

Program EVALPLOT  
(Version 2018-1)

by

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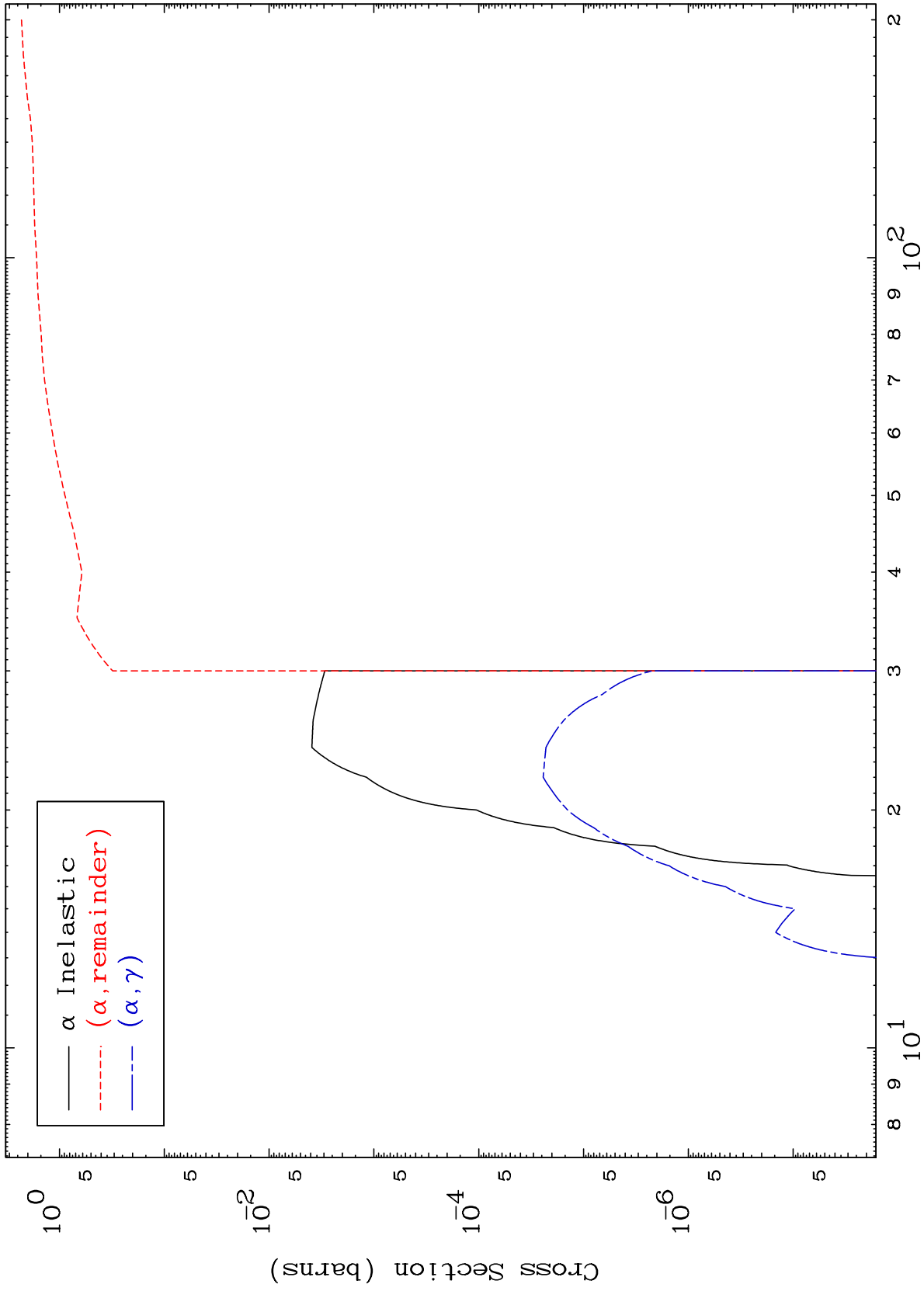
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8268

0 Kelvin  $\alpha$  Major  
Cross Sections

83-Bi-190



1

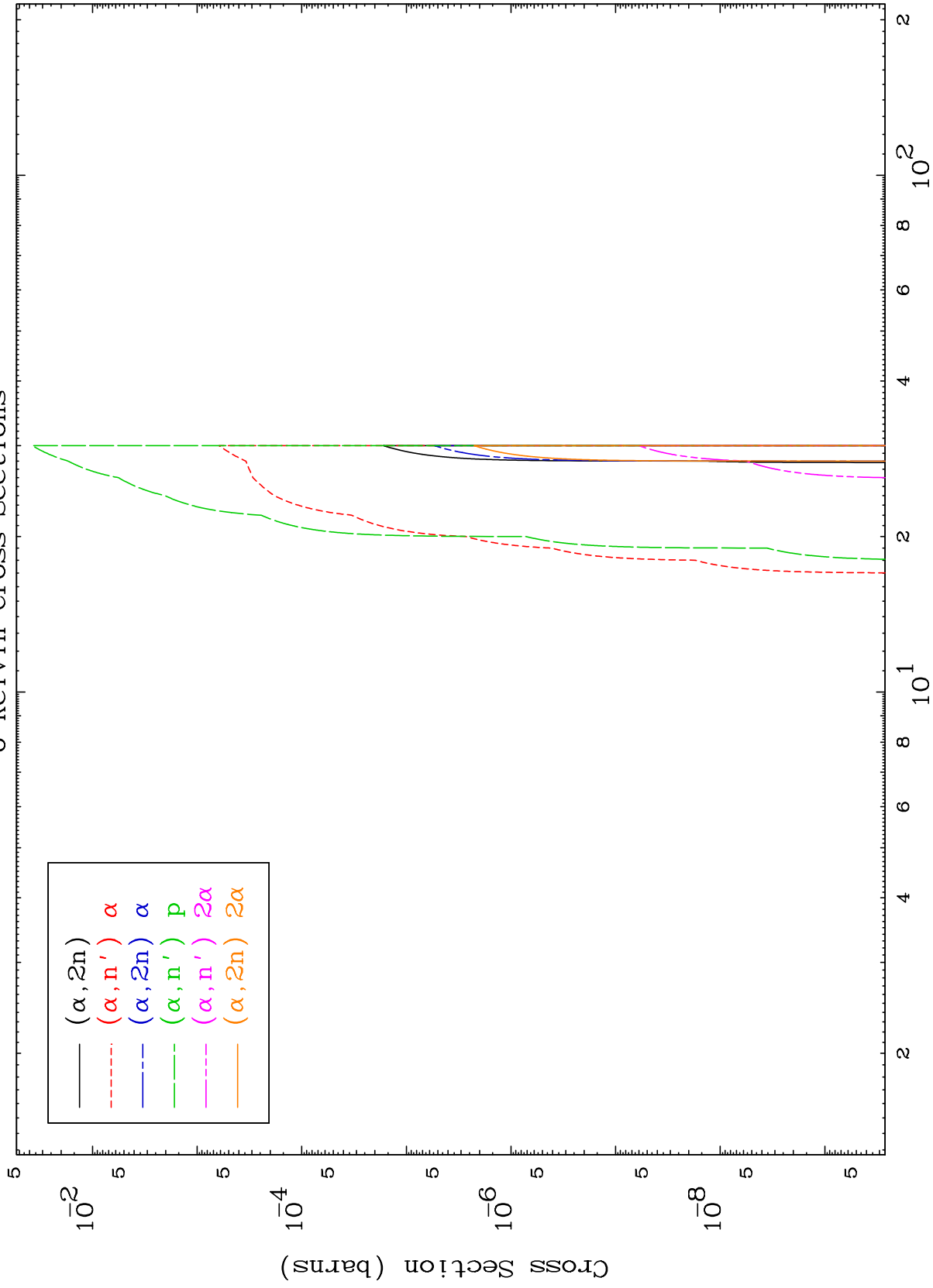
Incident Energy (MeV)

