

Program EVALPLOT  
(Version 2018-1)

by

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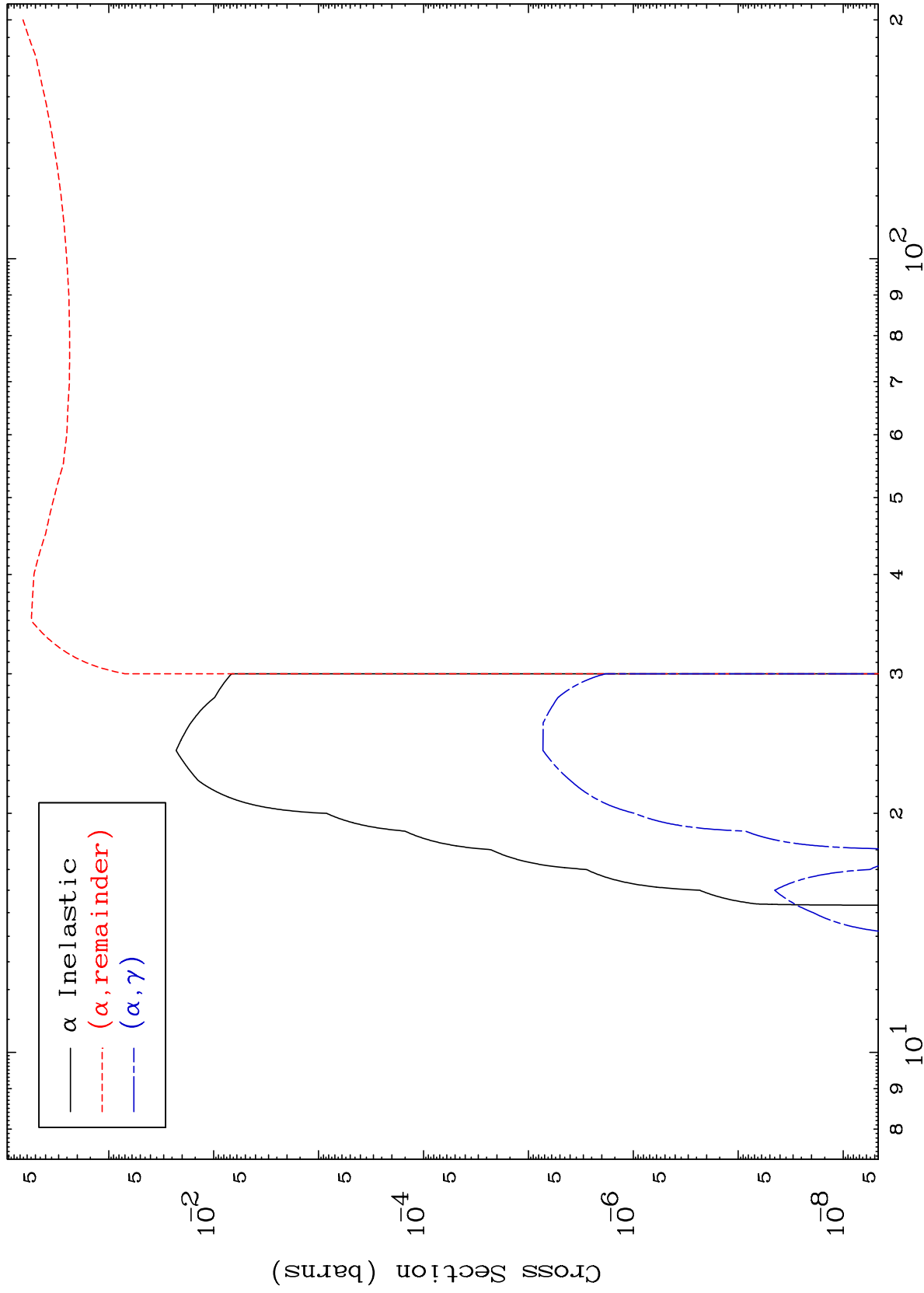
Press Mouse Button to Start

