

Program EVALPLOT  
(Version 2018-1)

by

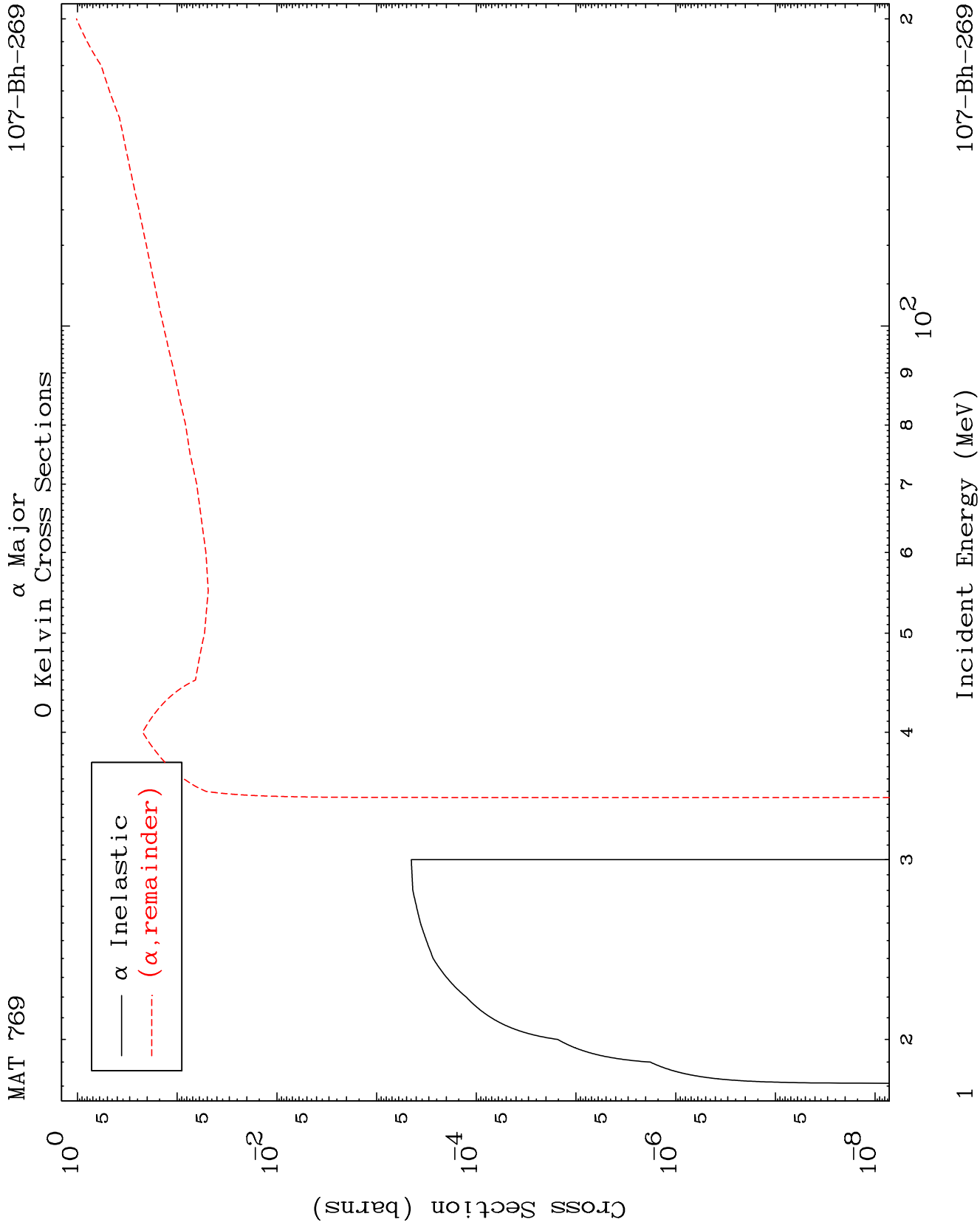
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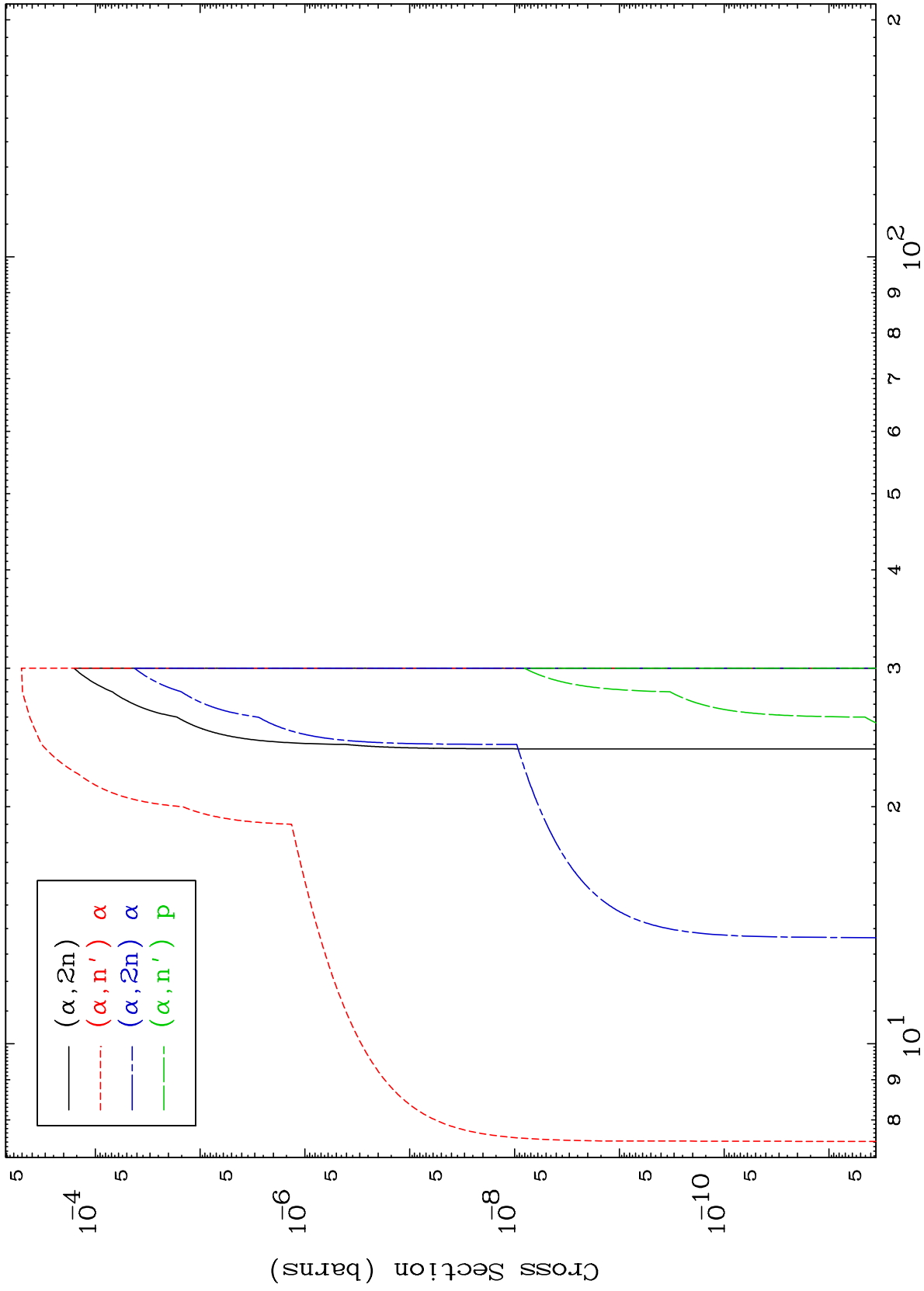
Press Mouse Button to Start