83-Bi-190

MAT 8268

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

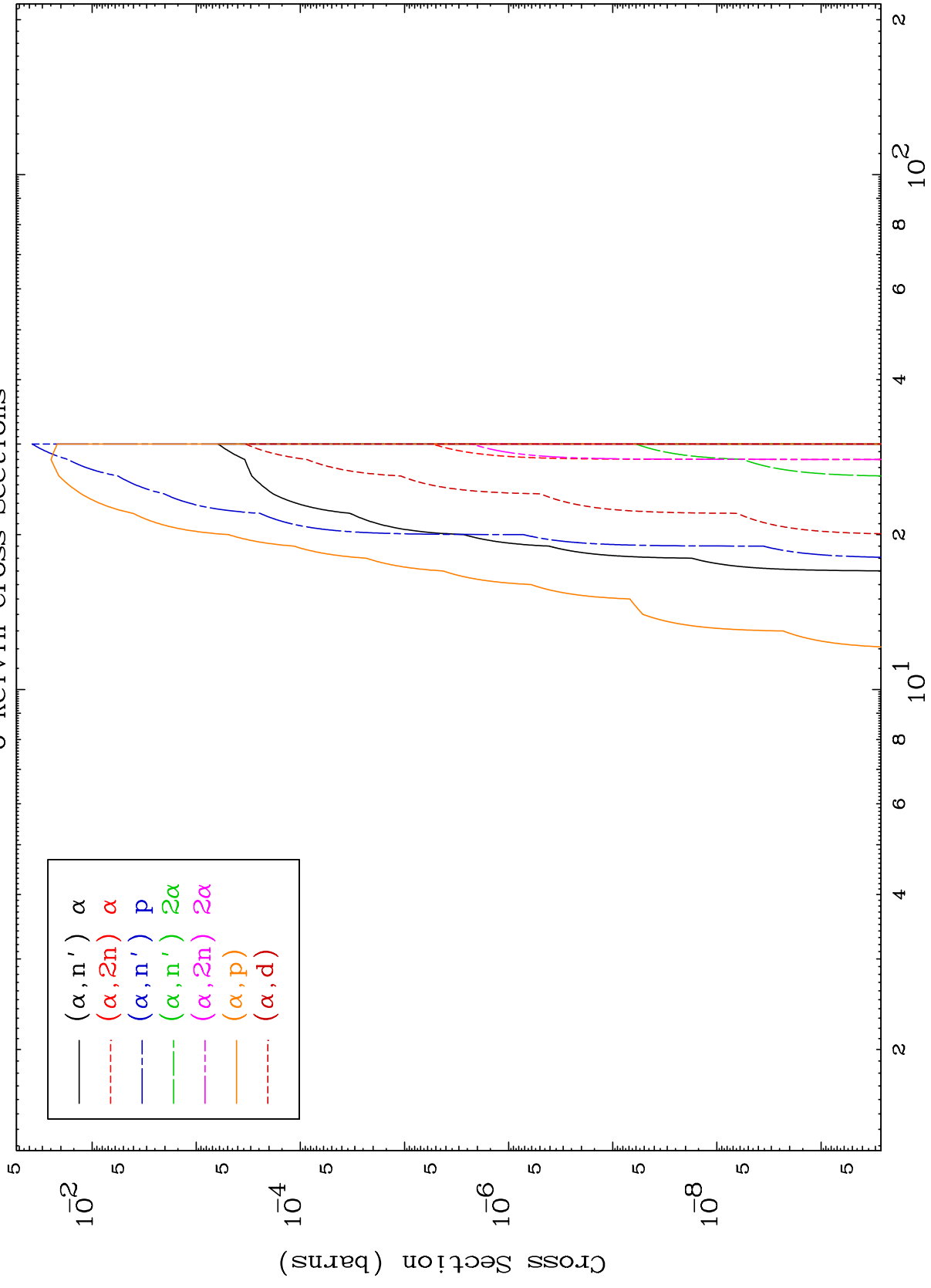
83-Bi-190



2

Incident Energy (MeV)

