

Program EVALPLOT  
(Version 2018-1)

by

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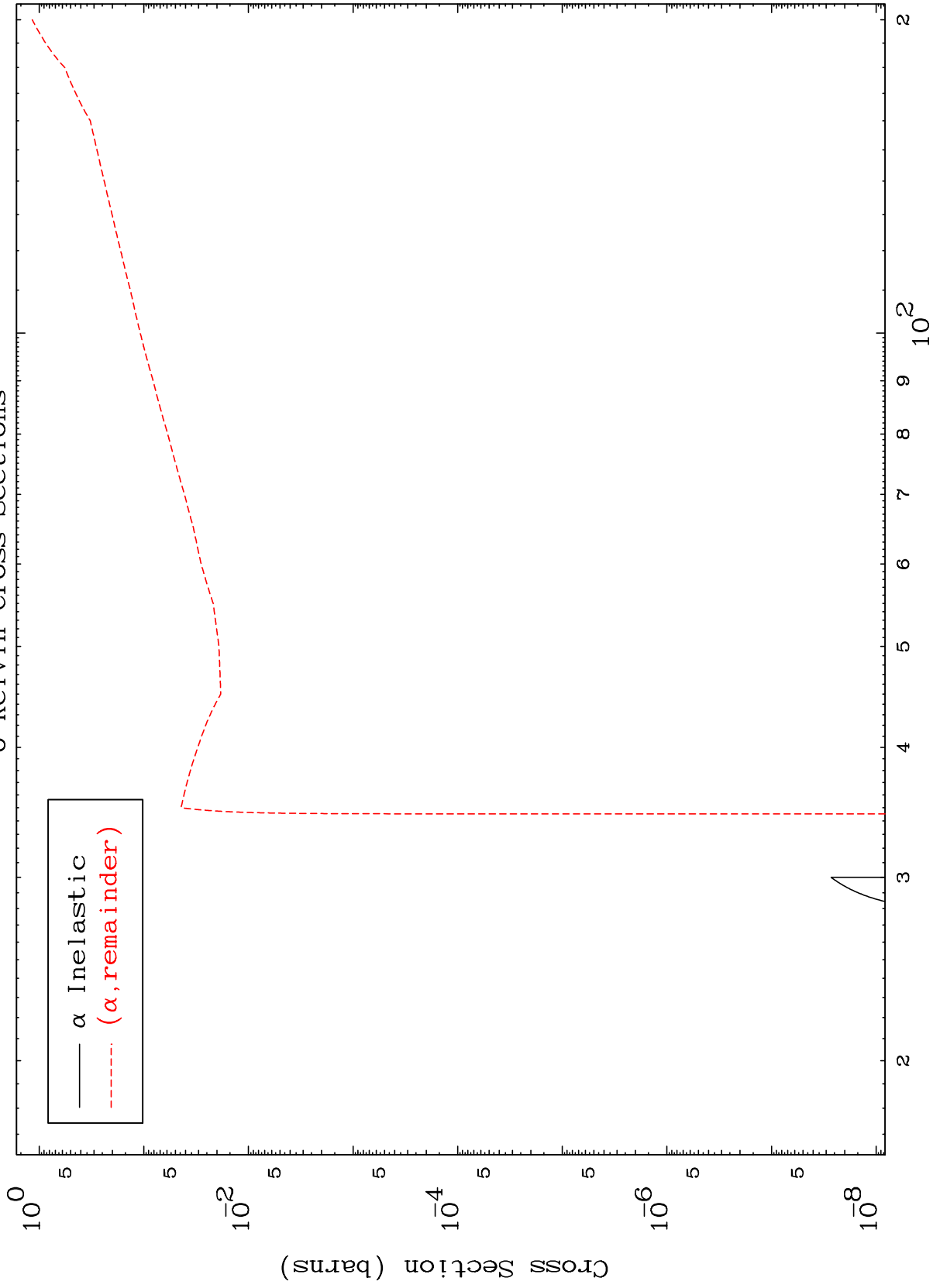
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9707

