

Program EVALPLOT  
(Version 2018-1)

by

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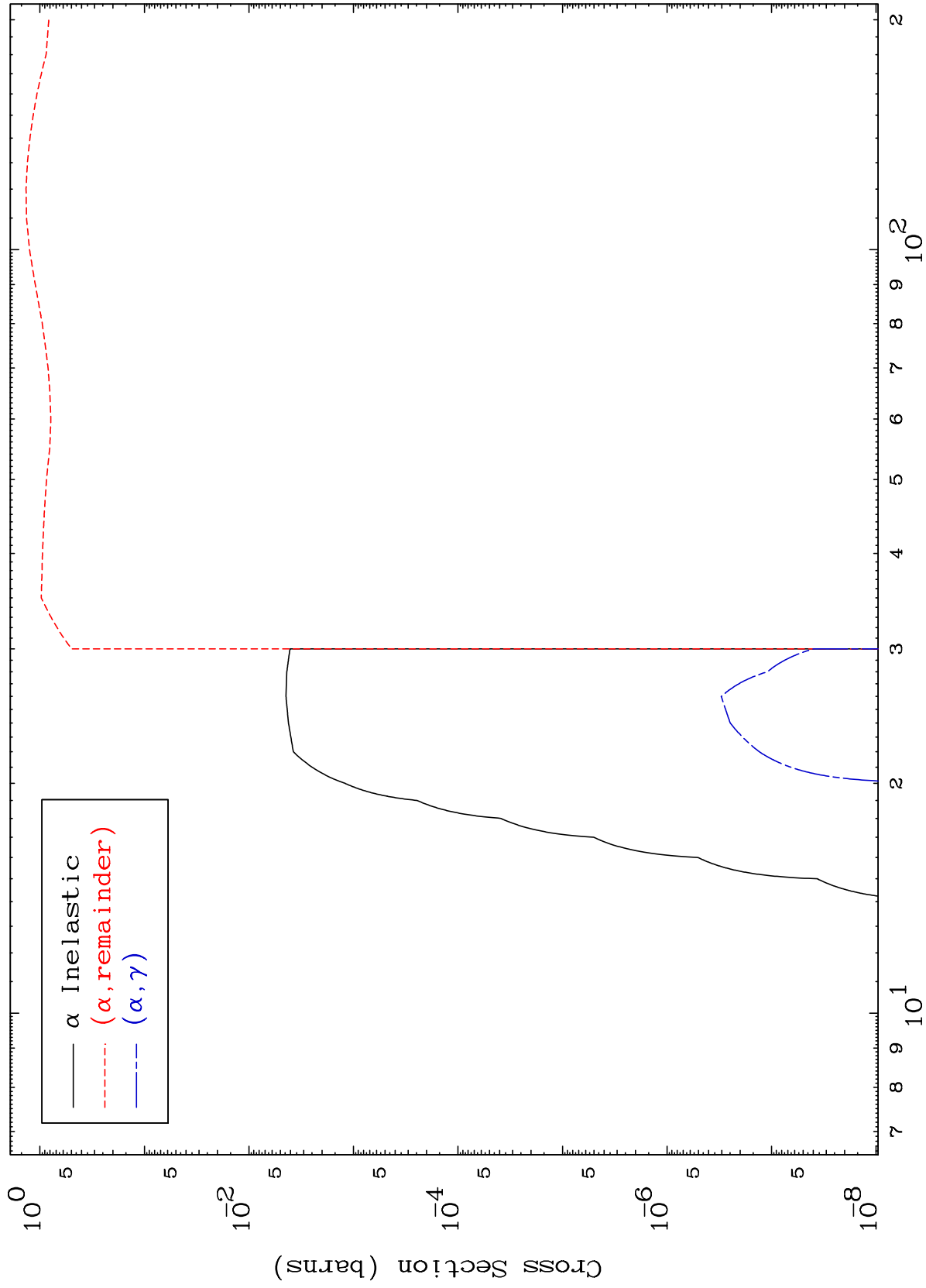
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9752

