

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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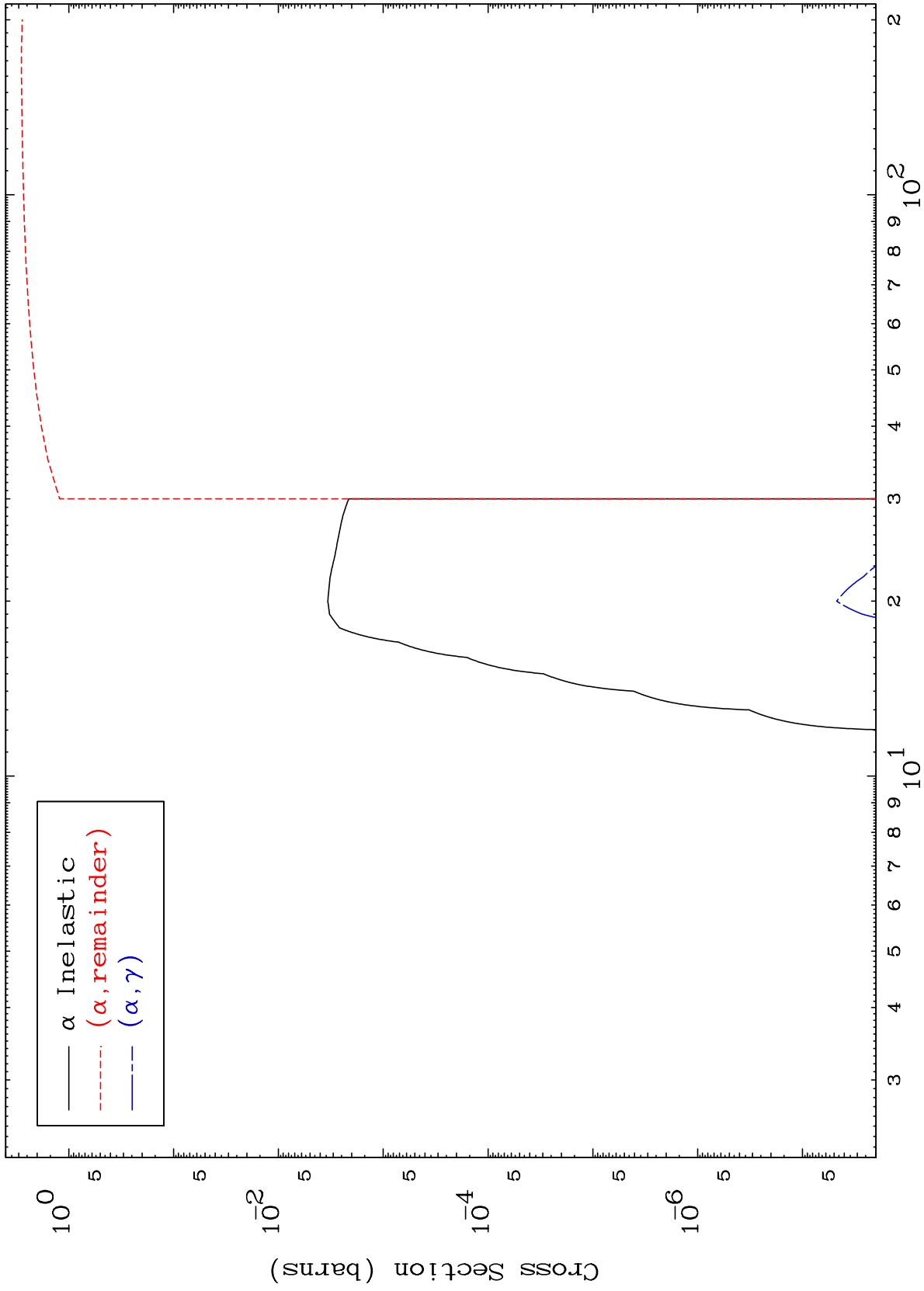
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7955