MAT 9843

$\alpha$  Major

0 Kelvin Cross Sections

98-Cf-246



1

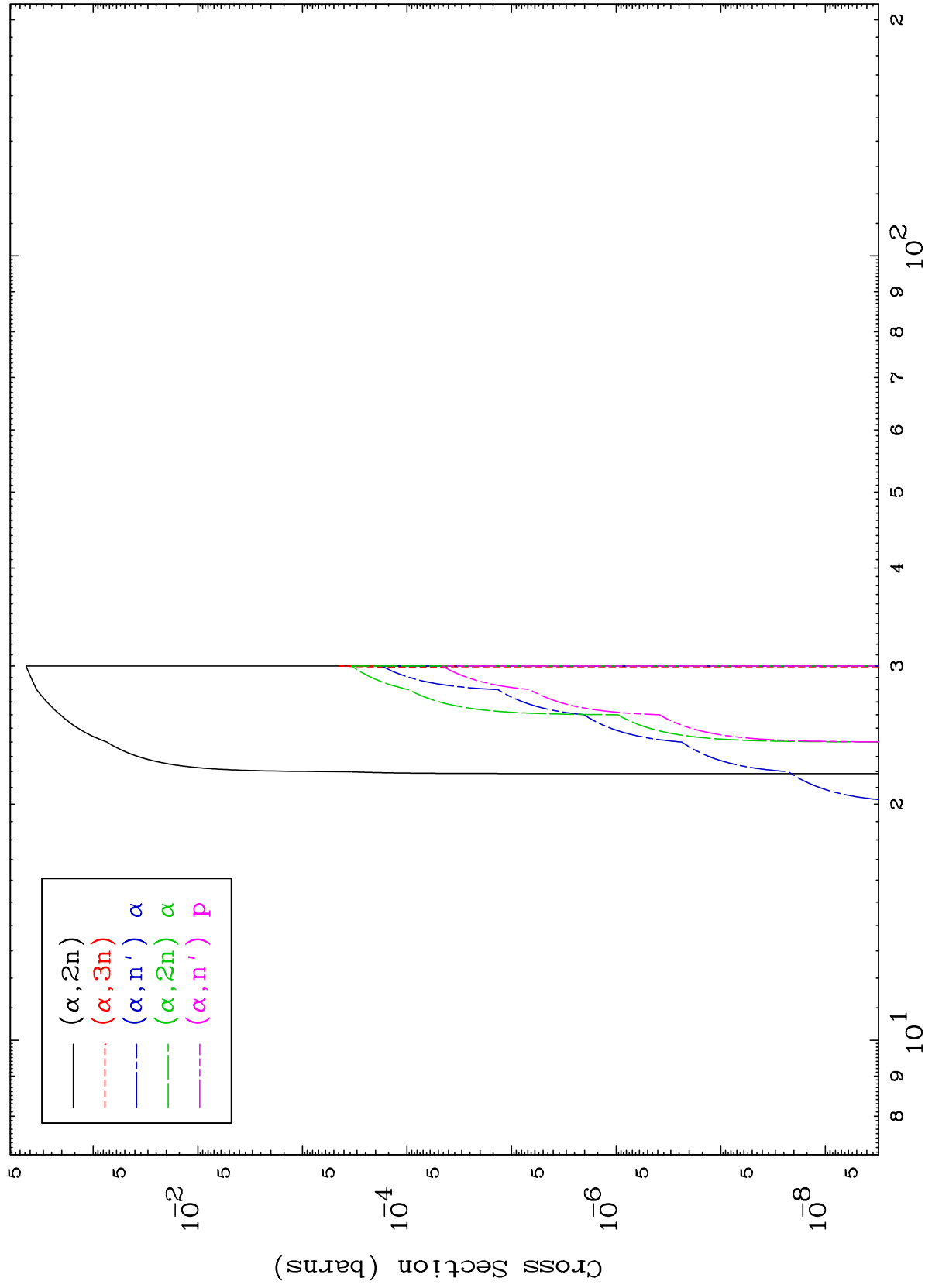
Incident Energy (MeV)

98-Cf-246

MAT 9843

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

98-Cf-246



2

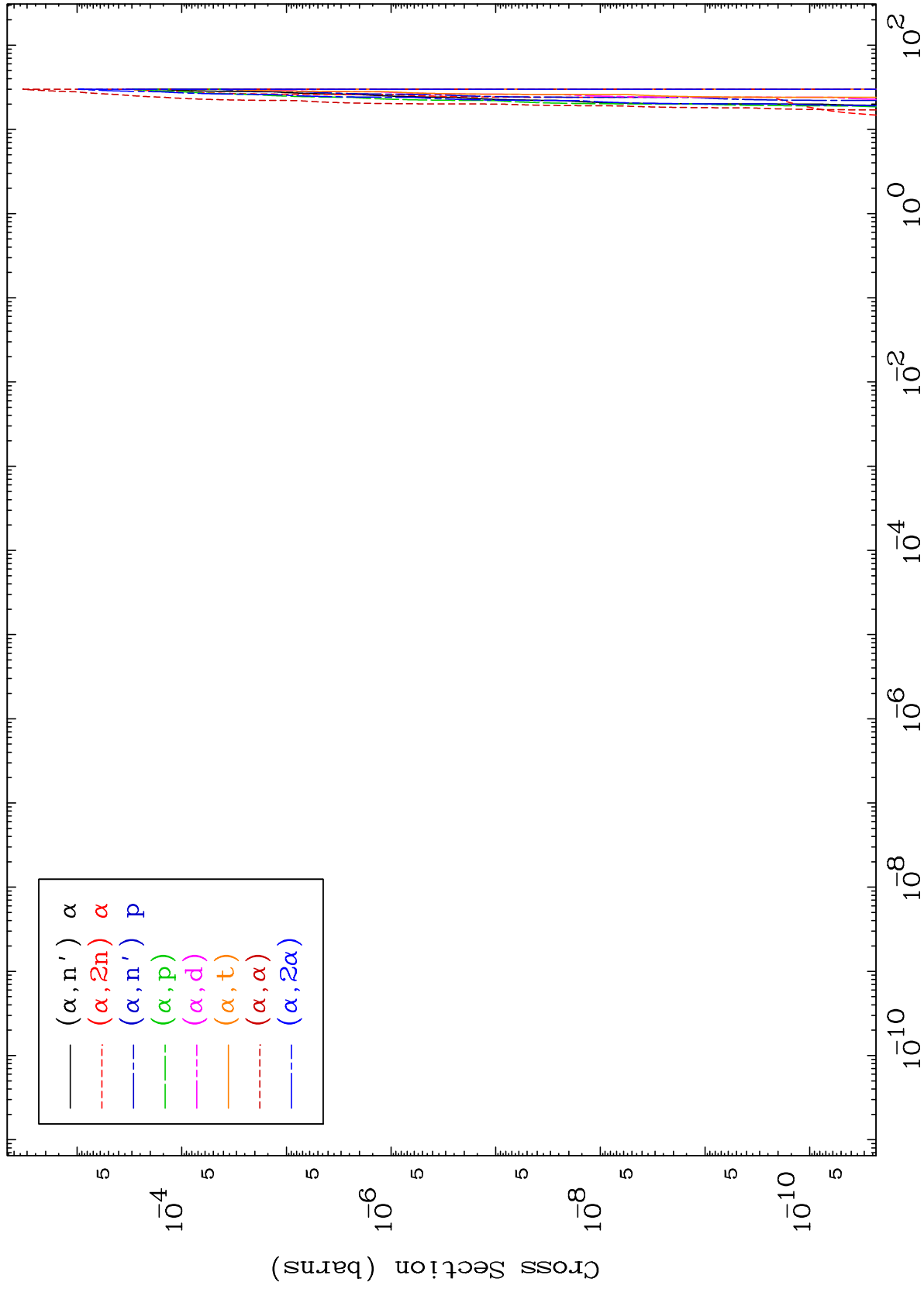
Incident Energy (MeV)

