

Program EVALPLOT
(Version 2018-1)

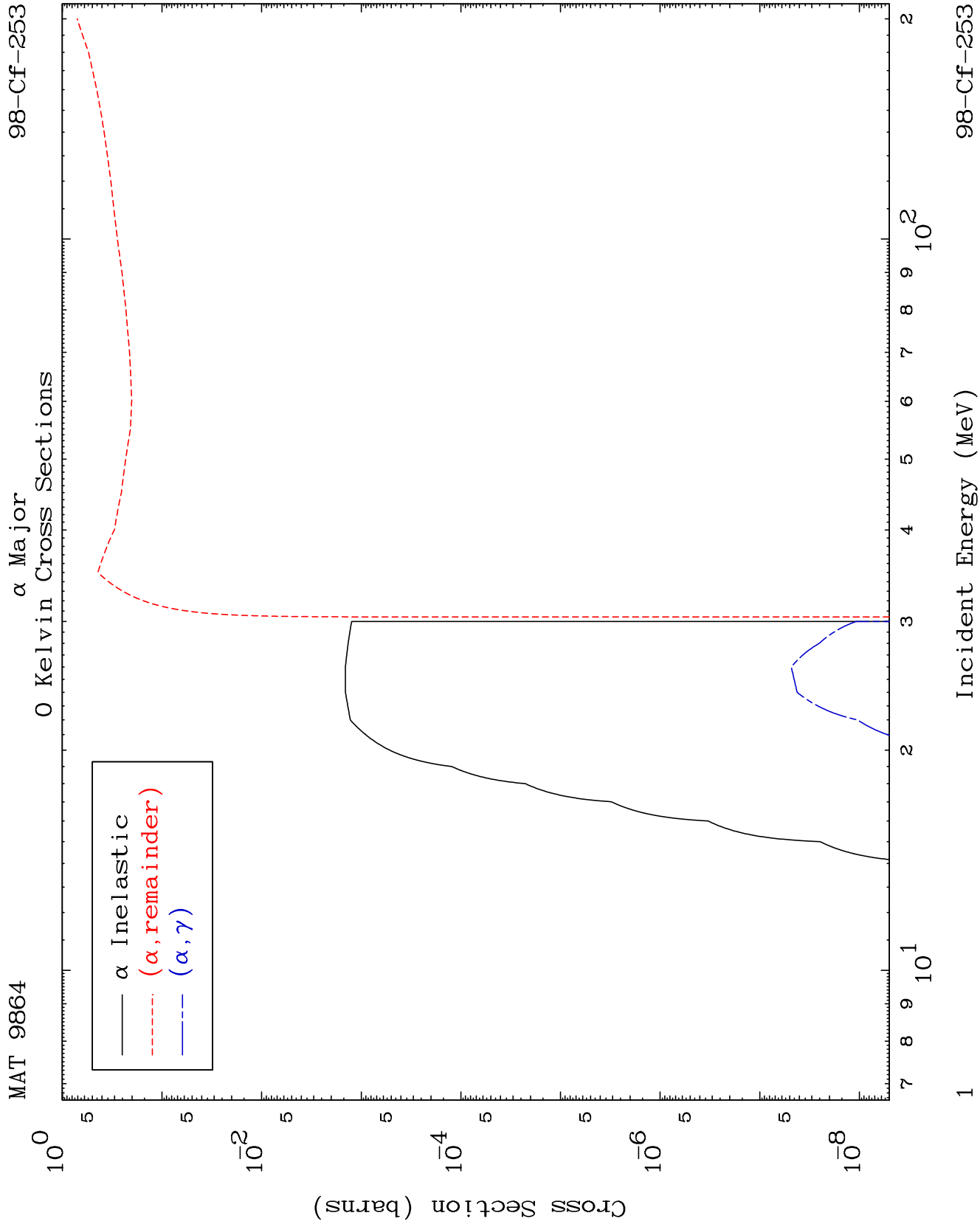
by