MAT 769

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

107-Bh-269



2

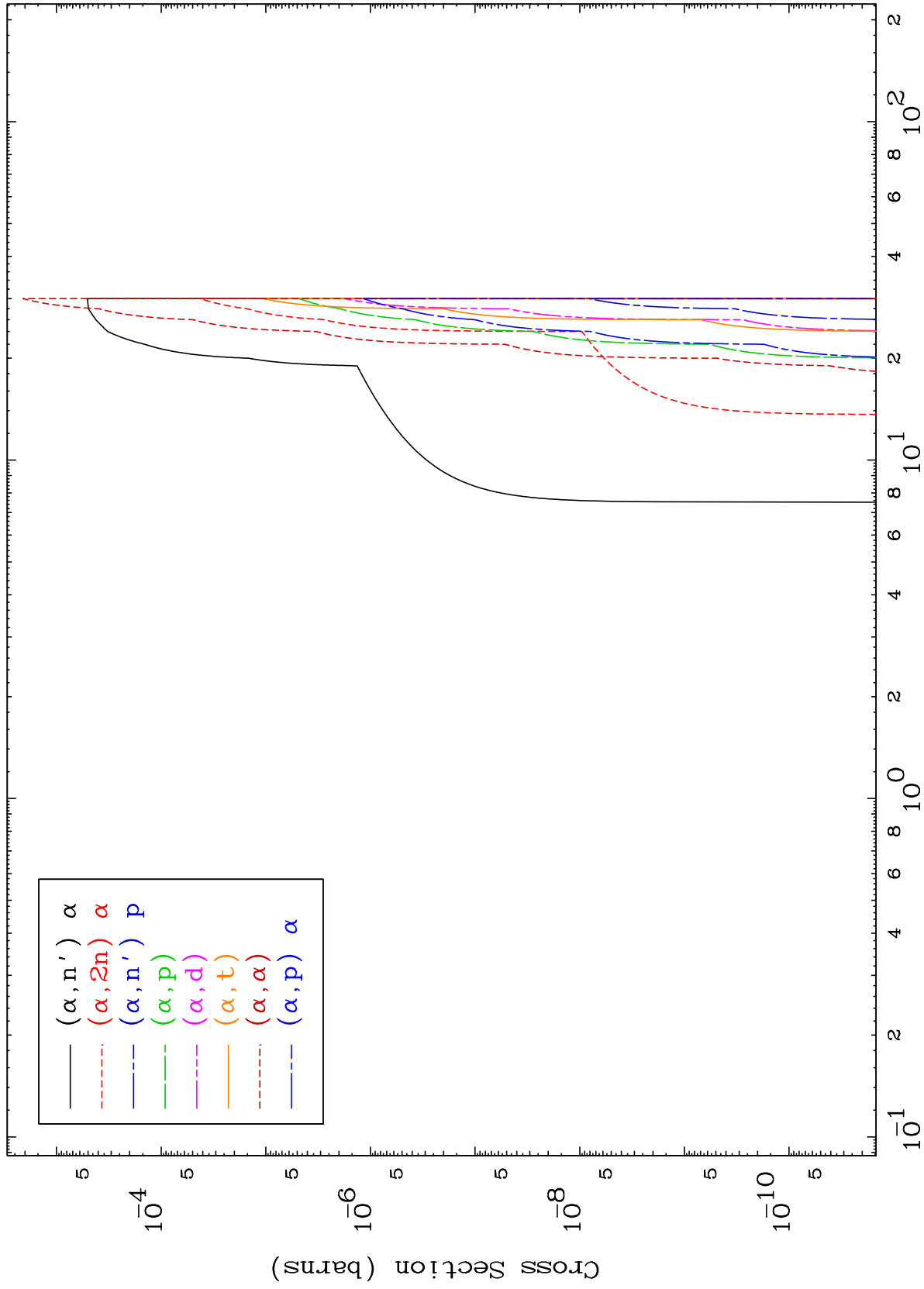
Incident Energy (MeV)

107-Bh-269

MAT 769

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

107-Bh-269



Incident Energy (MeV)

