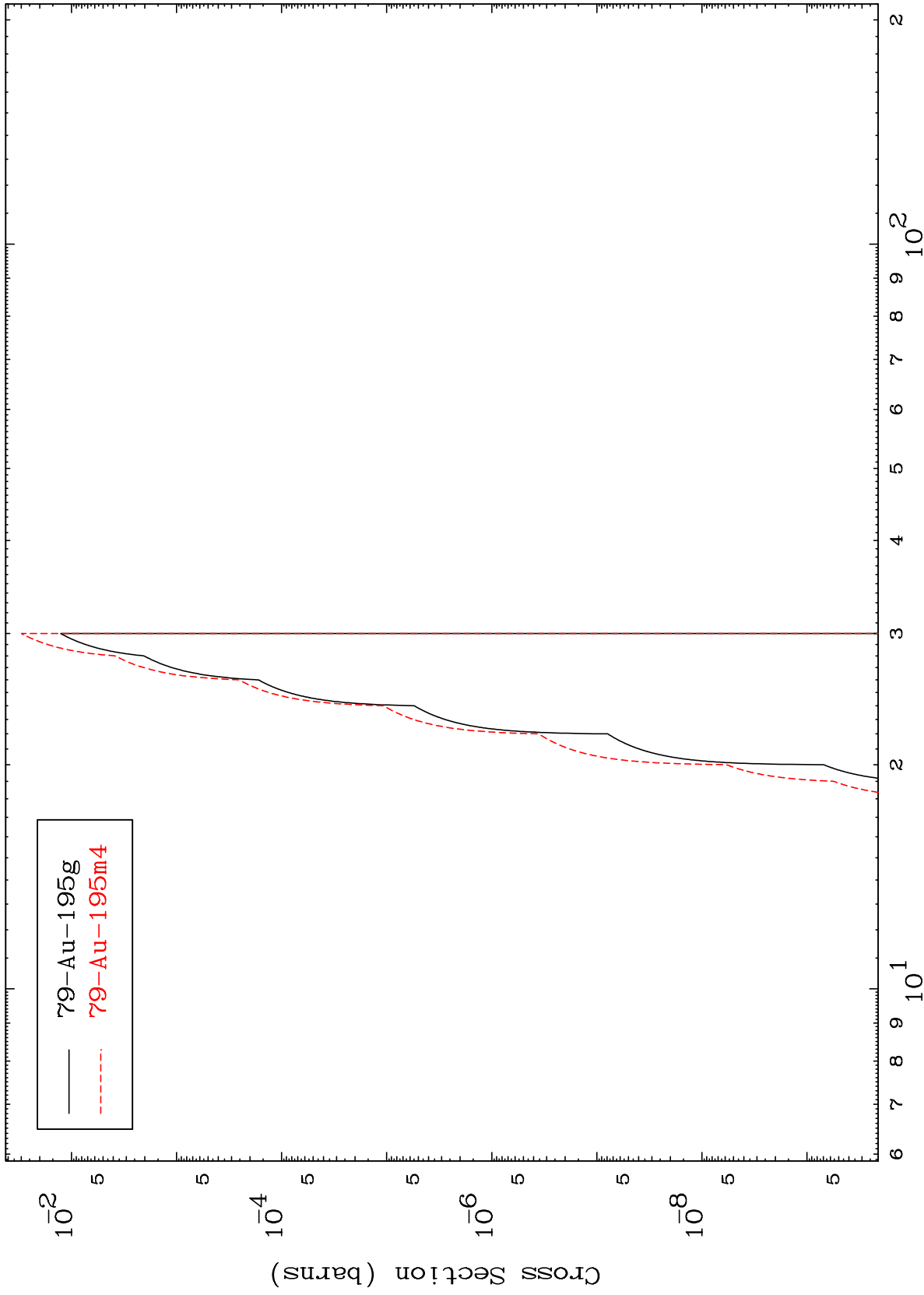
Incident Energy (MeV)

79-Au-196

MAT 7924

<sup>79</sup>Au-196

( $\alpha, n'$ )  $\alpha$   
Radionuclide Production Cross Section



14

Incident Energy (MeV)

