

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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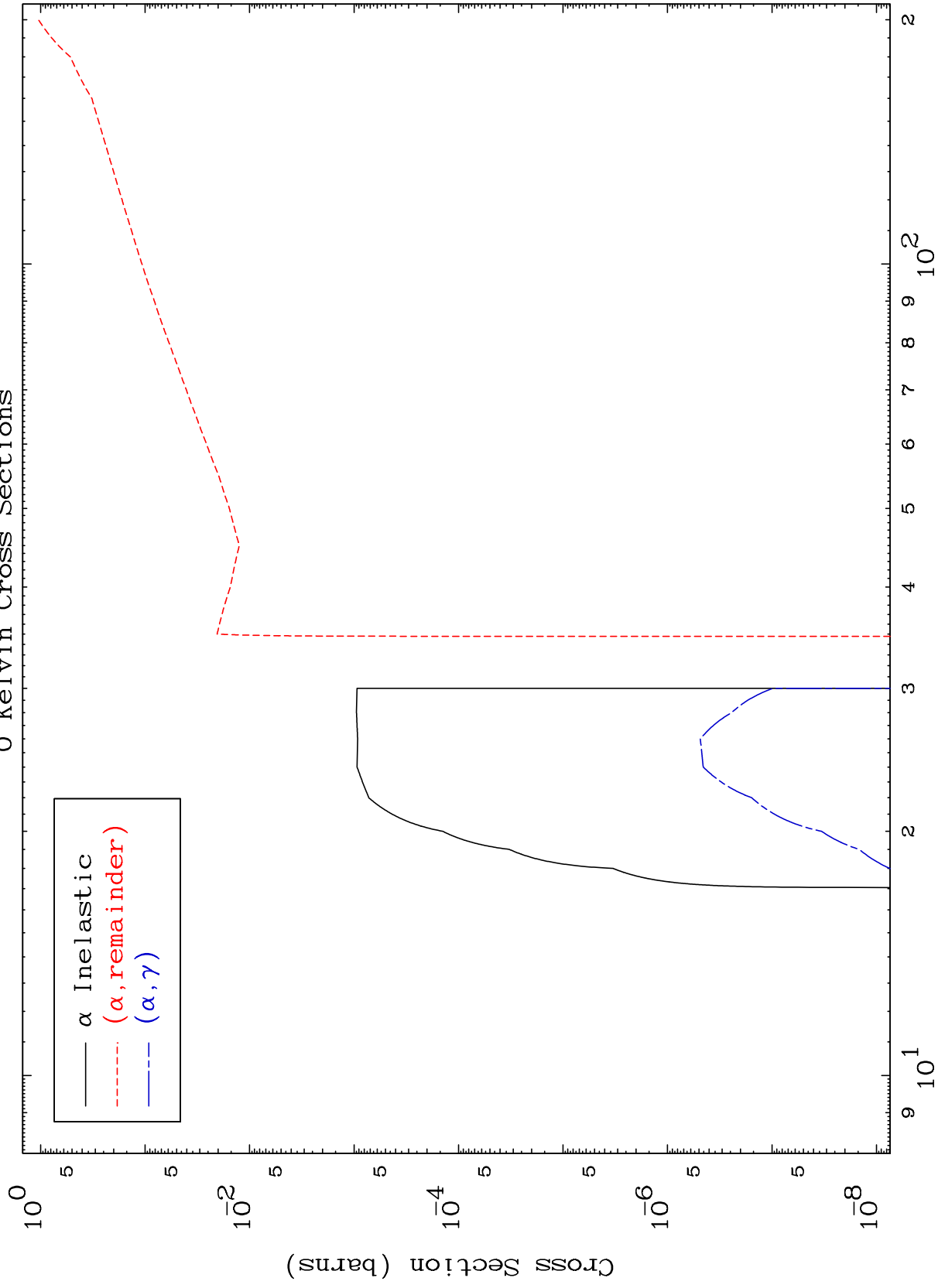
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9716