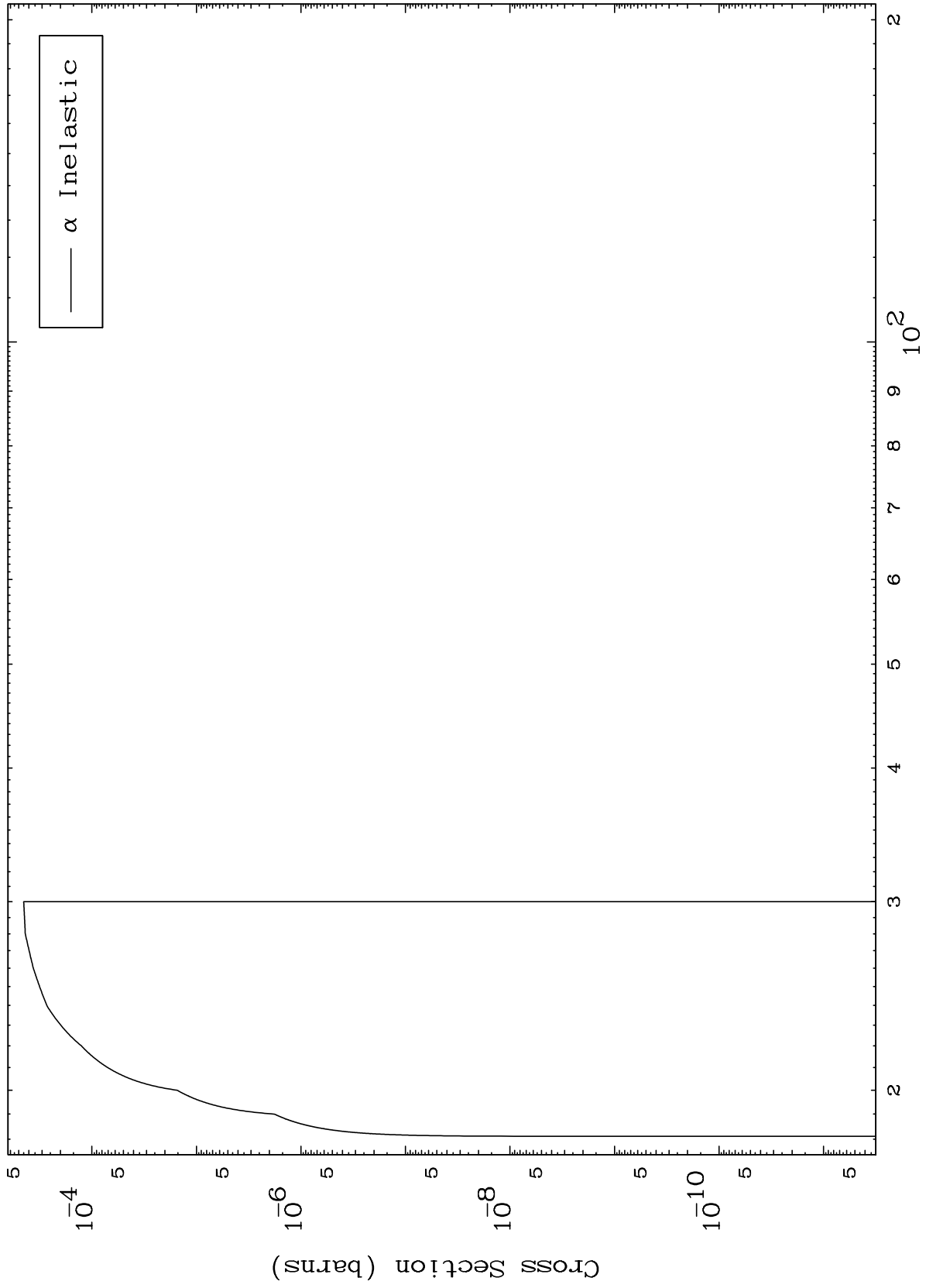
107-Bh-269

3

MAT 769

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