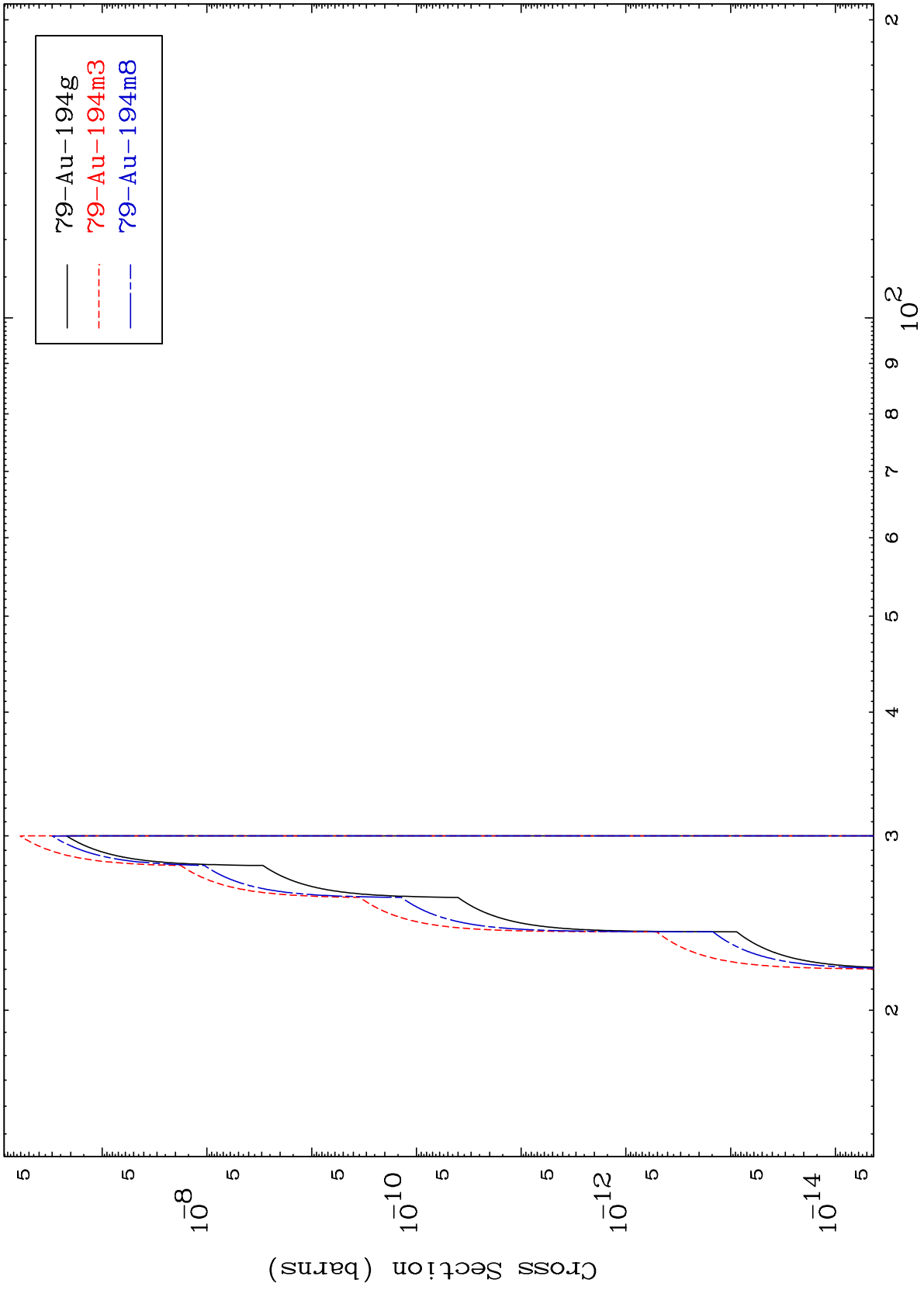
<sup>79</sup>Au-196

MAT 7924

$(\alpha, 2n) \alpha$

$^{79}\text{Au-196}$

Radionuclide Production Cross Section



15

Incident Energy (MeV)

