

Program EVALPLOT  
(Version 2018-1)

by

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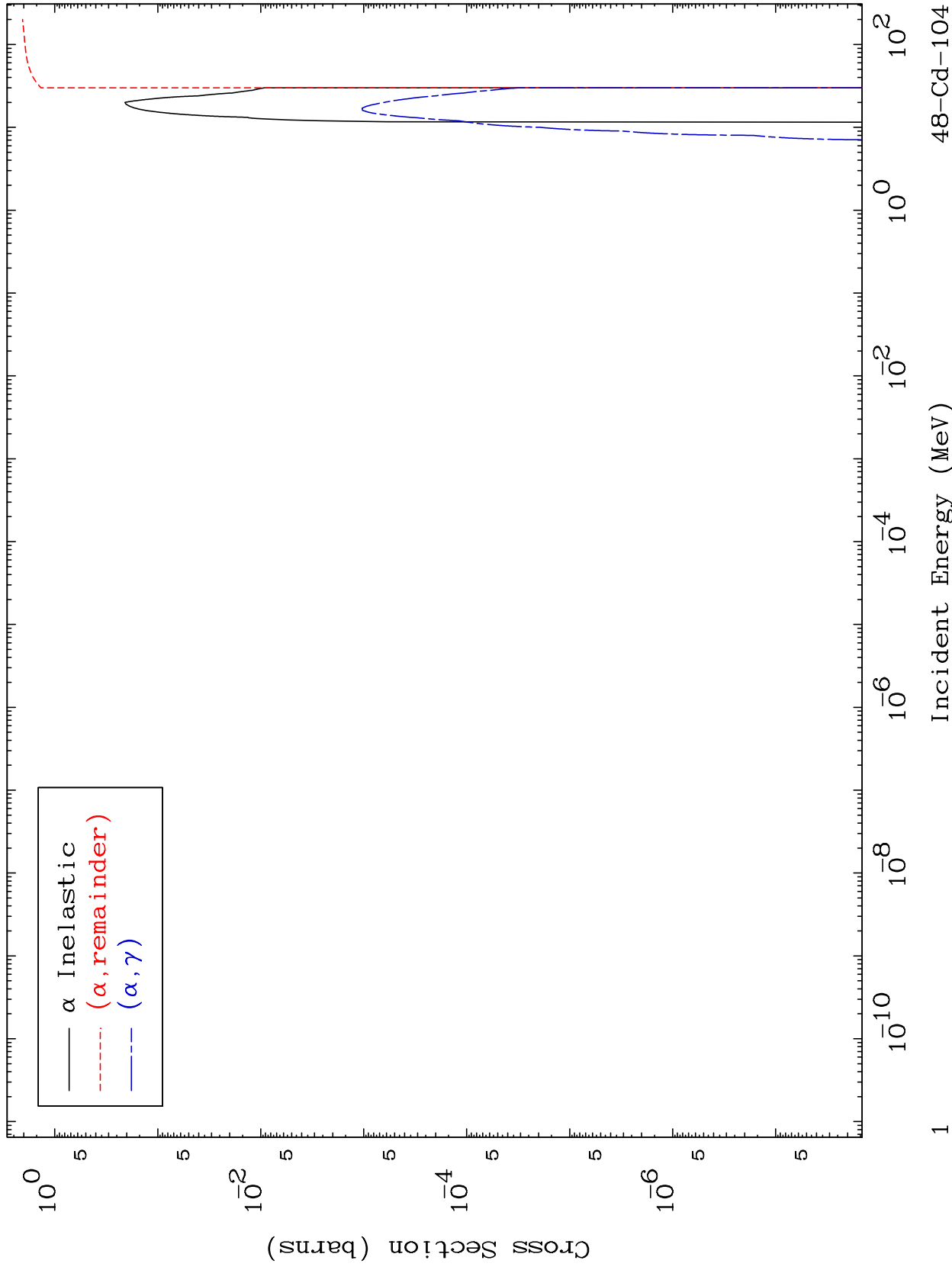
Press Mouse Button to Start