0 Kelvin  $\alpha$  Major  
Cross Sections

97-Bk-234



1

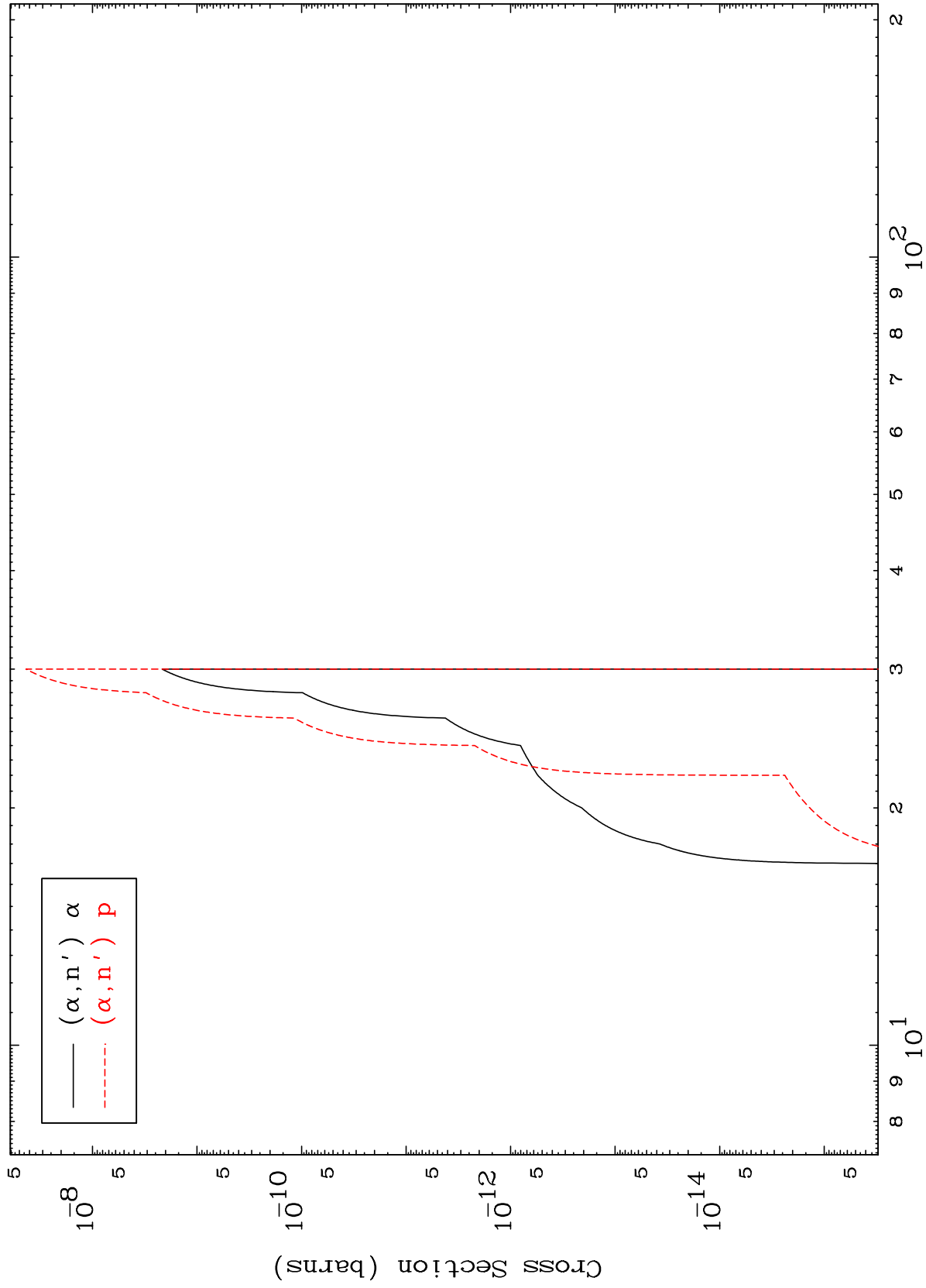
Incident Energy (MeV)