$^{79}\text{Au-196}$

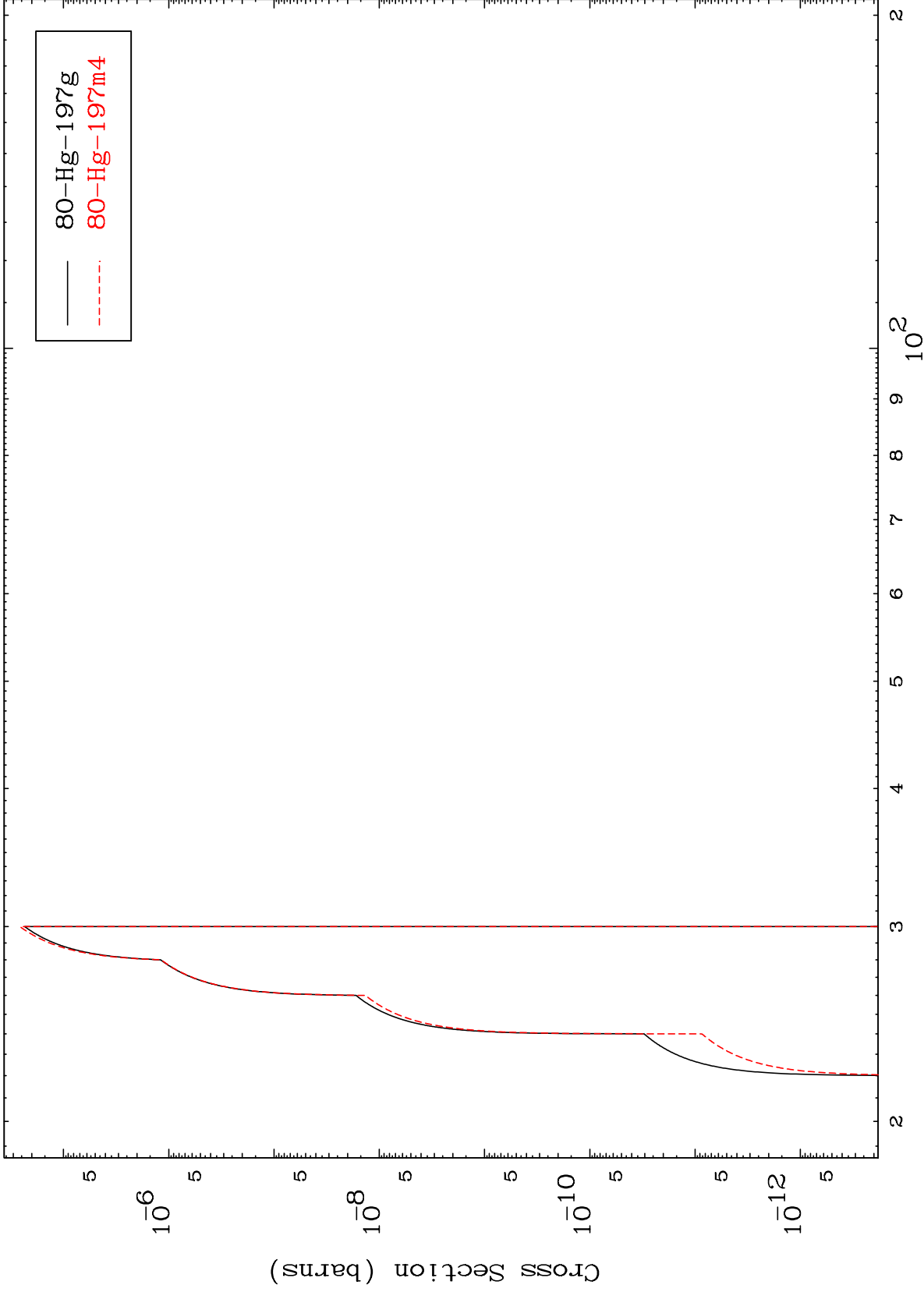


MAT 7924

( $\alpha, n'$ ) d

79-Au-196

Radionuclide Production Cross Section



16

Incident Energy (MeV)