MAT 4819

$\alpha$  Major

48-Cd-104

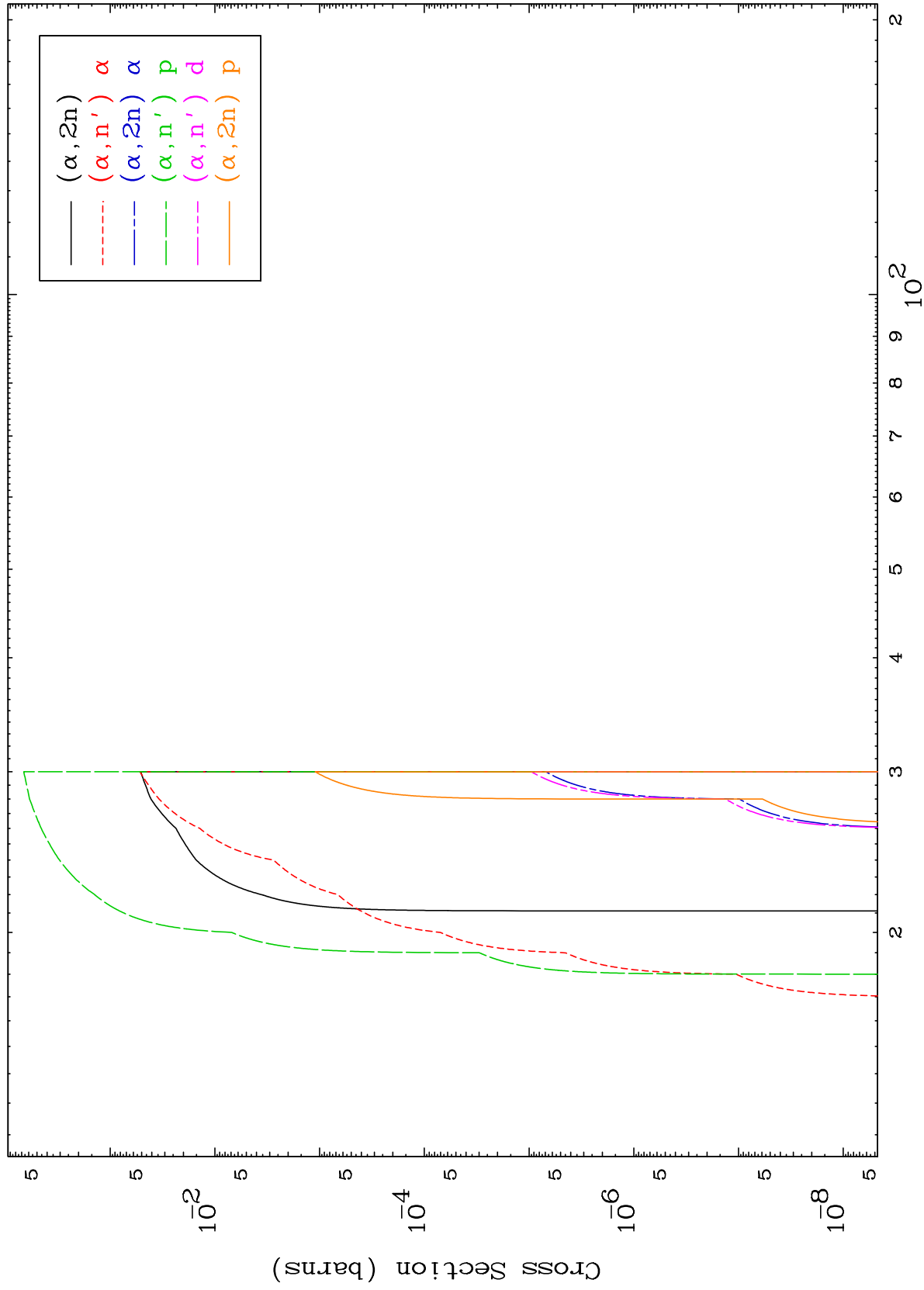
0 Kelvin Cross Sections



MAT 4819

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

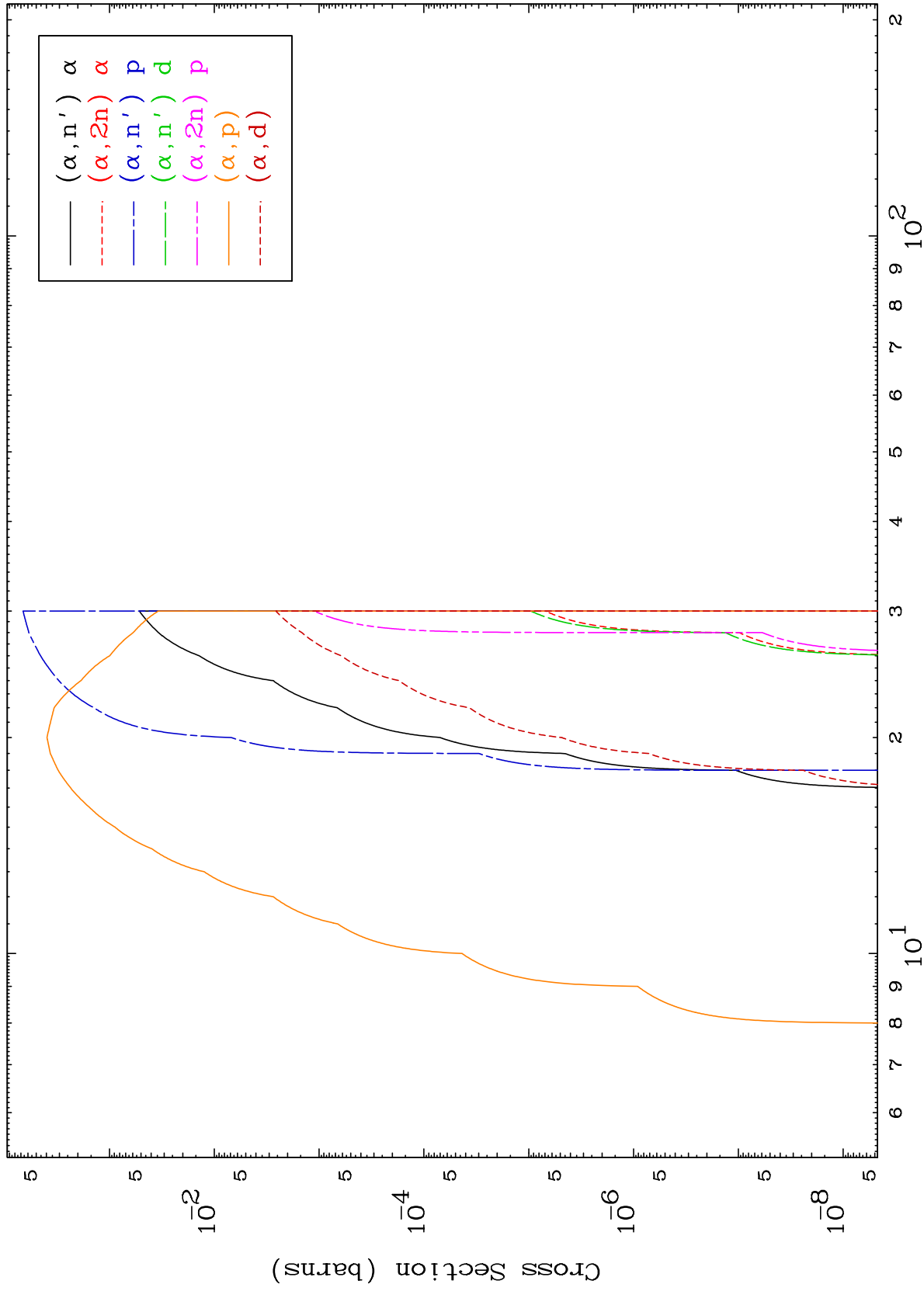
48-Cd-104



2

Incident Energy (MeV)

48-Cd-104

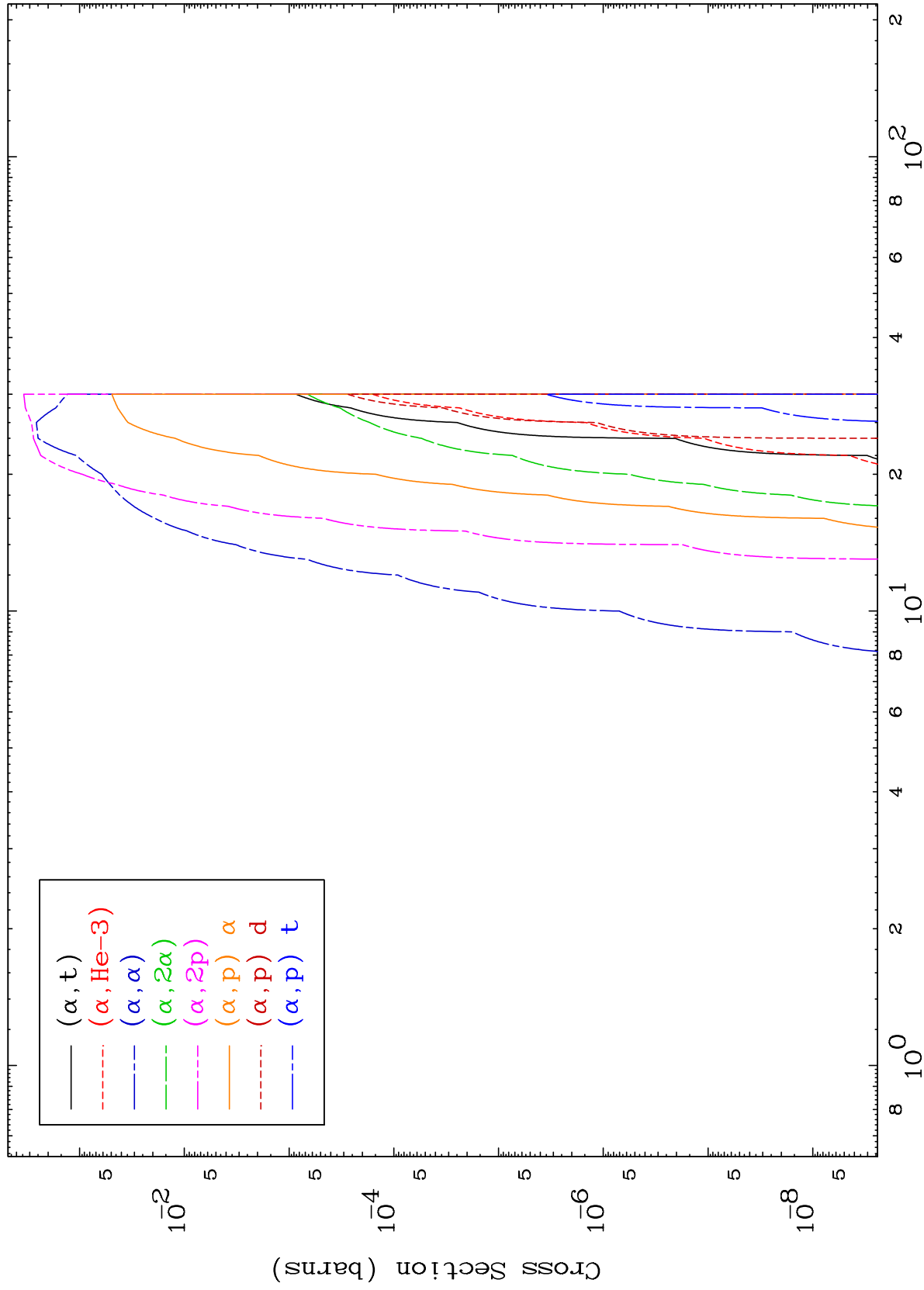


MAT 4819

$\alpha$  Charged Particle

48-Cd-104

0 Kelvin Cross Sections



4

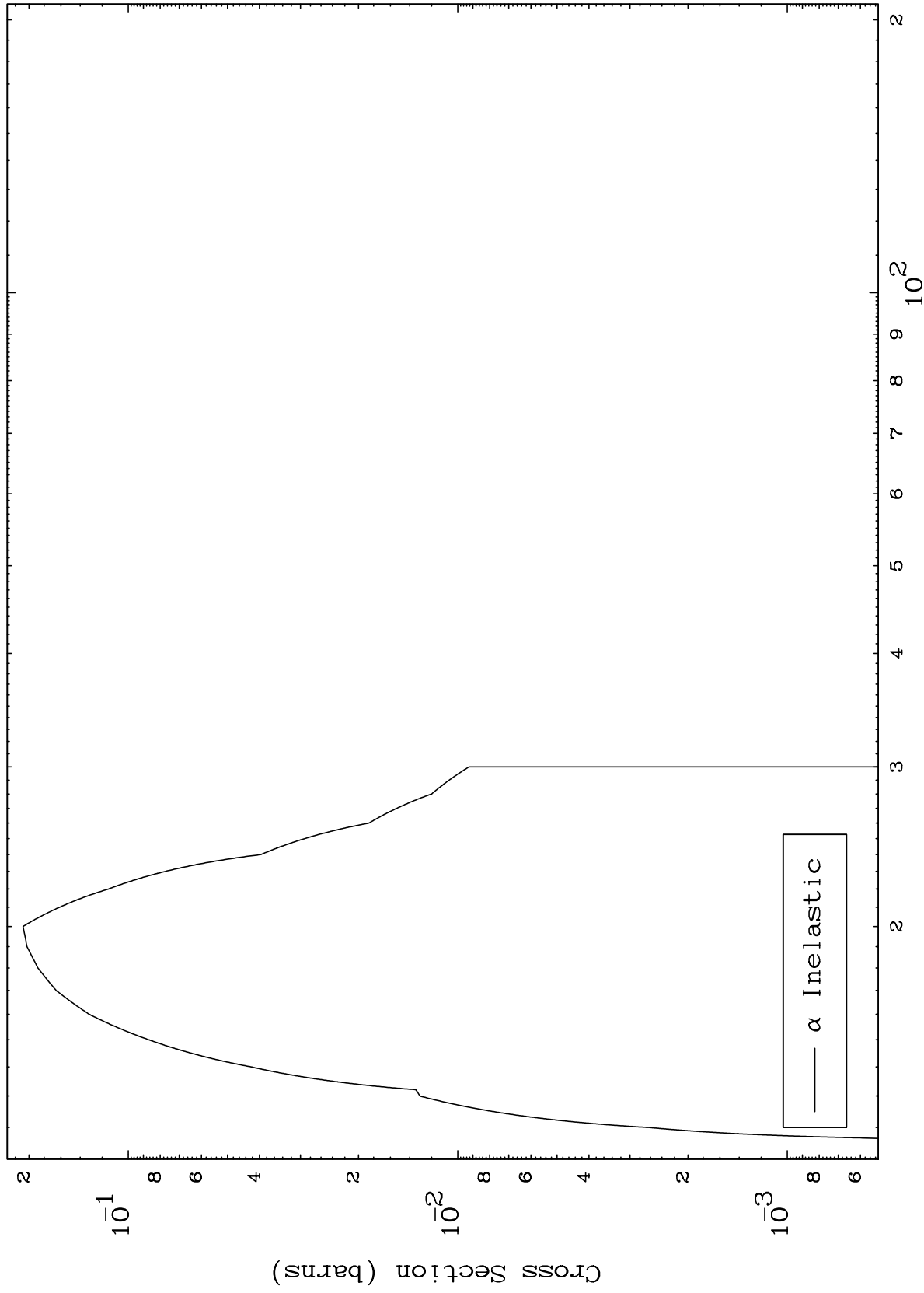
Incident Energy (MeV)