0 Kelvin  $\alpha$  Major  
Cross Sections

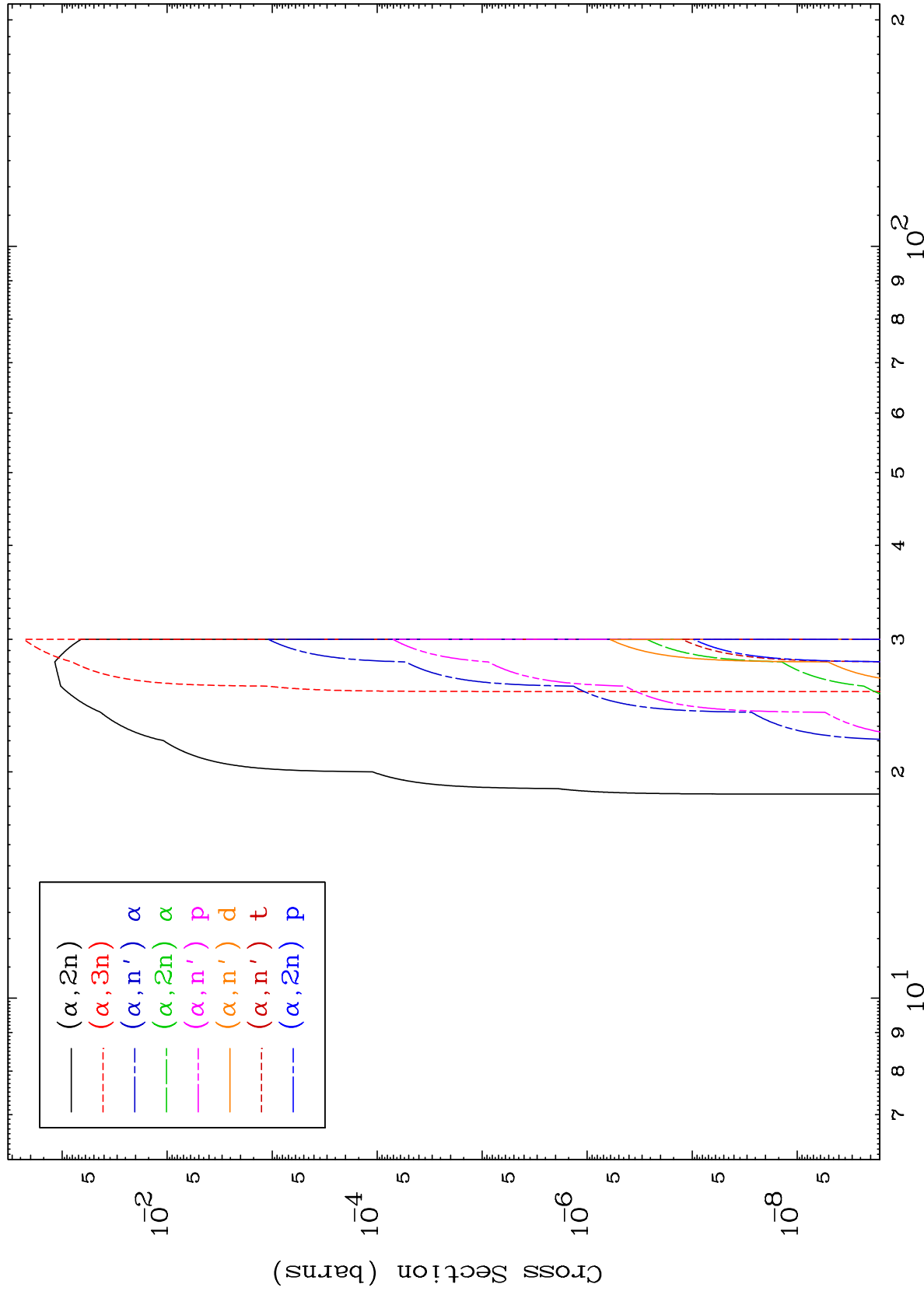
97-Bk-249



MAT 9752

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

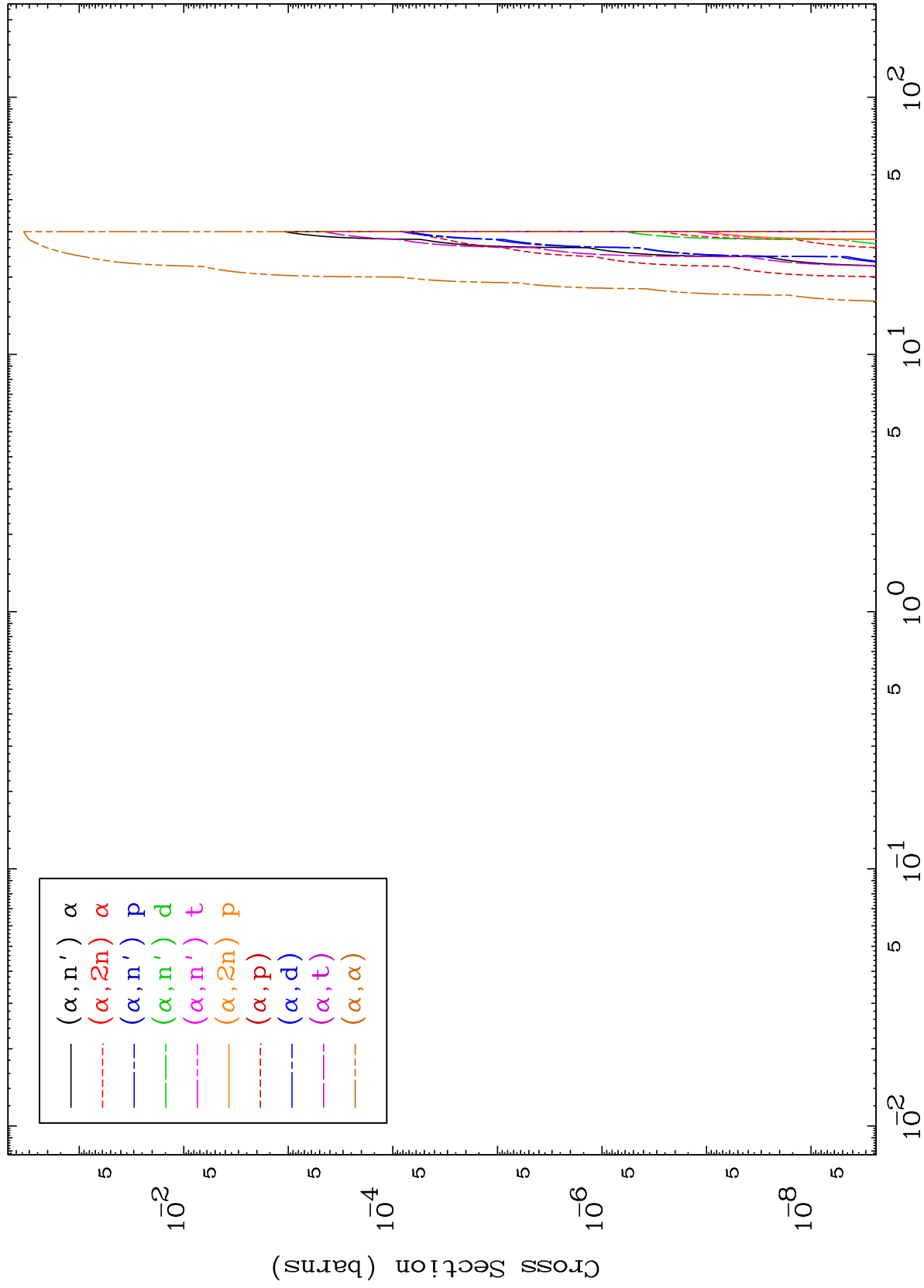
97-Bk-249



2

Incident Energy (MeV)