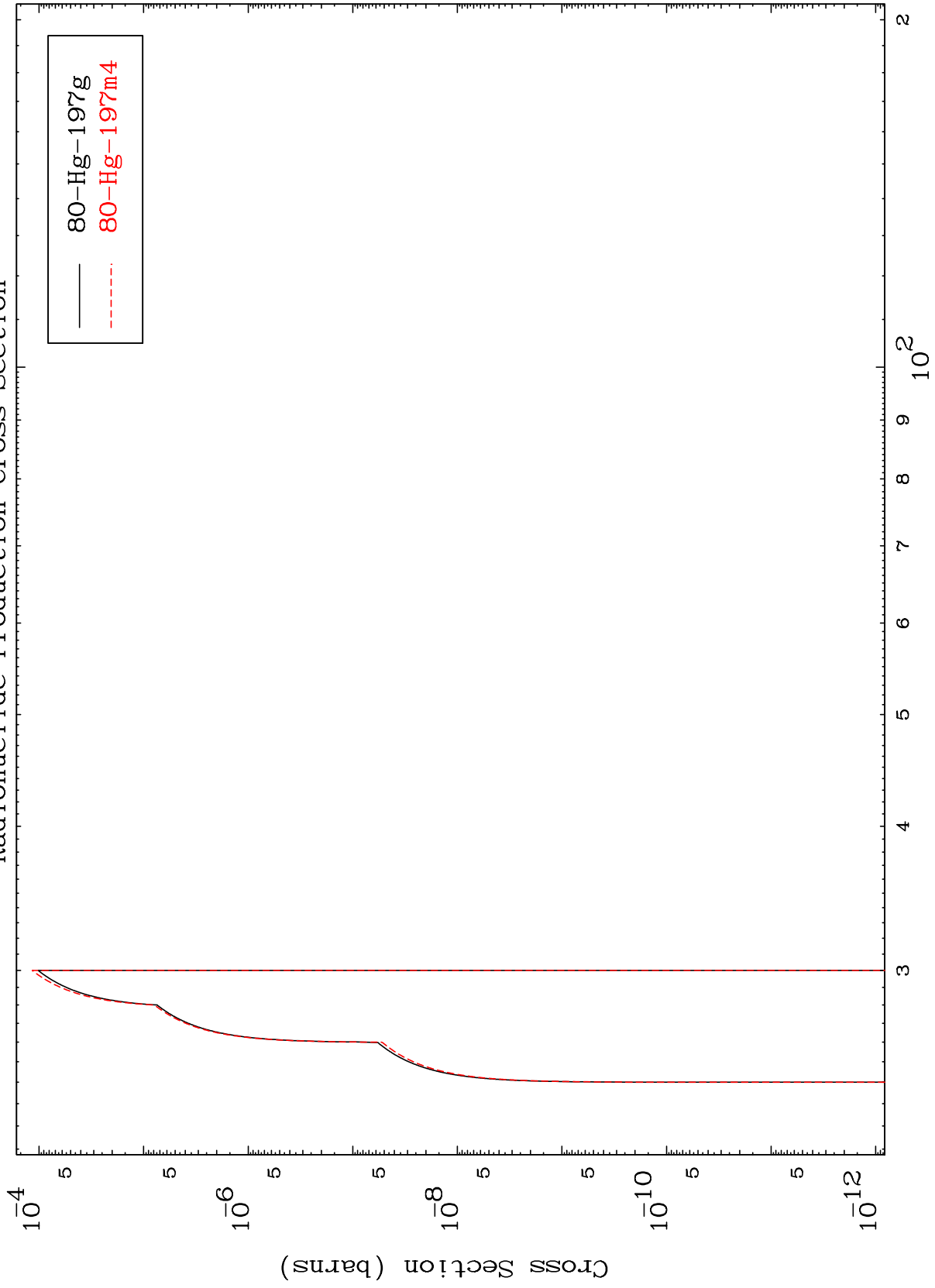
79-Au-196

MAT 7924

( $\alpha, 2n$ ) p

79-Au-196

Radionuclide Production Cross Section



17

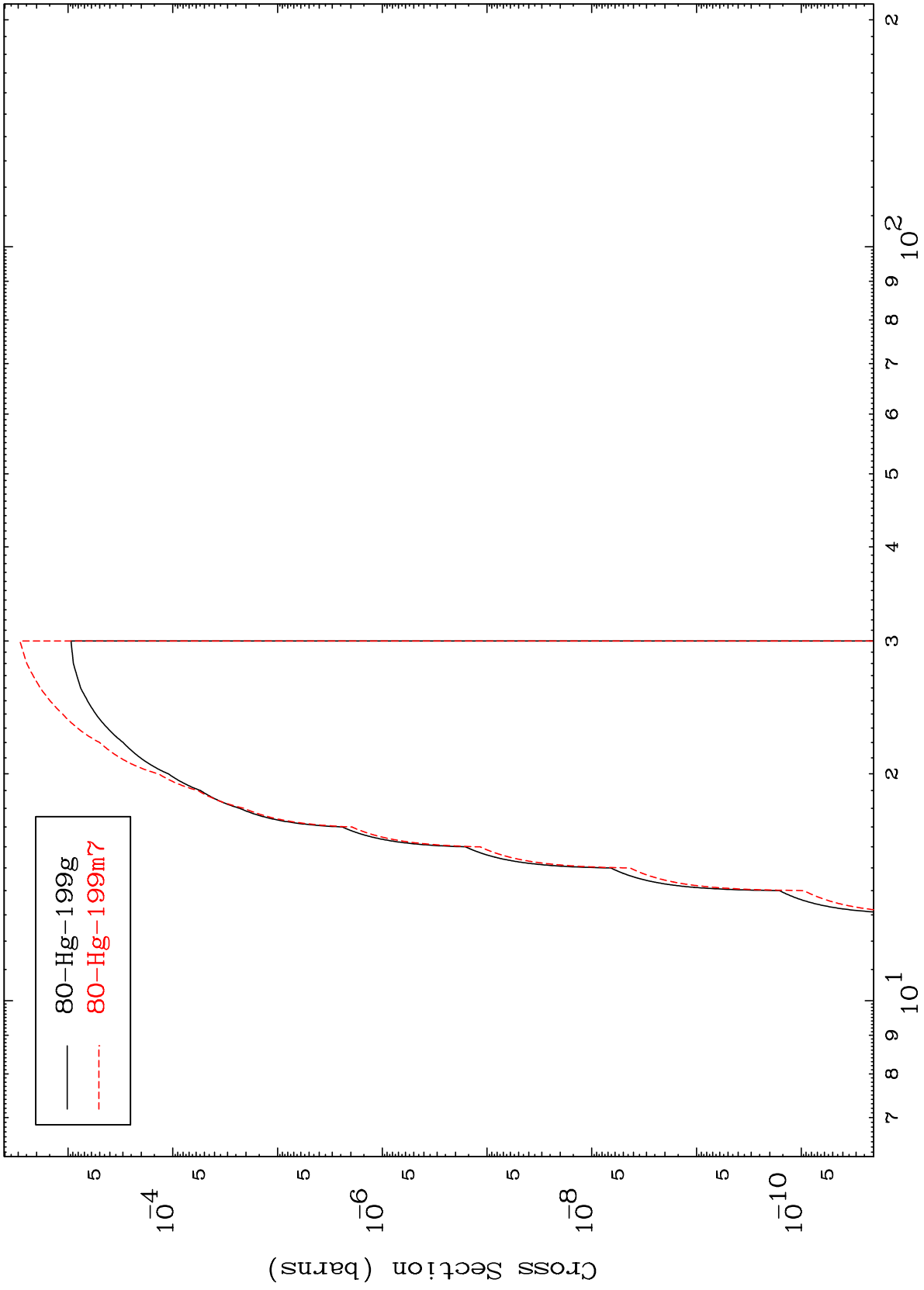
Incident Energy (MeV)