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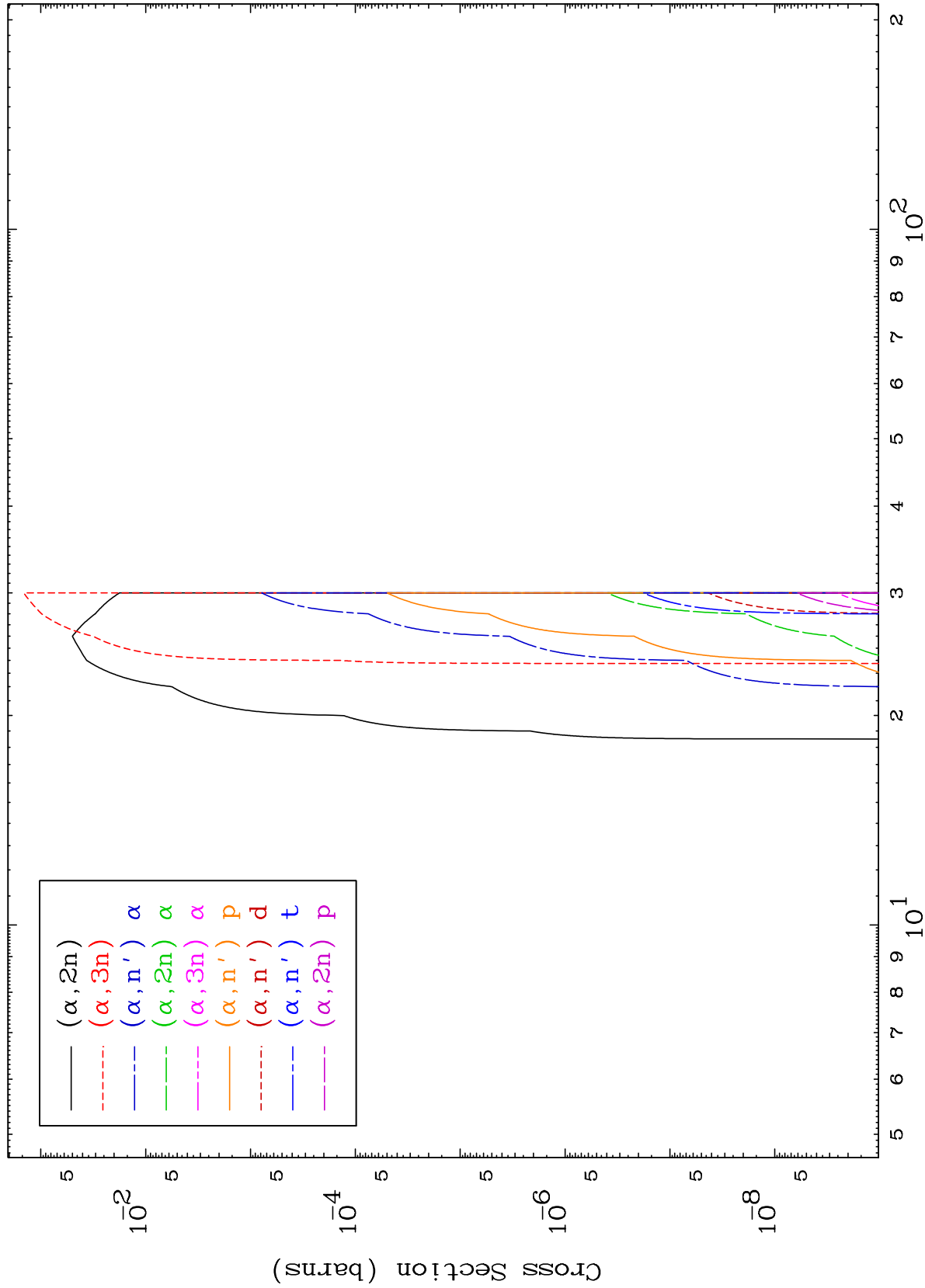
Press Mouse Button to Start