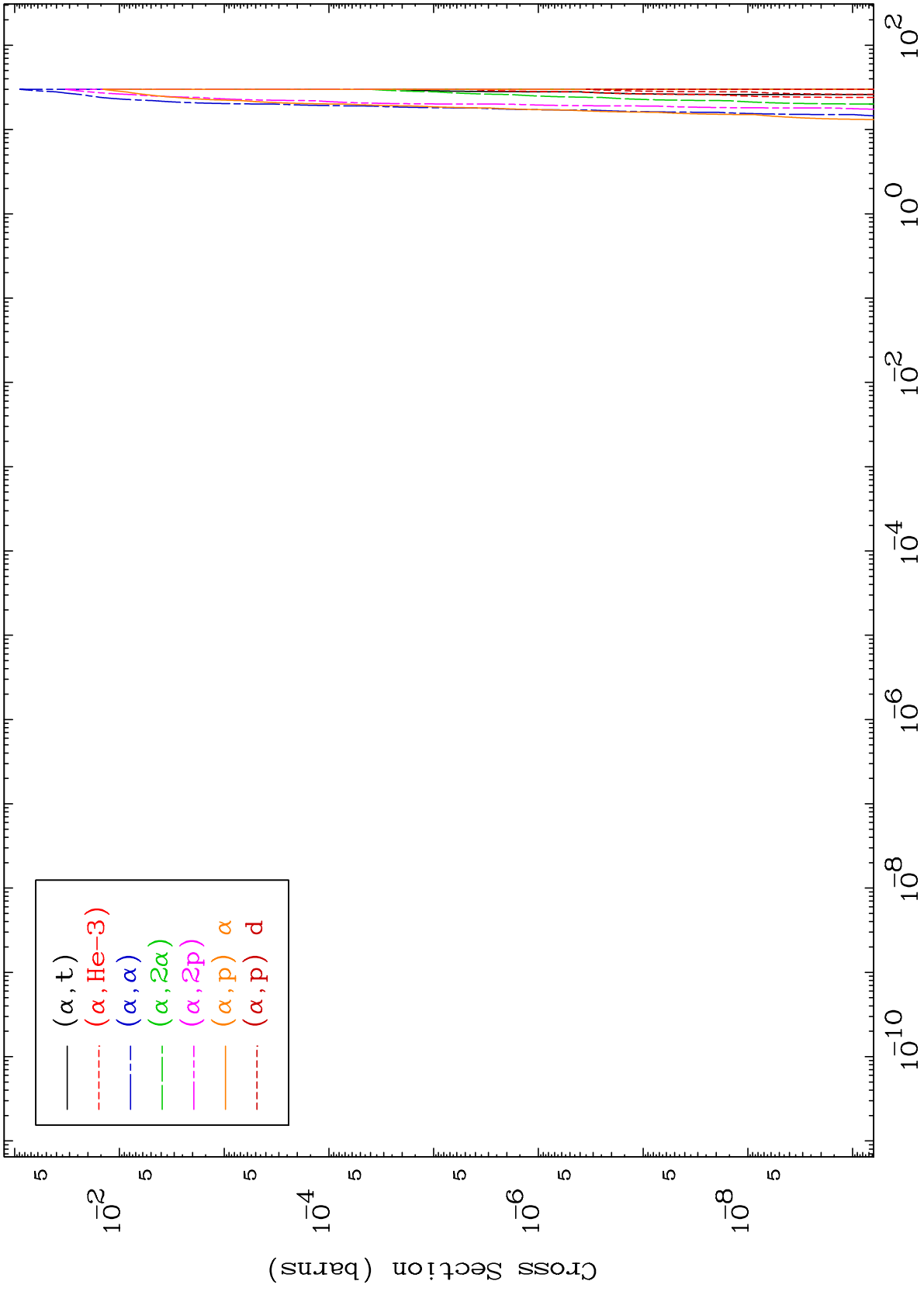
83-Bi-190



MAT 8268

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

83-Bi-190

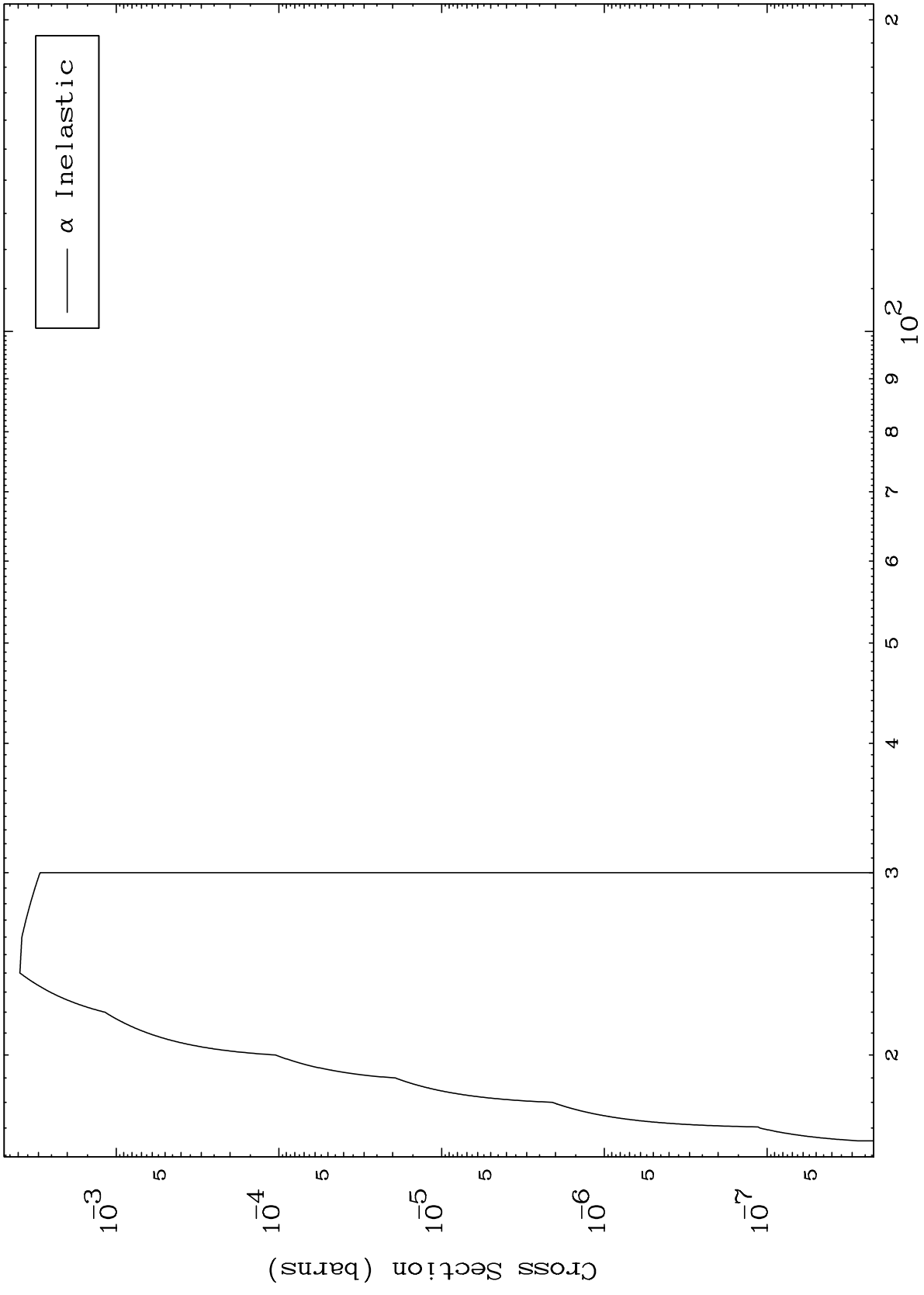


83-Bi-190

MAT 8268

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

83-Bi-190



5

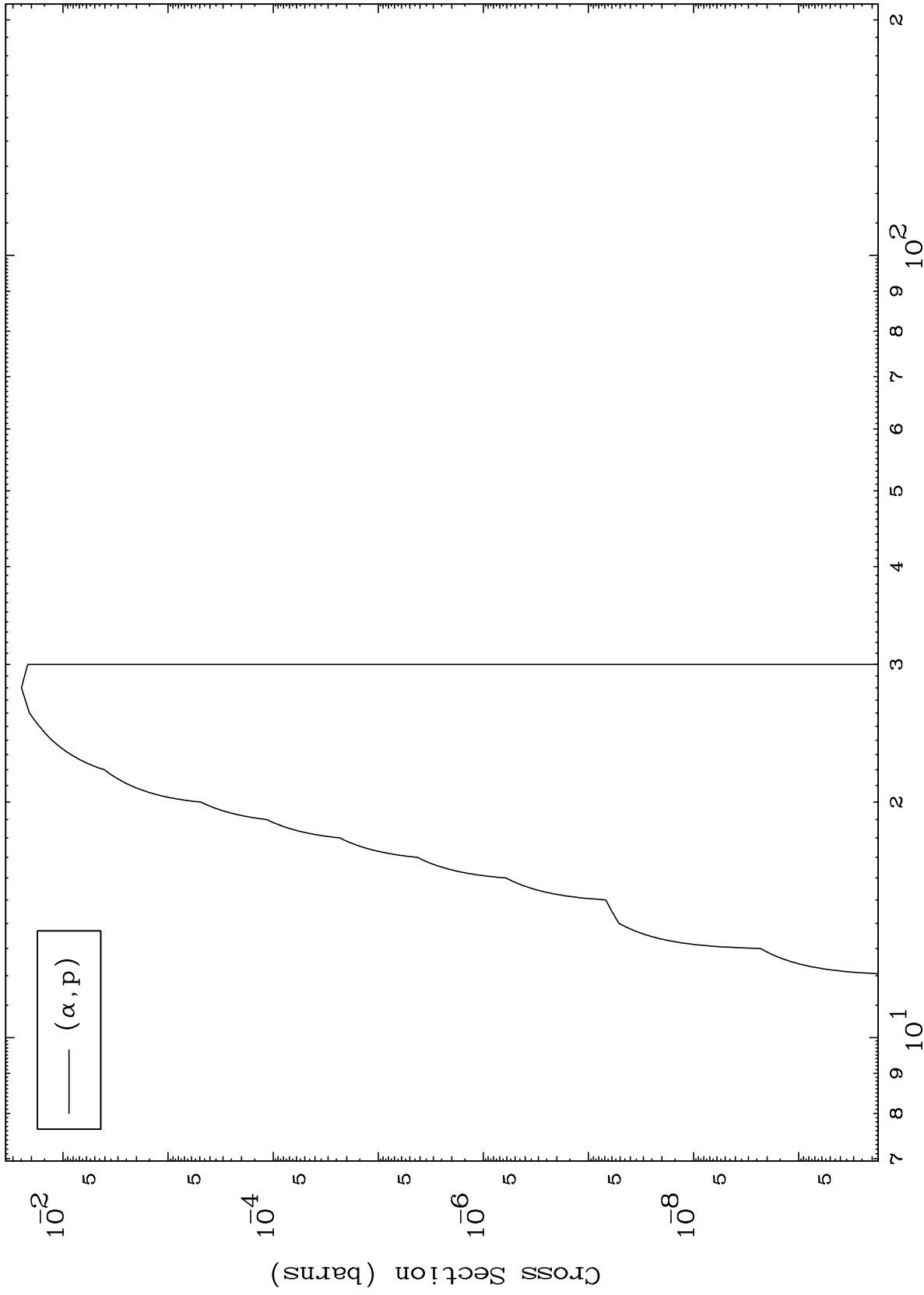
Incident Energy (MeV)

83-Bi-190

MAT 8268

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

83-Bi-190



6

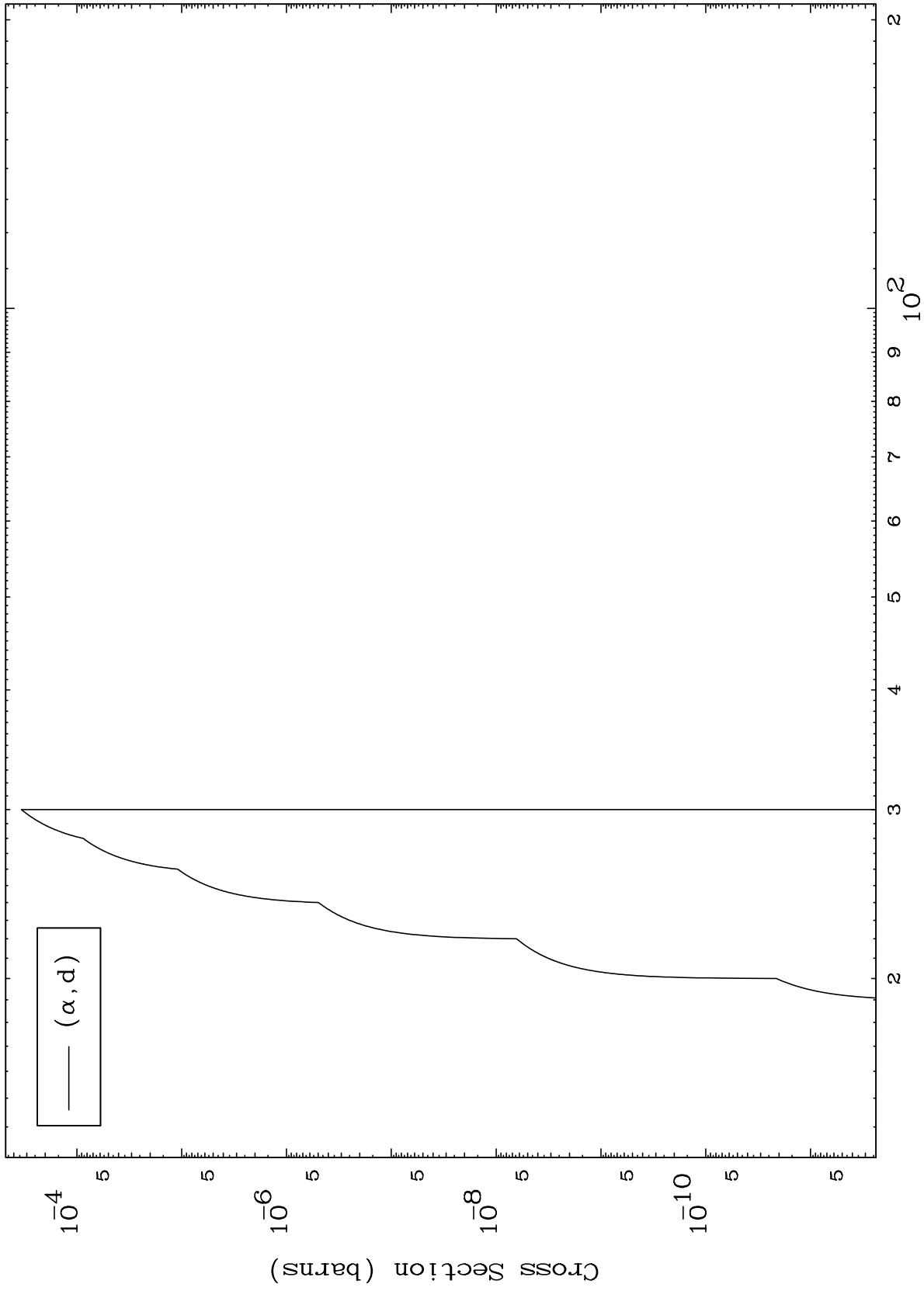
Incident Energy (MeV)

83-Bi-190

MAT 8268

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

83-Bi-190



7

Incident Energy (MeV)