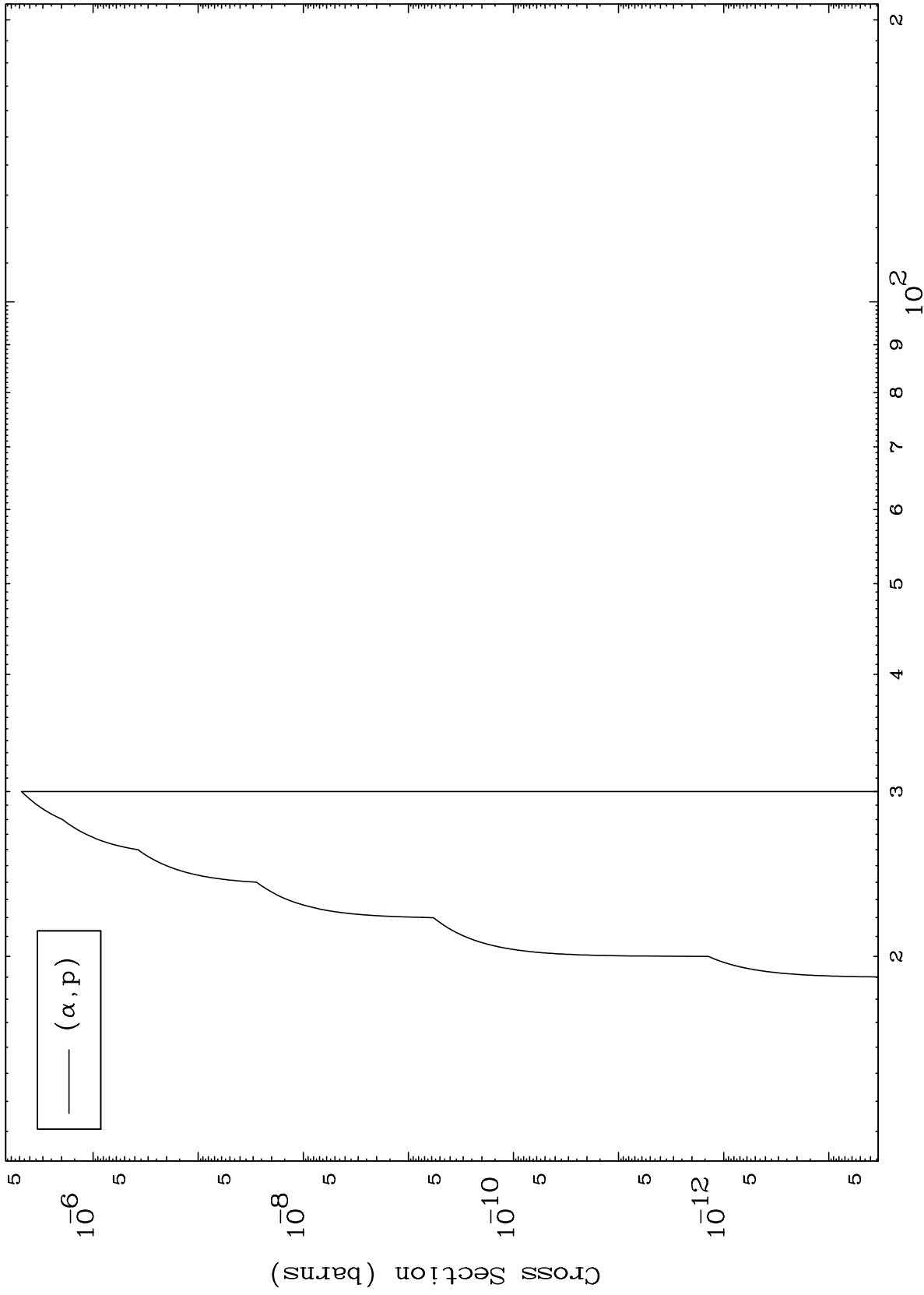
107-Bh-269



4

Incident Energy (MeV)