48-Cd-104

MAT 4819

48-Cd-104

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections



5

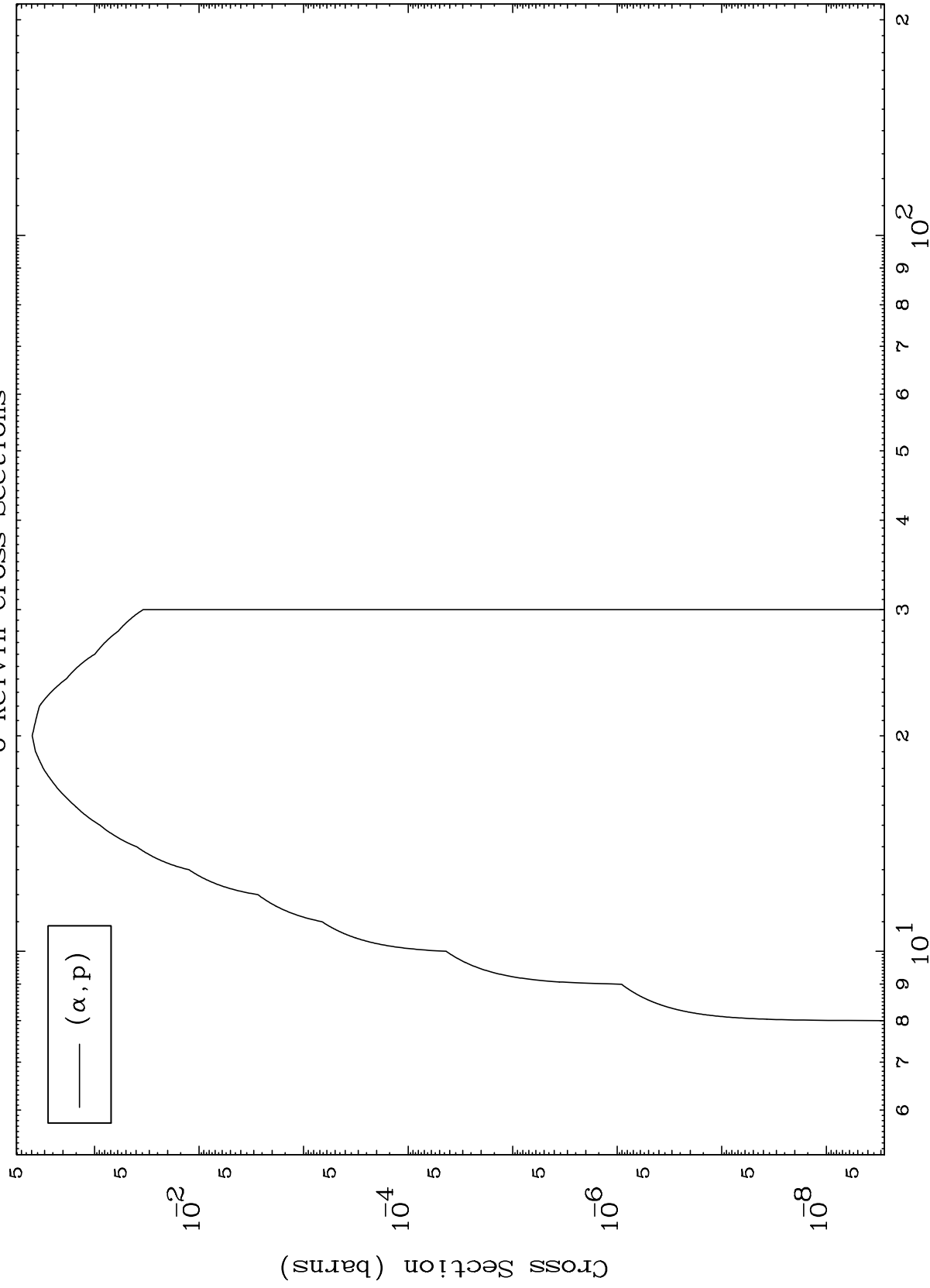
Incident Energy (MeV)

48-Cd-104

MAT 4819

48-Cd-104

( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections



6

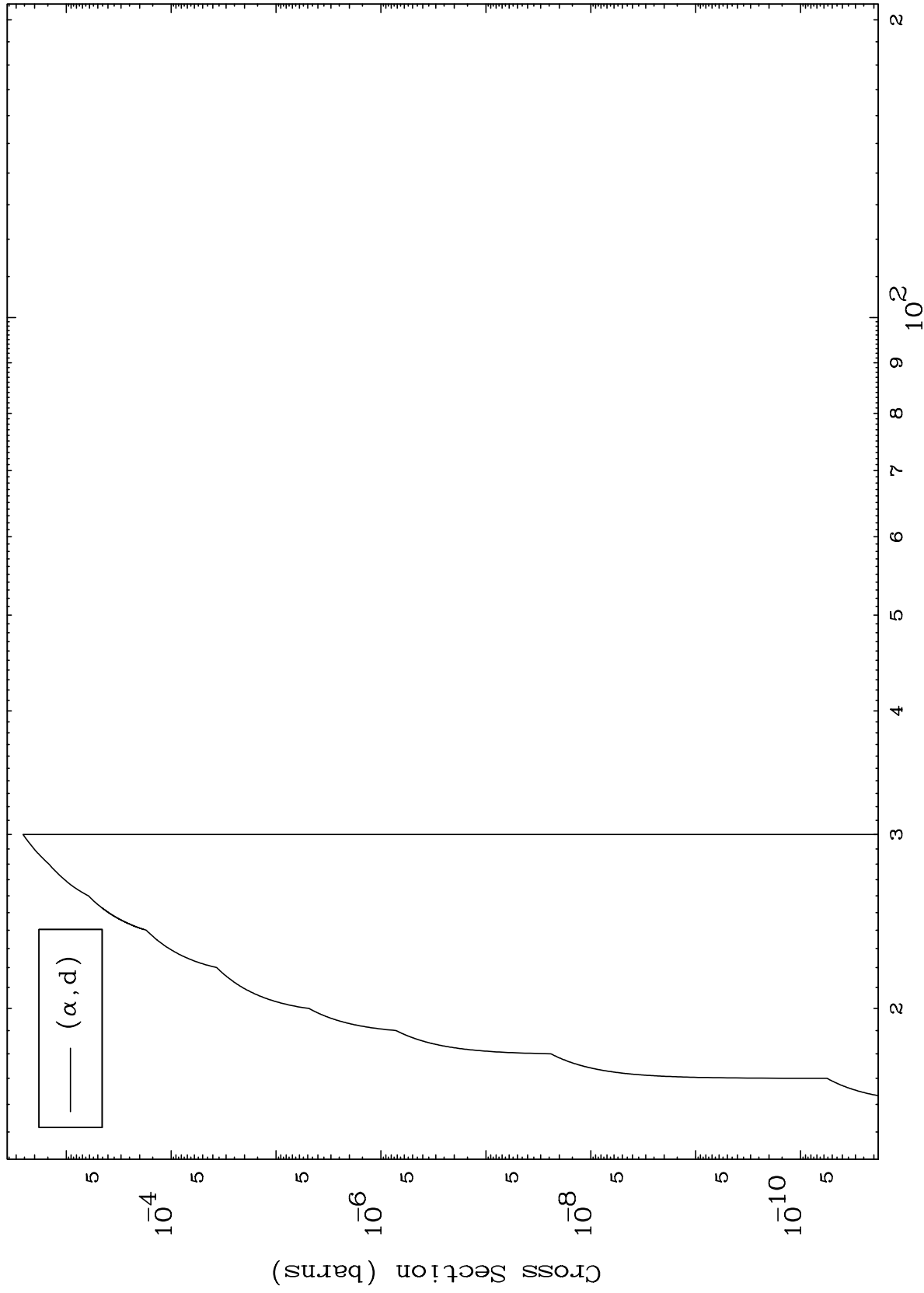
Incident Energy (MeV)

