

Program EVALPLOT  
(Version 2018-1)

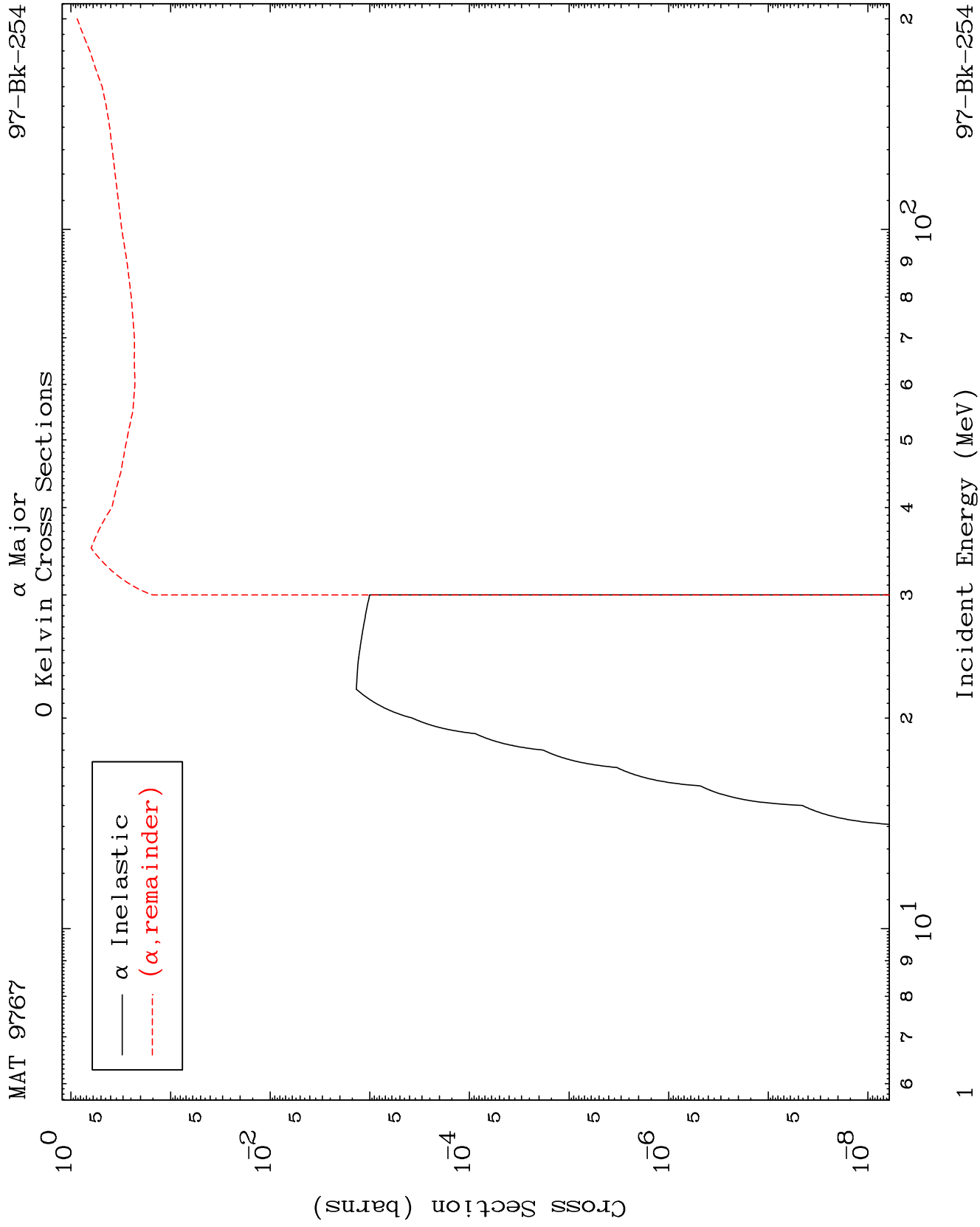
by

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Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