MAT 9864

α Neutron Production
0 Kelvin Cross Sections

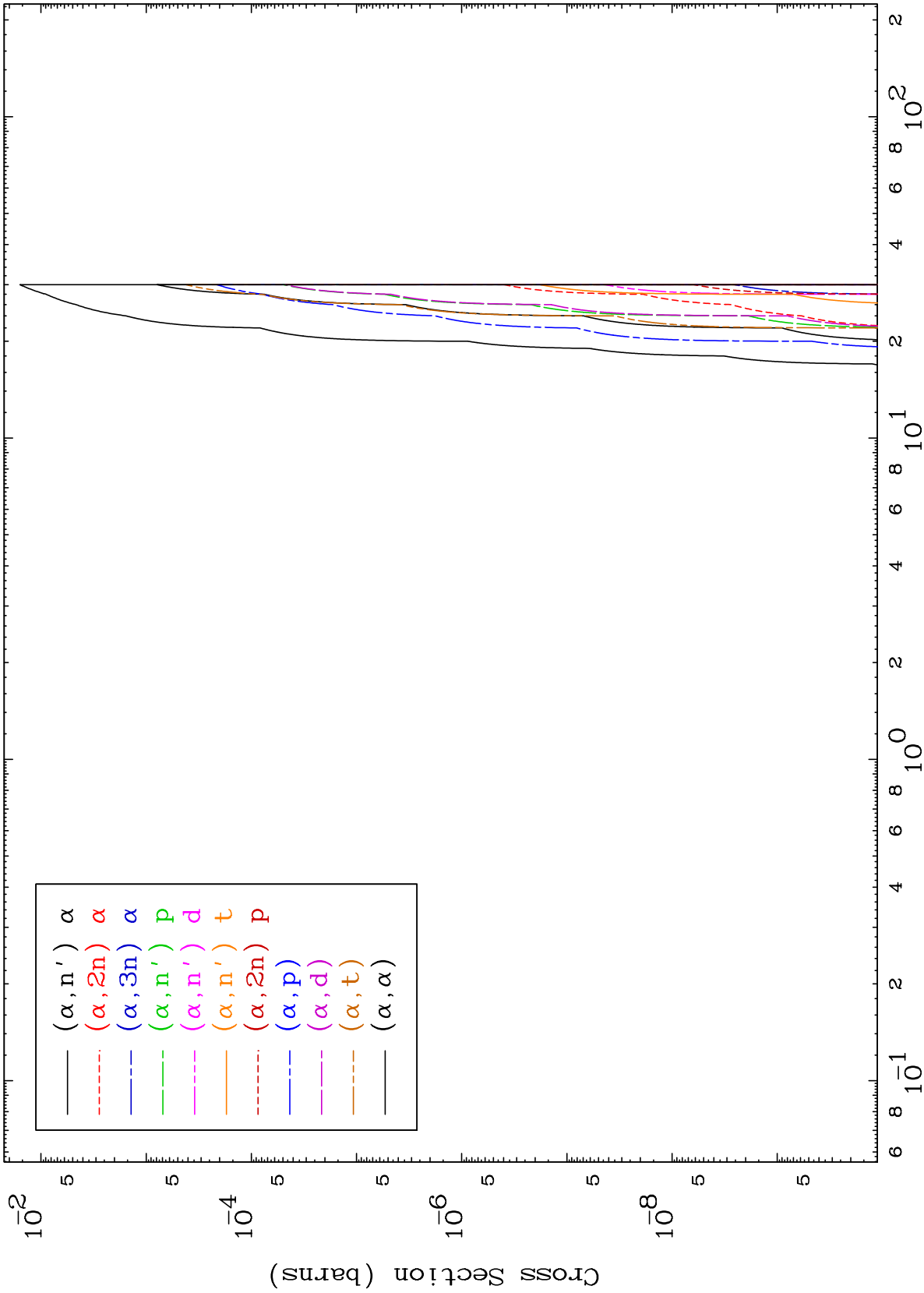
98-Cf-253



MAT 9864