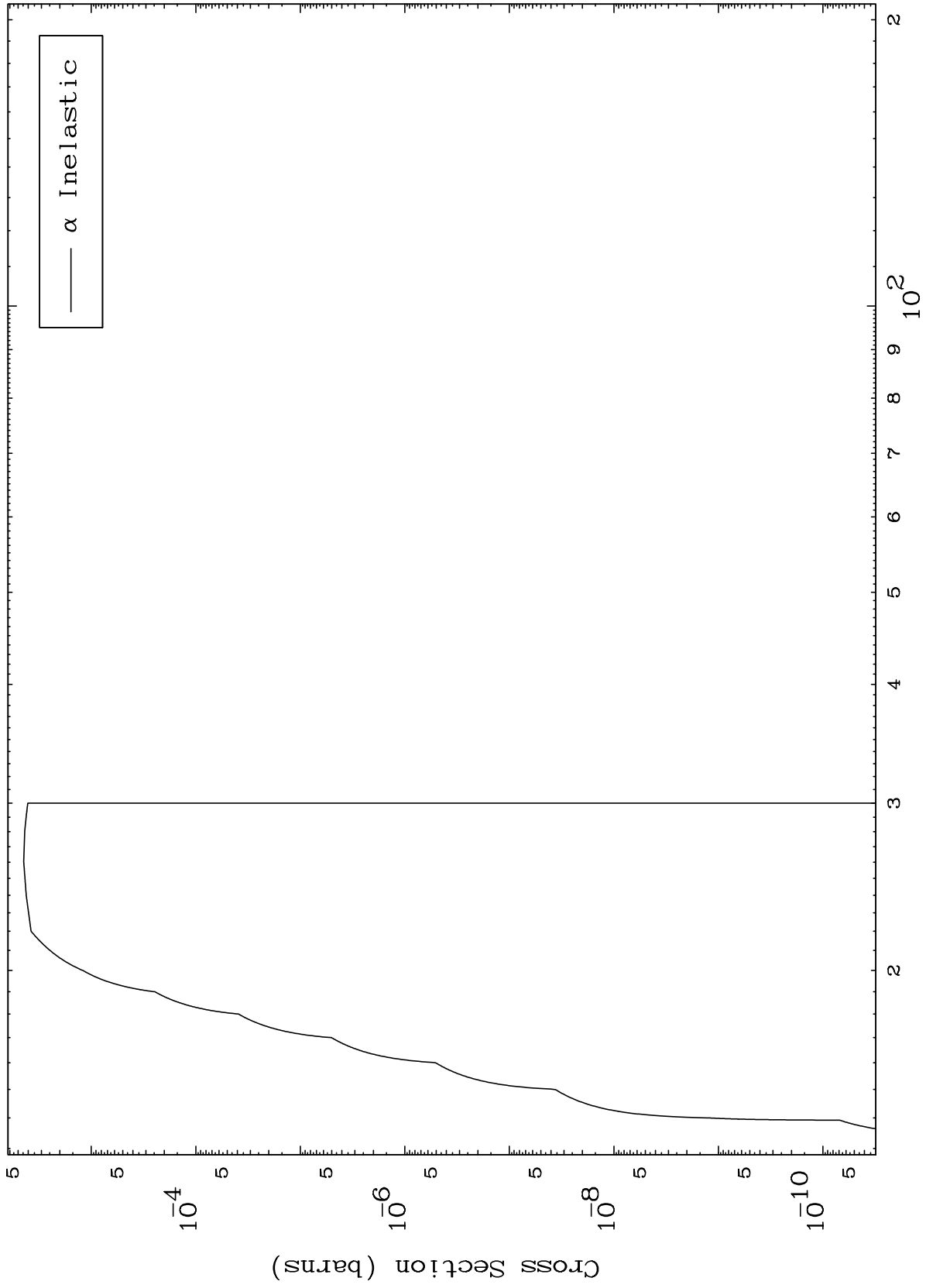
97-Bk-249



MAT 9752

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

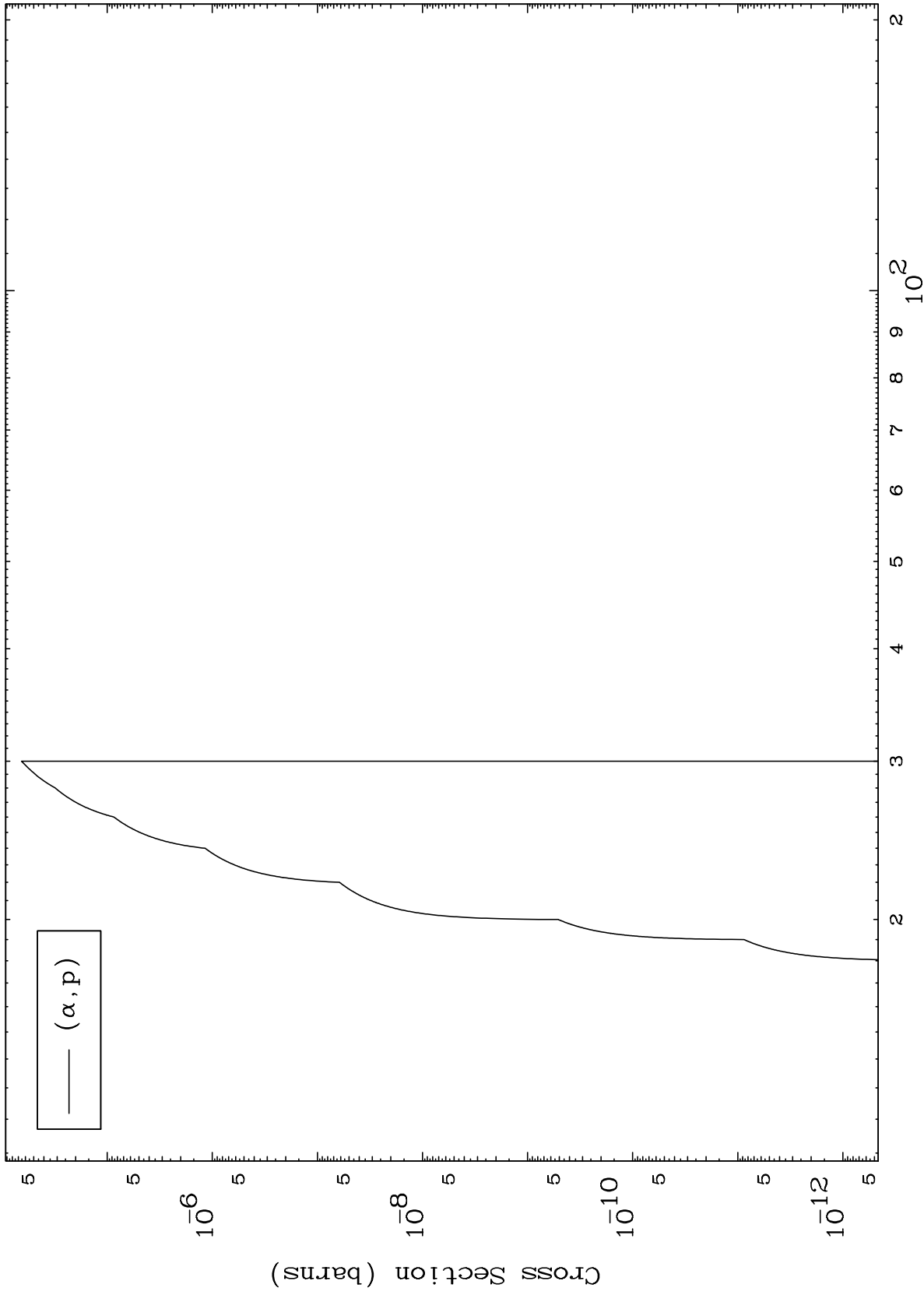
97-Bk-249



4

Incident Energy (MeV)