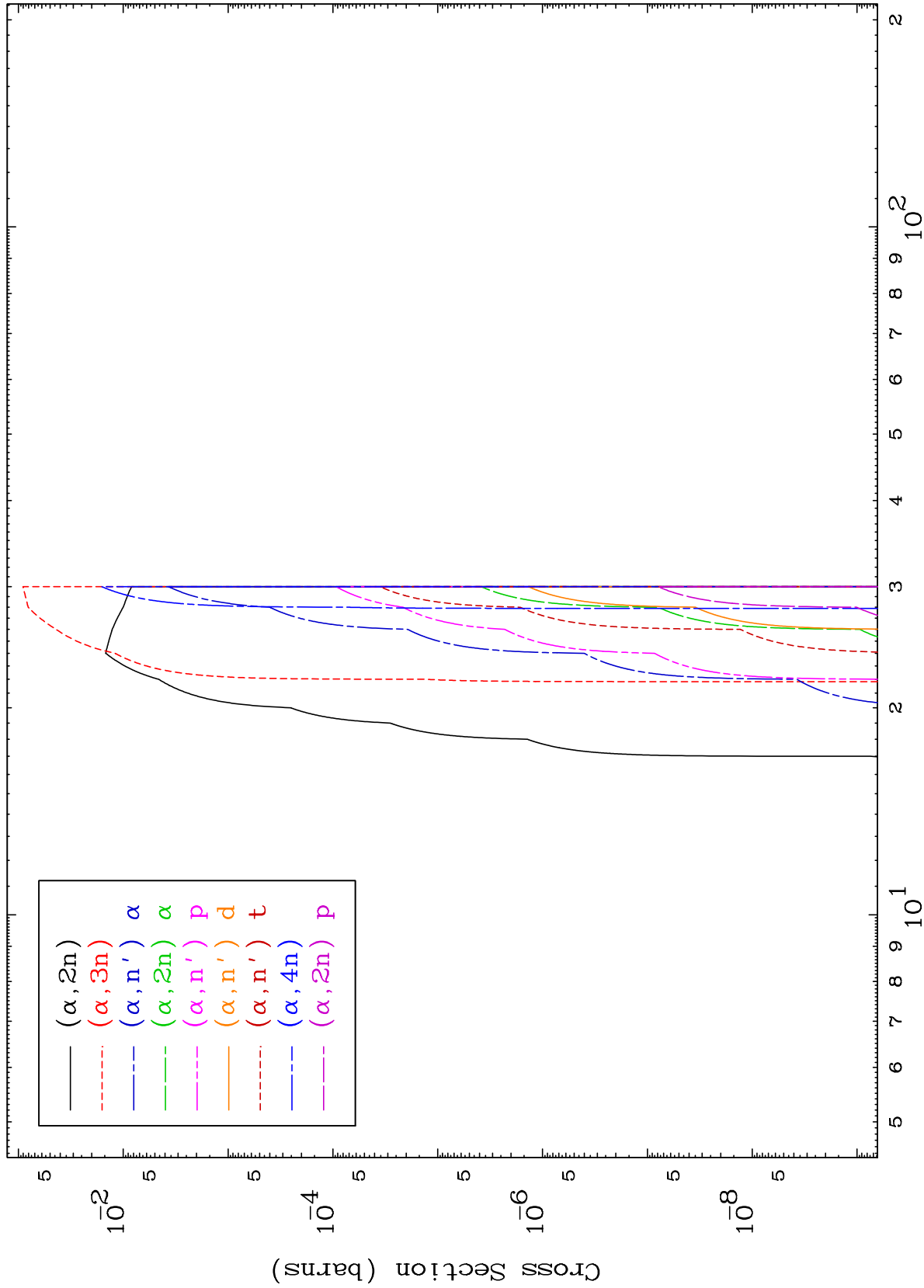
Press Mouse Button to Start



MAT 9767

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

97-Bk-254



2

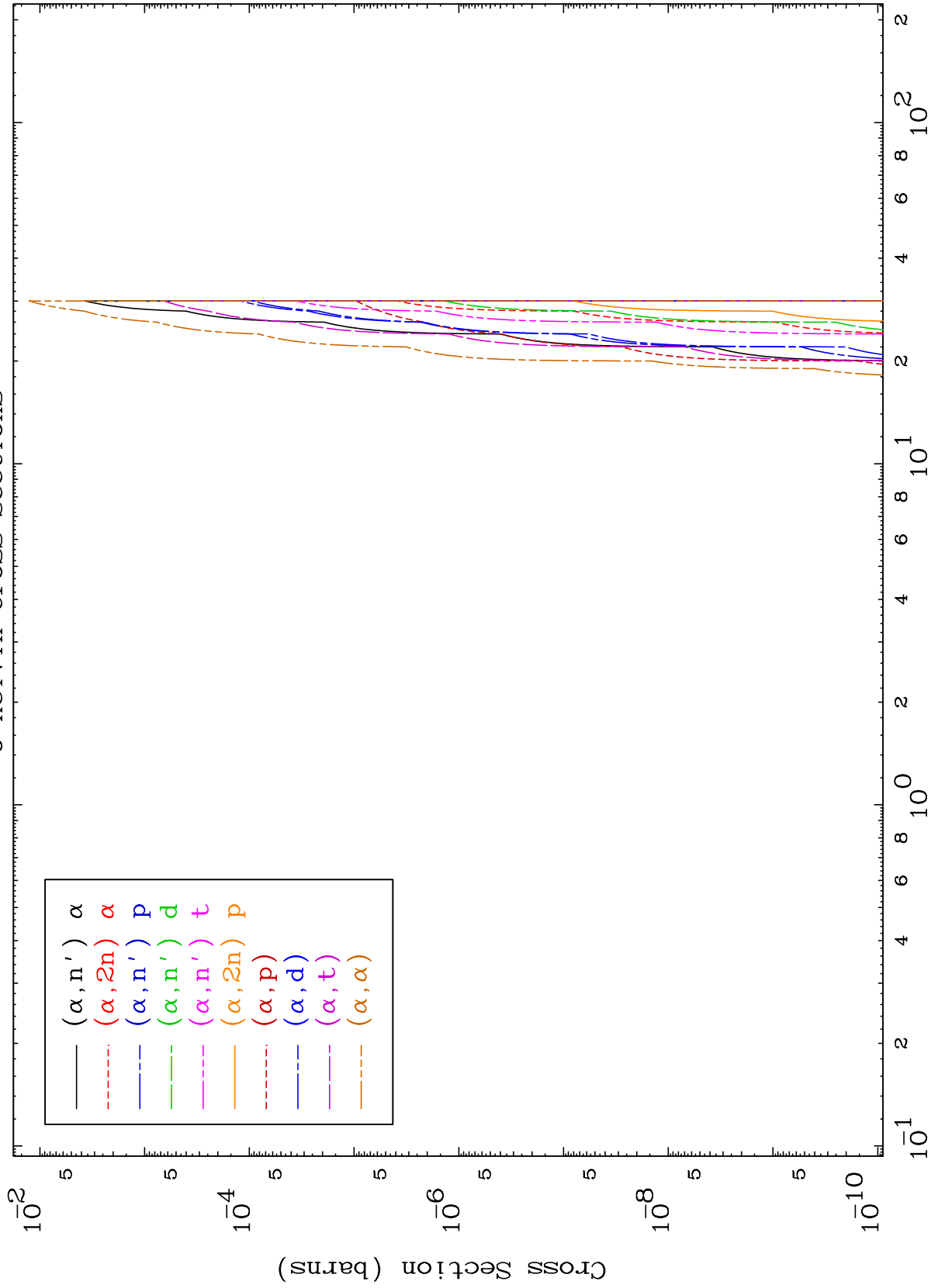
Incident Energy (MeV)