79-Au-196

MAT 7924

79-Au-196

( $\alpha, p$ )  
Radionuclide Production Cross Section



18

Incident Energy (MeV)

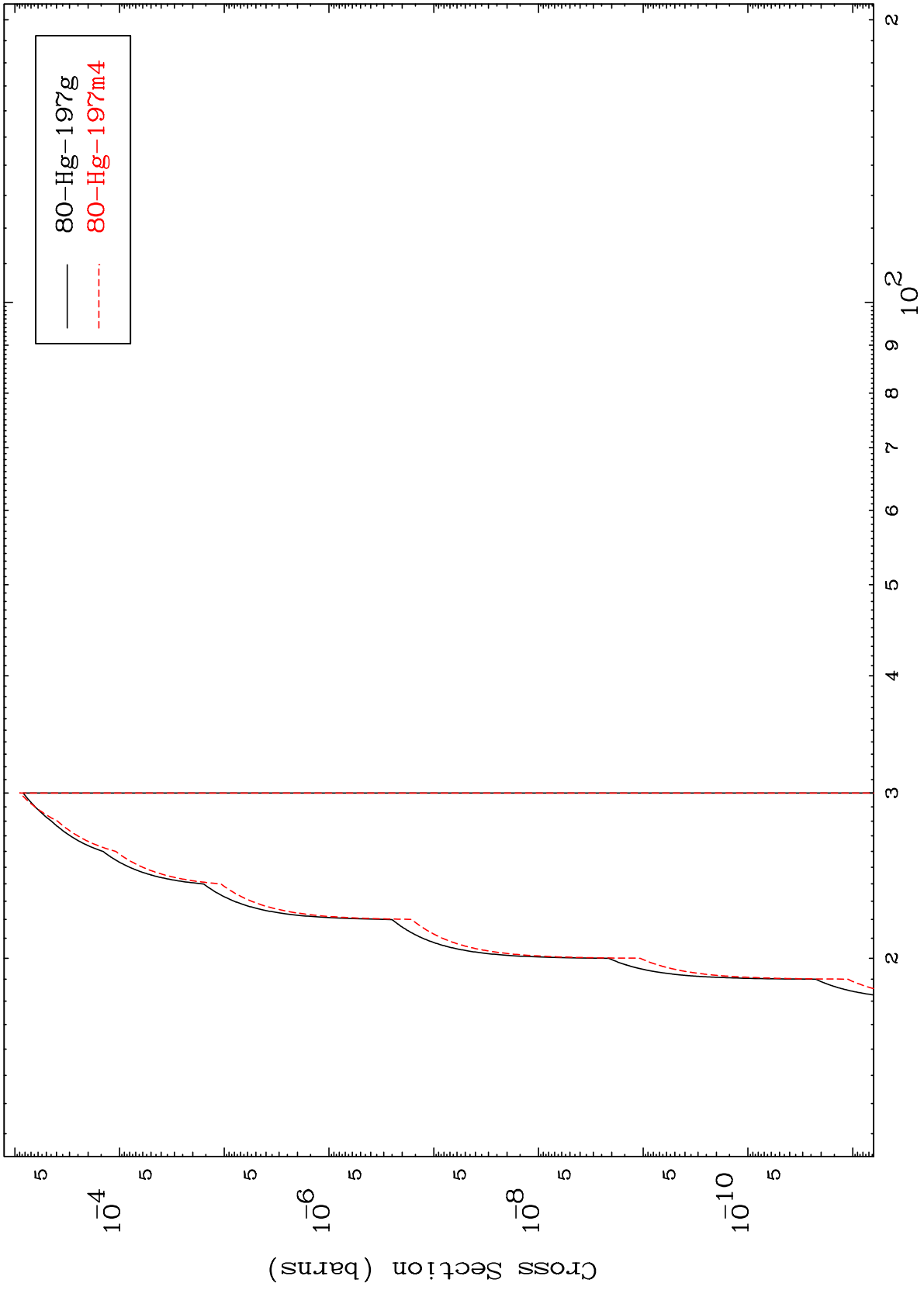
79-Au-196

MAT 7924

( $\alpha, t$ )

79-Au-196

Radionuclide Production Cross Section



19

Incident Energy (MeV)