98-Cf-246

MAT 9843

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

98-Cf-246



3

Incident Energy (MeV)

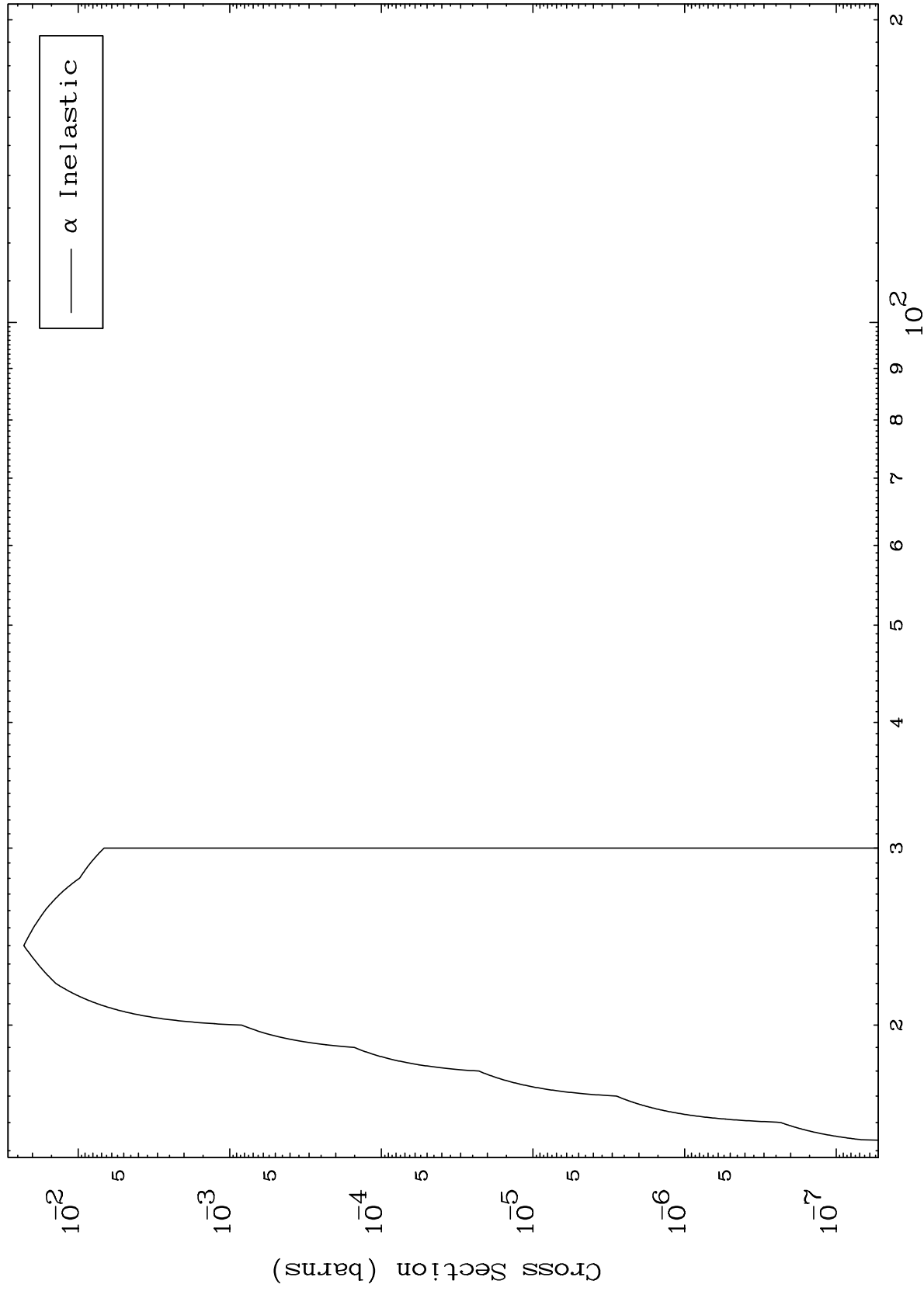
98-Cf-246

MAT 9843

( $\alpha, n'$ ) Level

98-Cf-246

0 Kelvin Cross Sections



4

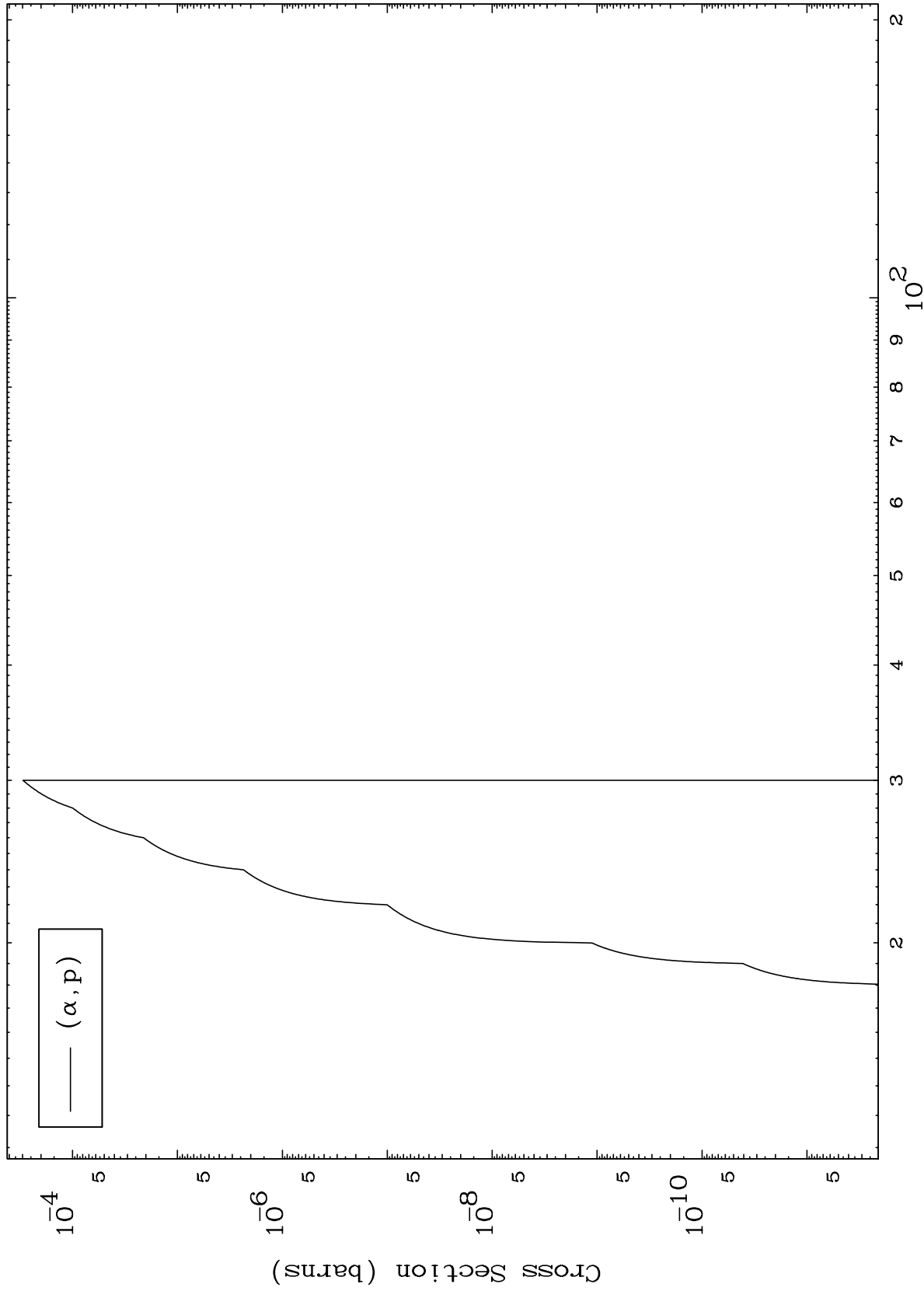
Incident Energy (MeV)