0 Kelvin  $\alpha$  Major  
Cross Sections

97-Bk-237



1

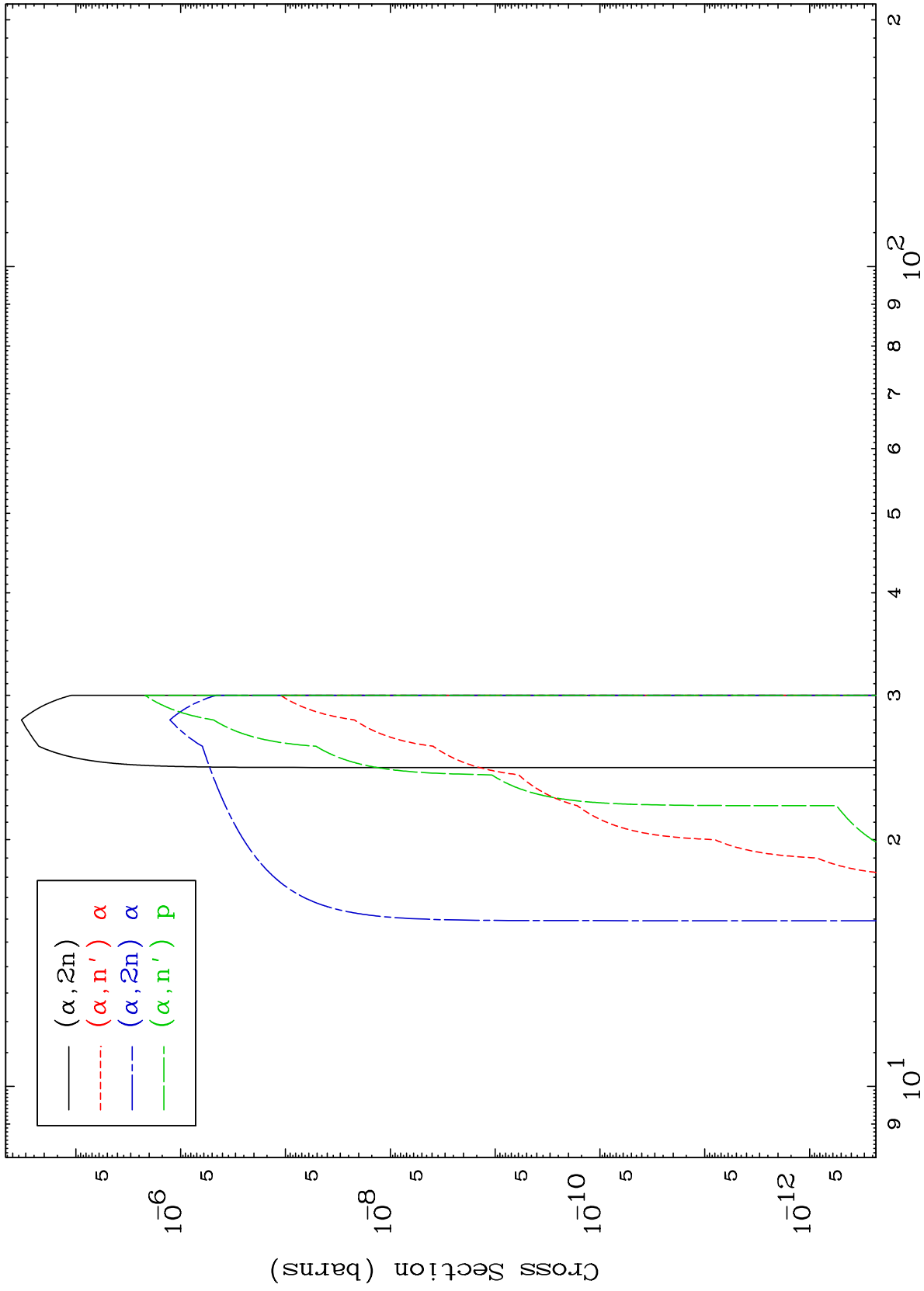
Incident Energy (MeV)