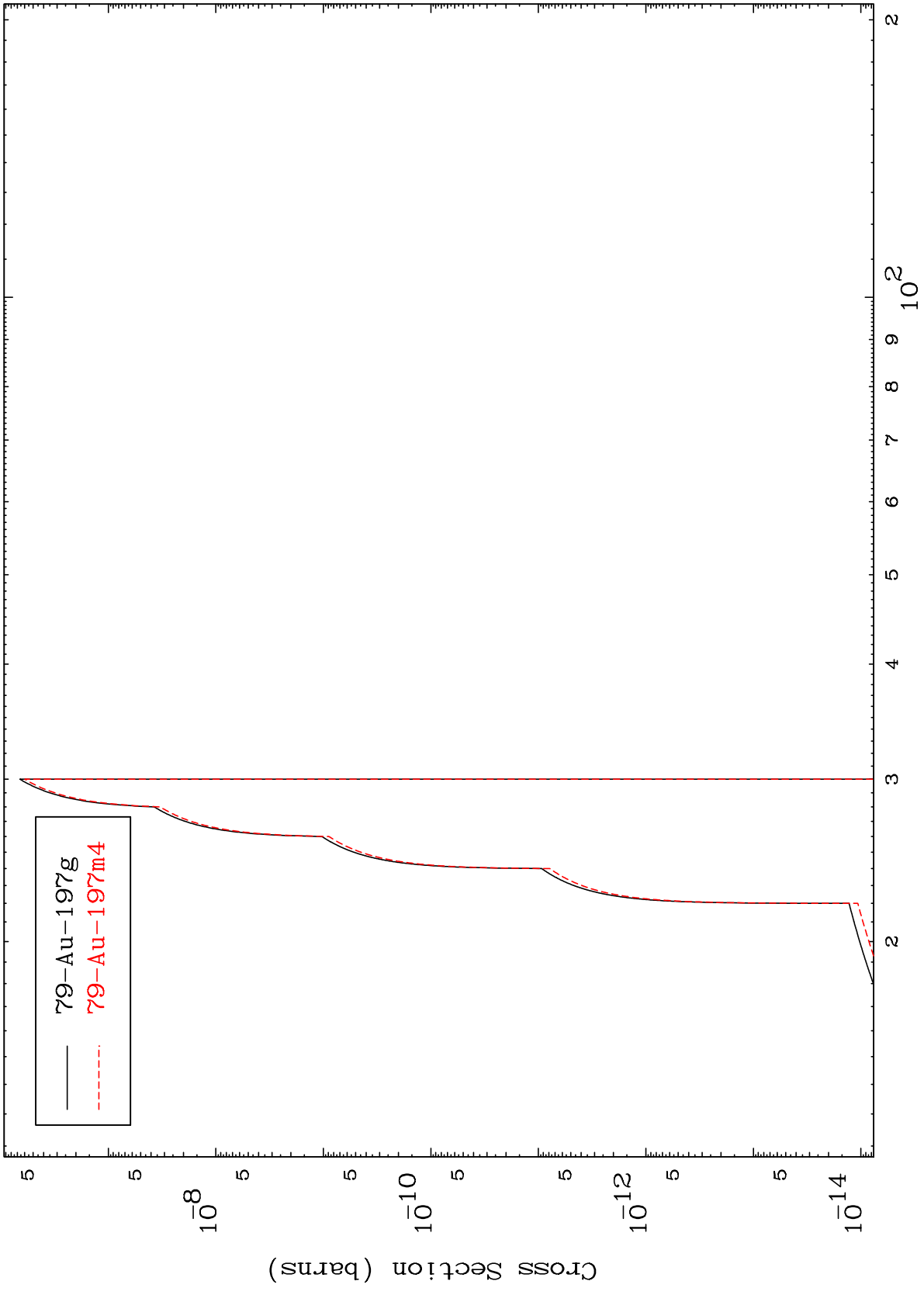
79-Au-196

MAT 7924

( $\alpha, \text{He-3}$ )

<sup>79</sup>Au-196

Radionuclide Production Cross Section



20

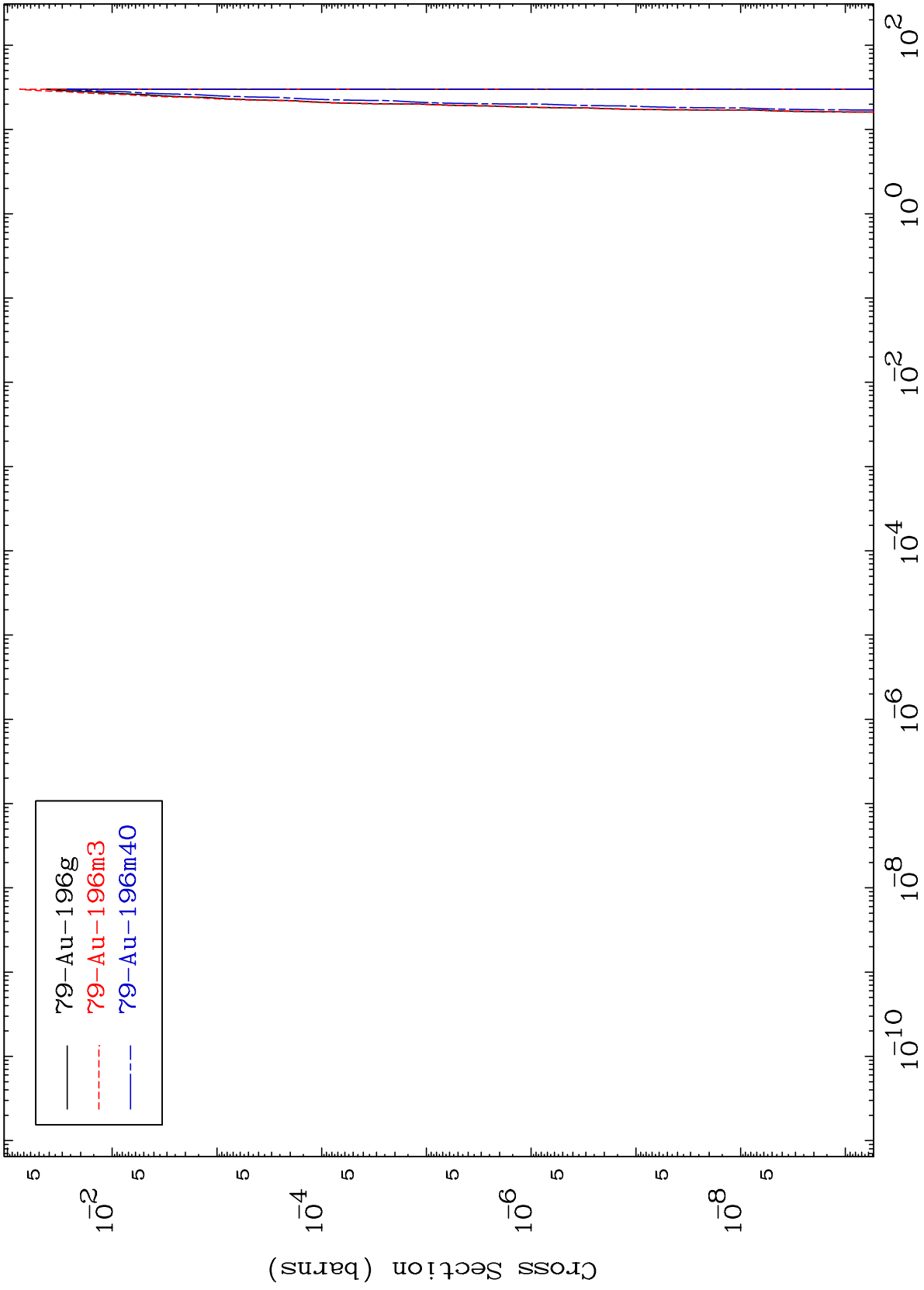
Incident Energy (MeV)