α Charged Particle
0 Kelvin Cross Sections

98-Cf-253



3

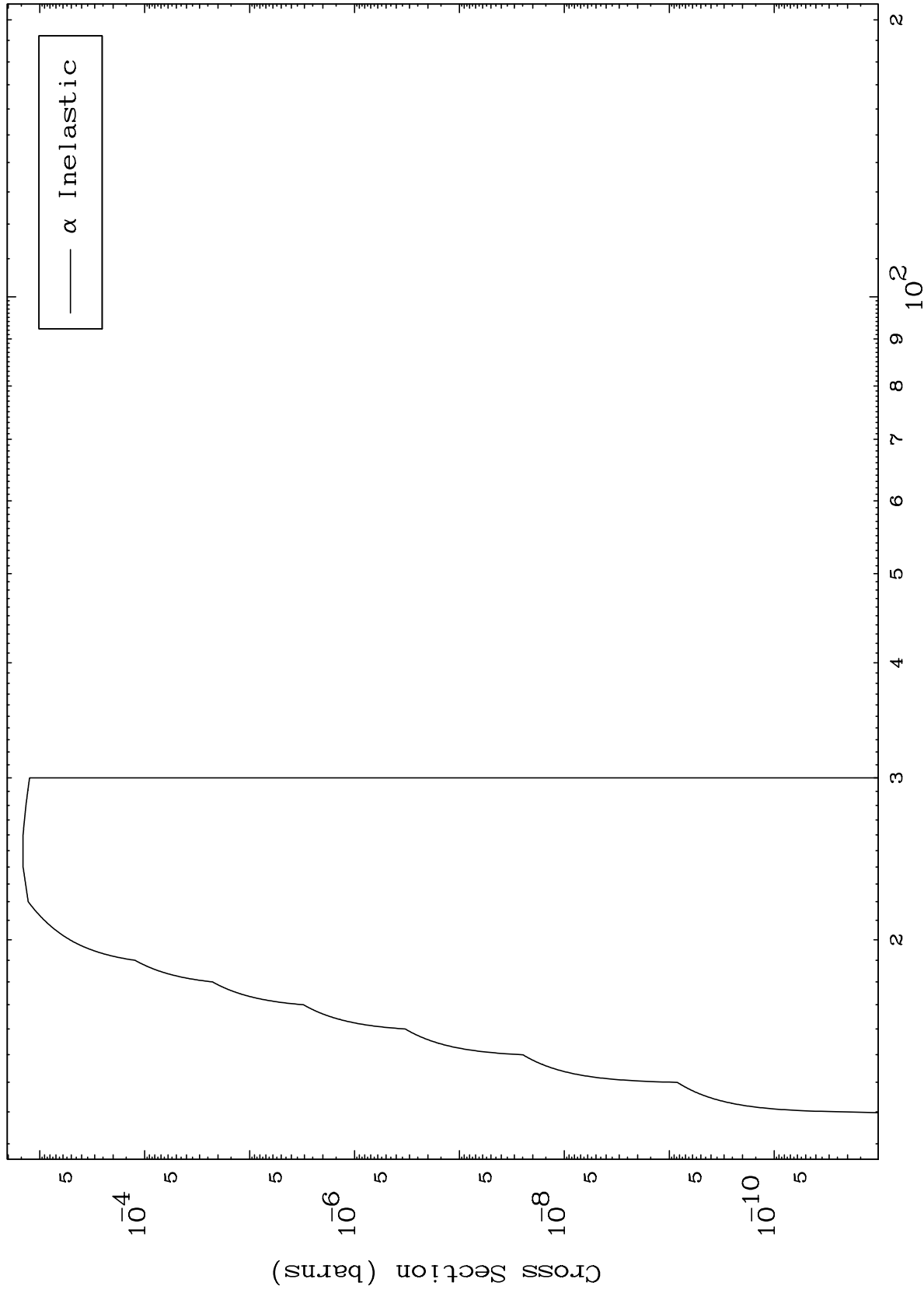
Incident Energy (MeV)