97-Bk-254

MAT 9767

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

97-Bk-254



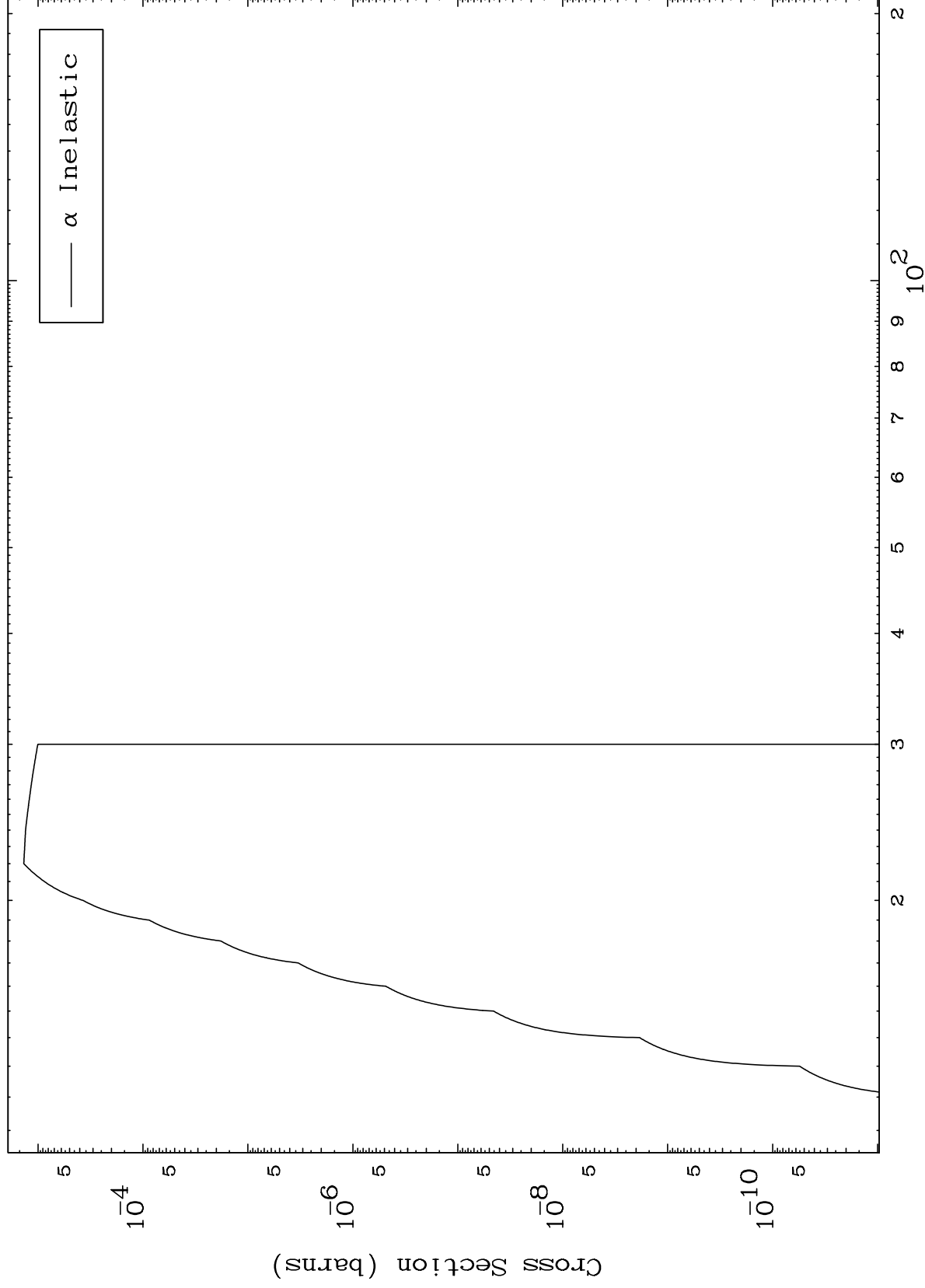
Incident Energy (MeV)