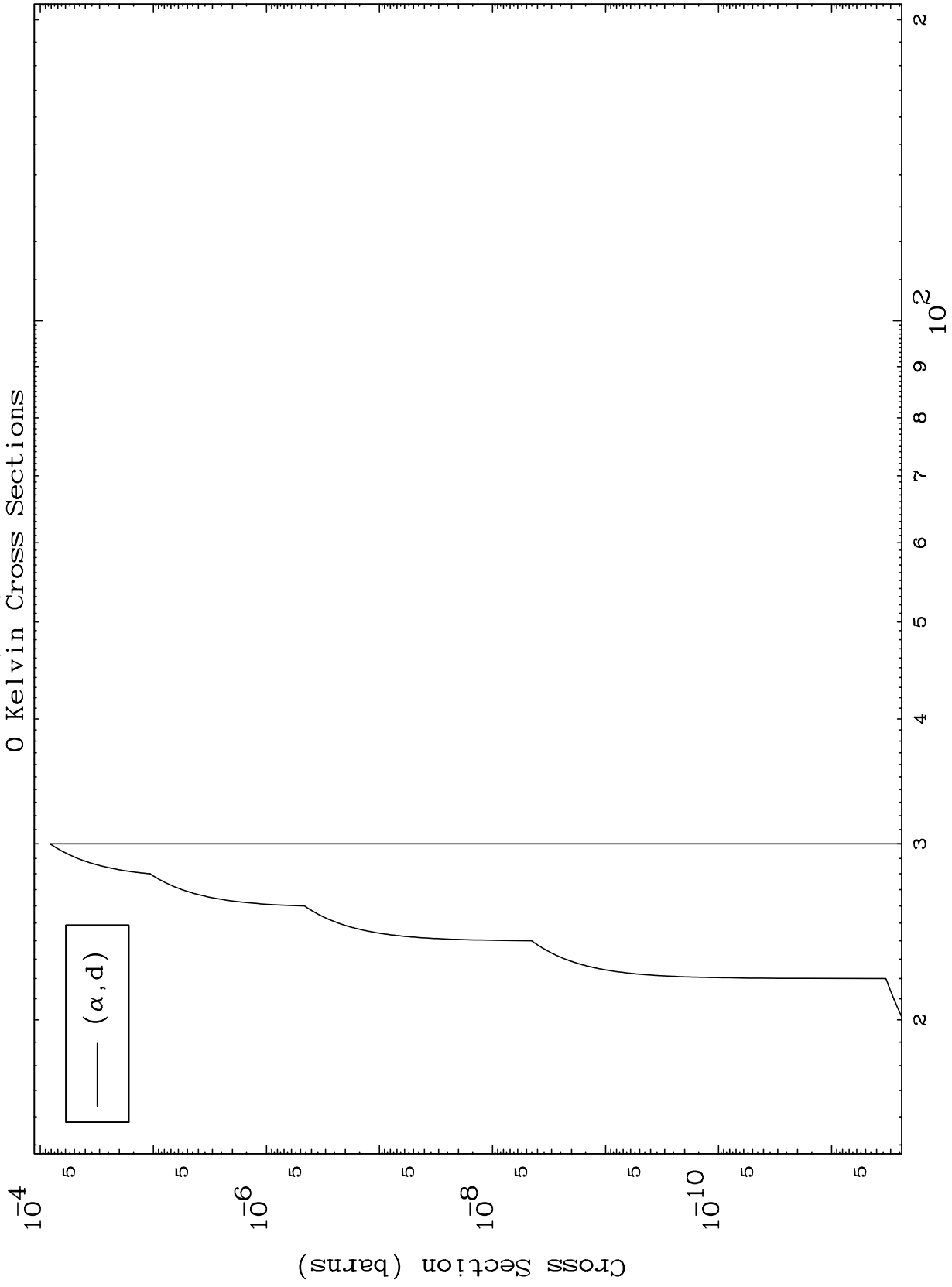
97-Bk-249



MAT 9752

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

97-Bk-249



6

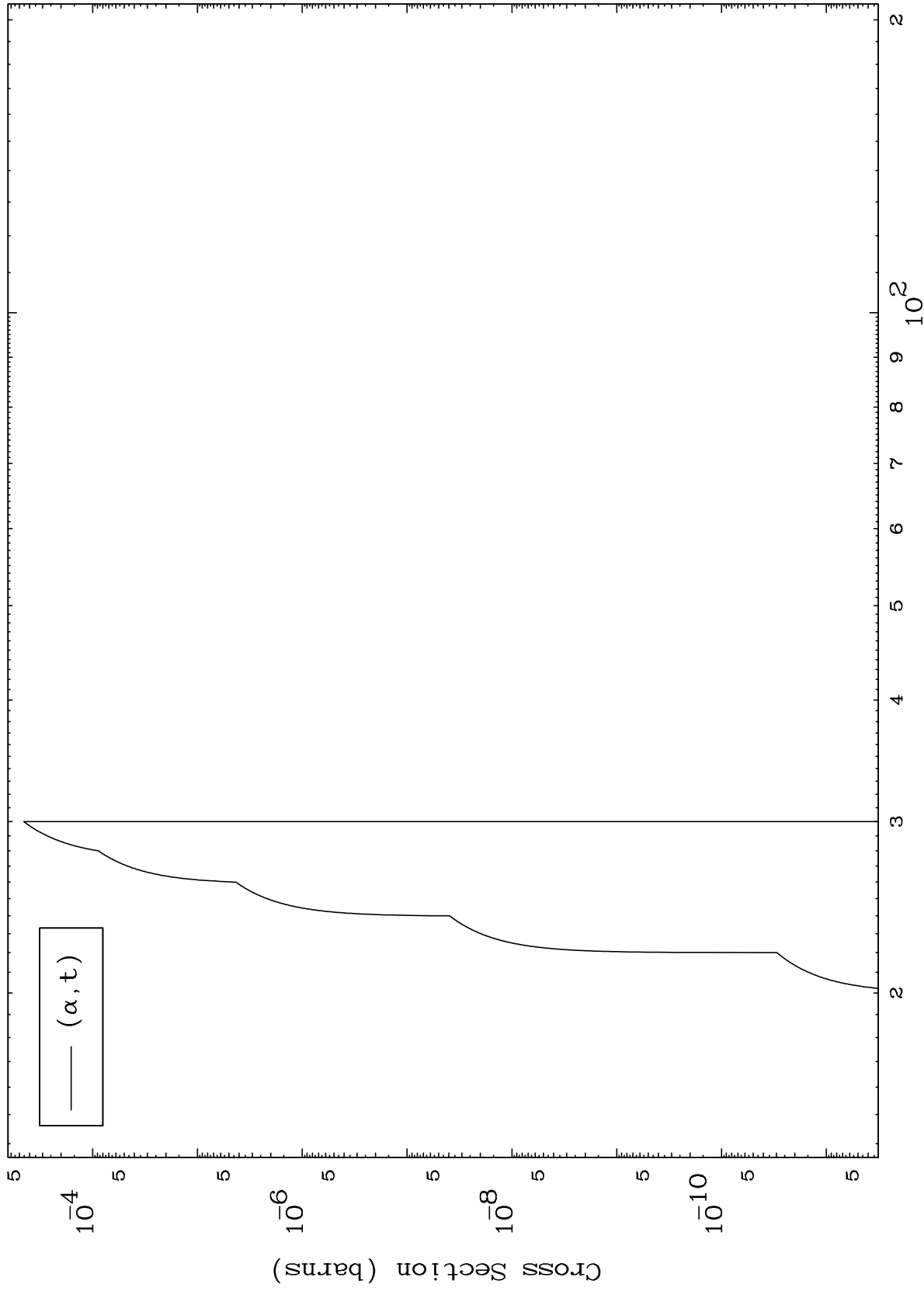
Incident Energy (MeV)