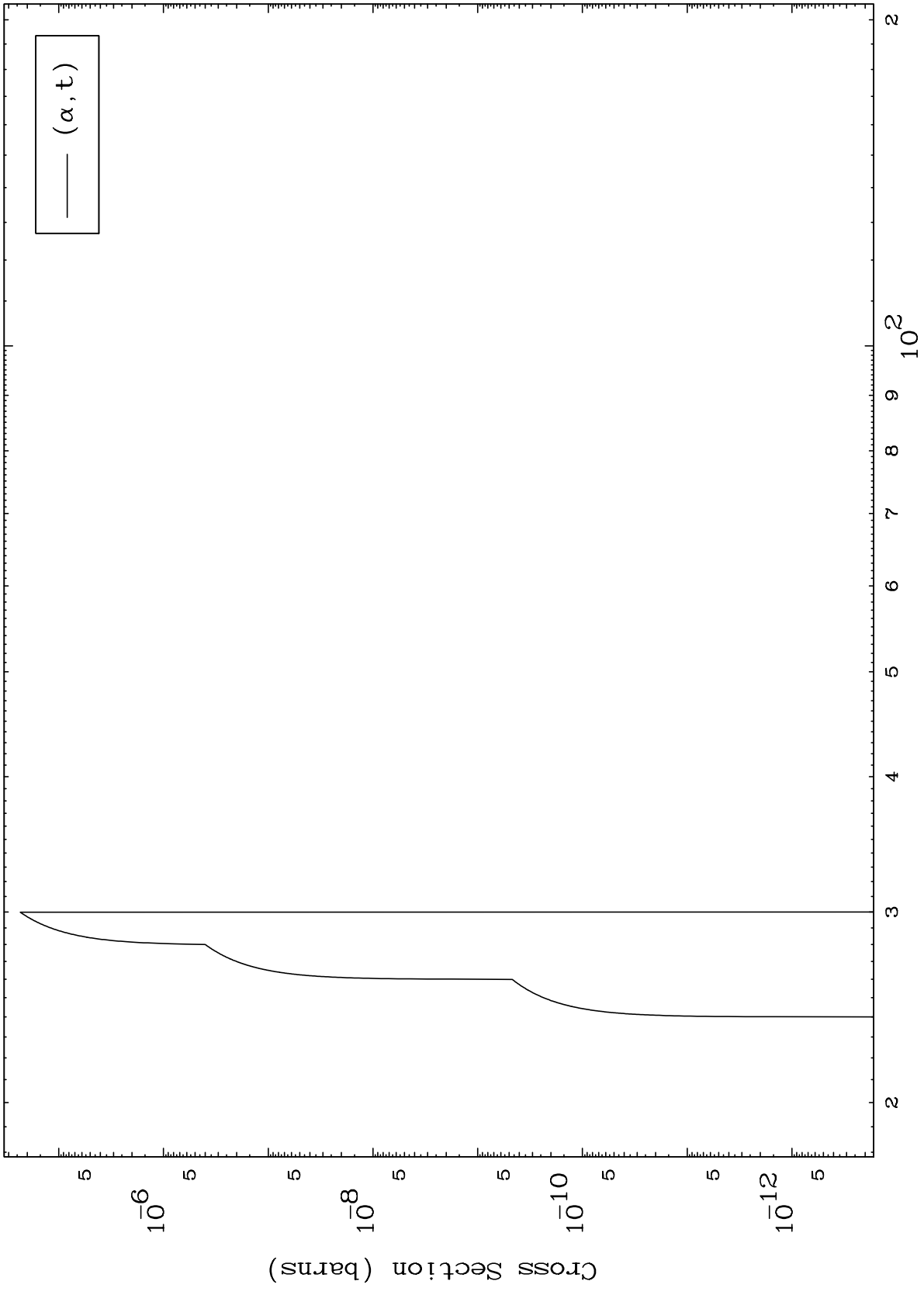
83-Bi-190



MAT 8268

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

83-Bi-190



8

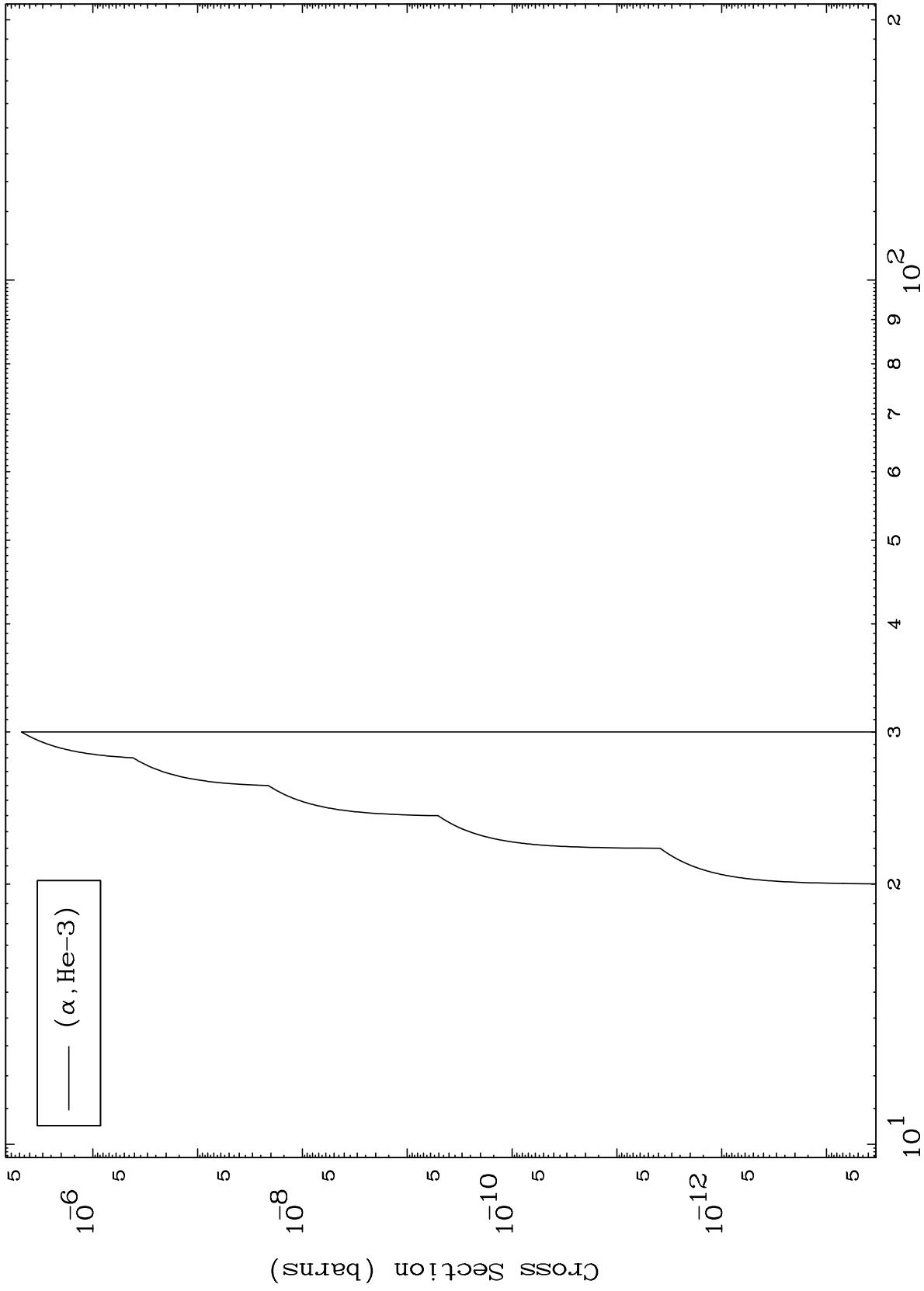
Incident Energy (MeV)

83-Bi-190

MAT 8268

( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections

83-Bi-190



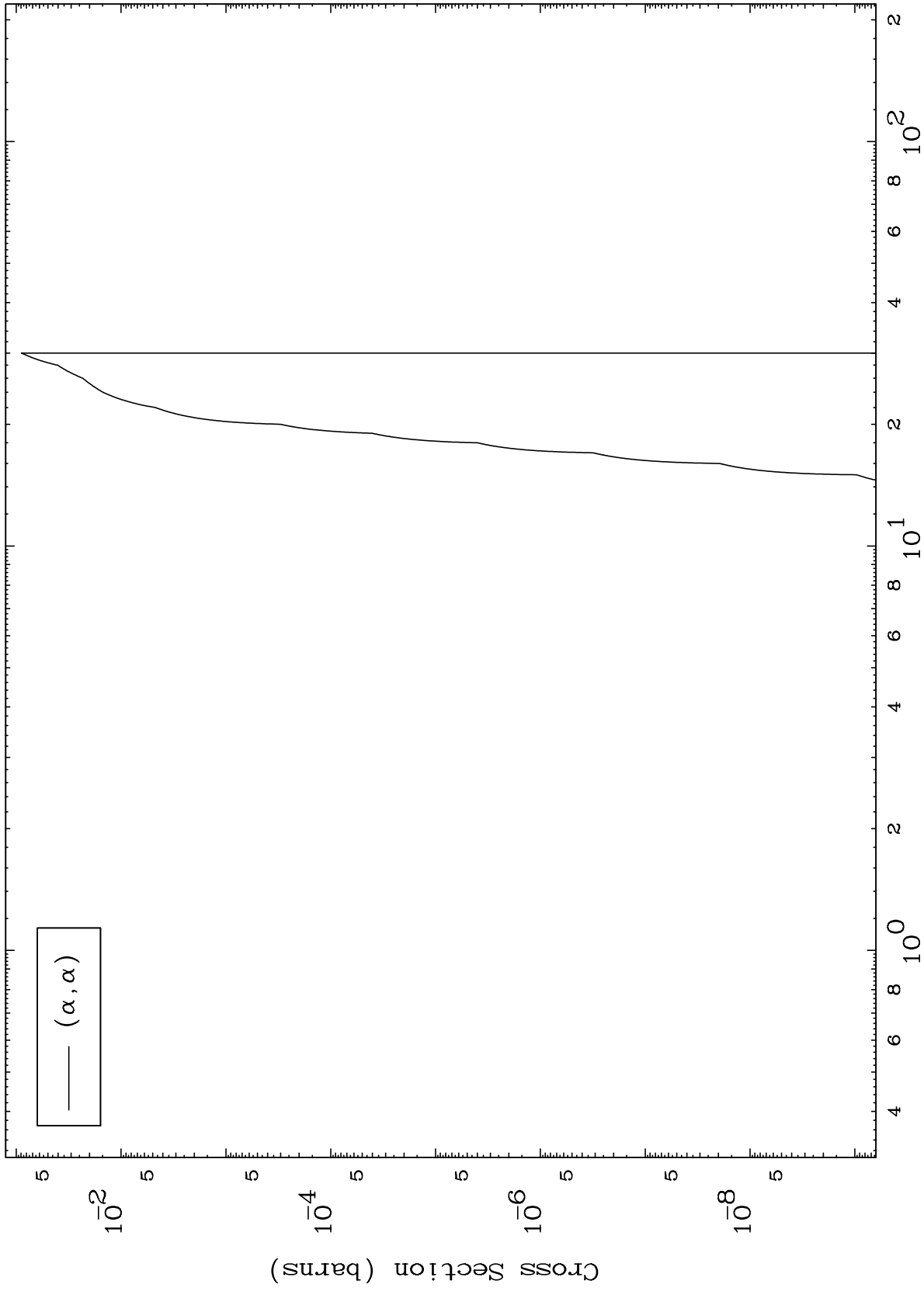
Incident Energy (MeV)