97-Bk-237

MAT 9716

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

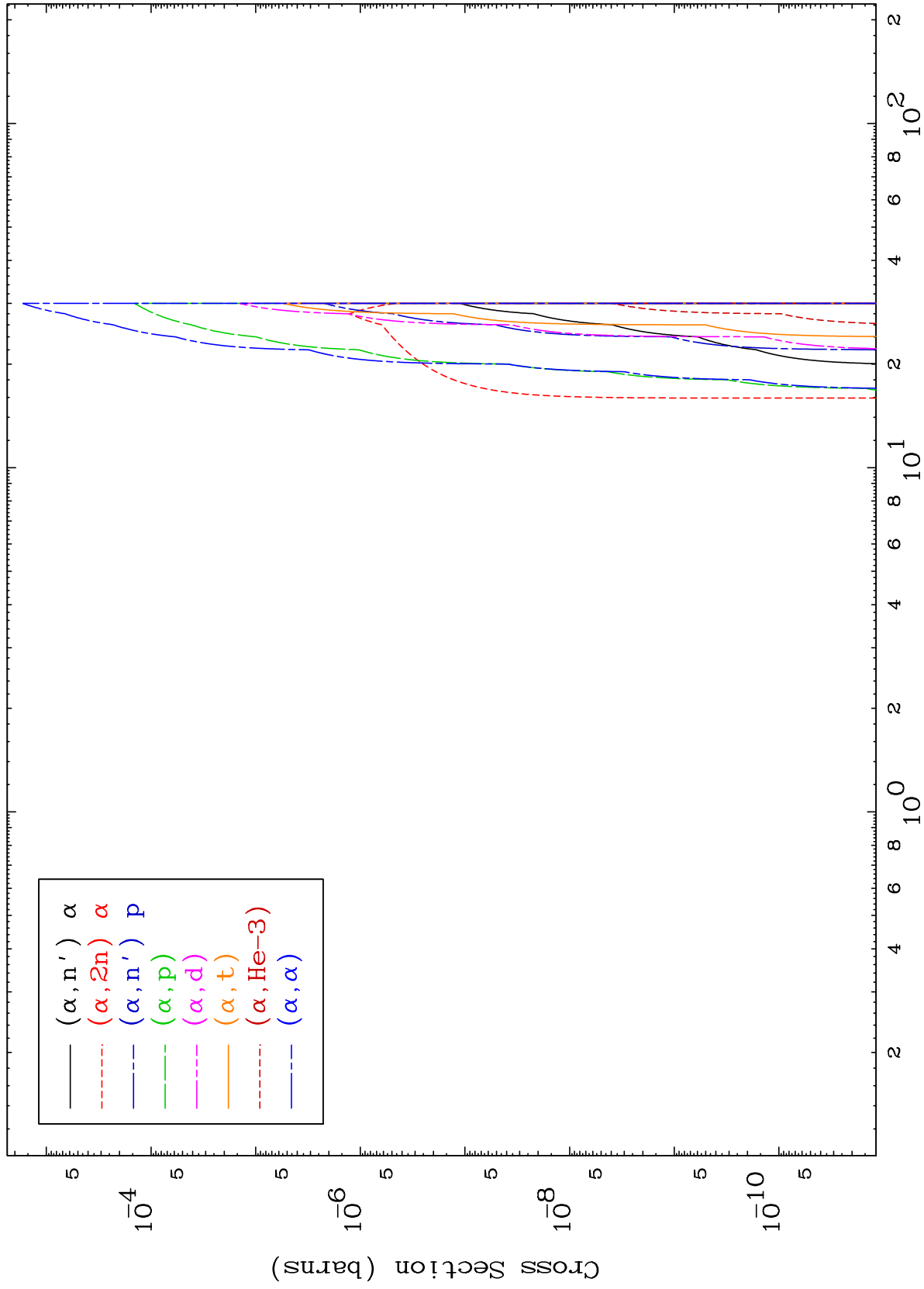
97-Bk-237



2

Incident Energy (MeV)