97-Bk-254

MAT 9767

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

97-Bk-254



4

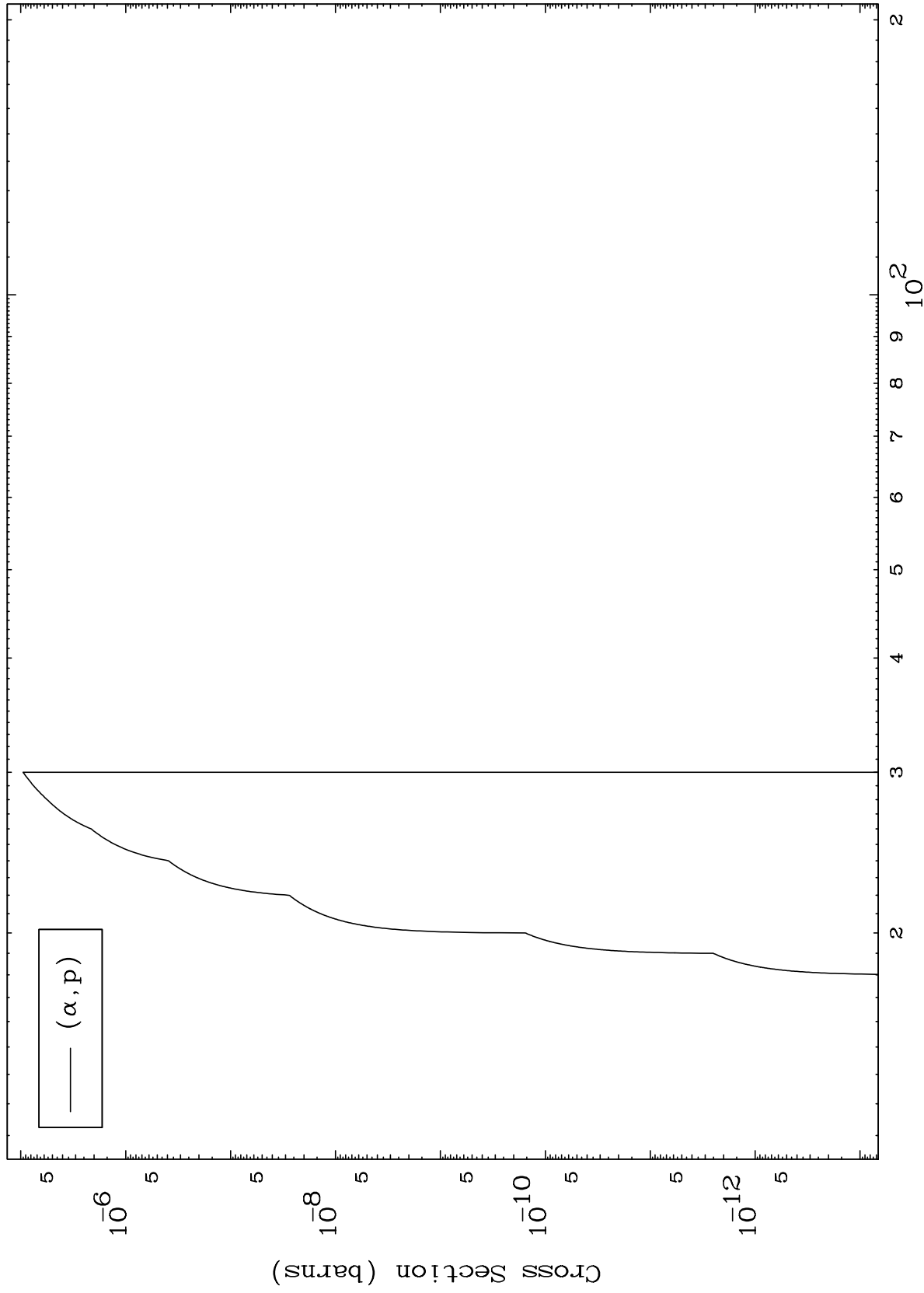
Incident Energy (MeV)

97-Bk-254

MAT 9767

97-Bk-254

( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections



5

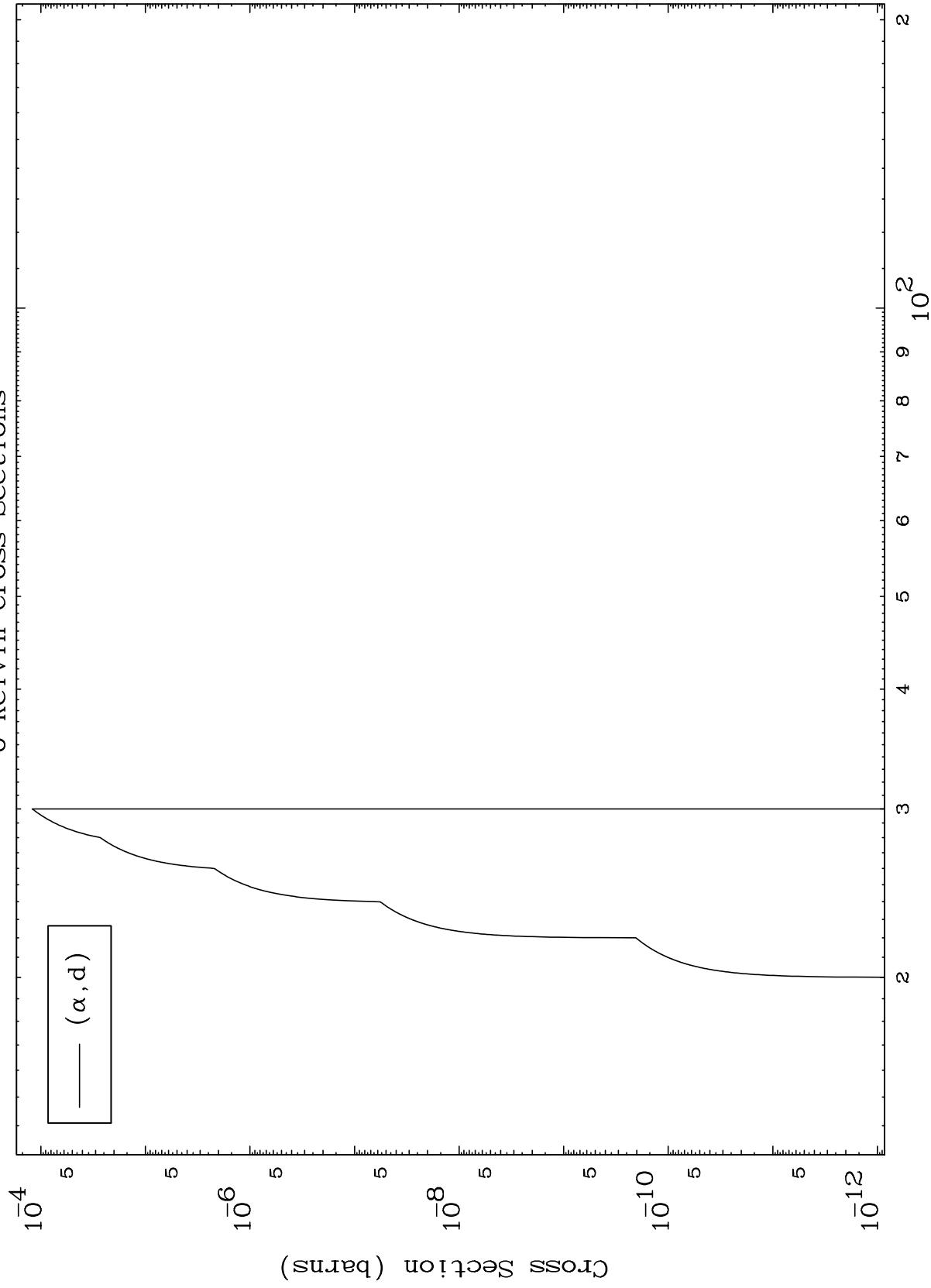
Incident Energy (MeV)