83-Bi-190

MAT 8268

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

83-Bi-190



10

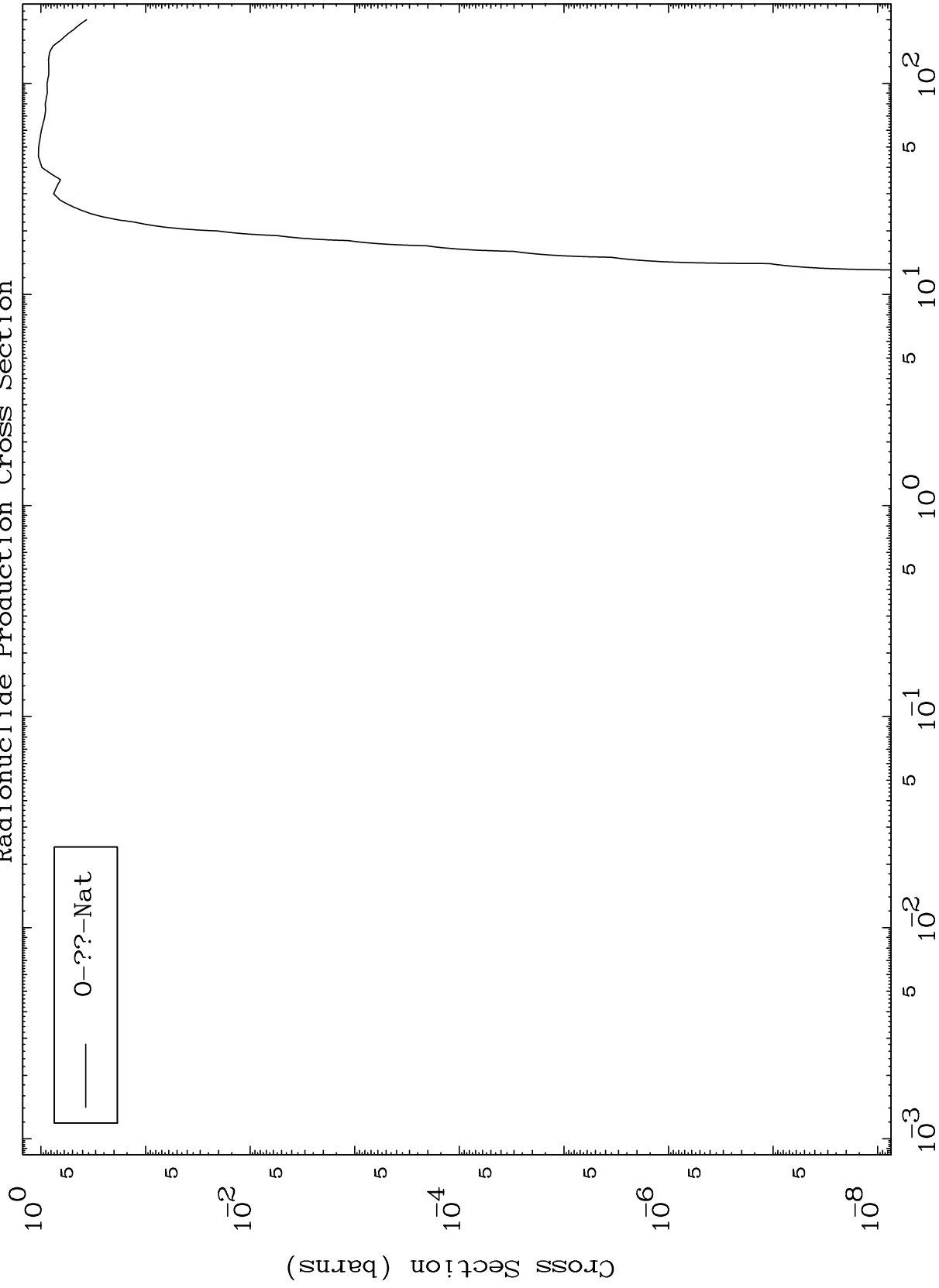
Incident Energy (MeV)

83-Bi-190

MAT 8268

83-Bi-190

$\alpha$  Fission  
Radionuclide Production Cross Section



11

Incident Energy (MeV)

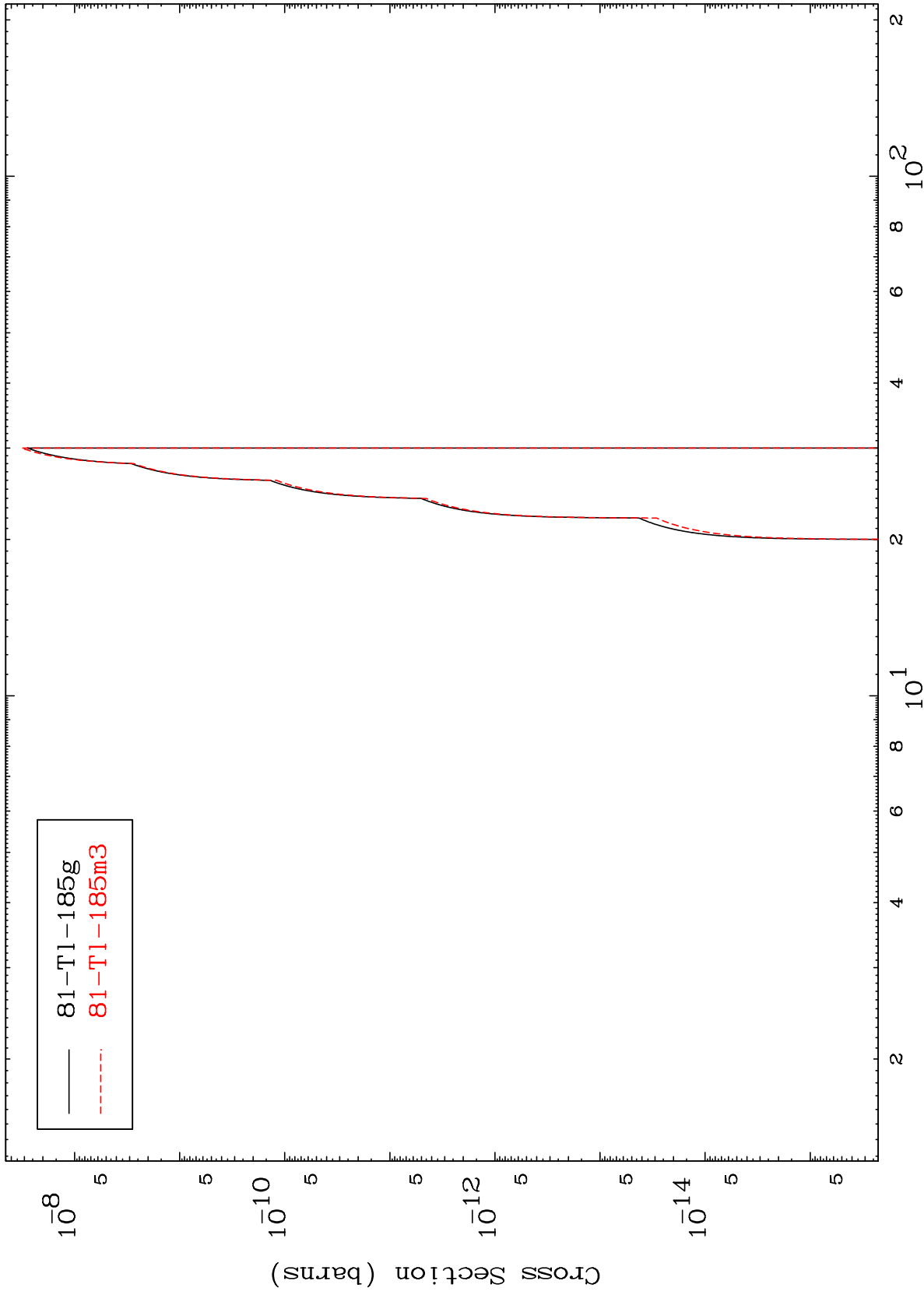
83-Bi-190

MAT 8268

( $\alpha, n'$ )  $2\alpha$

83-Bi-190

Radionuclide Production Cross Section



12

Incident Energy (MeV)