97-Bk-234

MAT 9707

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

97-Bk-234



2

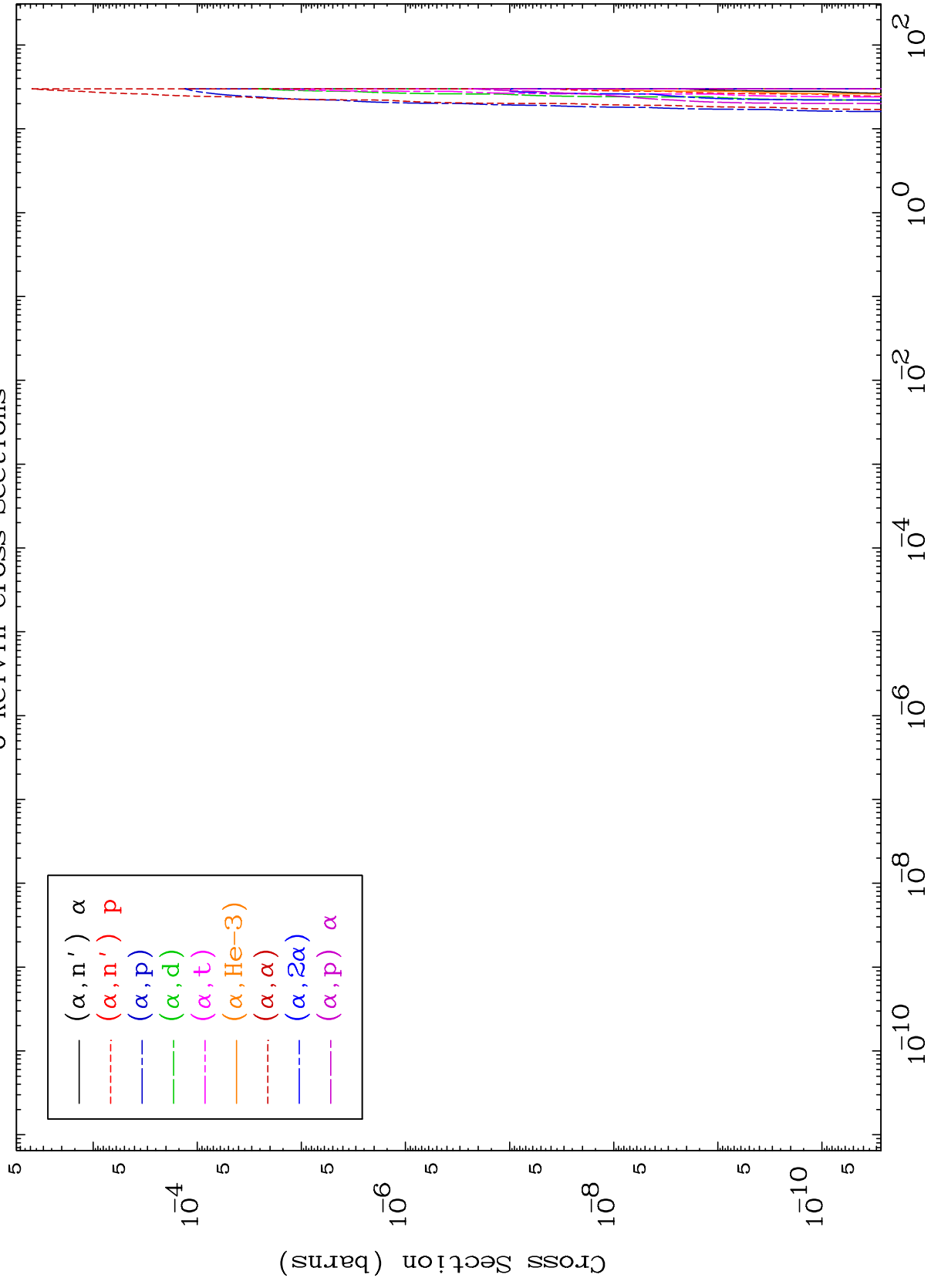
Incident Energy (MeV)