<sup>79</sup>Au-196

MAT 7924

$(\alpha, \alpha)$   
Radionuclide Production Cross Section

<sup>79</sup>Au-196



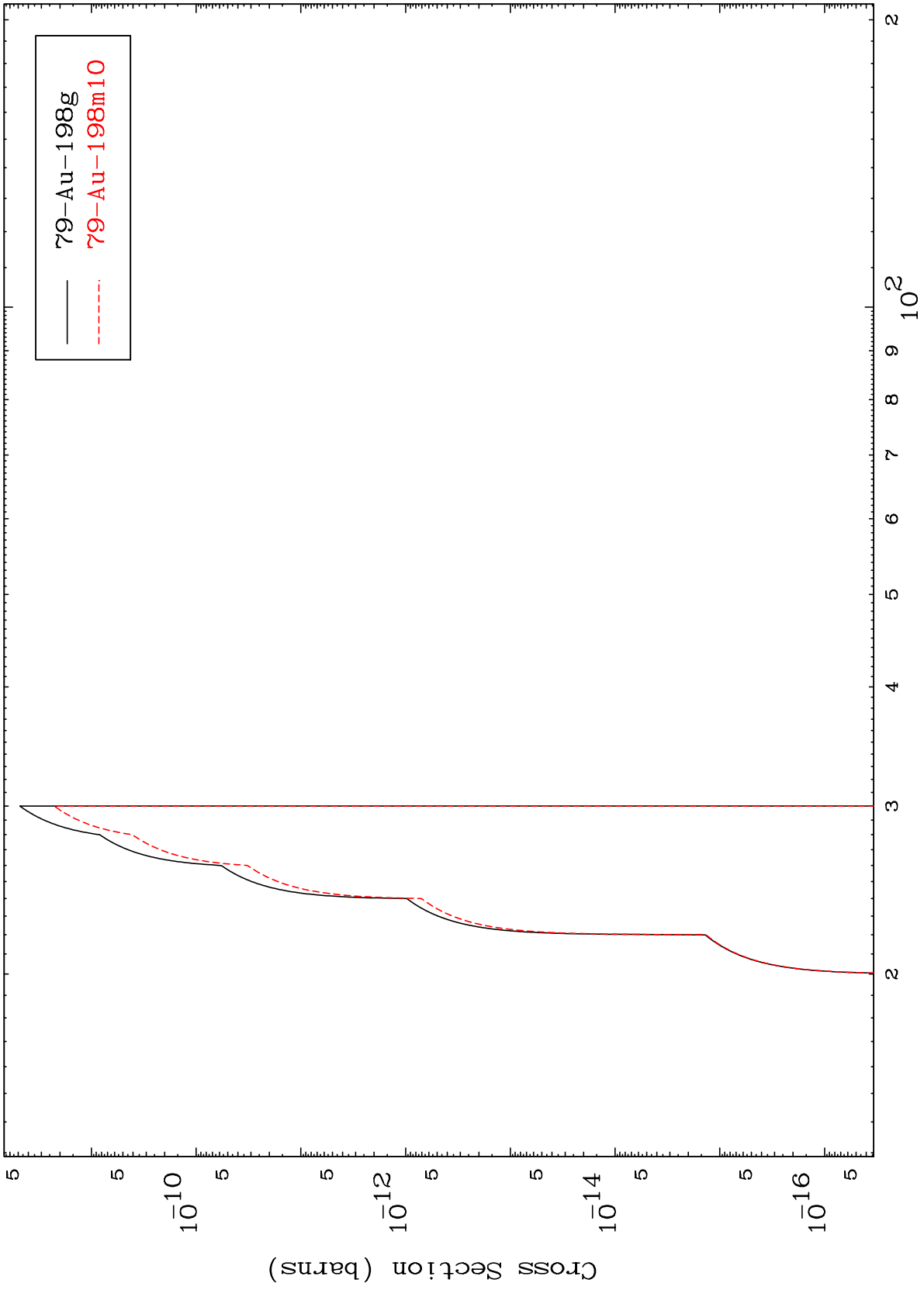
21

<sup>79</sup>Au-196

MAT 7924

79-Au-196