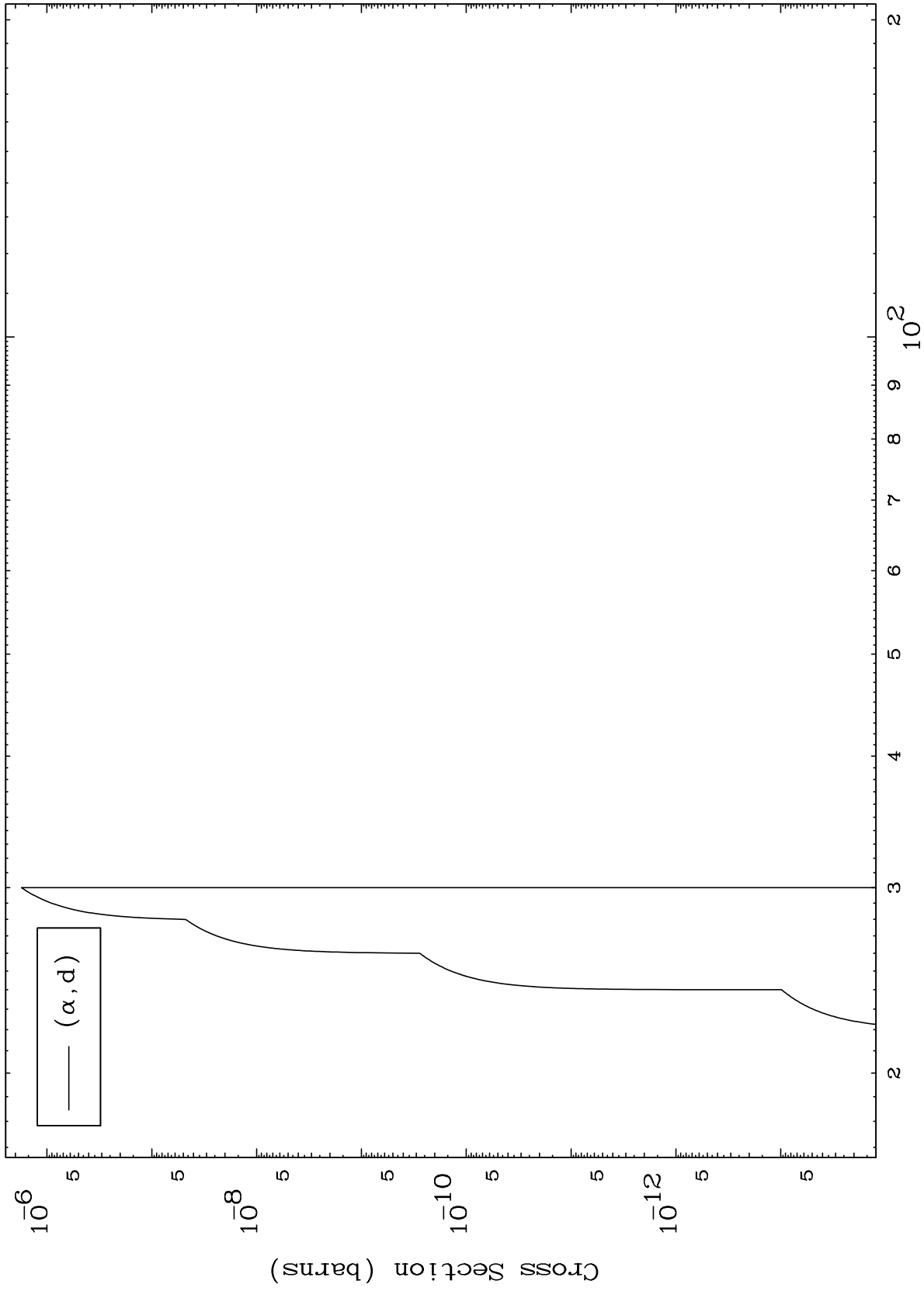
107-Bh-269



MAT 769

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