98-Cf-246

MAT 9843

98-Cf-246

( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections



5

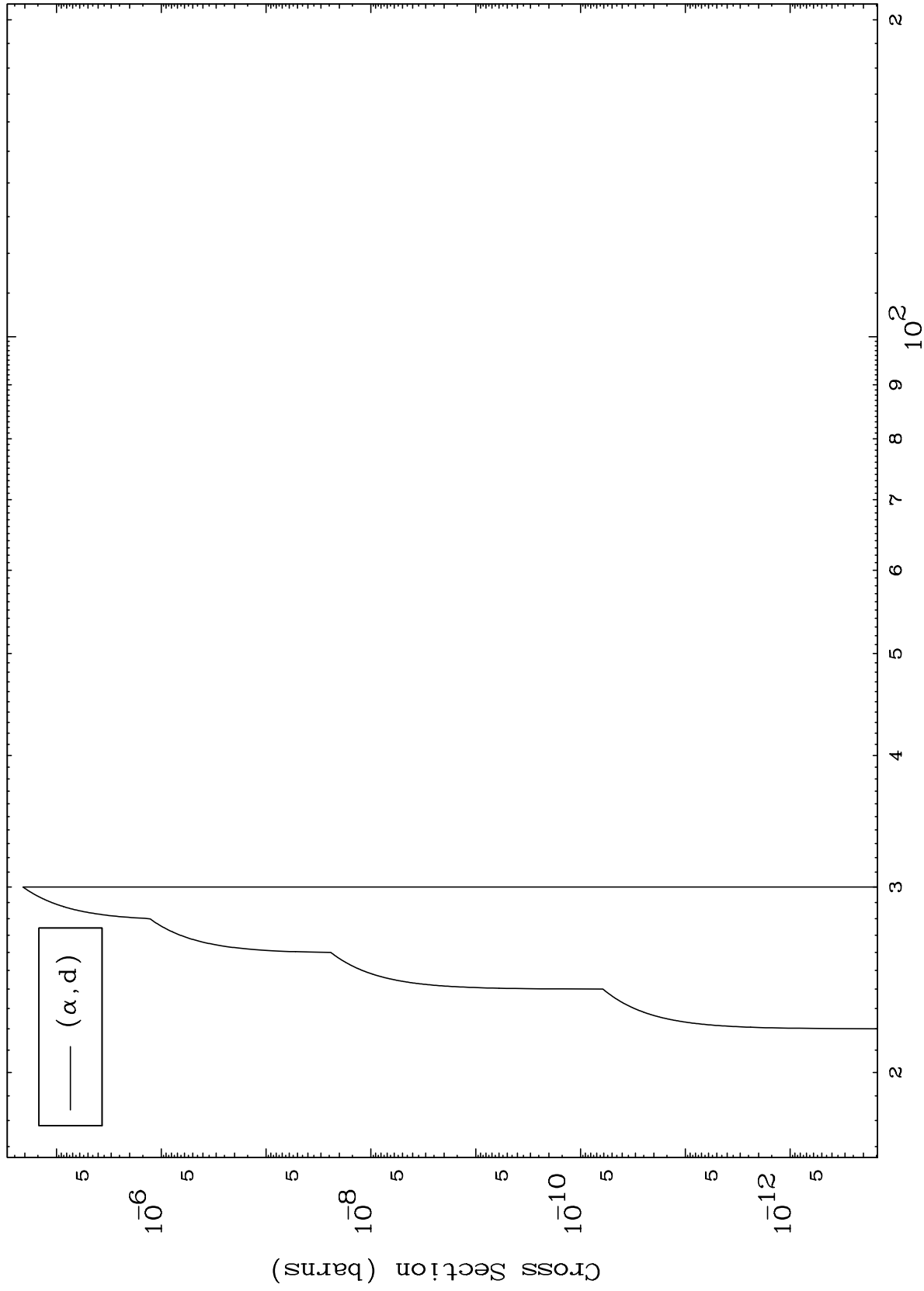
Incident Energy (MeV)