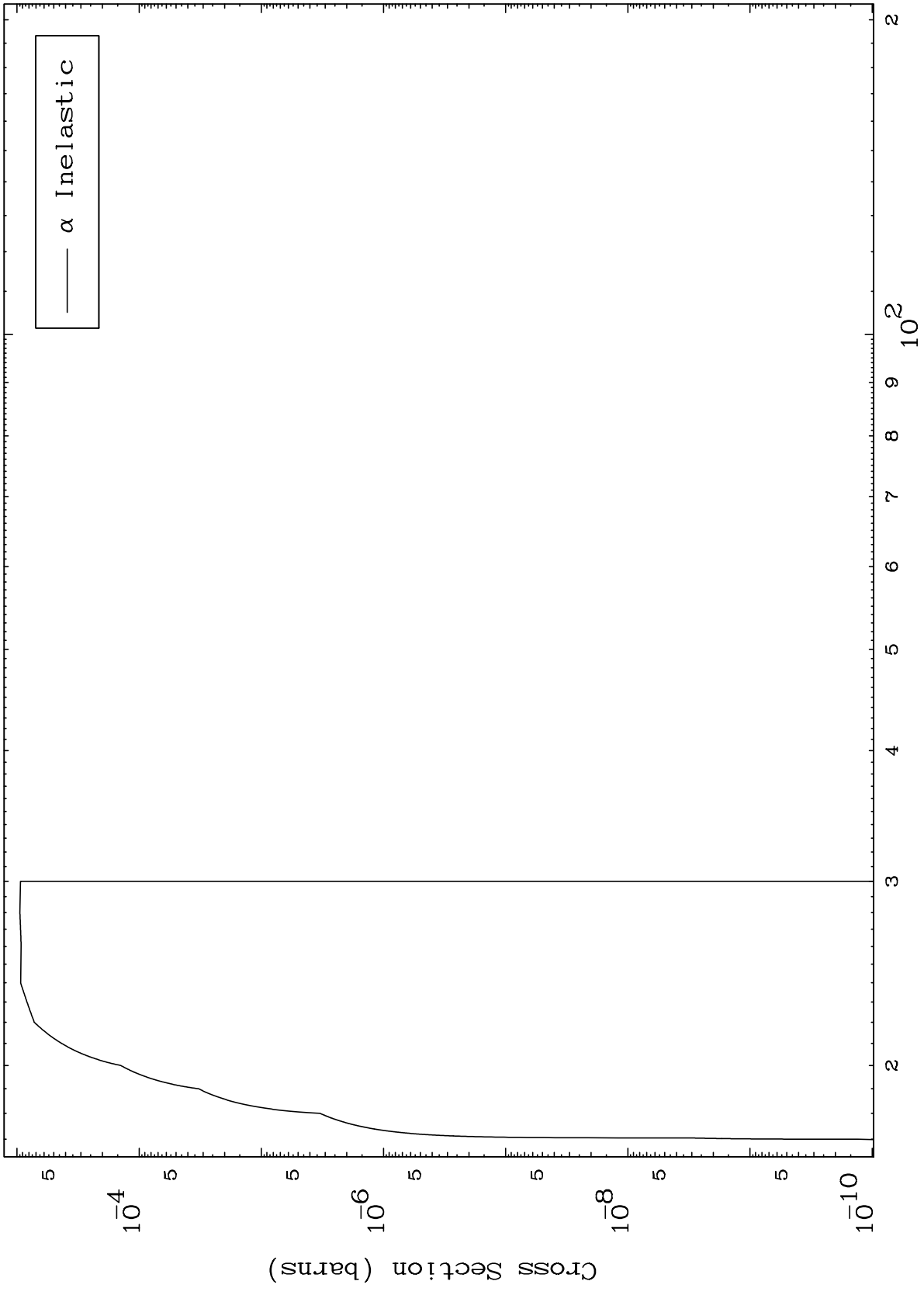
97-Bk-237



MAT 9716

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

97-Bk-237



4

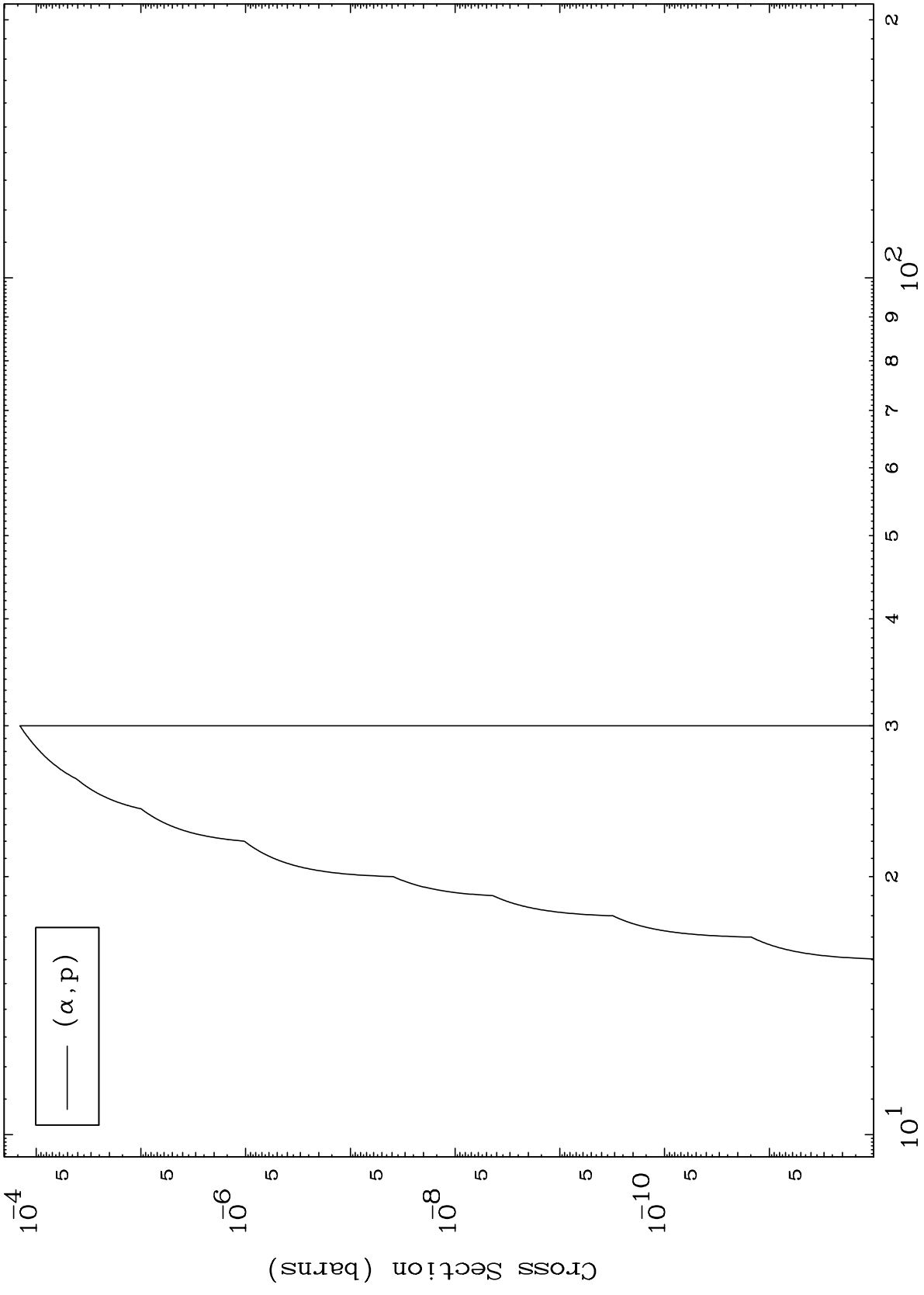
Incident Energy (MeV)