107-Bh-269



6

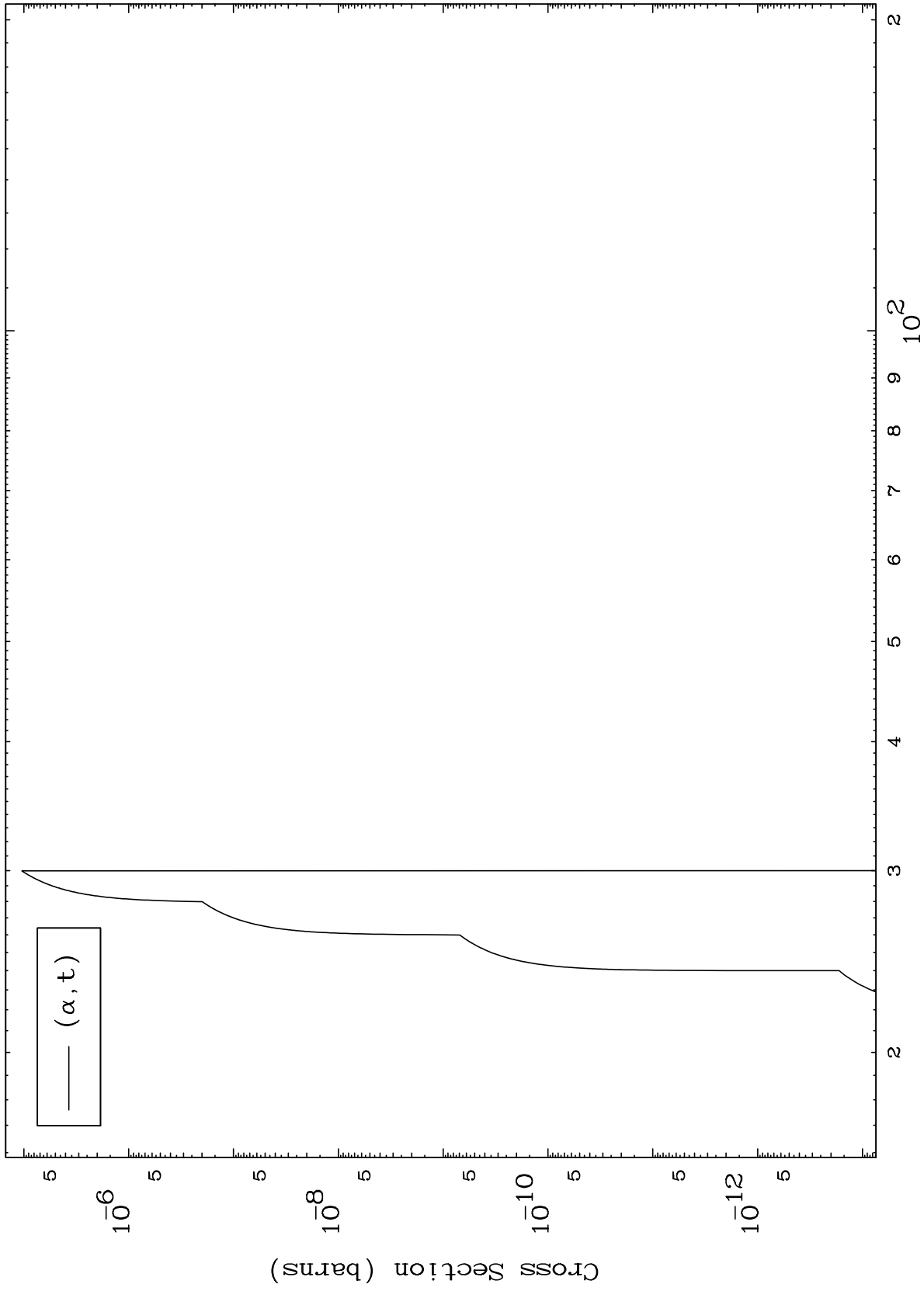
Incident Energy (MeV)