0 Kelvin  $\alpha$  Major  
Cross Sections

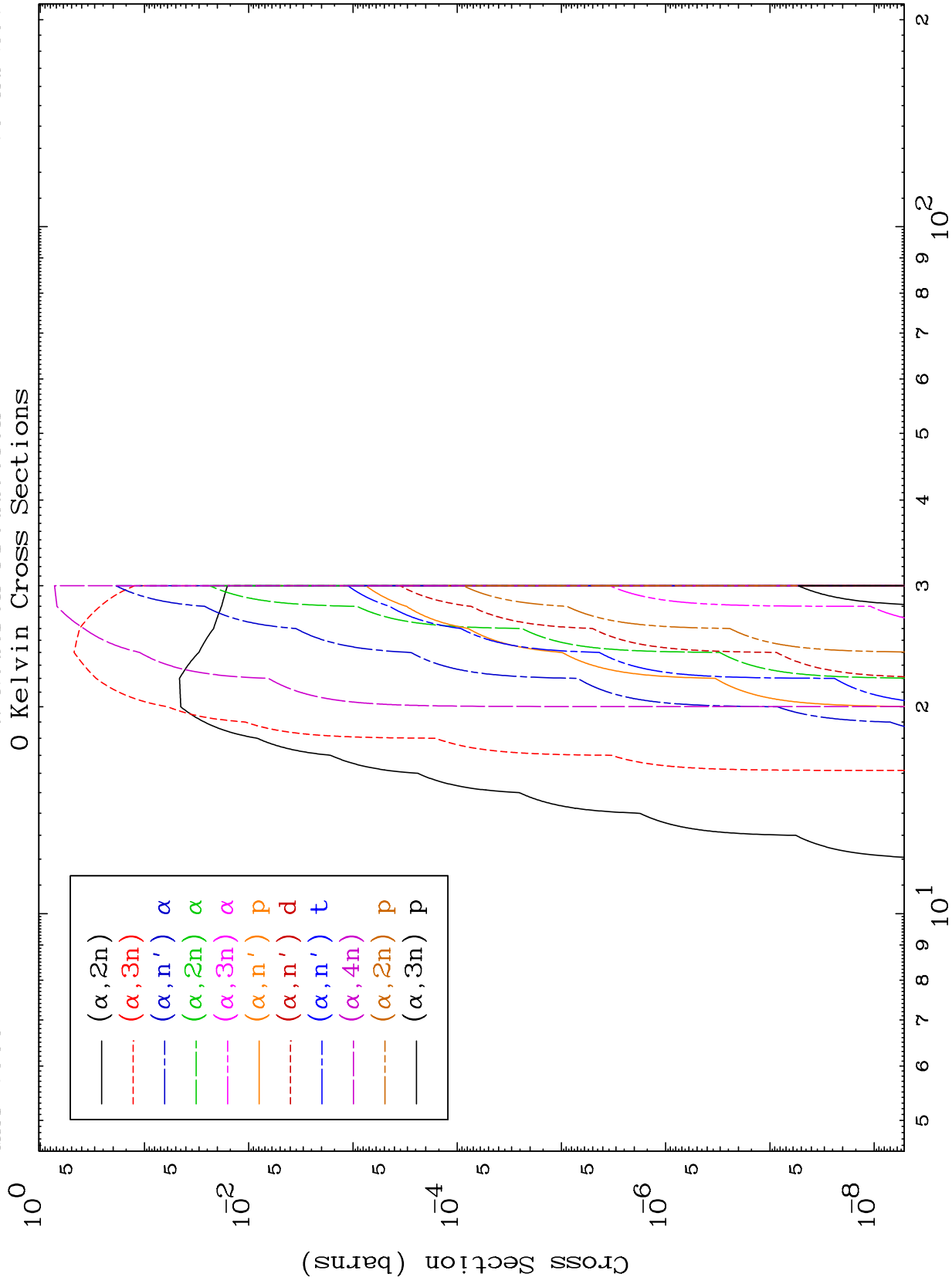
79-Au-207



MAT 7955

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