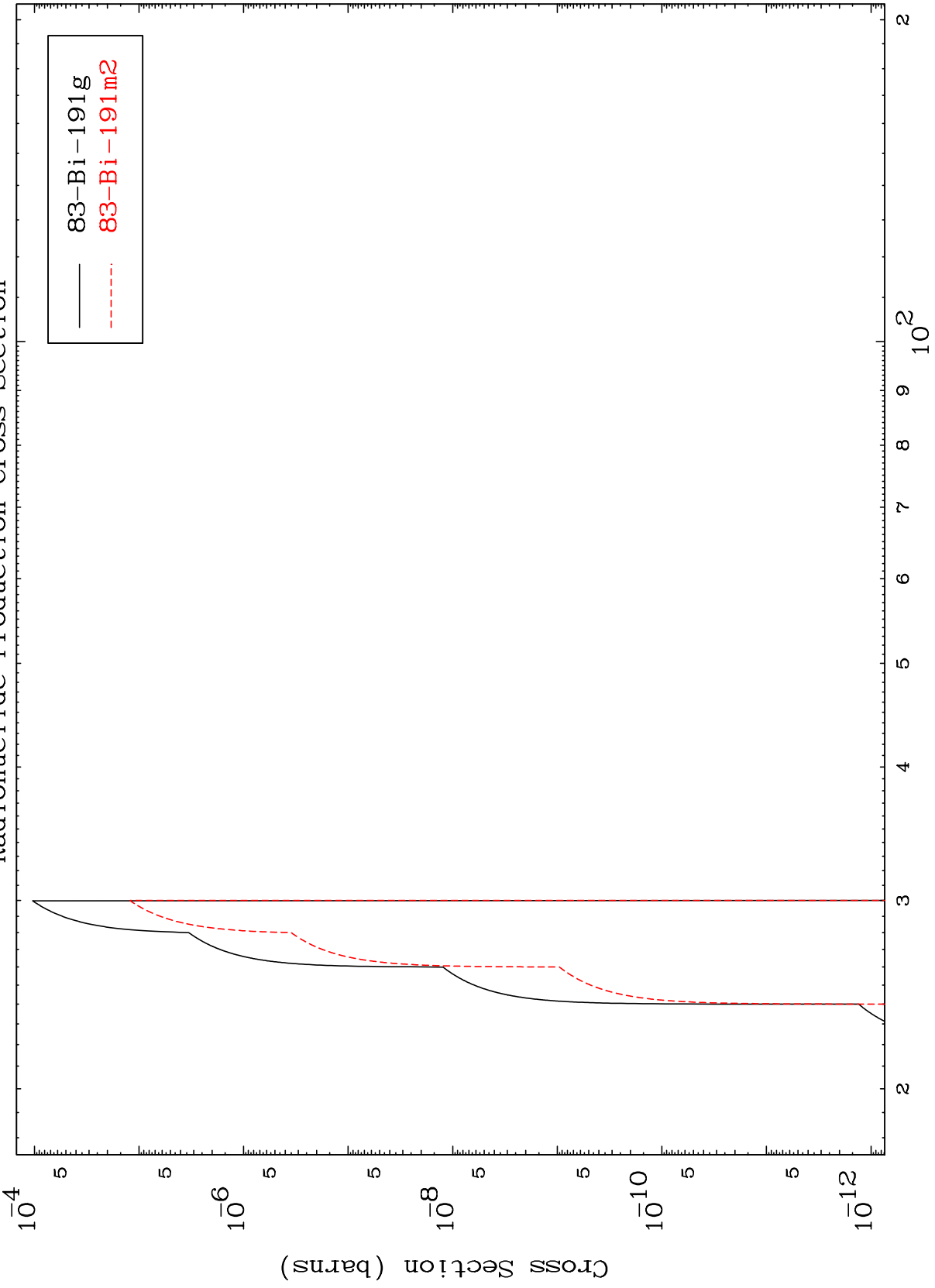
83-Bi-190

MAT 8268

( $\alpha, 2n$ ) p

83-Bi-190

Radionuclide Production Cross Section



13

Incident Energy (MeV)

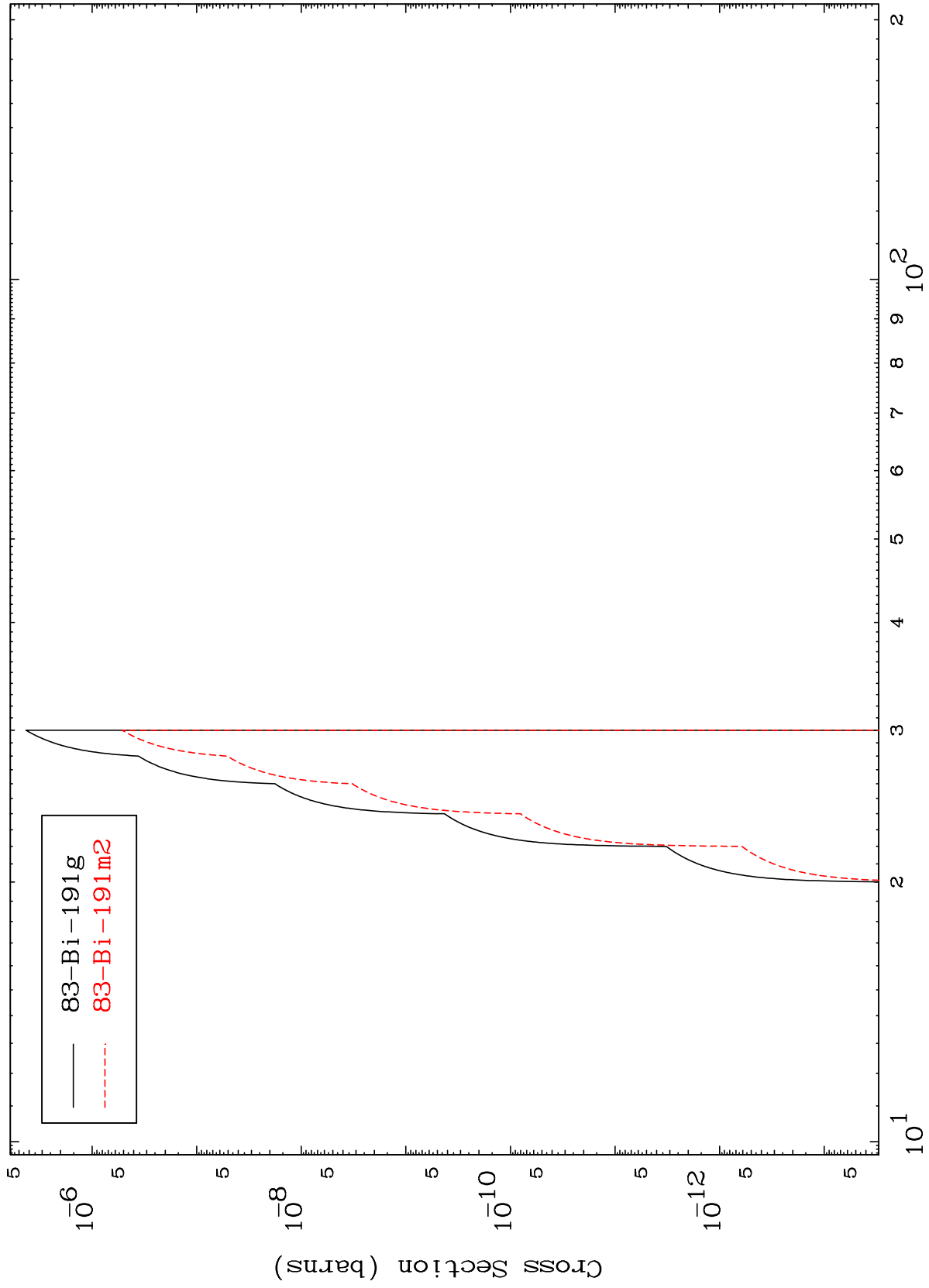
83-Bi-190

MAT 8268

( $\alpha, \text{He-3}$ )

83-Bi-190

Radionuclide Production Cross Section



14

Incident Energy (MeV)