97-Bk-234

MAT 9707

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

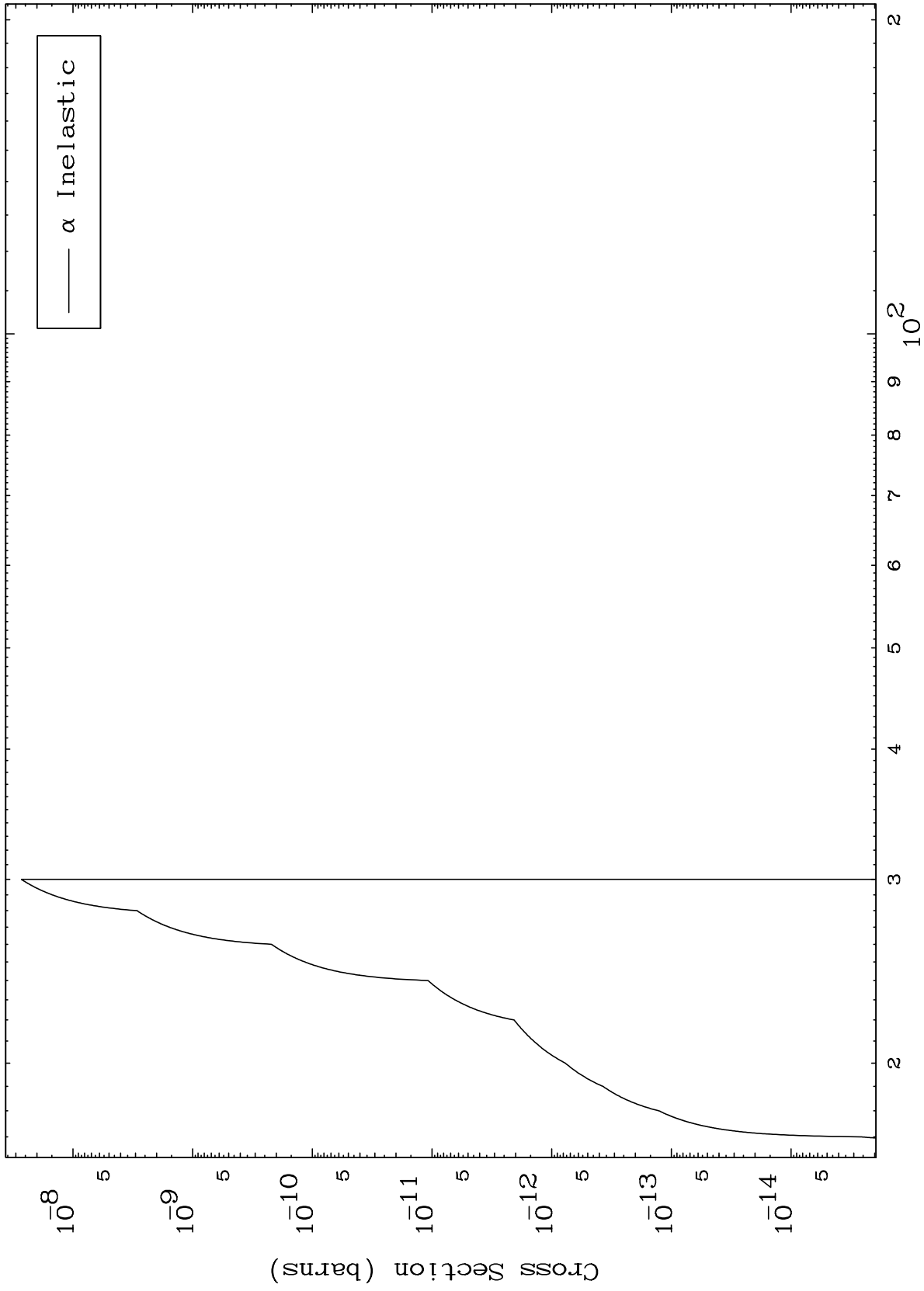
97-Bk-234



97-Bk-234

Incident Energy (MeV)