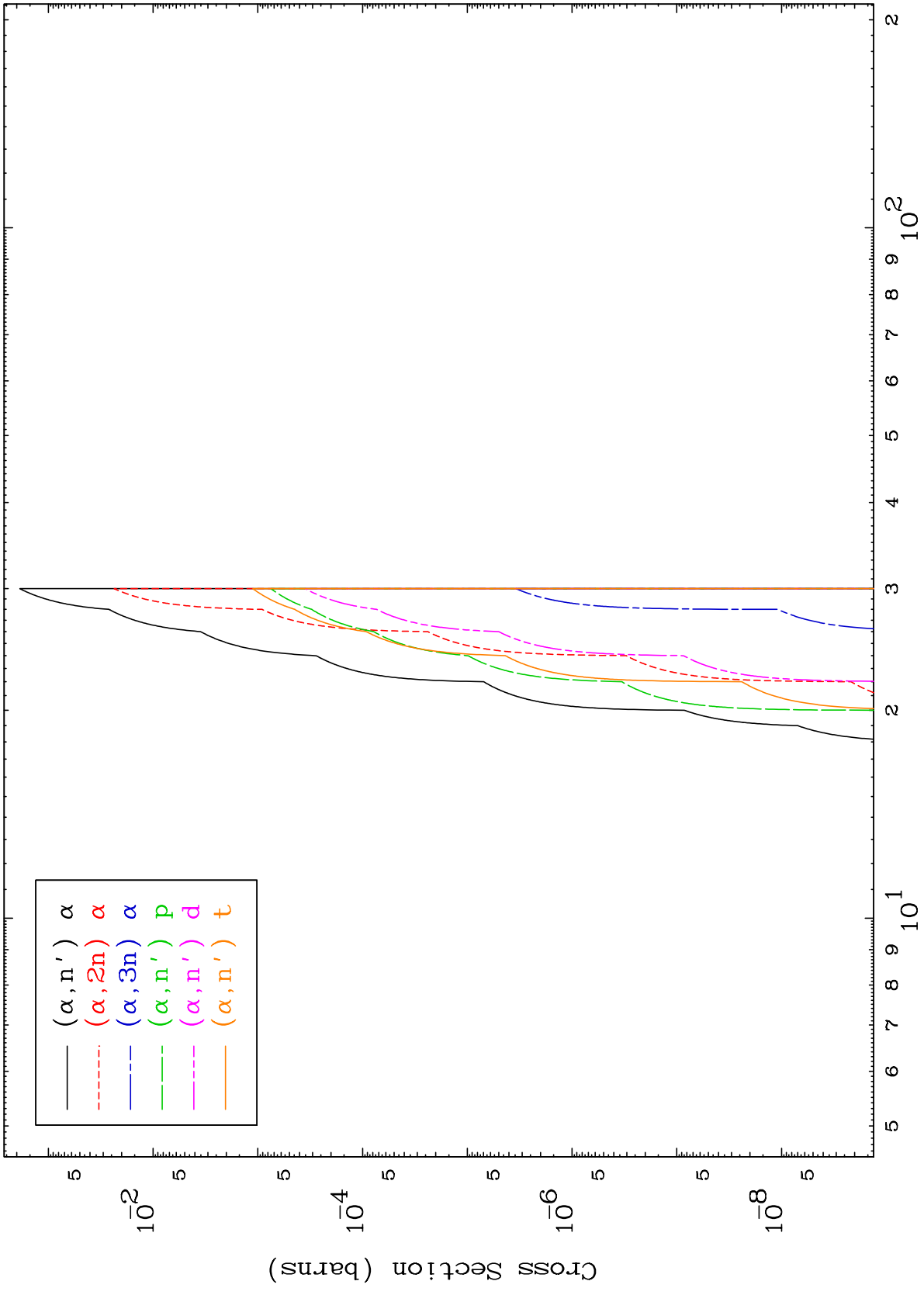
79-Au-207

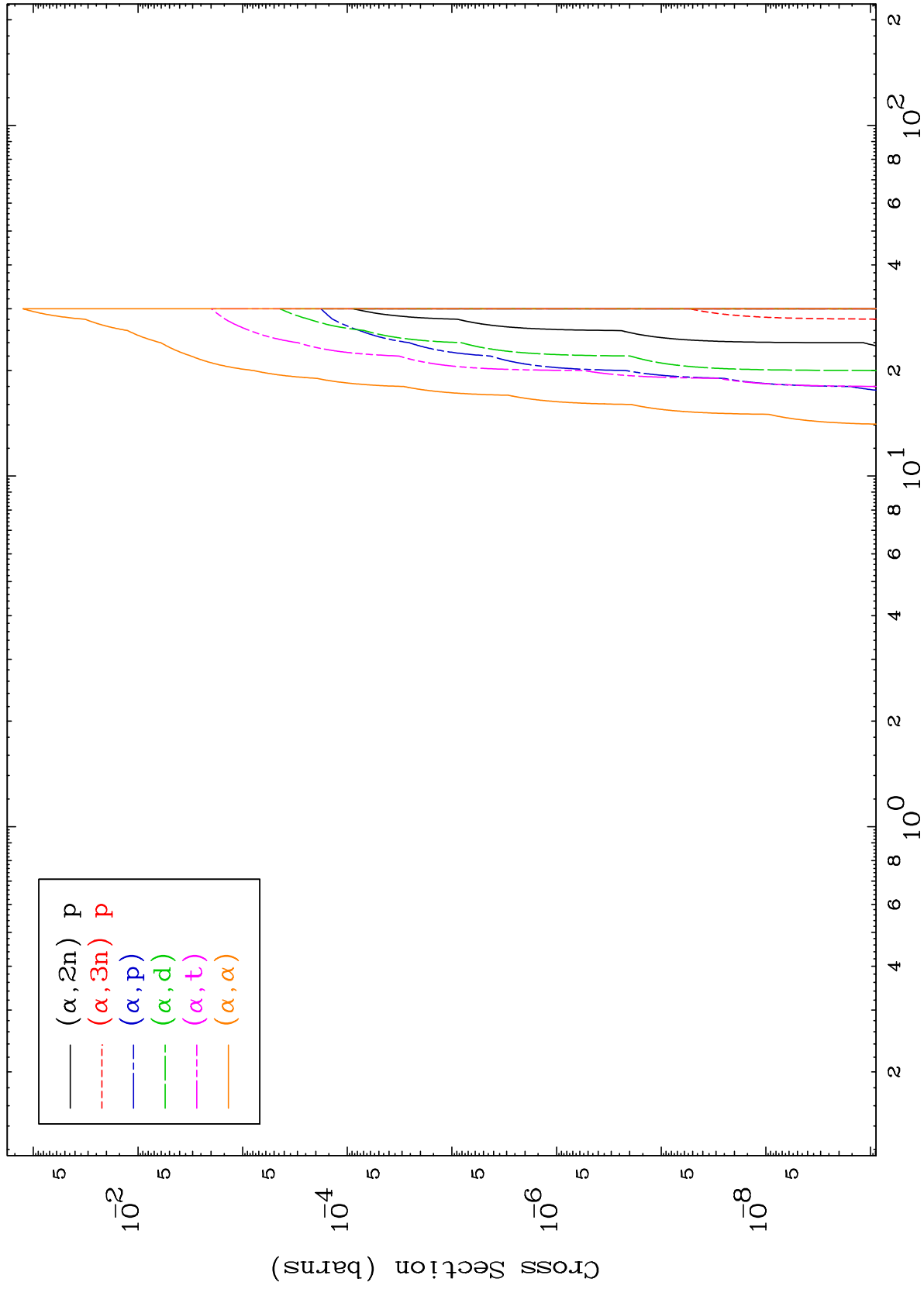


2

Incident Energy (MeV)