97-Bk-237

MAT 9716

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

97-Bk-237



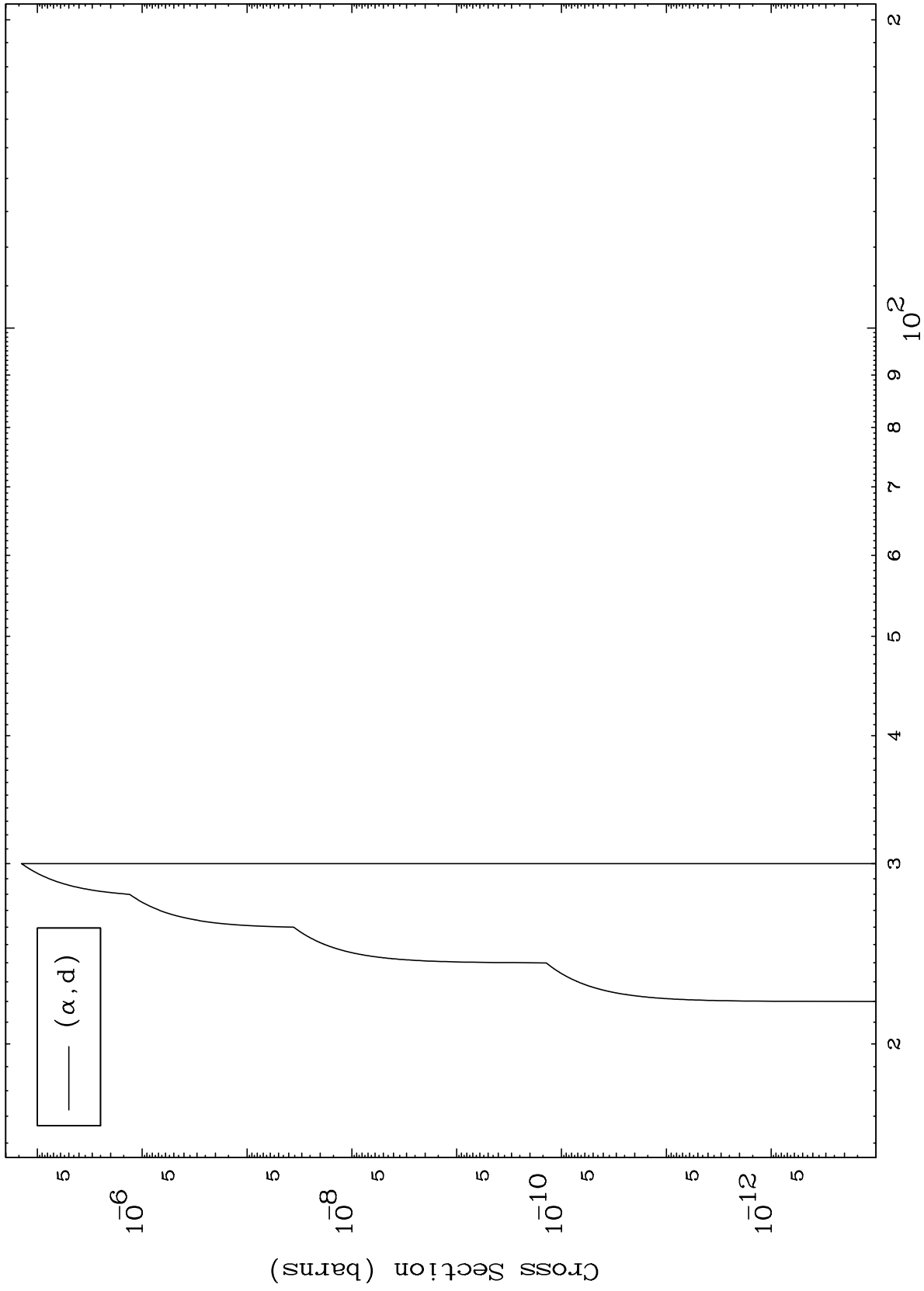
Incident Energy (MeV)