48-Cd-104

MAT 4819

48-Cd-104

( $\alpha$ ,d) Levels  
0 Kelvin Cross Sections



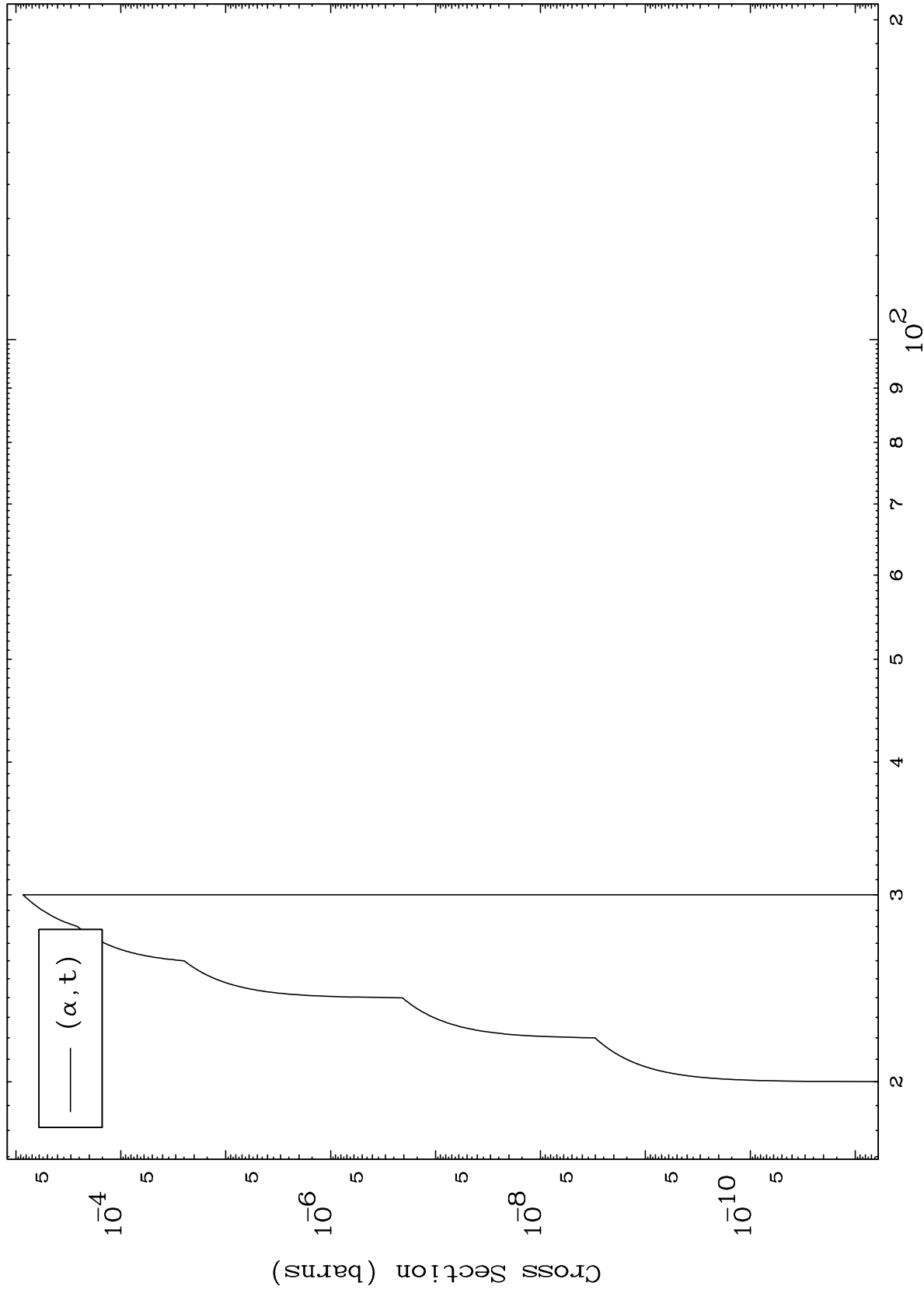
7



MAT 4819

48-Cd-104

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



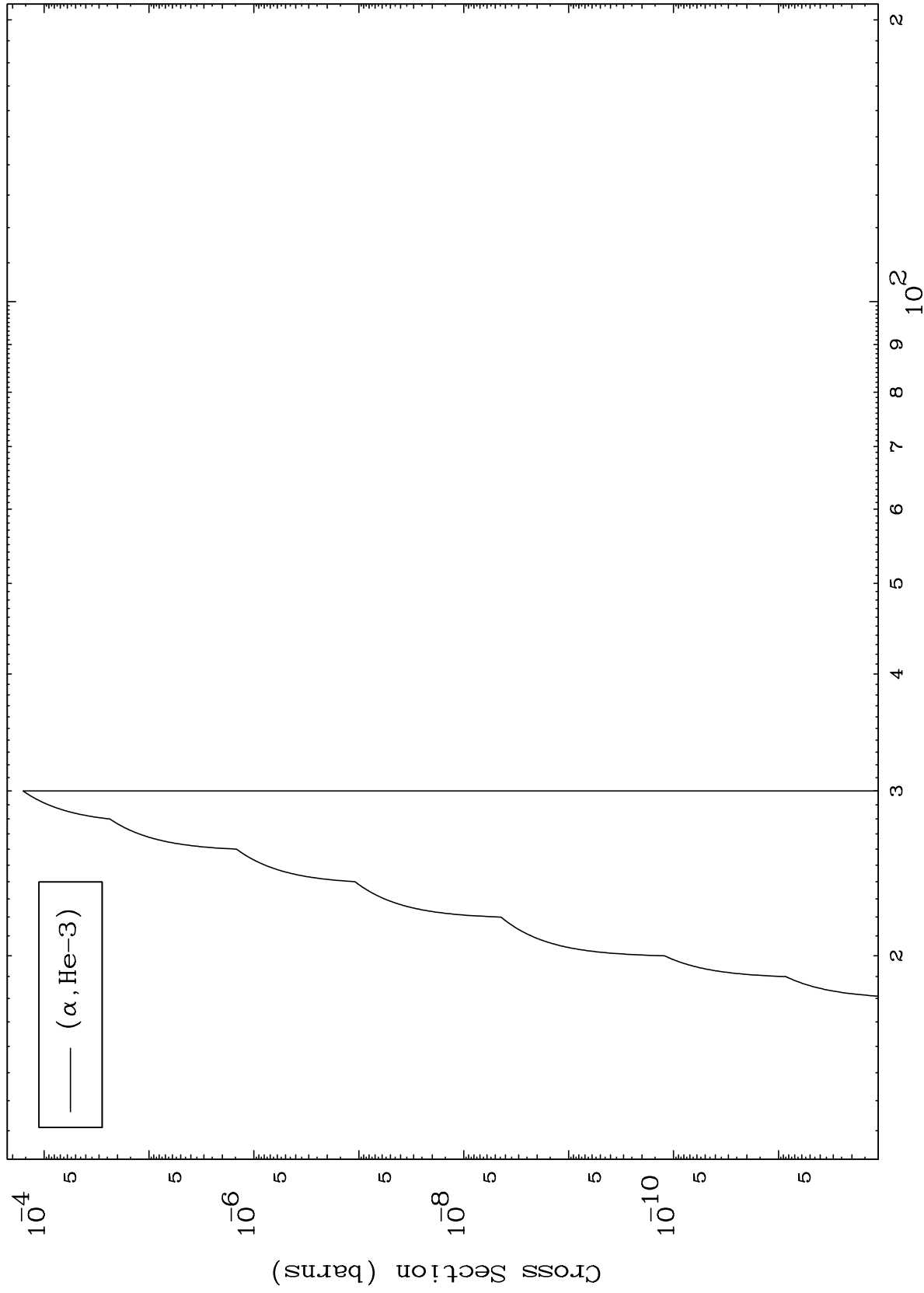
8

48-Cd-104

MAT 4819

( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections

48-Cd-104



9

Incident Energy (MeV)