79-Au-207

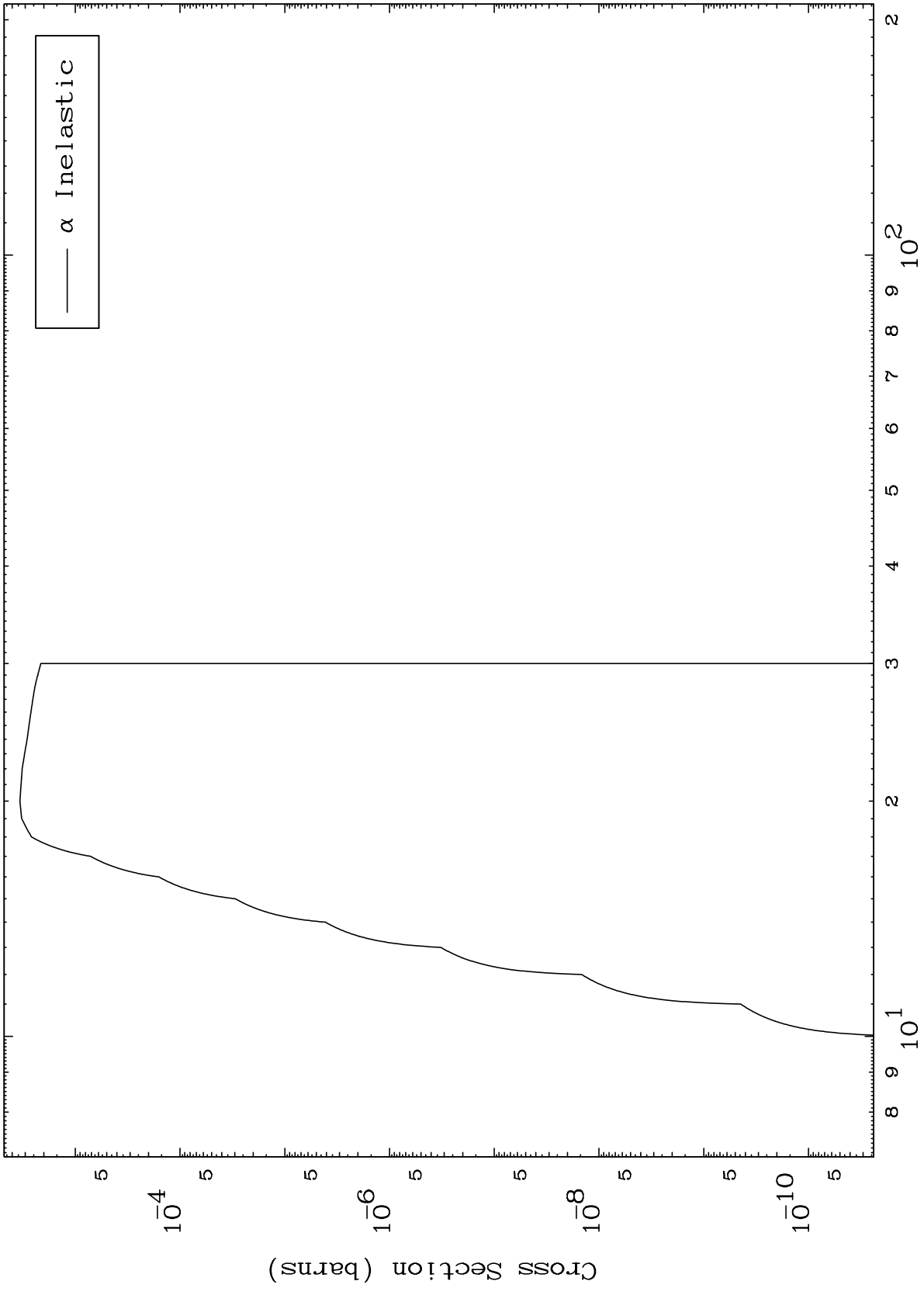




MAT 7955

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

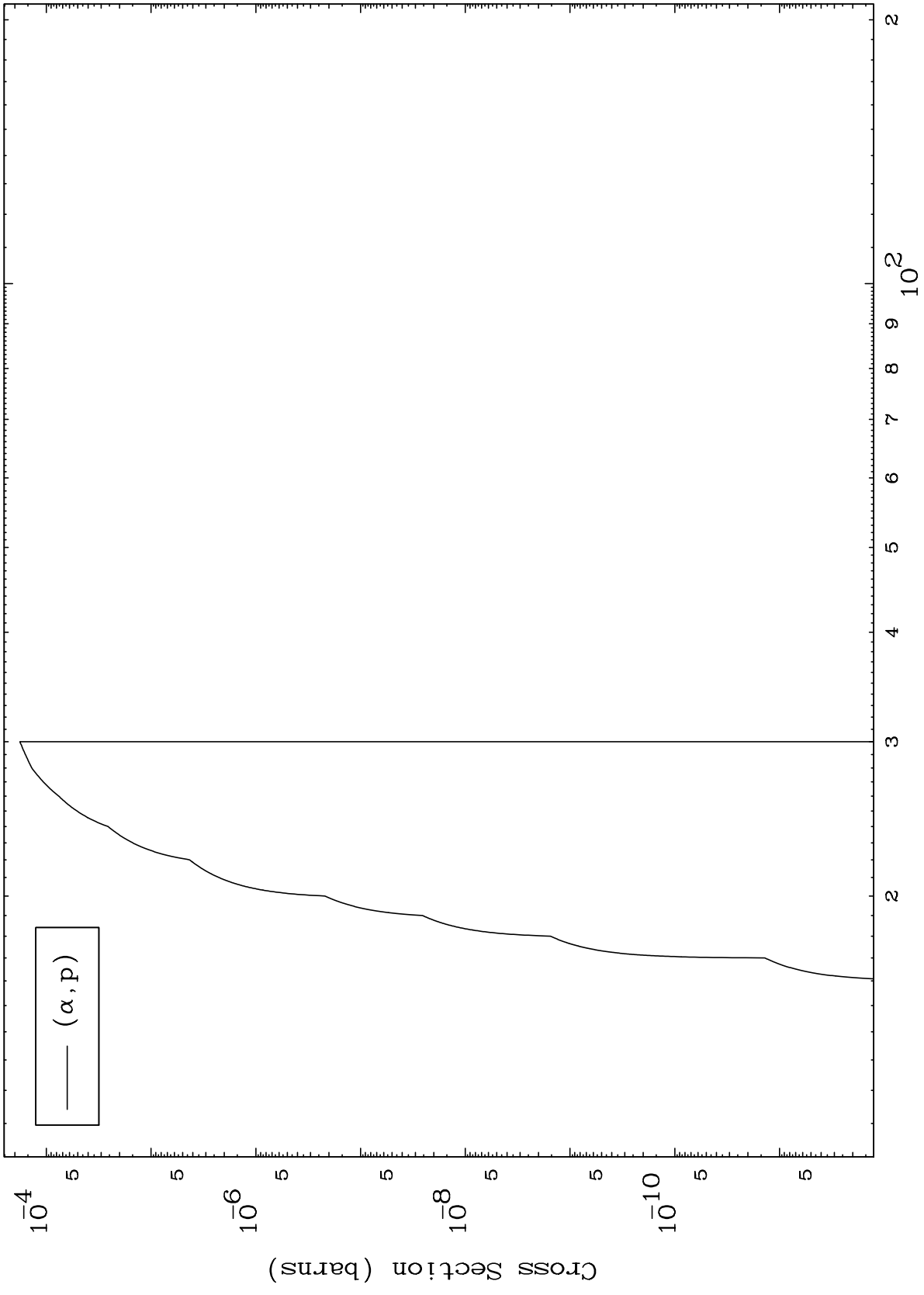
79-Au-207



5