97-Bk-254

MAT 9767

( $\alpha$ ,d) Levels  
0 Kelvin Cross Sections

97-Bk-254



6

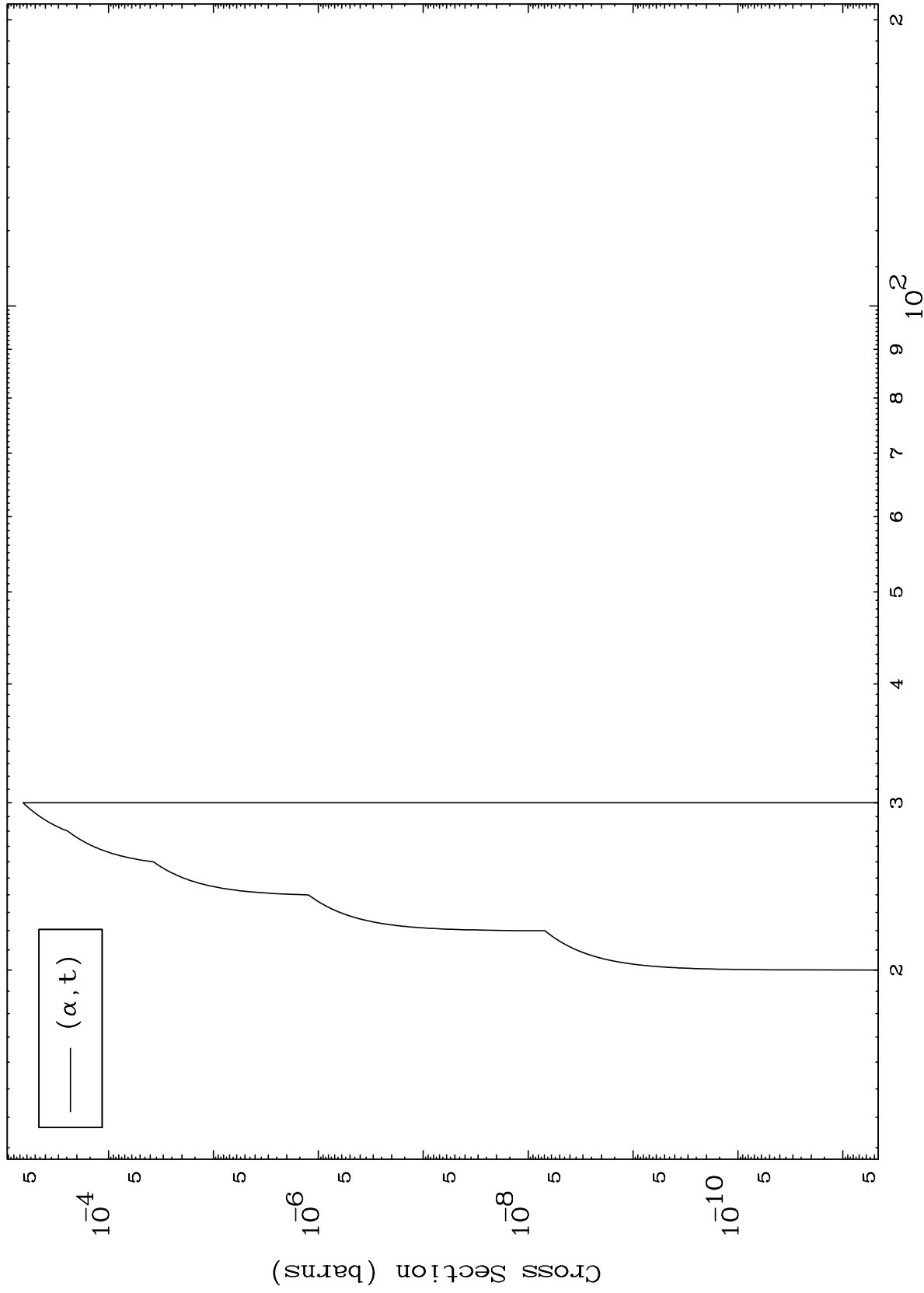
Incident Energy (MeV)