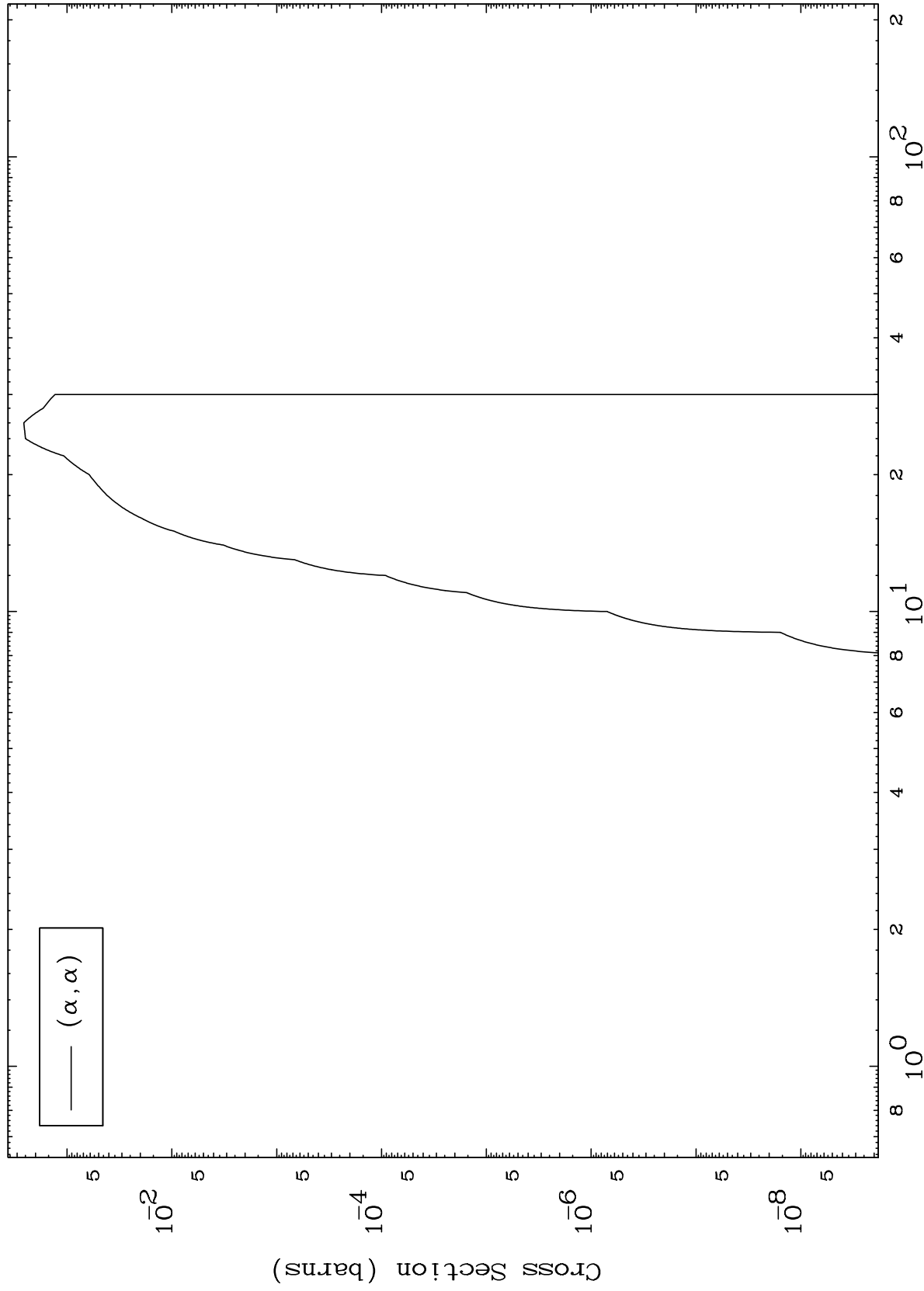
48-Cd-104

MAT 4819

( $\alpha, \alpha$ ) Levels

48-Cd-104

0 Kelvin Cross Sections



Incident Energy (MeV)

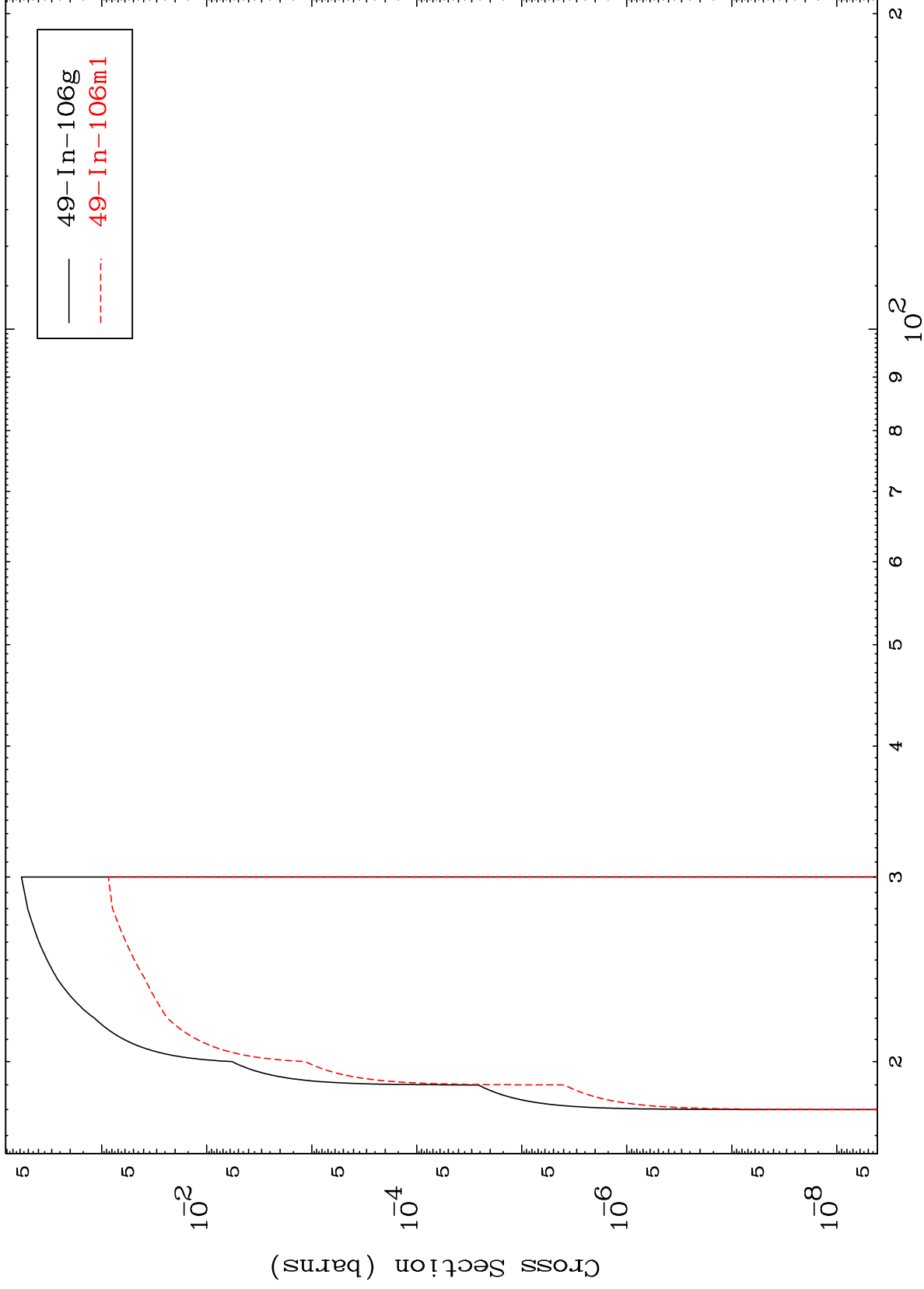
48-Cd-104

MAT 4819

$(\alpha, n')$  p

48-Cd-104

Radionuclide Production Cross Section



11

Incident Energy (MeV)