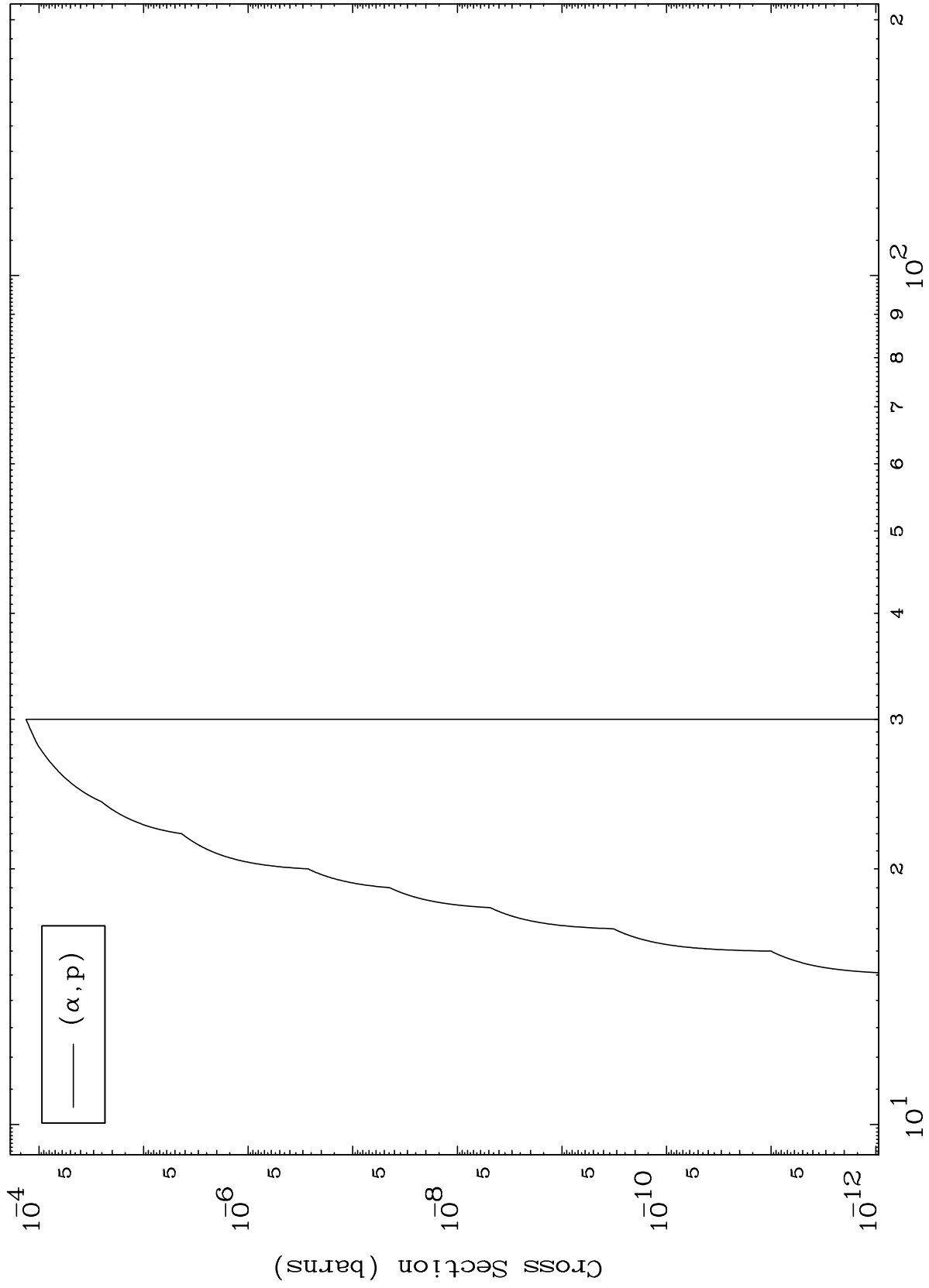
3



MAT 9707

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

97-Bk-234



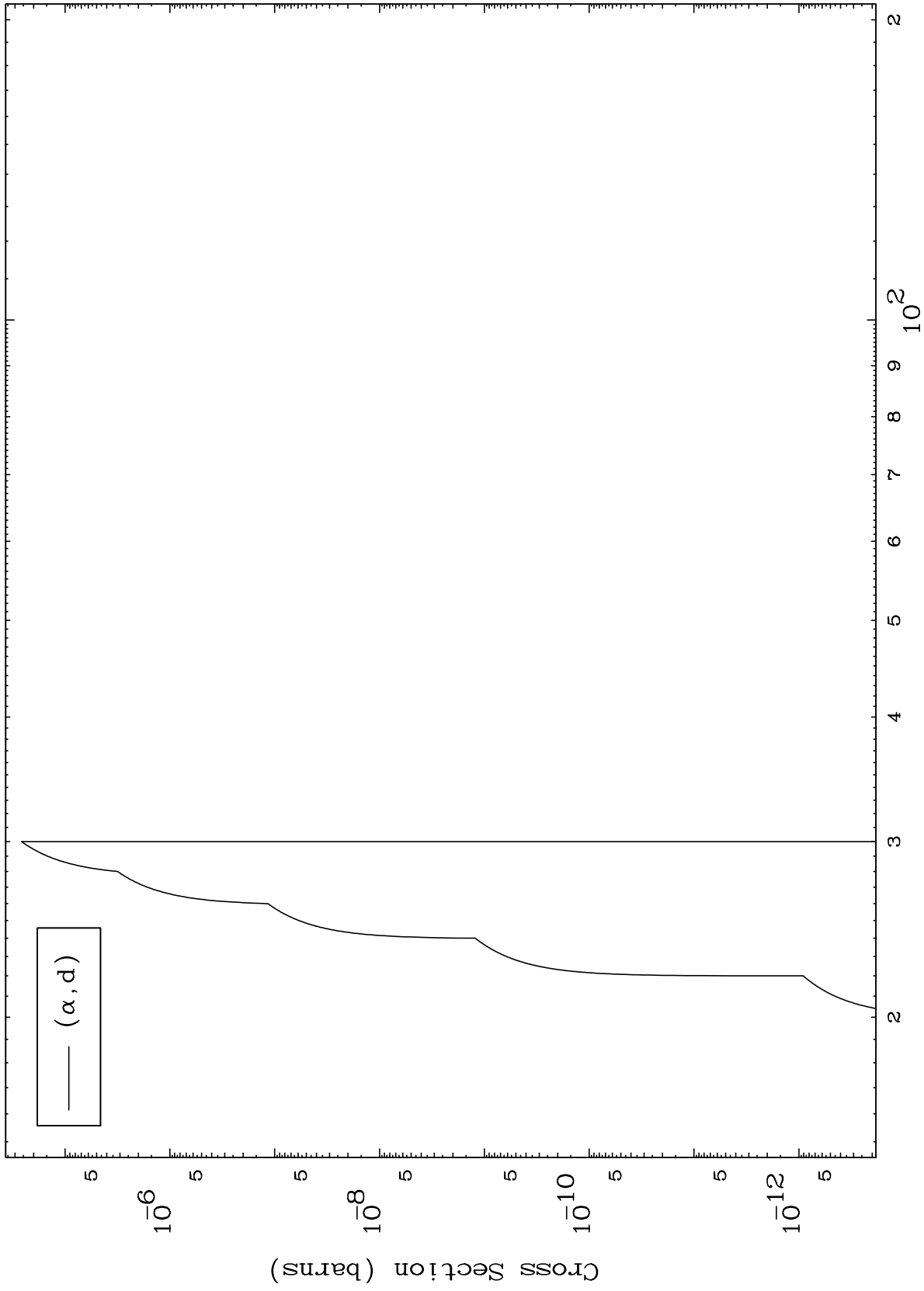
Incident Energy (MeV)