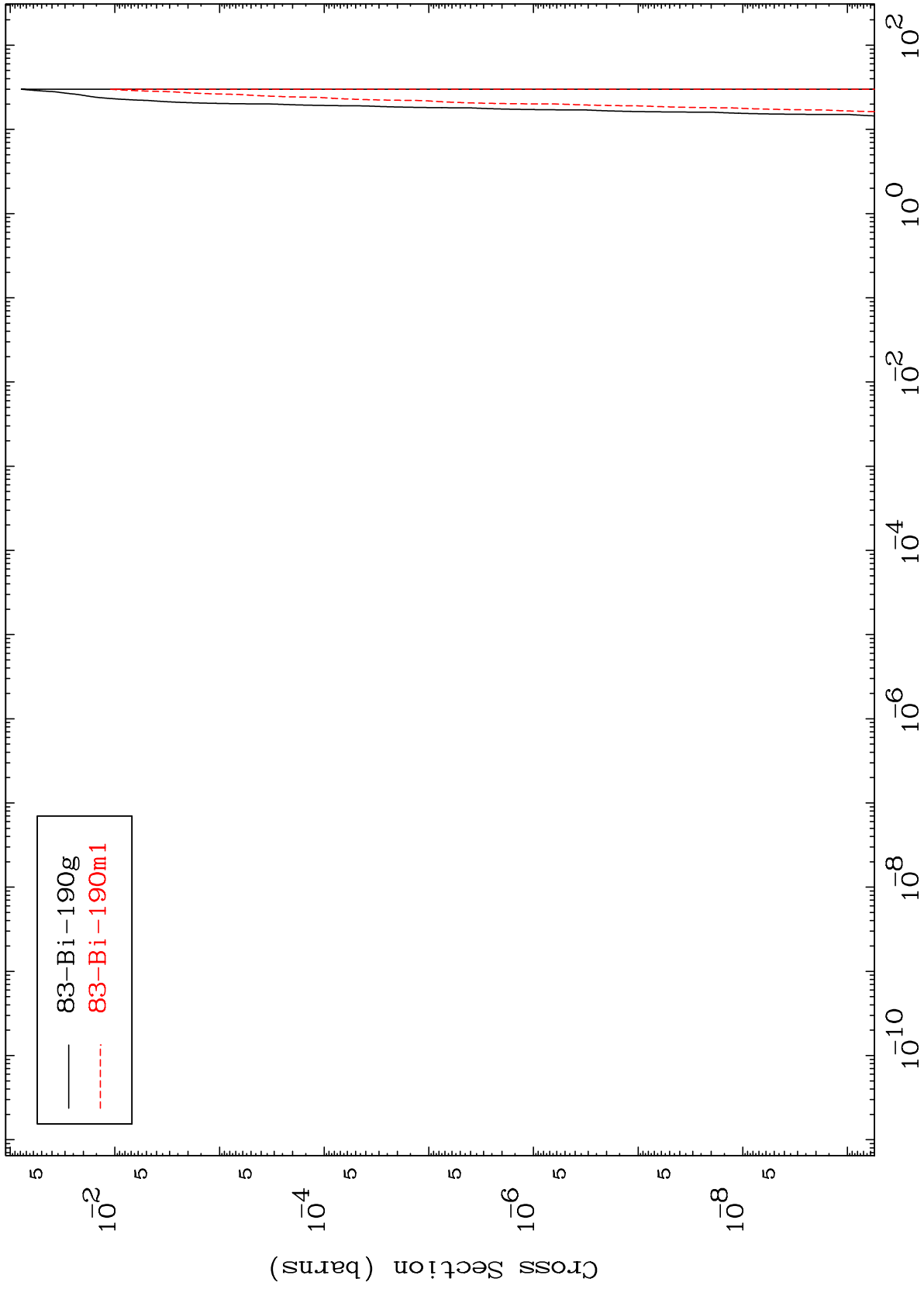
83-Bi-190

MAT 8268

( $\alpha, \alpha$ )

83-Bi-190

Radionuclide Production Cross Section



15

Incident Energy (MeV)

83-Bi-190

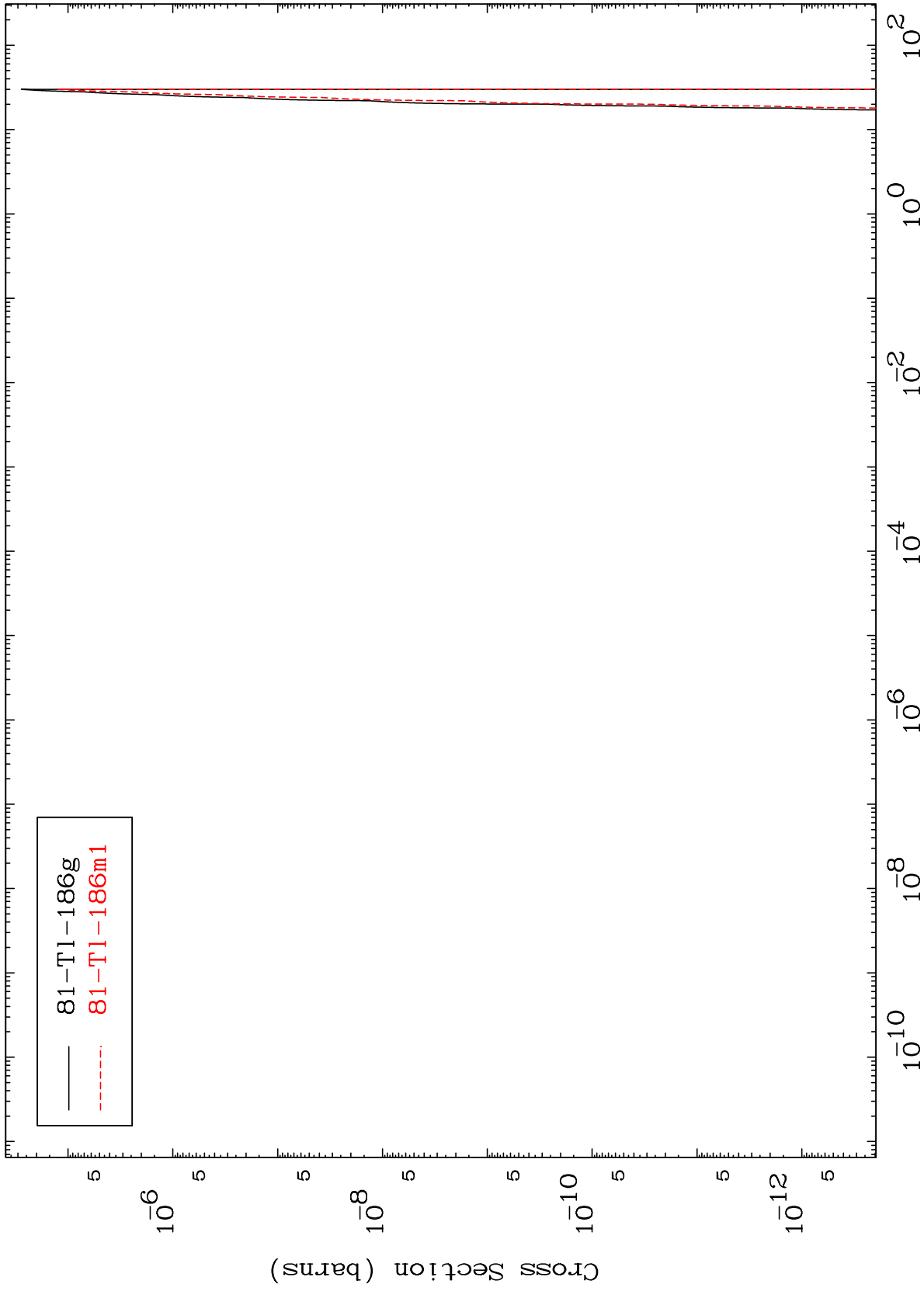


MAT 8268

( $\alpha, 2\alpha$ )

83-Bi-190

Radionuclide Production Cross Section



16

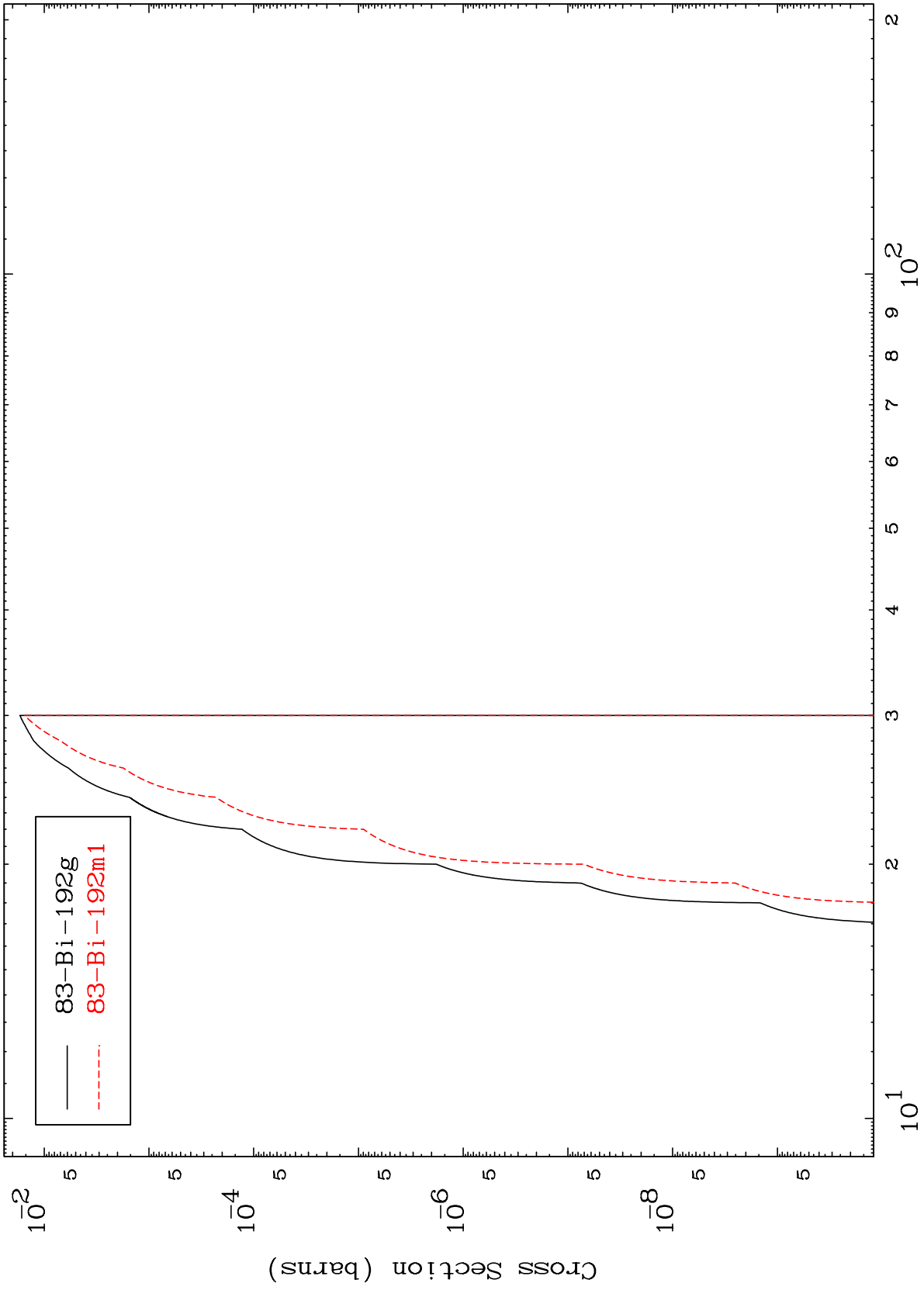
Incident Energy (MeV)