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



22

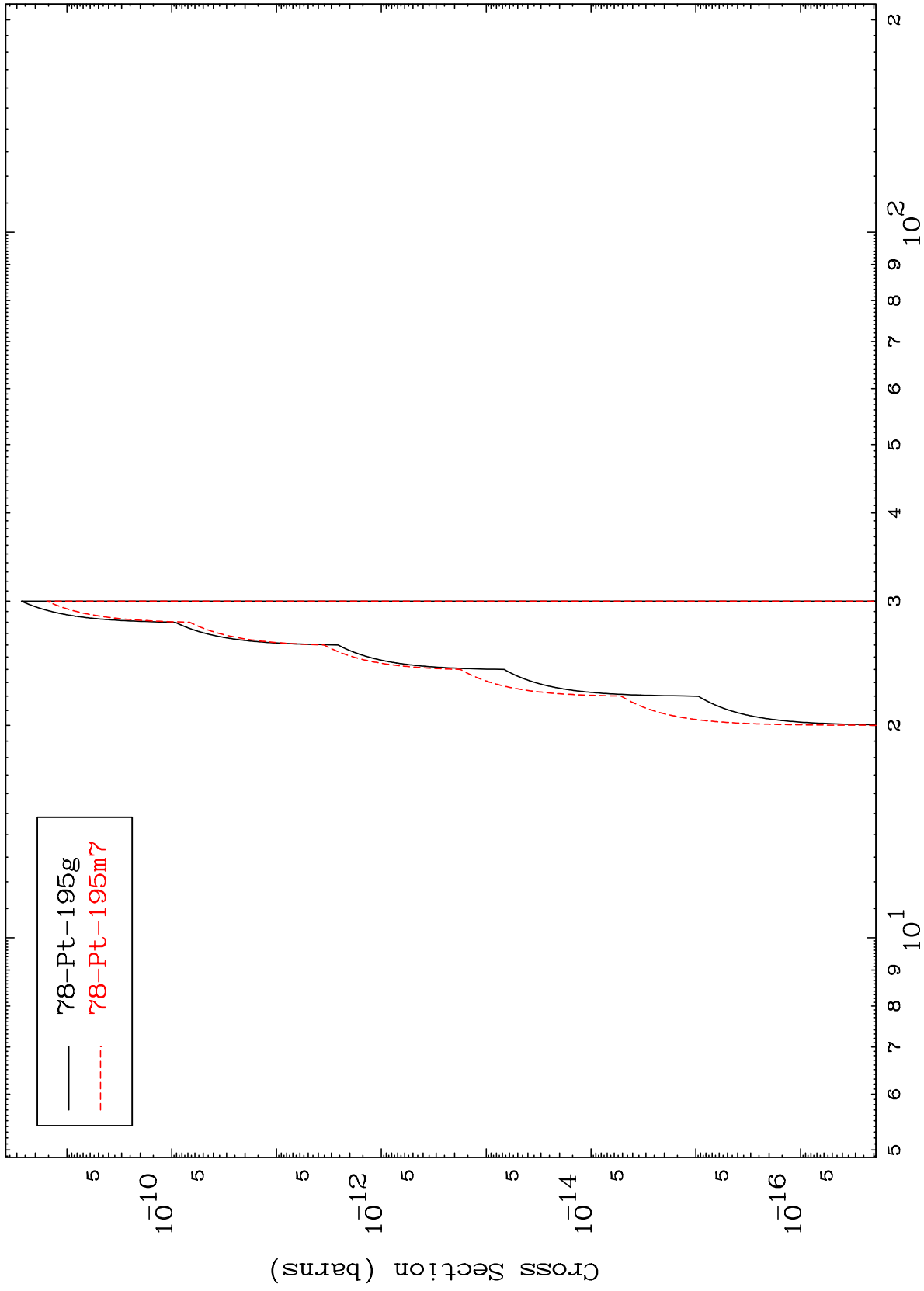
79-Au-196

MAT 7924

( $\alpha, p$ )  $\alpha$

79-Au-196

Radionuclide Production Cross Section



23

Incident Energy (MeV)

79-Au-196