97-Bk-234

MAT 9707

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

97-Bk-234



6

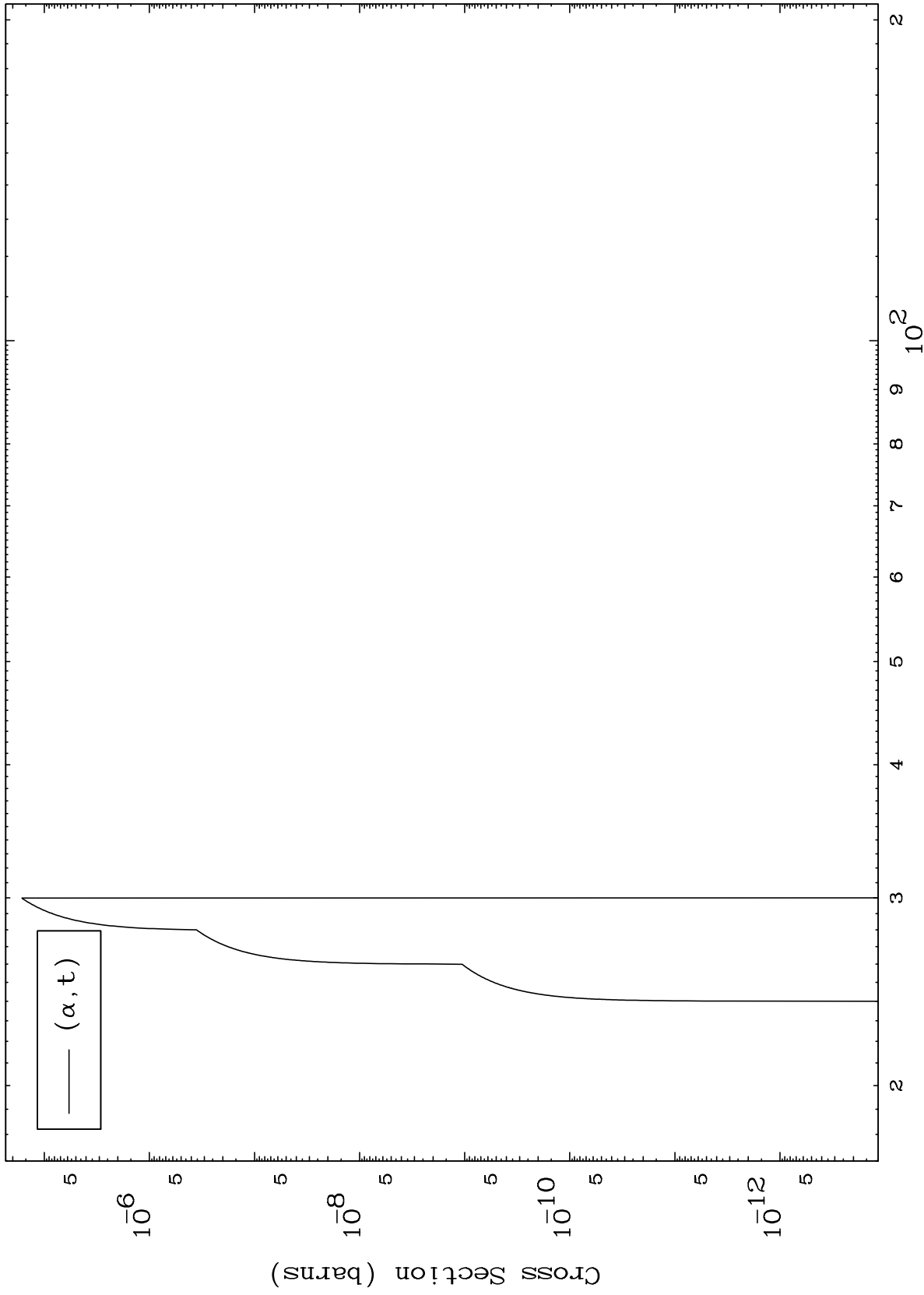
Incident Energy (MeV)

97-Bk-234

MAT 9707

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

97-Bk-234



7

Incident Energy (MeV)