97-Bk-254

MAT 9767

97-Bk-254

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



7

Incident Energy (MeV)

97-Bk-254

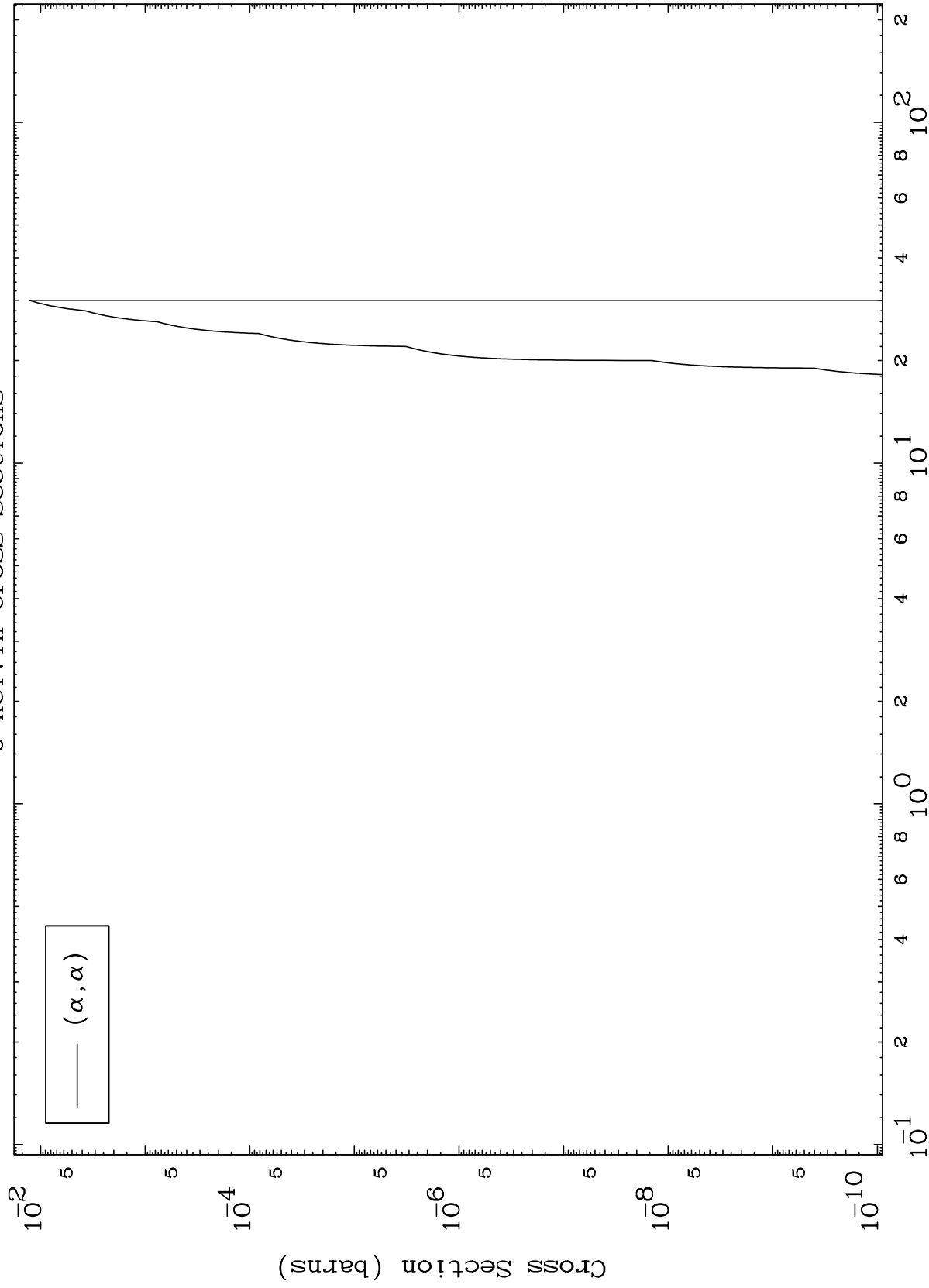


MAT 9767

$(\alpha, \alpha)$  Levels

97-Bk-254

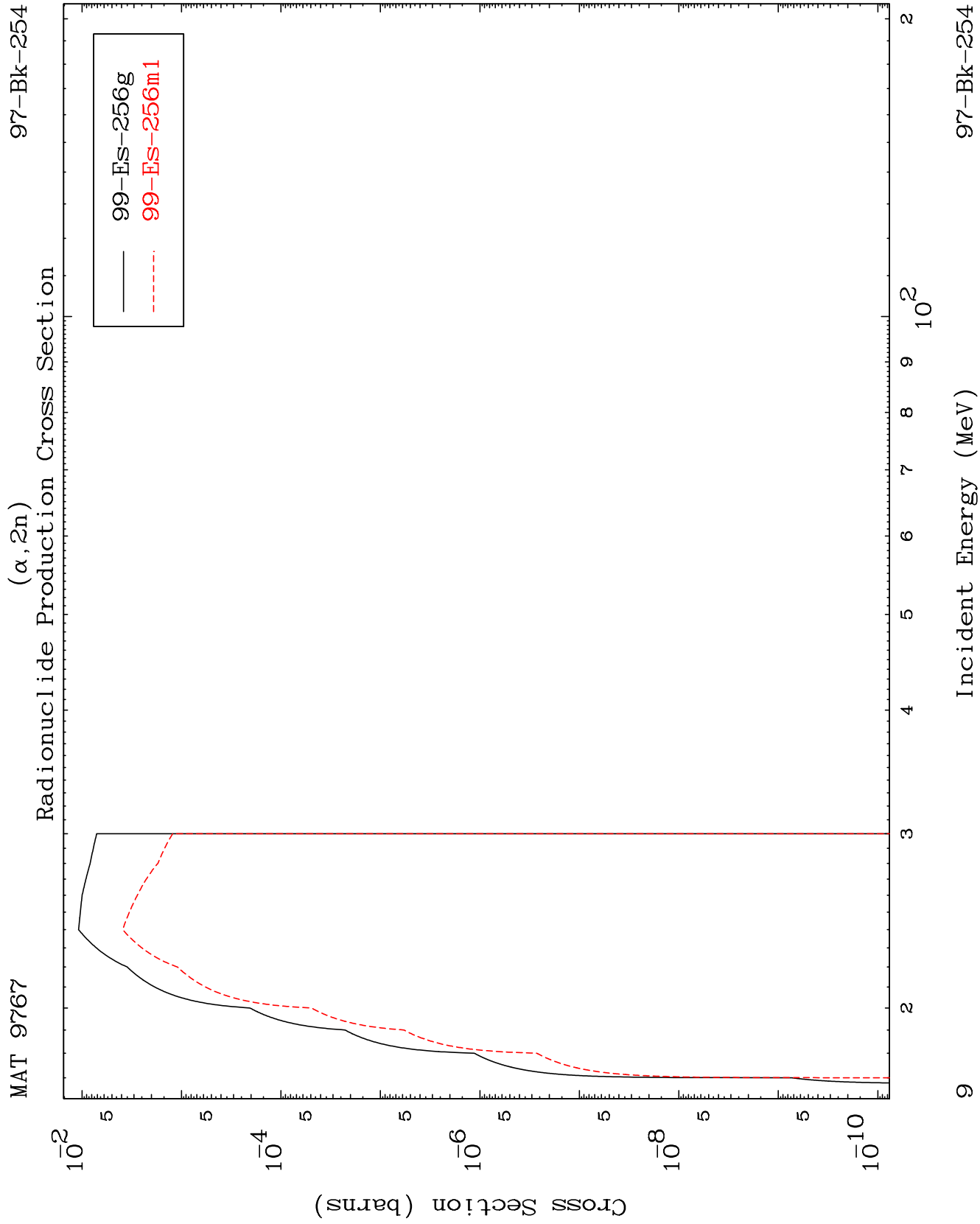
0 Kelvin Cross Sections



8

Incident Energy (MeV)