83-Bi-190

MAT 8268

83-Bi-190

Radionuclide Production Cross Section  
( $\alpha, 2p$ )



17

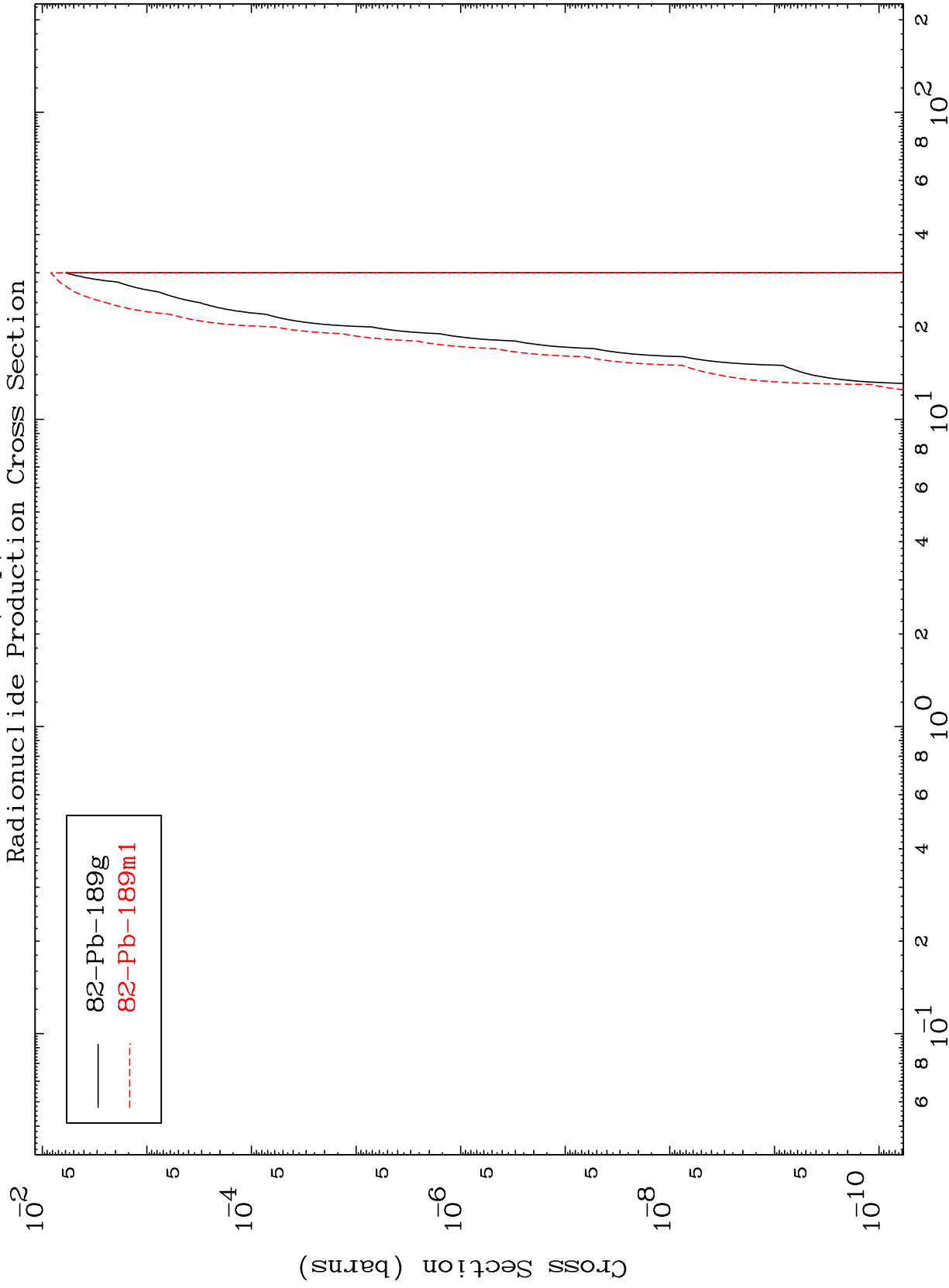
Incident Energy (MeV)

83-Bi-190

MAT 8268

( $\alpha, p$ )  $\alpha$

83-Bi-190



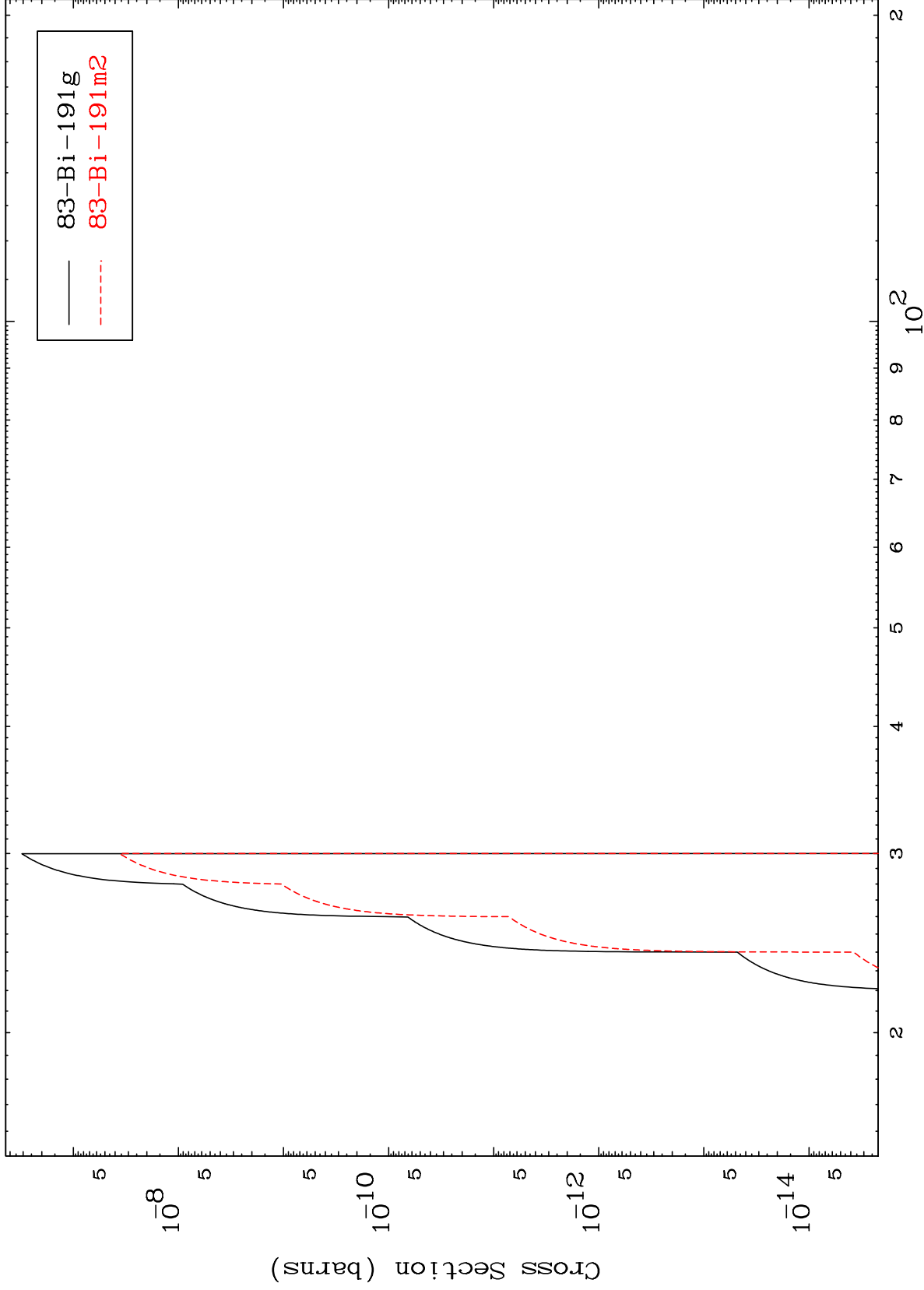
82-Pb-189g  
82-Pb-189m1

MAT 8268

( $\alpha, p$ ) d

$^{83}\text{Bi}-190$

Radionuclide Production Cross Section



19

Incident Energy (MeV)

$^{83}\text{Bi}-190$