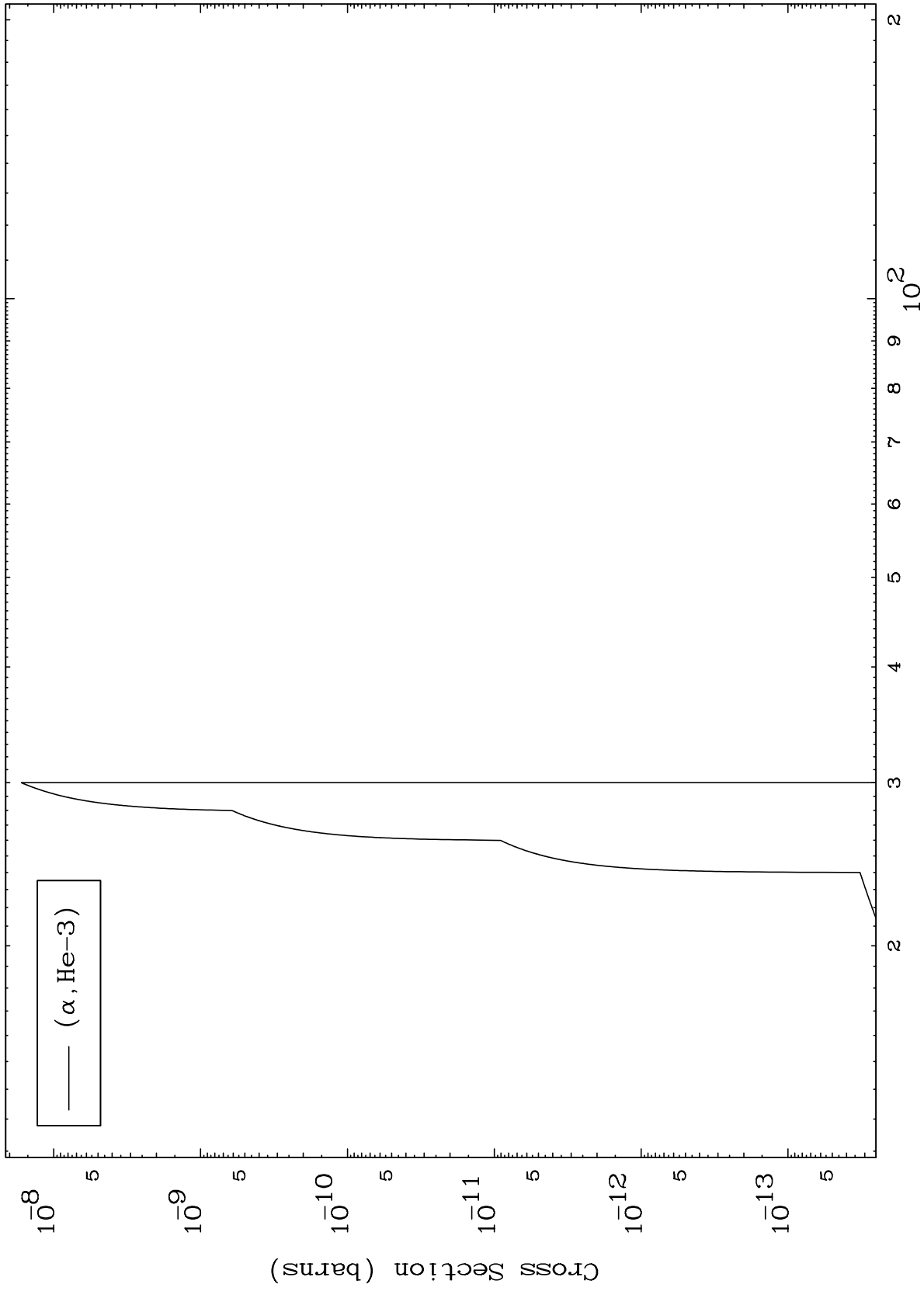
97-Bk-234



MAT 9707

( $\alpha$ , He3) Levels  
0 Kelvin Cross Sections

97-Bk-234



8

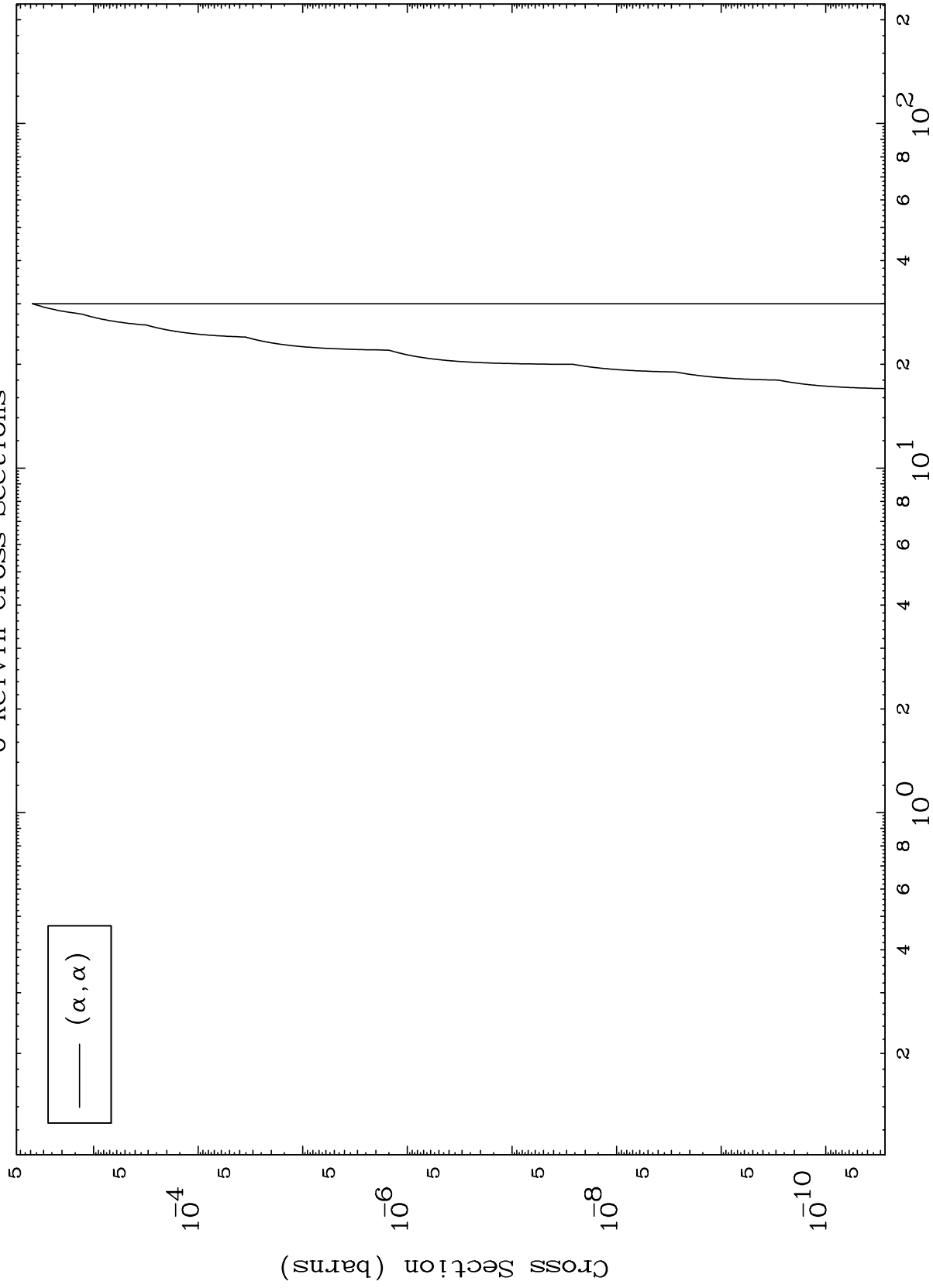
Incident Energy (MeV)