98-Cf-246

MAT 9843

( $\alpha$ ,d) Levels  
0 Kelvin Cross Sections

98-Cf-246



6

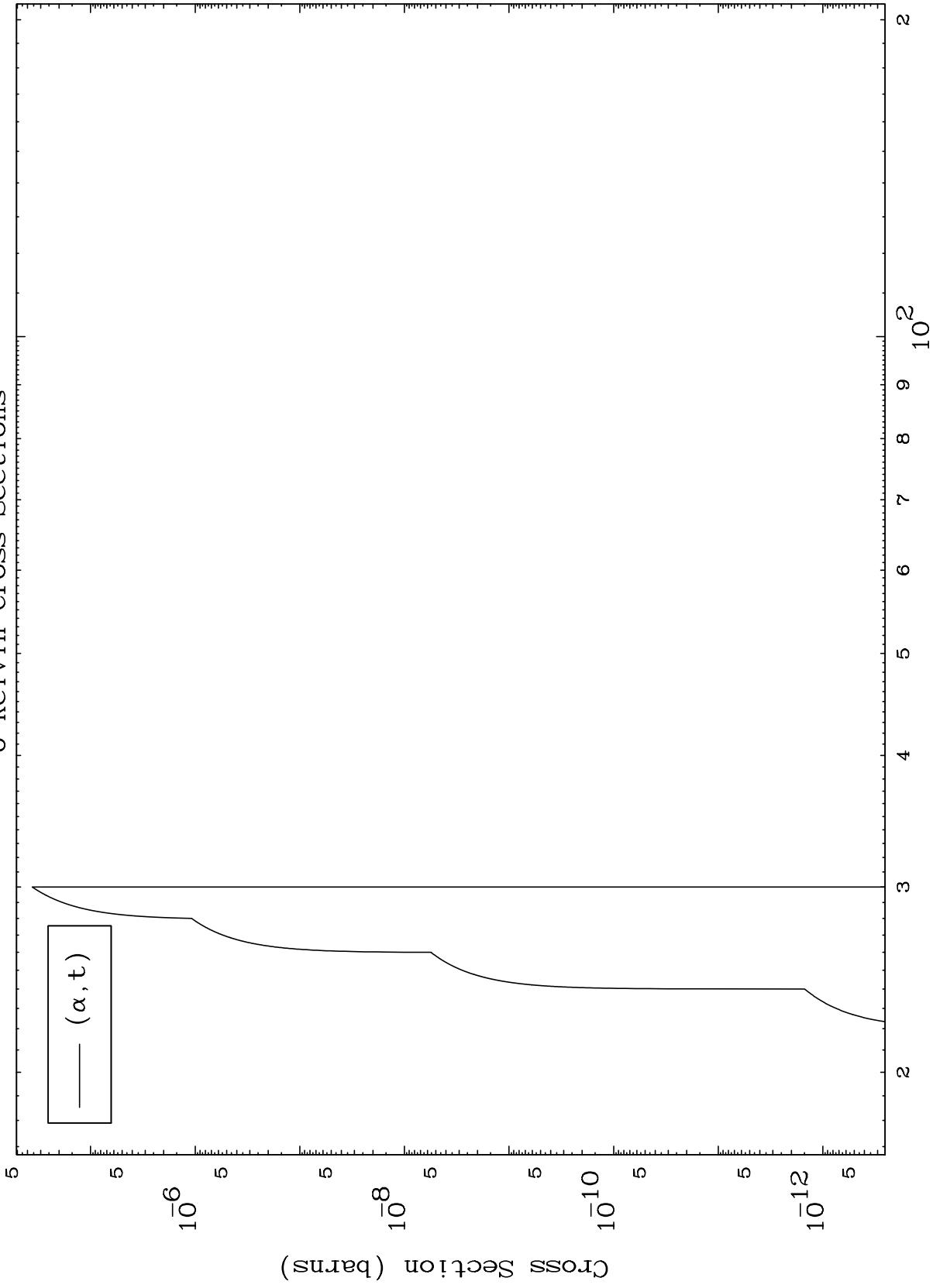
Incident Energy (MeV)

98-Cf-246

MAT 9843

98-Cf-246

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



7

Incident Energy (MeV)