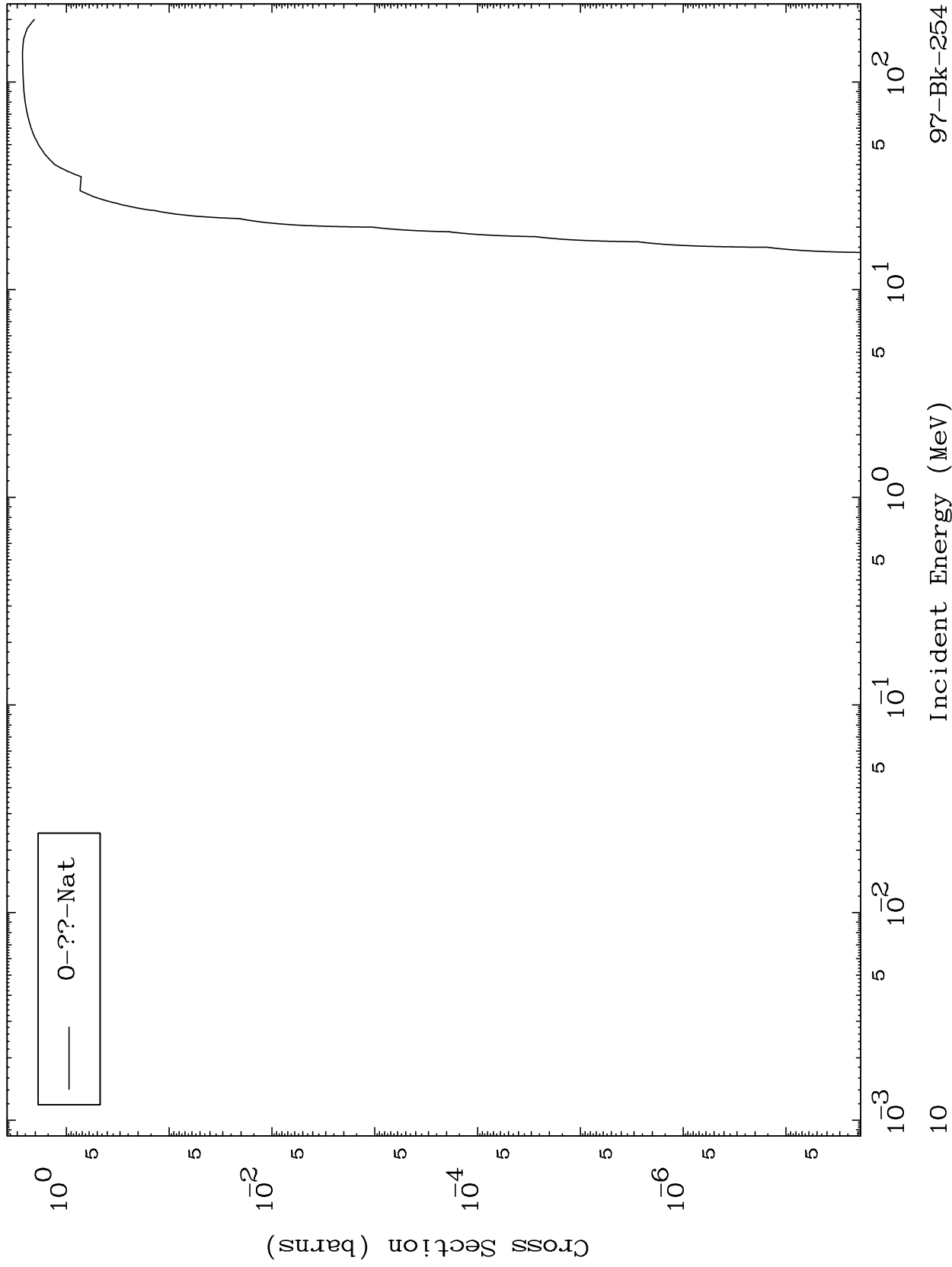
97-Bk-254



MAT 9767

97-Bk-254

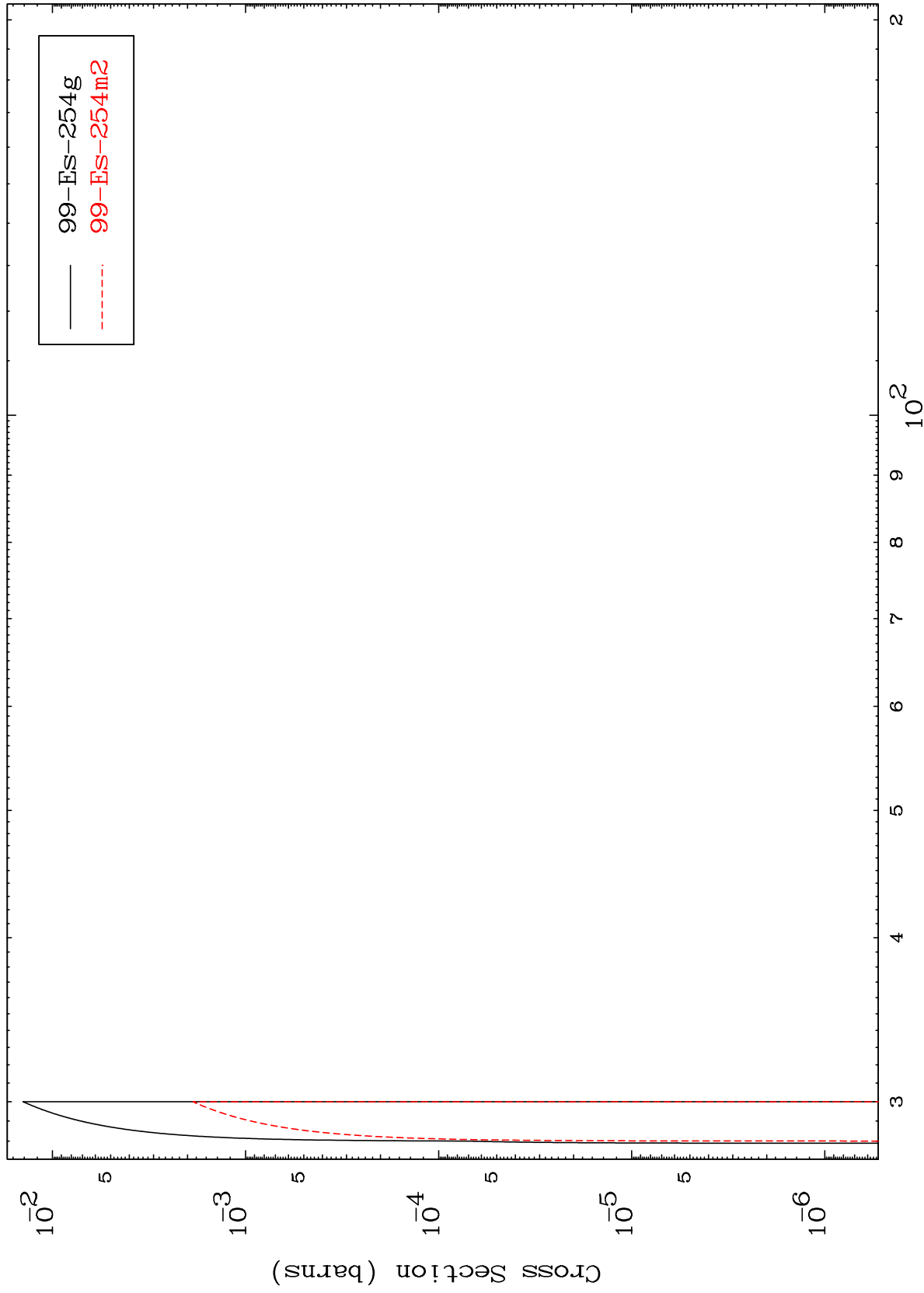
$\alpha$  Fission  
Radionuclide Production Cross Section



MAT 9767

97-Bk-254

$(\alpha, 4n)$   
Radionuclide Production Cross Section



11

Incident Energy (MeV)

97-Bk-254