97-Bk-234

MAT 9707

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

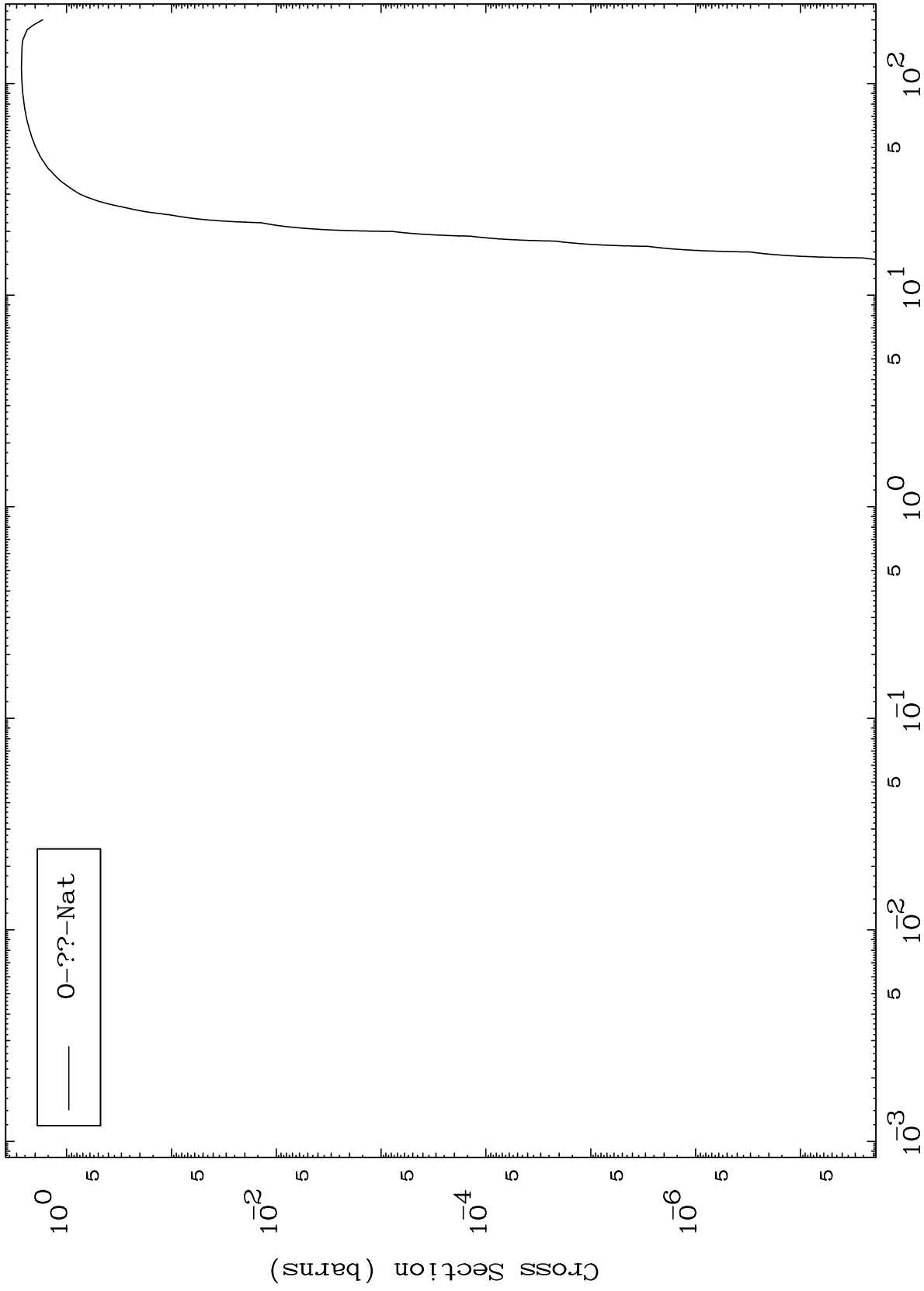
97-Bk-234



MAT 9707

$\alpha$  Fission  
Radionuclide Production Cross Section

97-Bk-234



Incident Energy (MeV)

97-Bk-234