97-Bk-249

MAT 9752

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

97-Bk-249



7

Incident Energy (MeV)

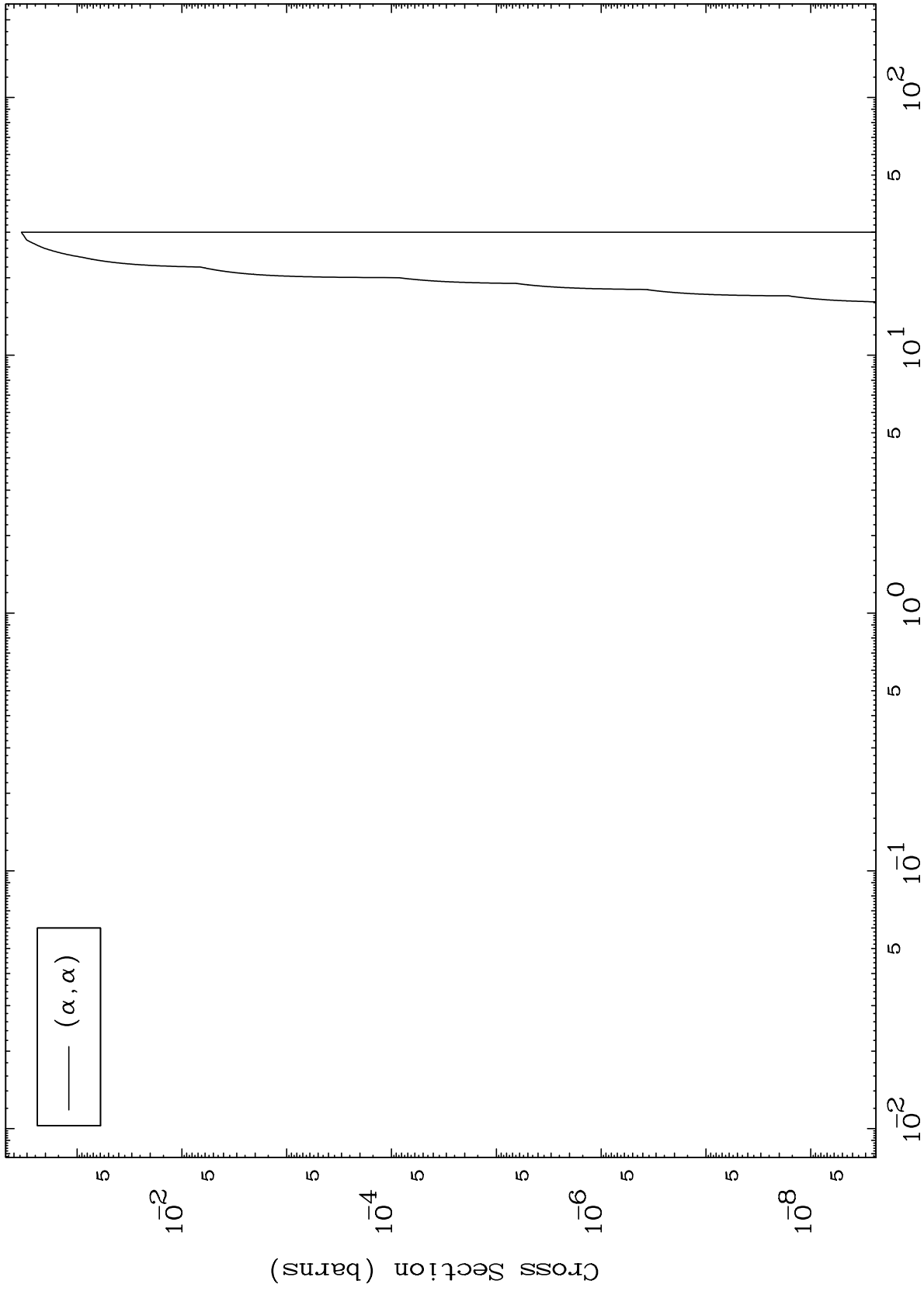
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MAT 9752