Incident Energy (MeV)

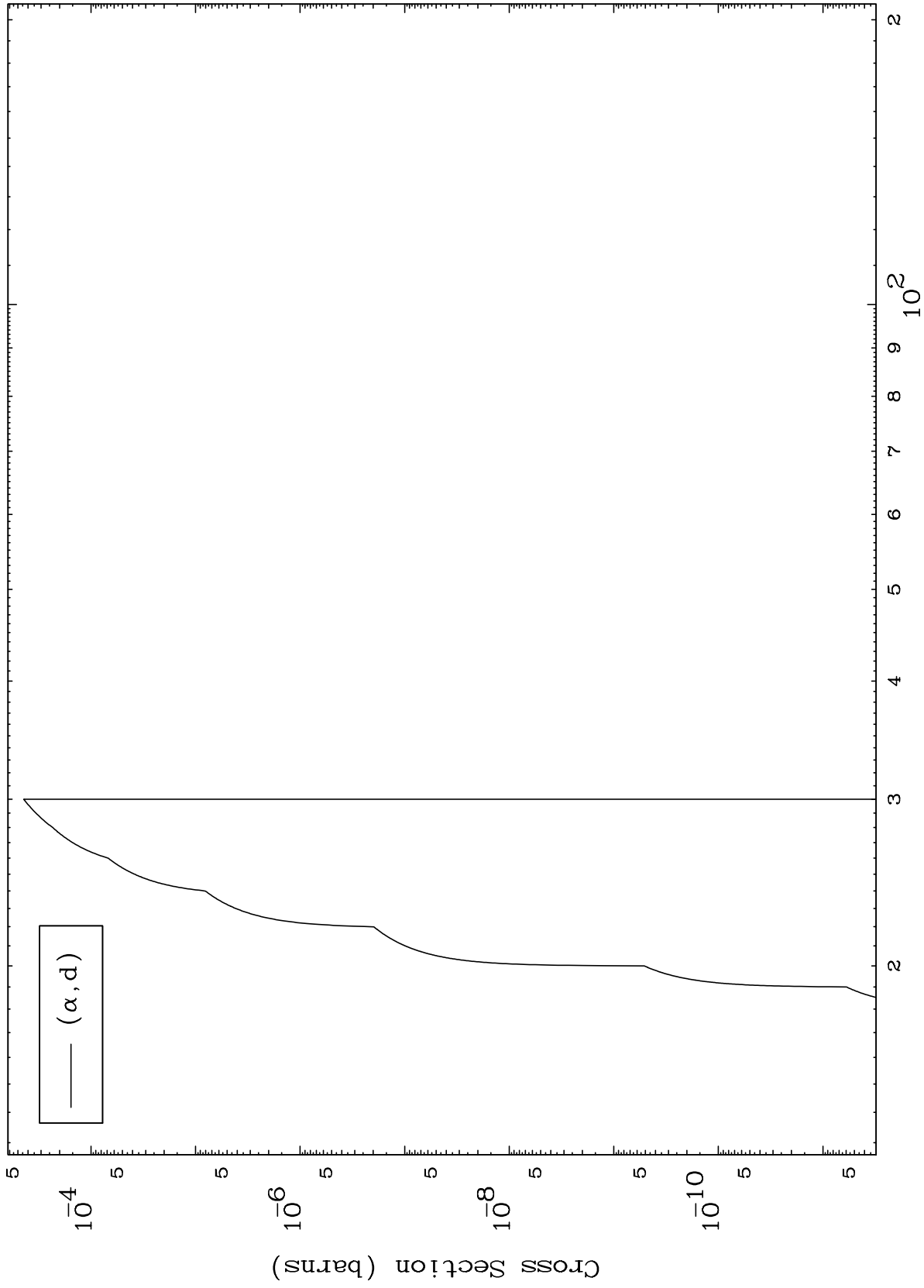
79-Au-207



MAT 7955

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

79-Au-207



7

Incident Energy (MeV)