107-Bh-269

MAT 769

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

107-Bh-269



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Incident Energy (MeV)

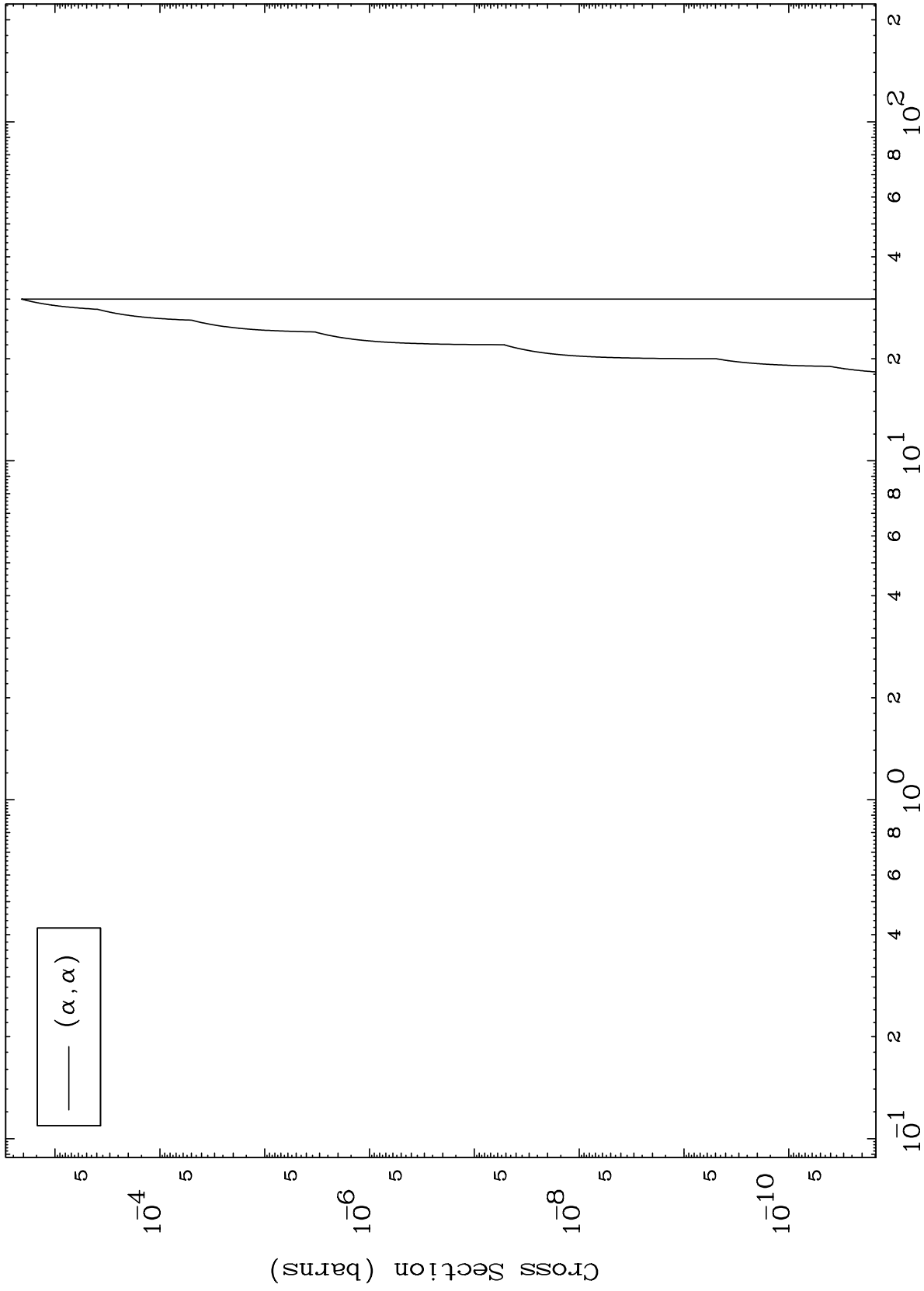
107-Bh-269



MAT 769

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