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

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8

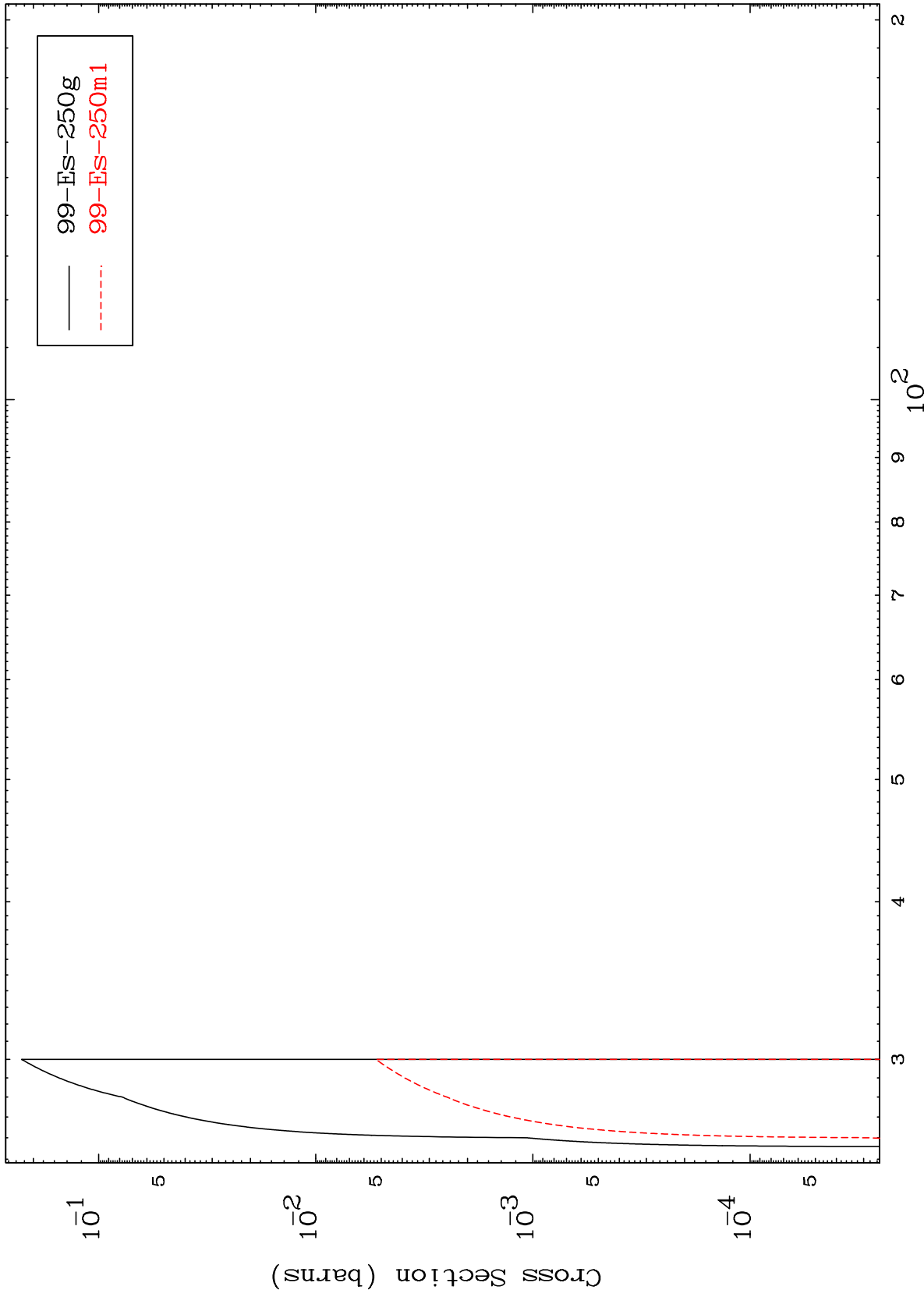
Incident Energy (MeV)

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MAT 9752