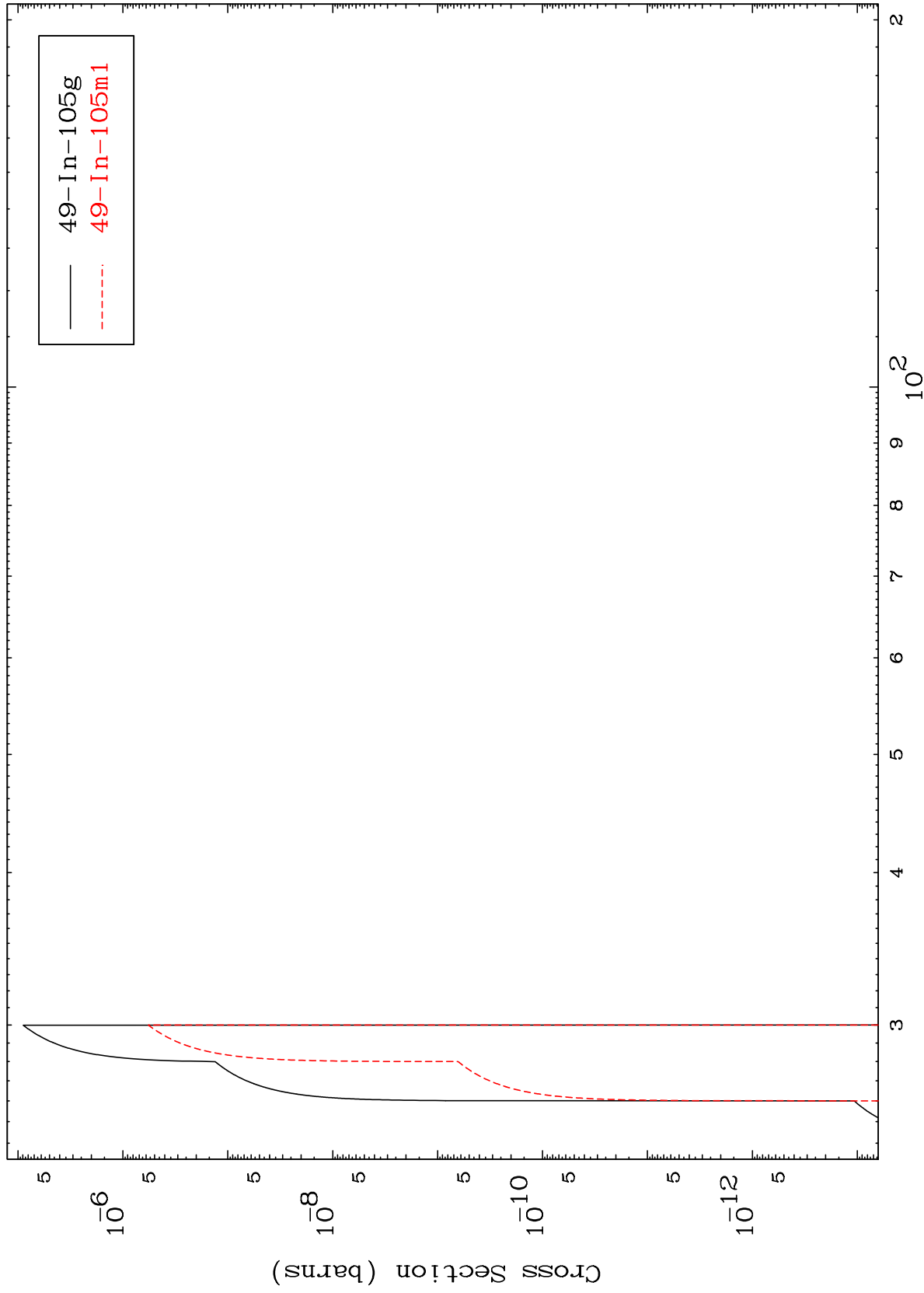
48-Cd-104

MAT 4819

$(\alpha, n')$  d

48-Cd-104

Radionuclide Production Cross Section



12

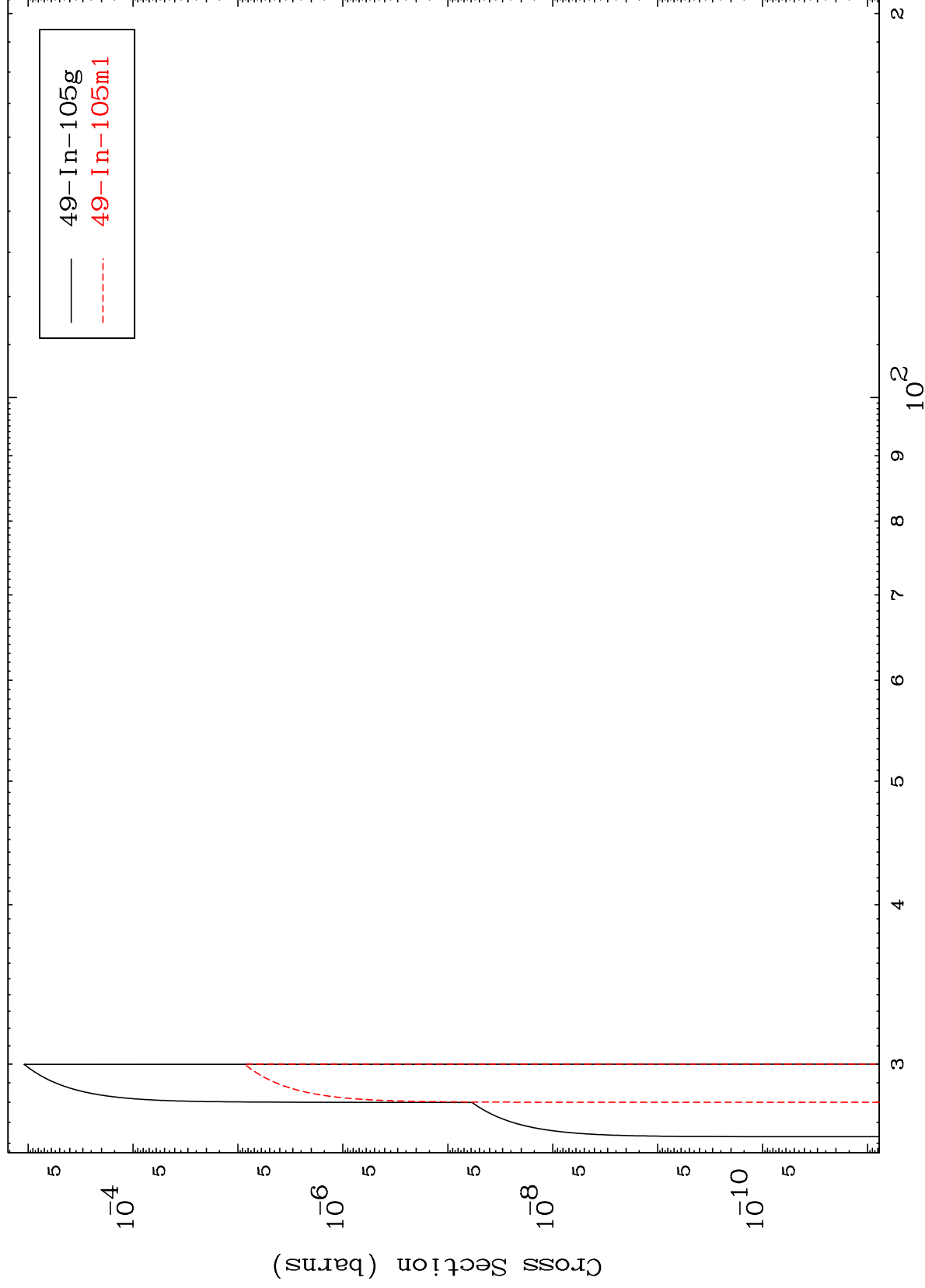
Incident Energy (MeV)

48-Cd-104

MAT 4819

$(\alpha, 2n)$  p  
Radionuclide Production Cross Section

48-Cd-104



13

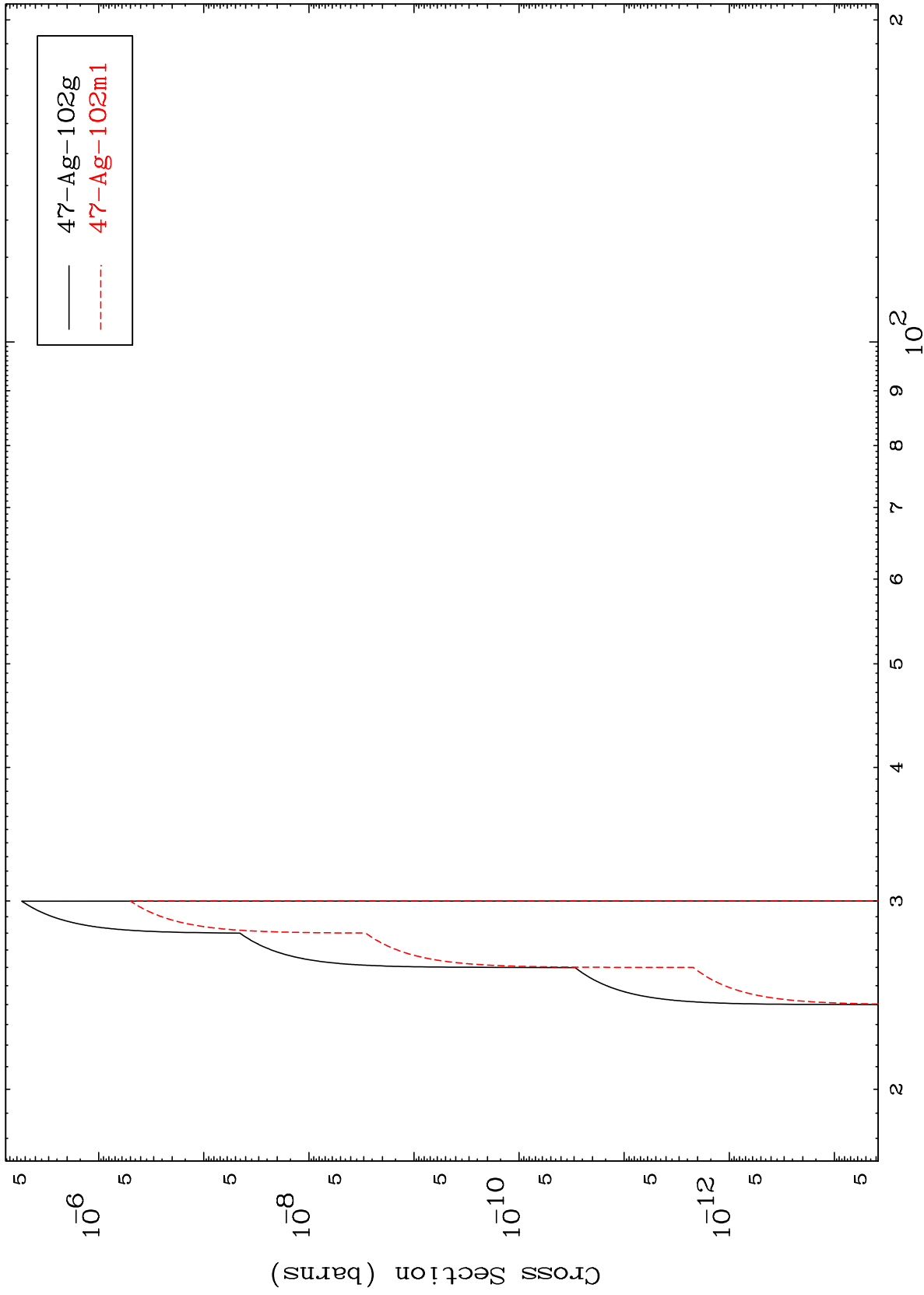
Incident Energy (MeV)