97-Bk-237

MAT 9716

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

97-Bk-237



6

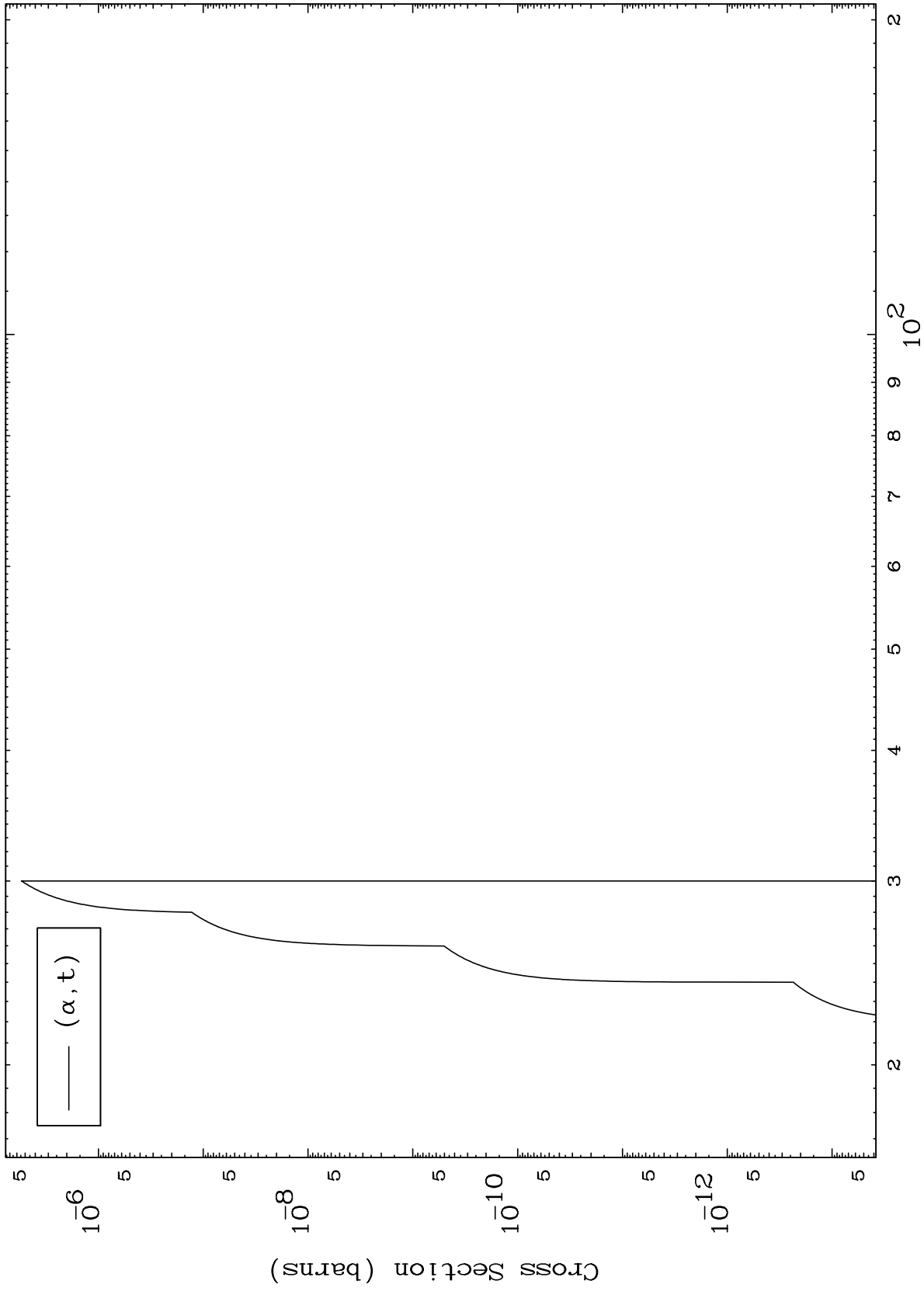
Incident Energy (MeV)