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Radionuclide Production Cross Section  
( $\alpha, 3n$ )



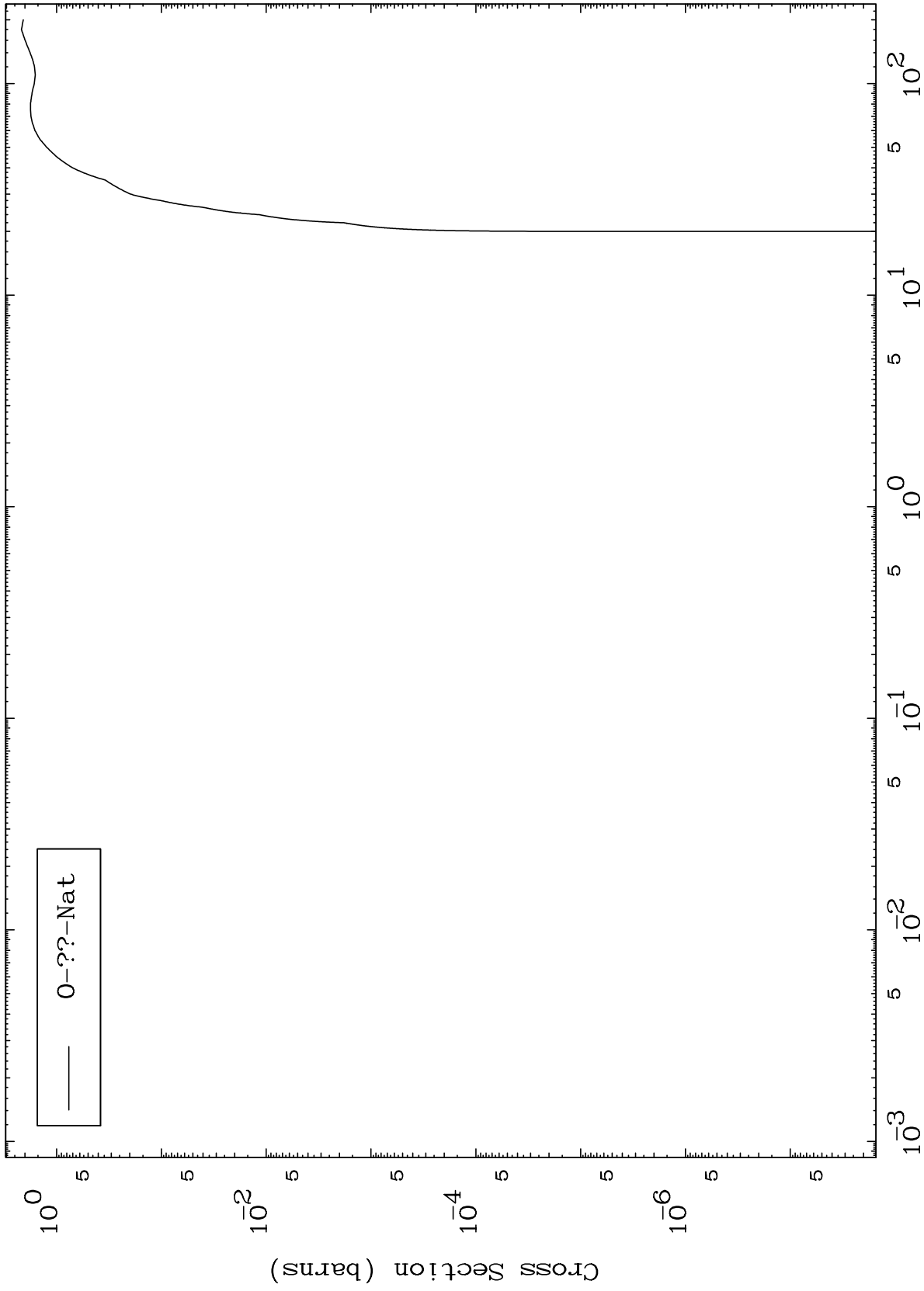
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MAT 9752