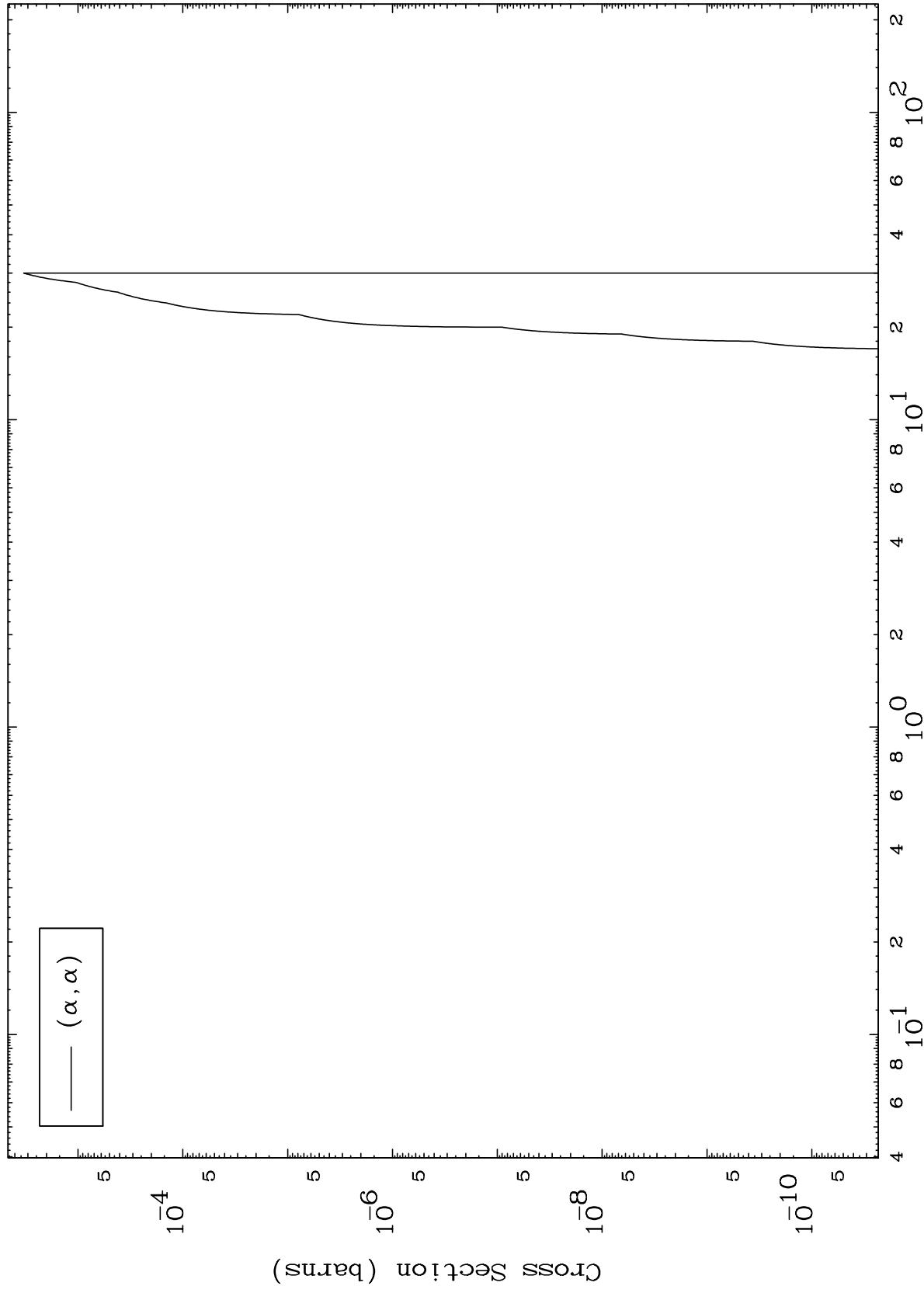
98-Cf-246

MAT 9843

( $\alpha, \alpha$ ) Levels

98-Cf-246

0 Kelvin Cross Sections



8

Incident Energy (MeV)