97-Bk-237

MAT 9716

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

97-Bk-237

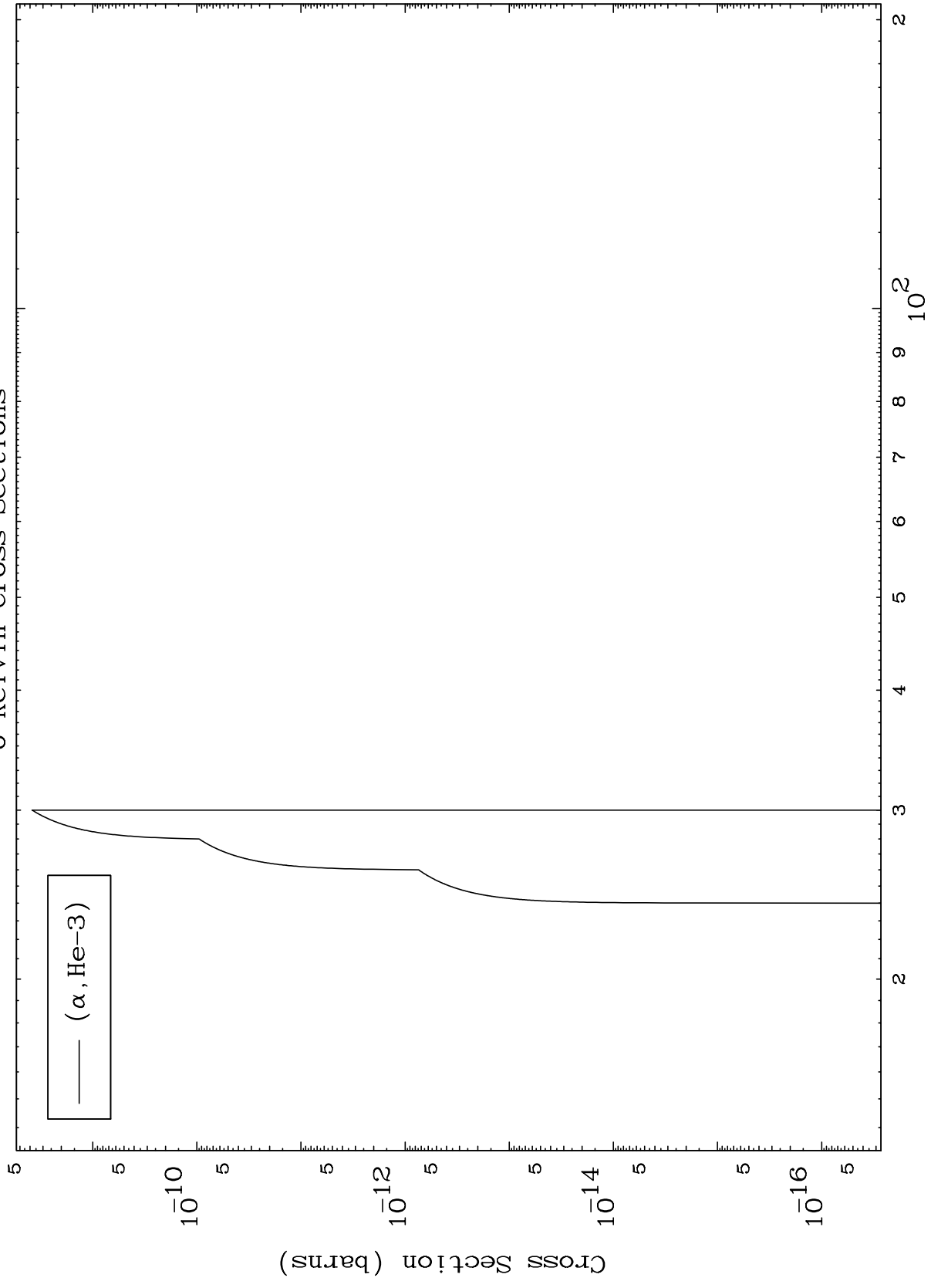


7

Incident Energy (MeV)