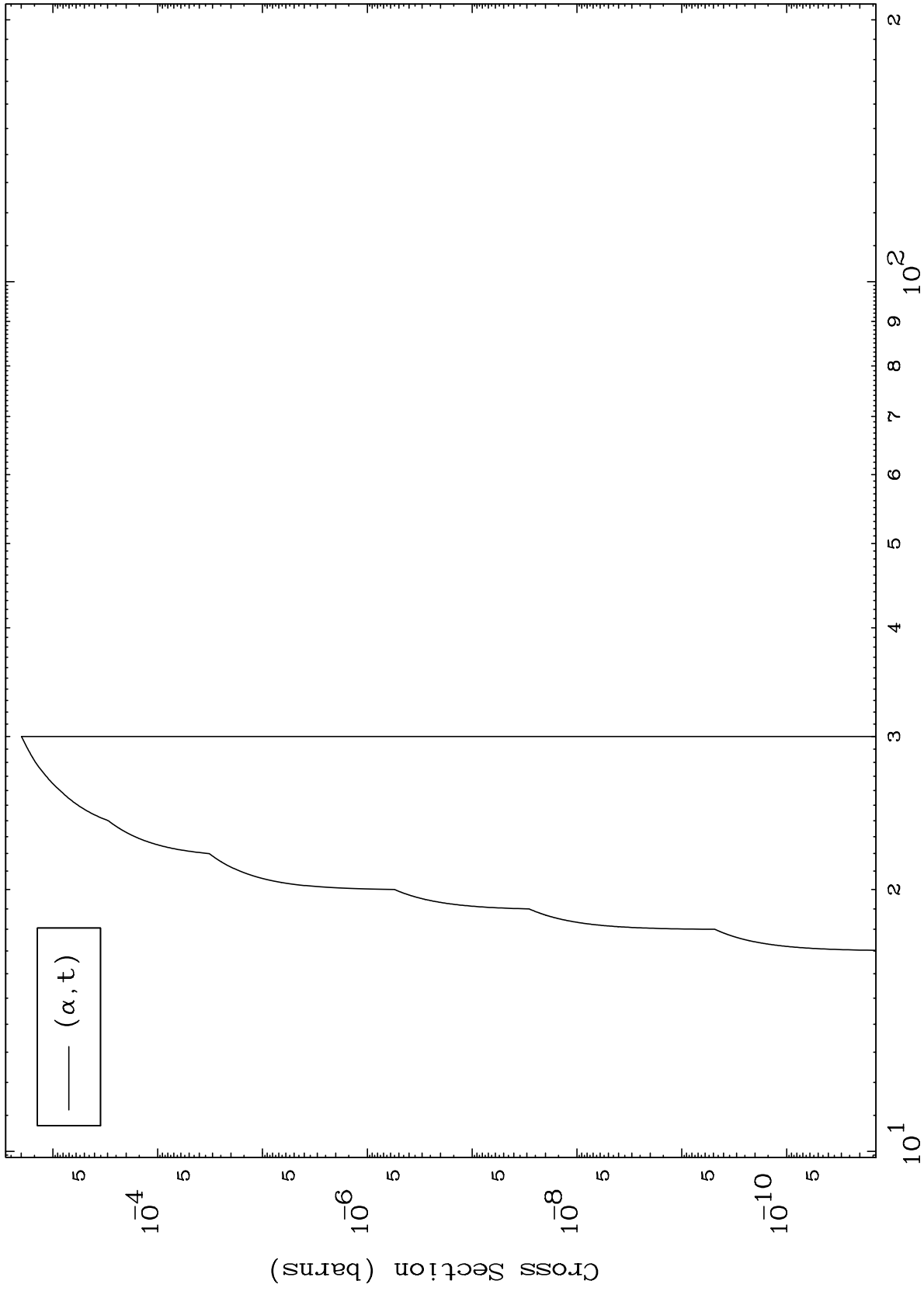
79-Au-207

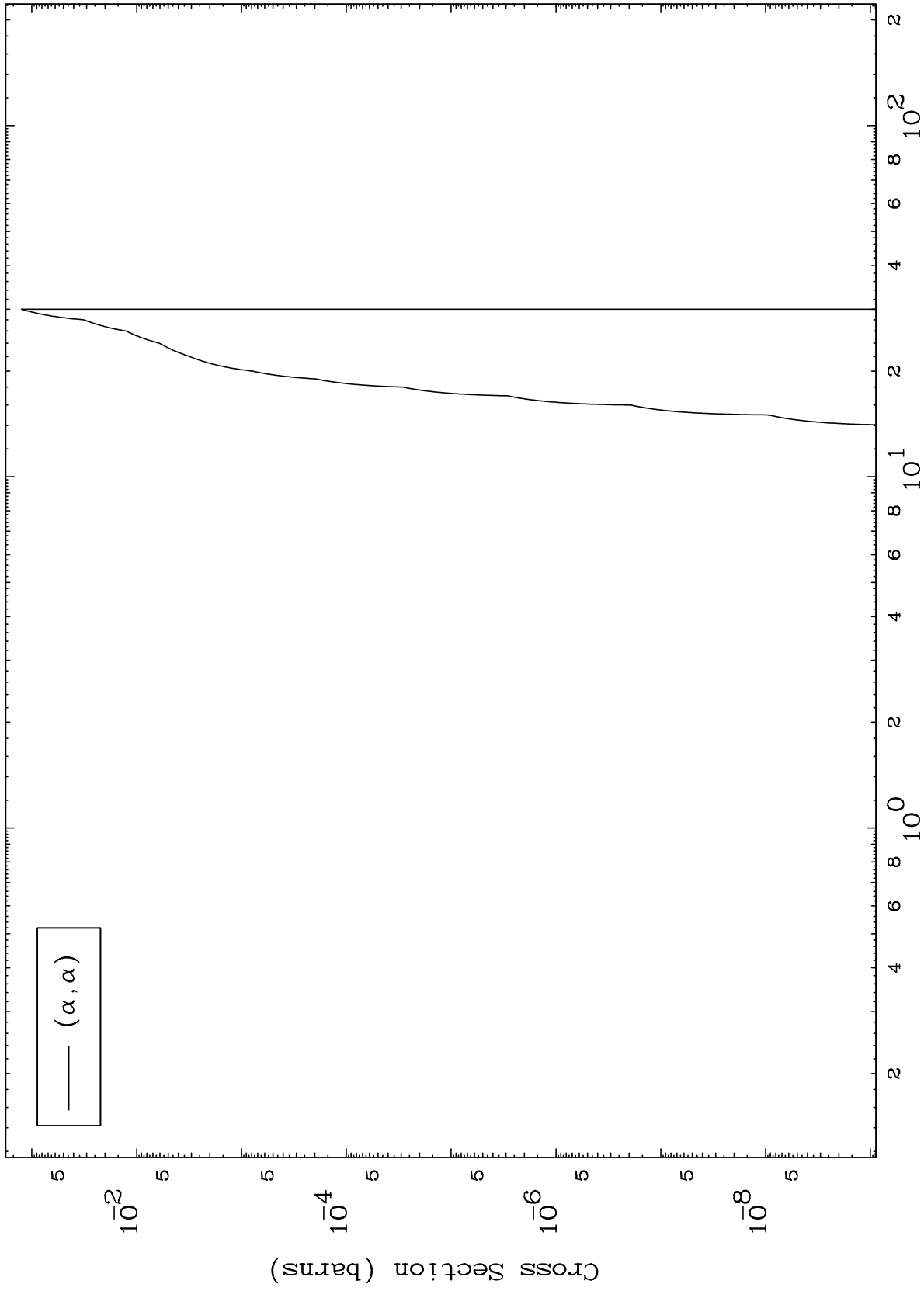


MAT 7955

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