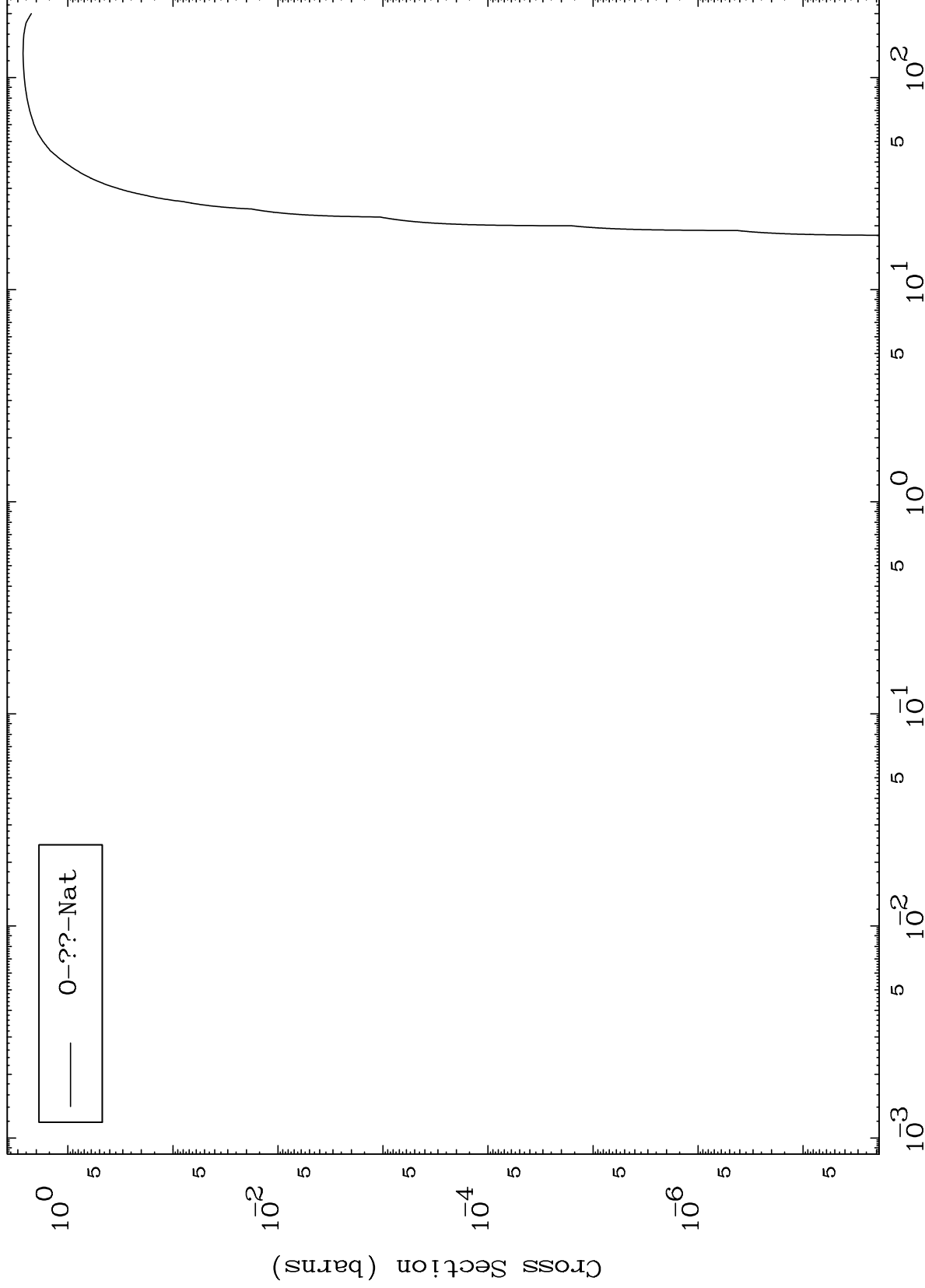
98-Cf-246

MAT 9843

$\alpha$  Fission

98-Cf-246

Radionuclide Production Cross Section



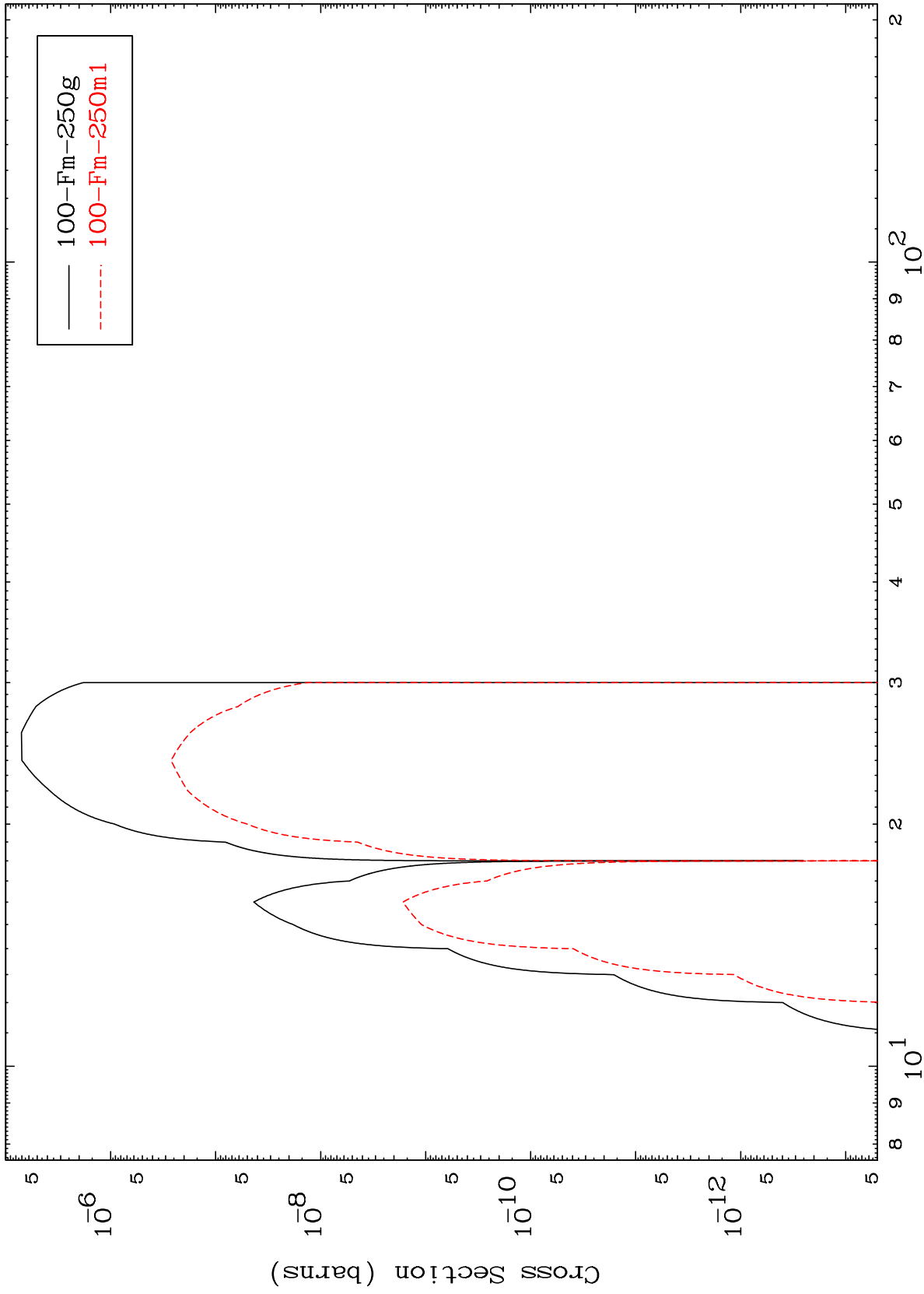
Incident Energy (MeV)

98-Cf-246

MAT 9843

98-Cf-246

$(\alpha, \gamma)$   
Radionuclide Production Cross Section



98-Cf-246

Incident Energy (MeV)