97-Bk-237

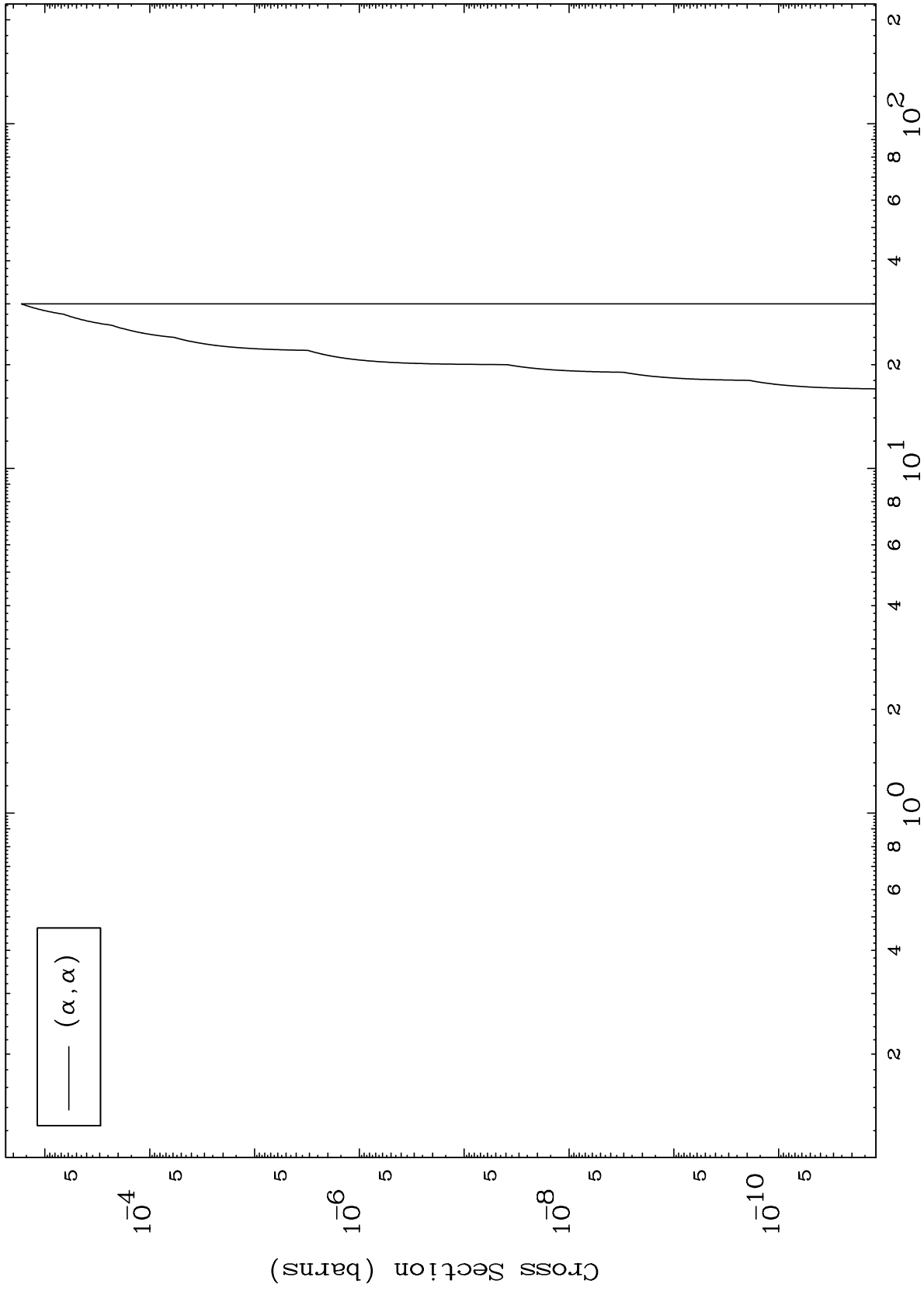




MAT 9716

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

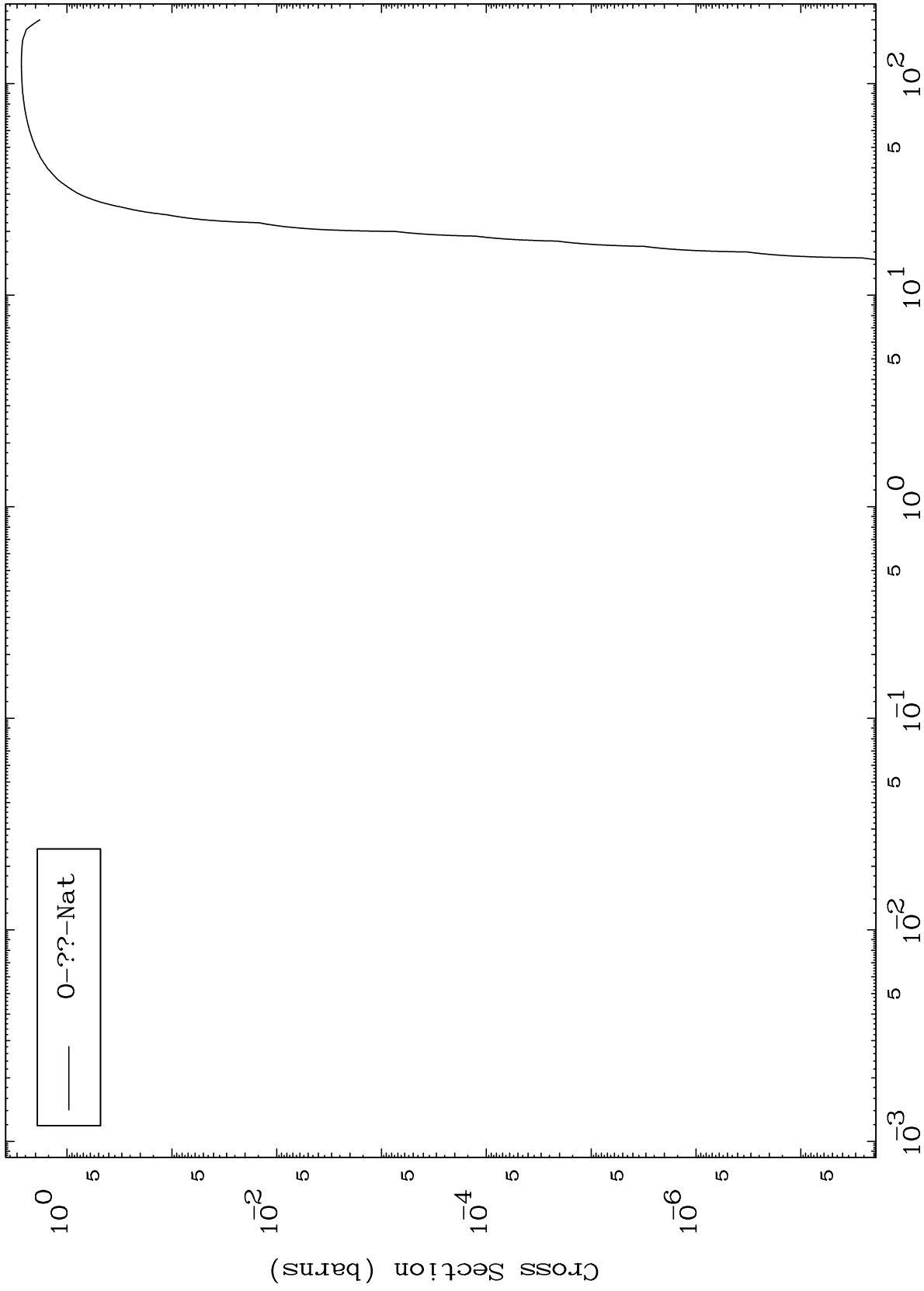
97-Bk-237



MAT 9716

$\alpha$  Fission  
Radionuclide Production Cross Section

97-Bk-237



Incident Energy (MeV)

97-Bk-237