79-Au-207

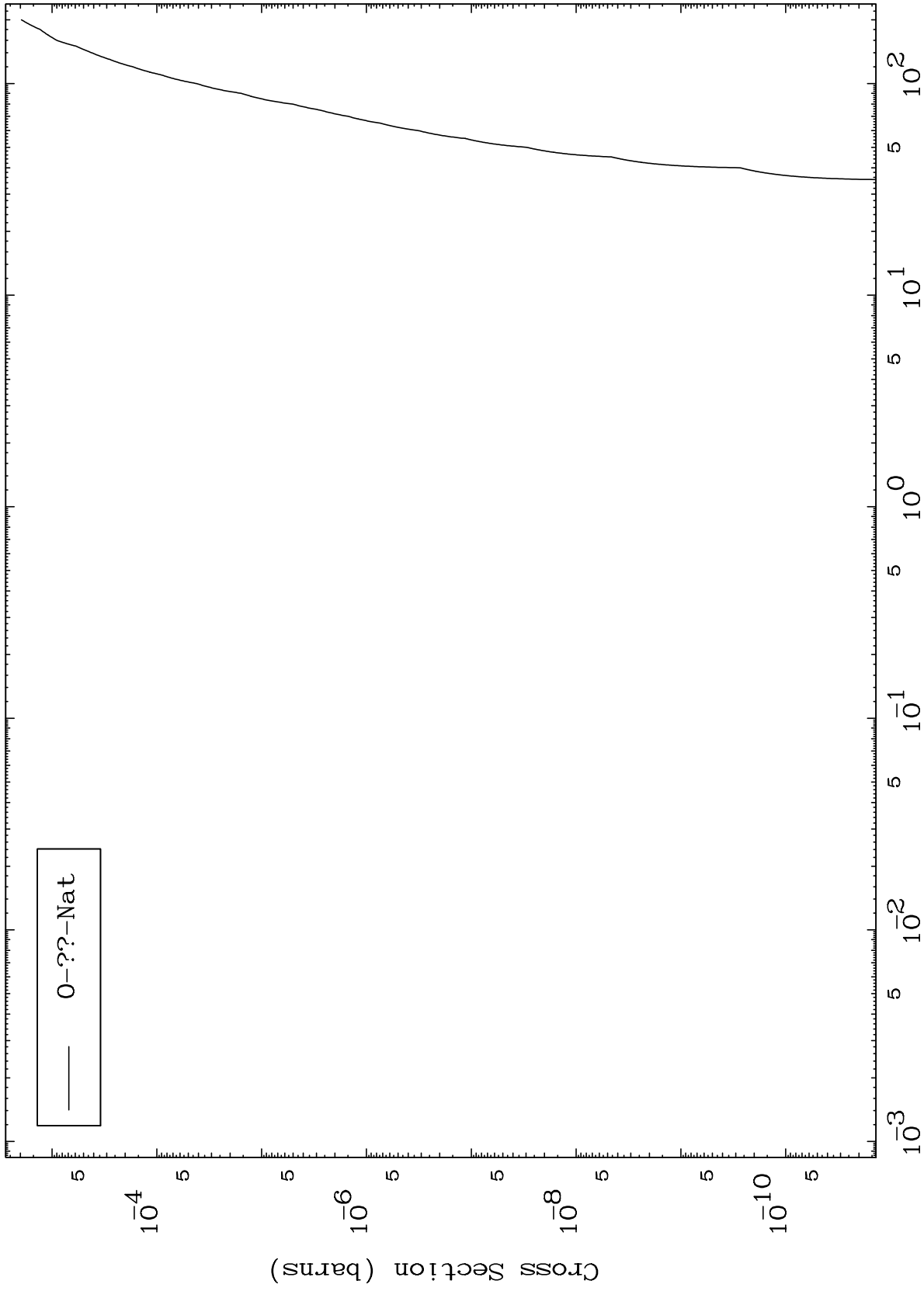




MAT 7955

$\alpha$  Fission  
Radionuclide Production Cross Section

79-Au-207



Incident Energy (MeV)