107-Bh-269



8

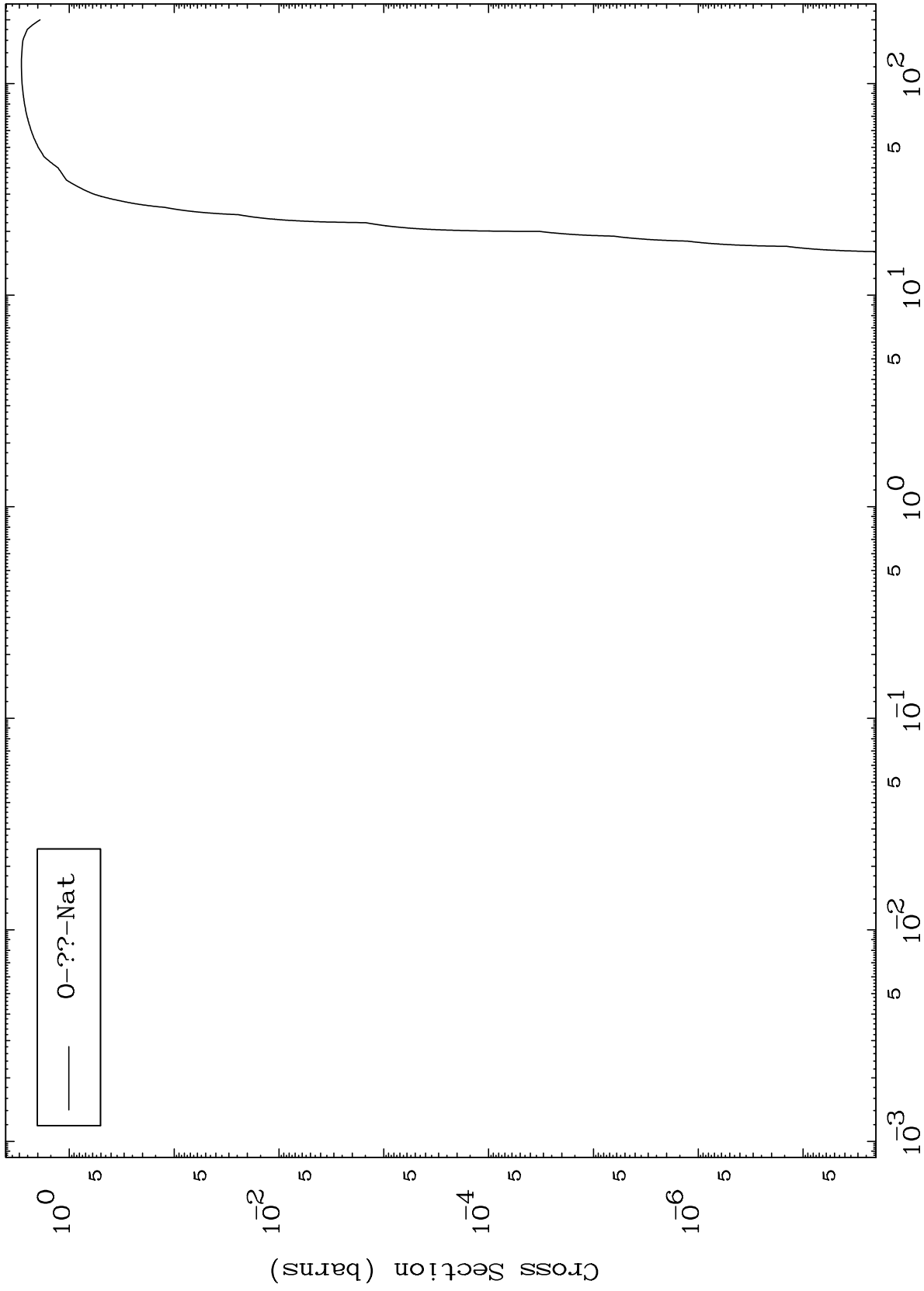
Incident Energy (MeV)

107-Bh-269

MAT 769

$\alpha$  Fission  
Radionuclide Production Cross Section

107-Bh-269



Incident Energy (MeV)

107-Bh-269

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