48-Cd-104

MAT 4819

48-Cd-104

$(\alpha, n')$  p  $\alpha$   
Radionuclide Production Cross Section



14

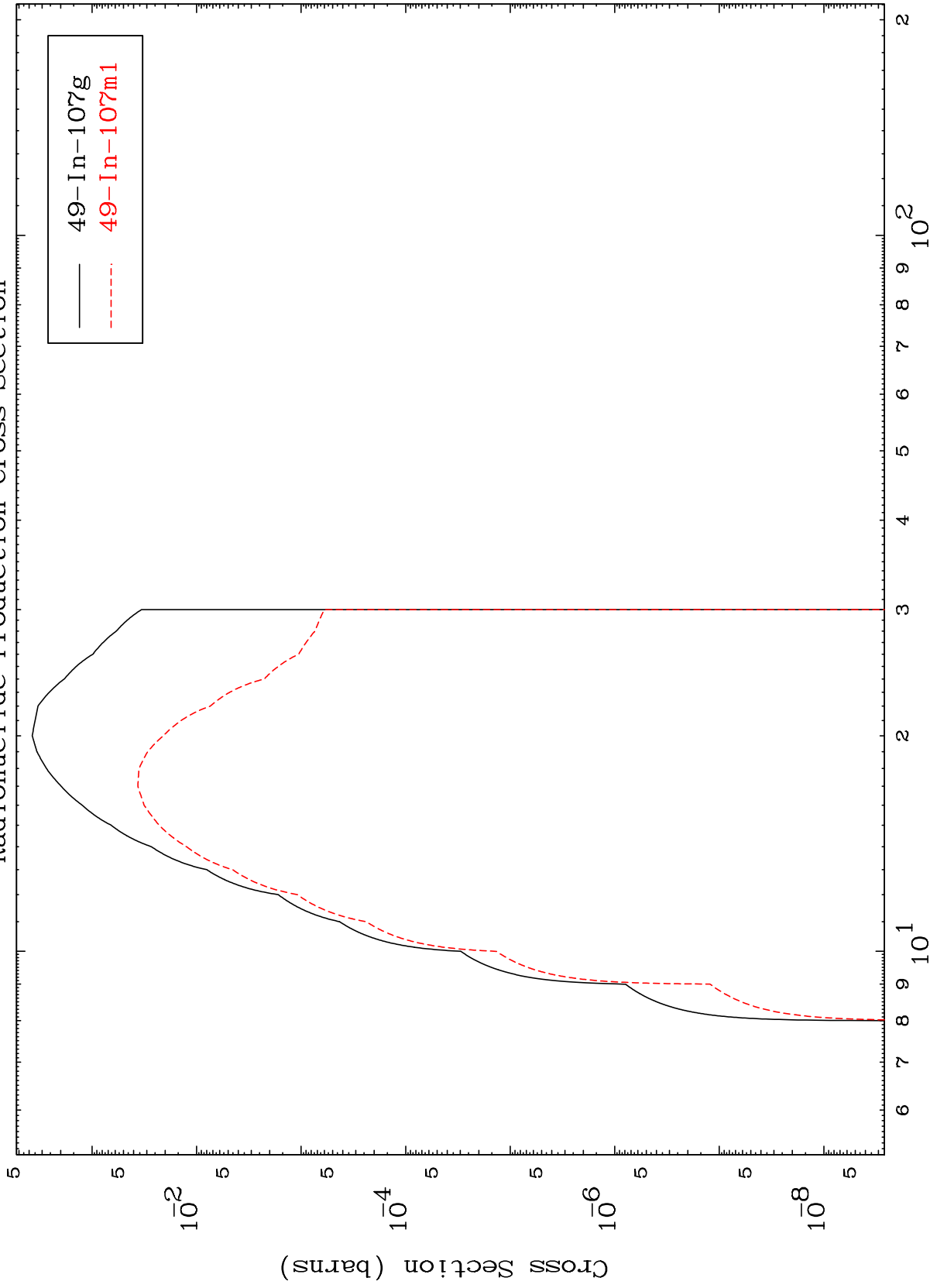
Incident Energy (MeV)

48-Cd-104

MAT 4819

48-Cd-104

$(\alpha, p)$   
Radionuclide Production Cross Section



15

Incident Energy (MeV)

48-Cd-104

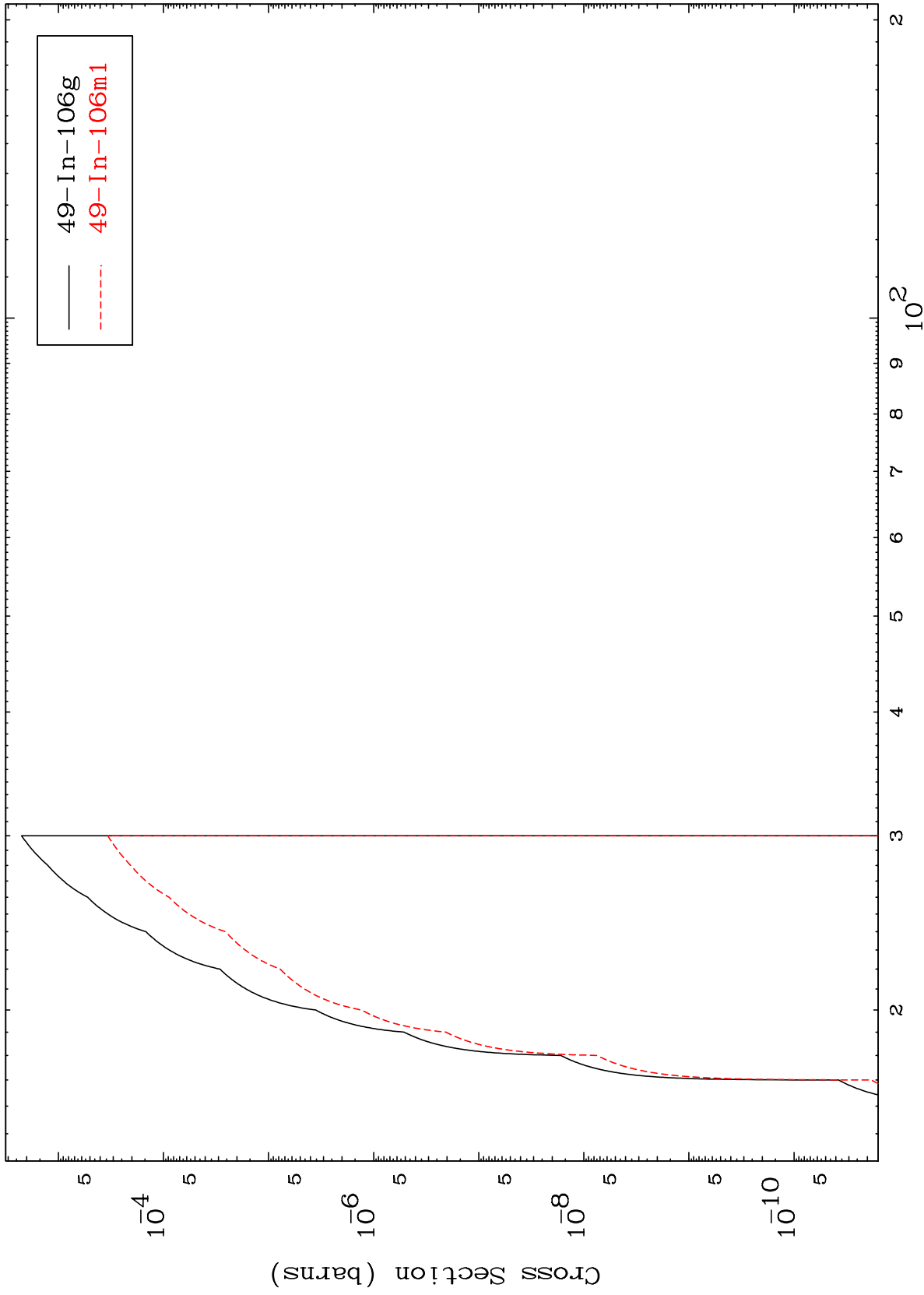


MAT 4819

( $\alpha, d$ )

Radionuclide Production Cross Section

48-Cd-104



16

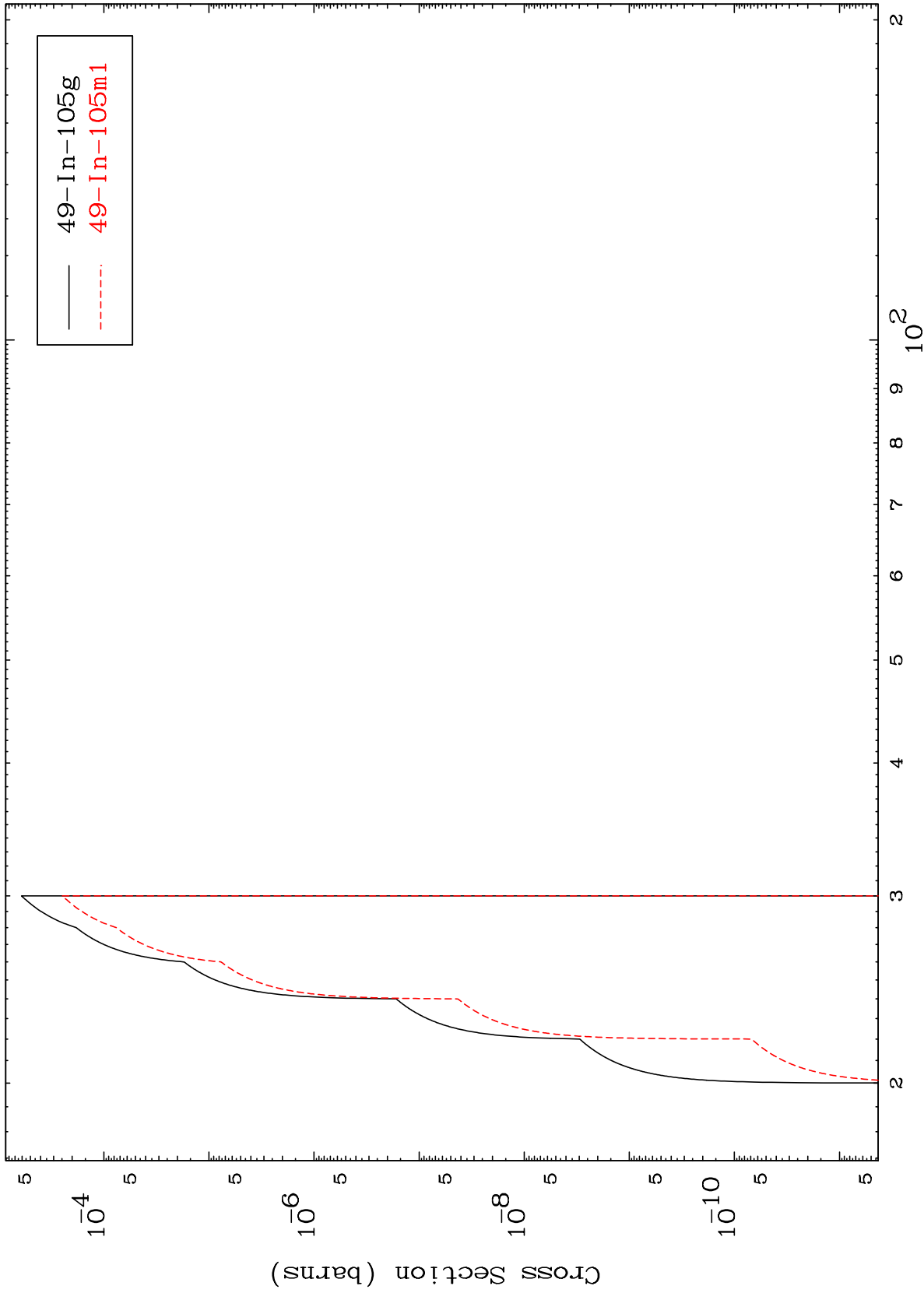
Incident Energy (MeV)