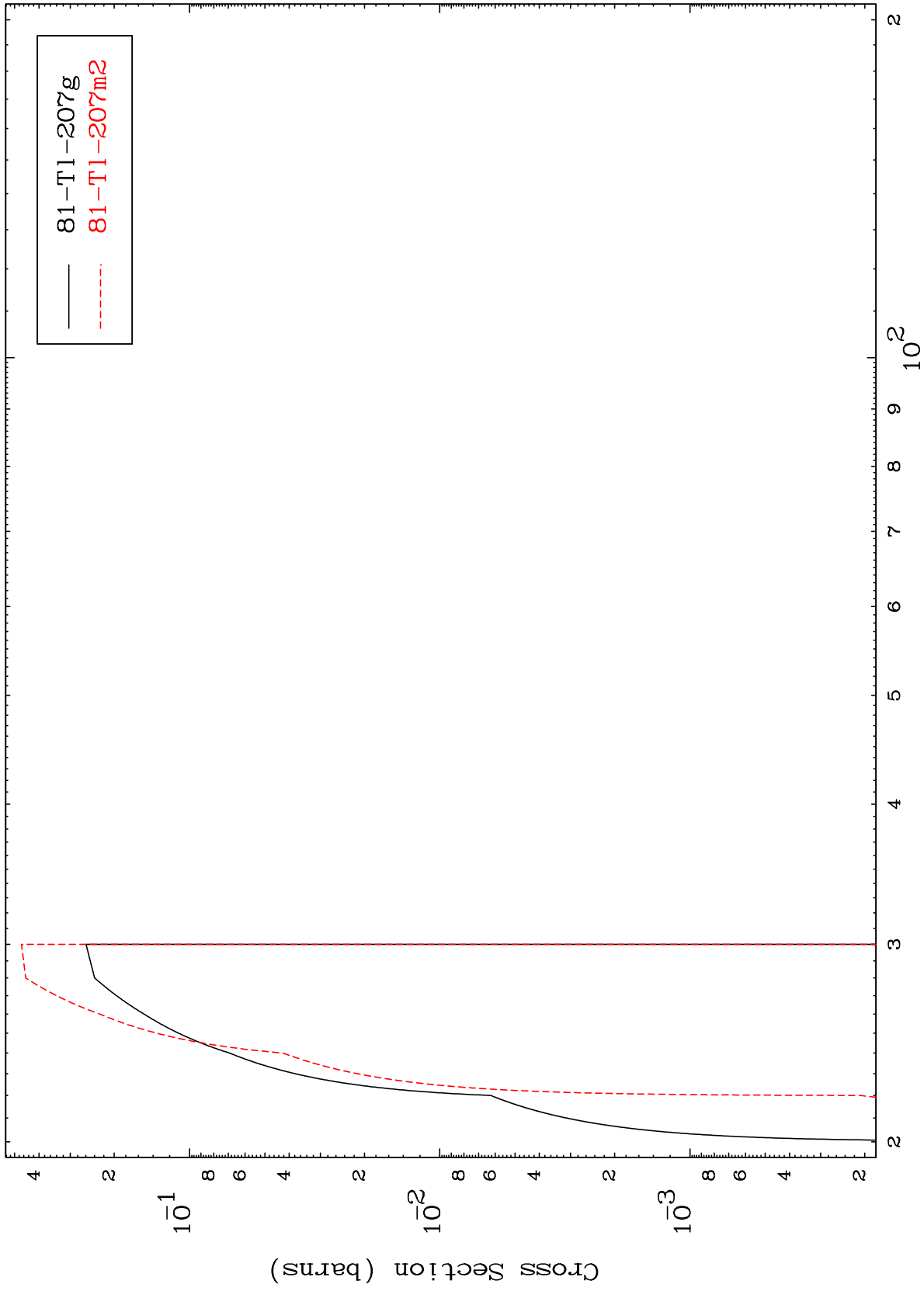
79-Au-207

MAT 7955

( $\alpha, 4n$ )

79-Au-207

Radionuclide Production Cross Section



11

Incident Energy (MeV)

79-Au-207