$\alpha$  Fission  
Radionuclide Production Cross Section

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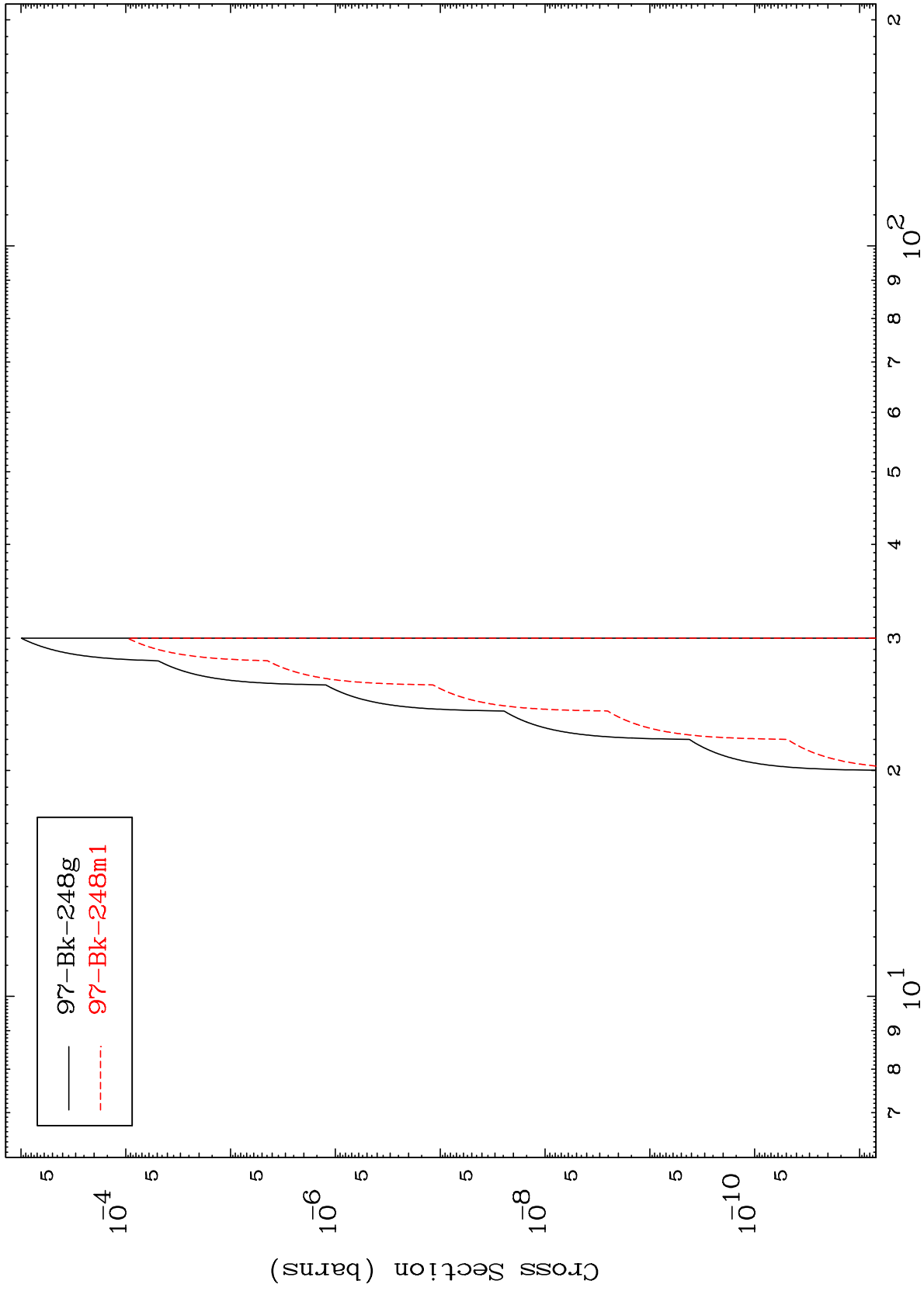
Incident Energy (MeV)

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MAT 9752

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( $\alpha, n'$ )  $\alpha$   
Radionuclide Production Cross Section



11

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