48-Cd-104

MAT 4819

48-Cd-104

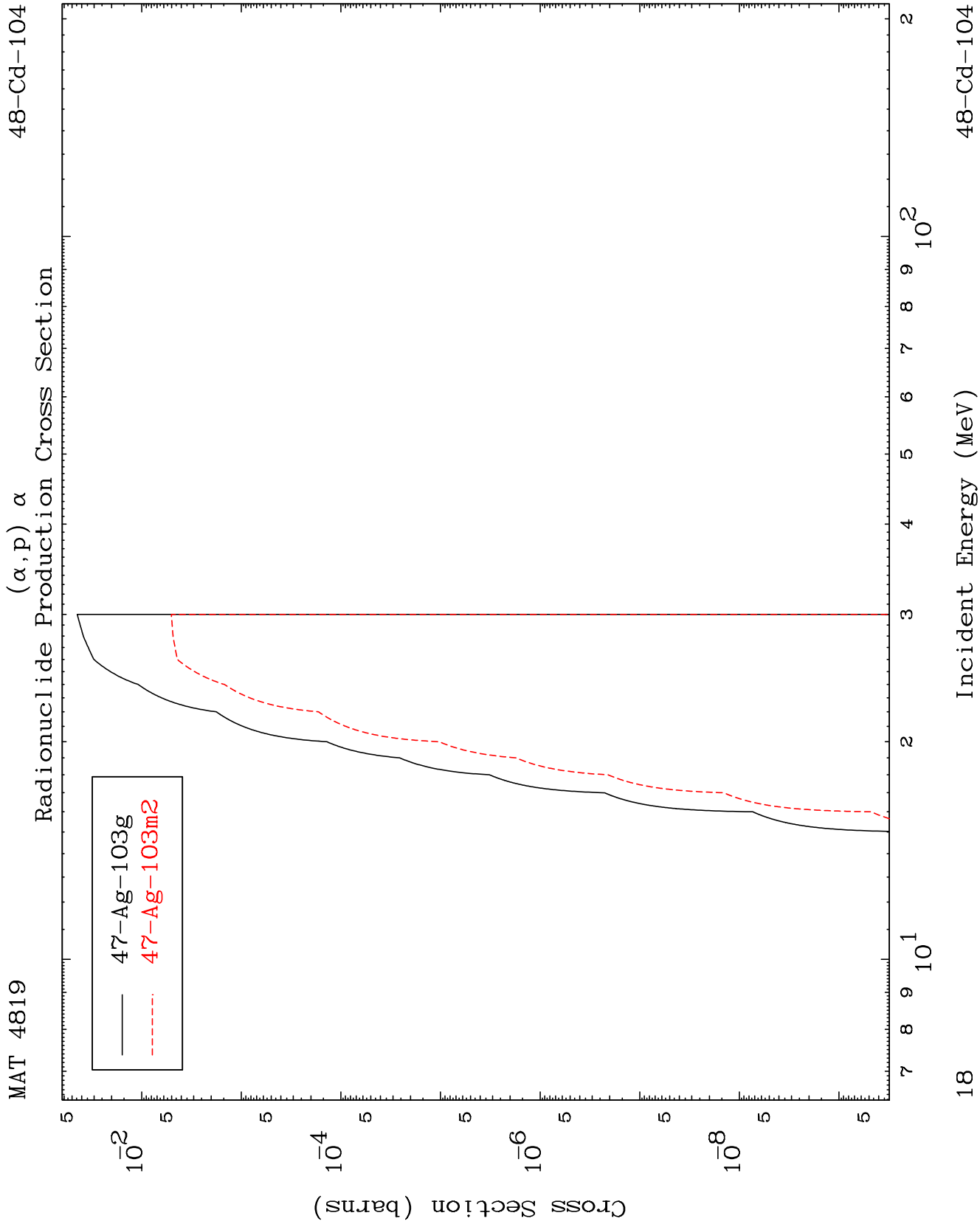
$(\alpha, t)$   
Radionuclide Production Cross Section



17

Incident Energy (MeV)

48-Cd-104

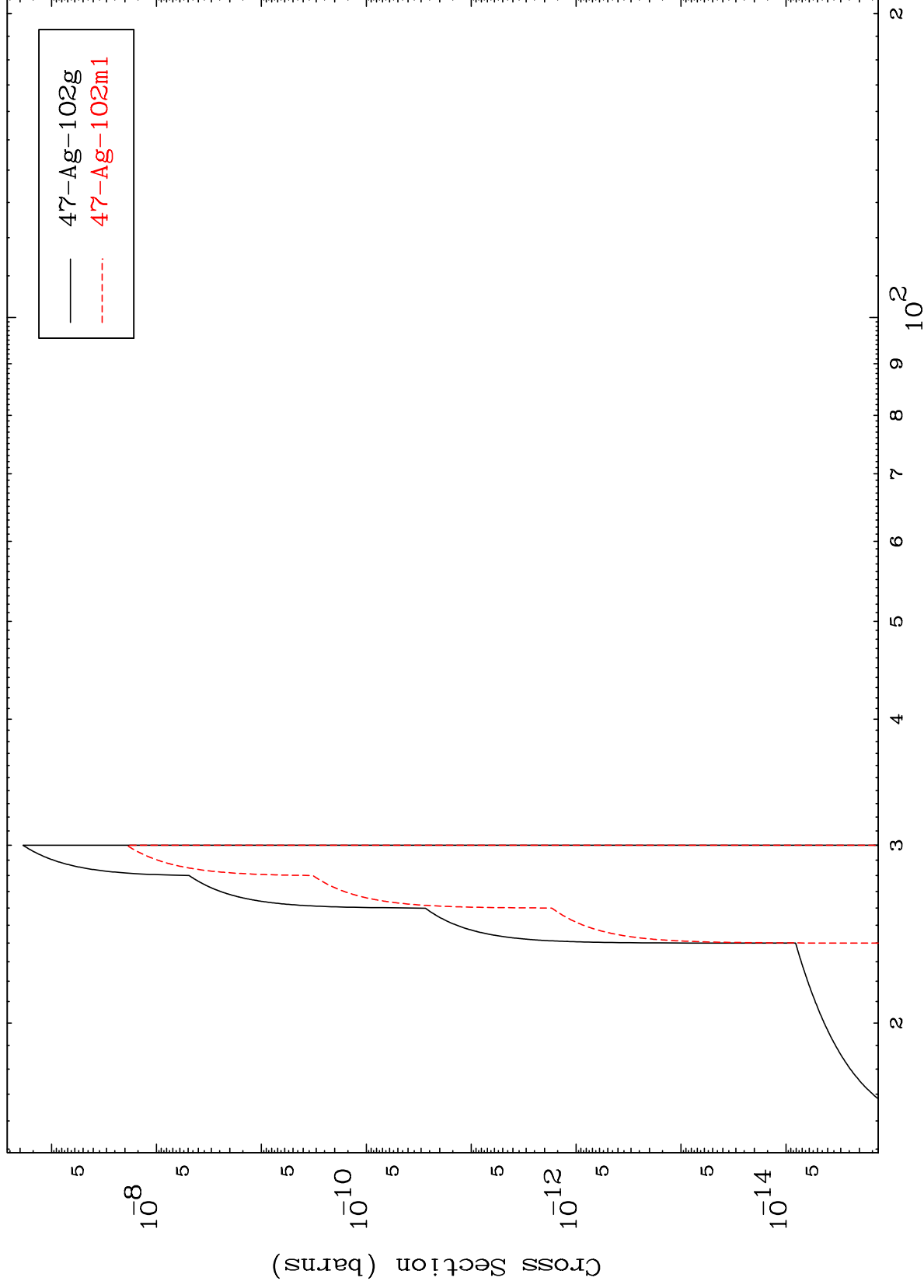


MAT 4819

( $\alpha$ ,d)  $\alpha$

48-Cd-104

Radionuclide Production Cross Section



19

Incident Energy (MeV)

48-Cd-104