98-Cf-253

MAT 9864

(α, n') Level
0 Kelvin Cross Sections

98-Cf-253



4

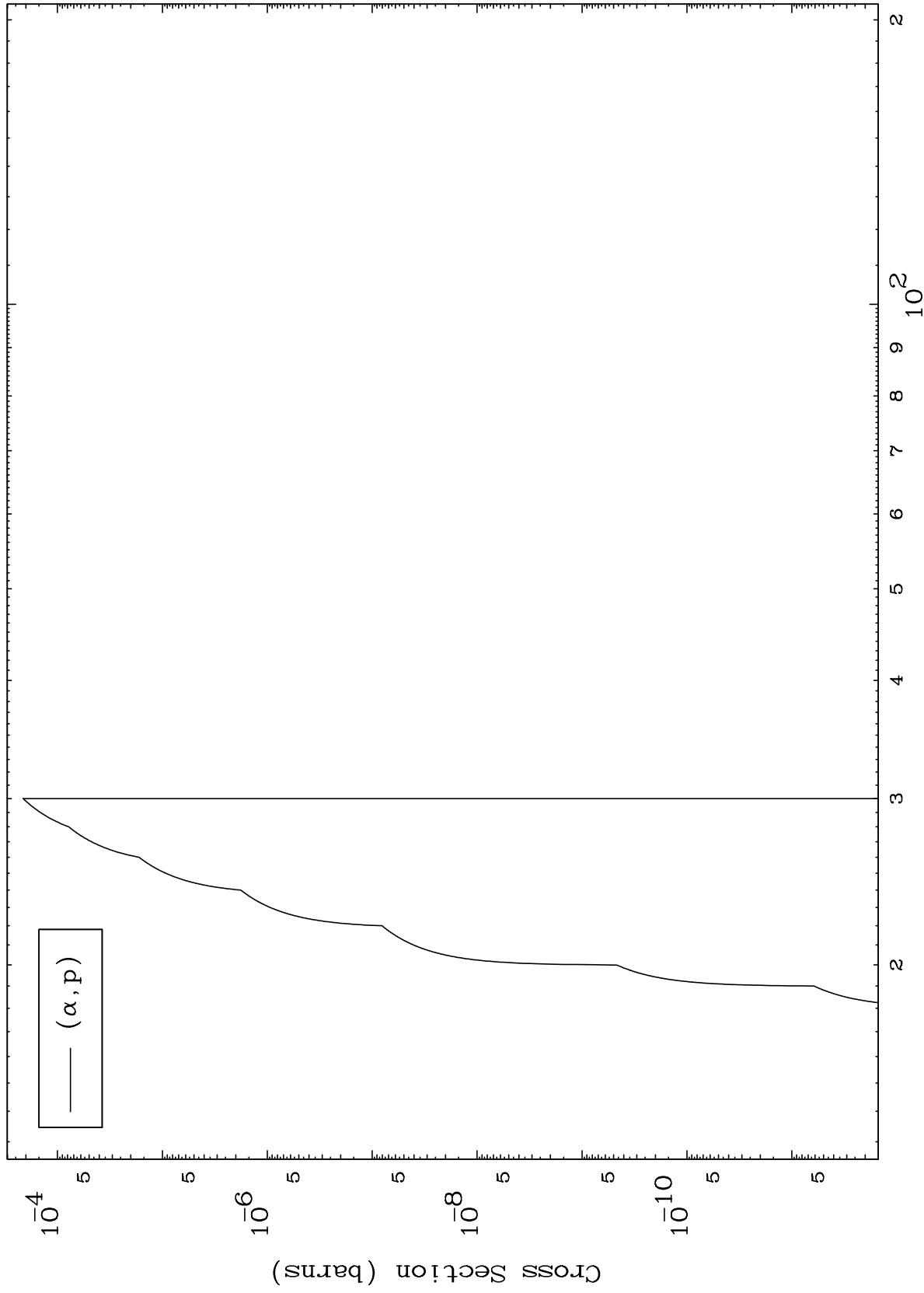
Incident Energy (MeV)

98-Cf-253

MAT 9864

98-Cf-253

(α ,p) Levels
0 Kelvin Cross Sections



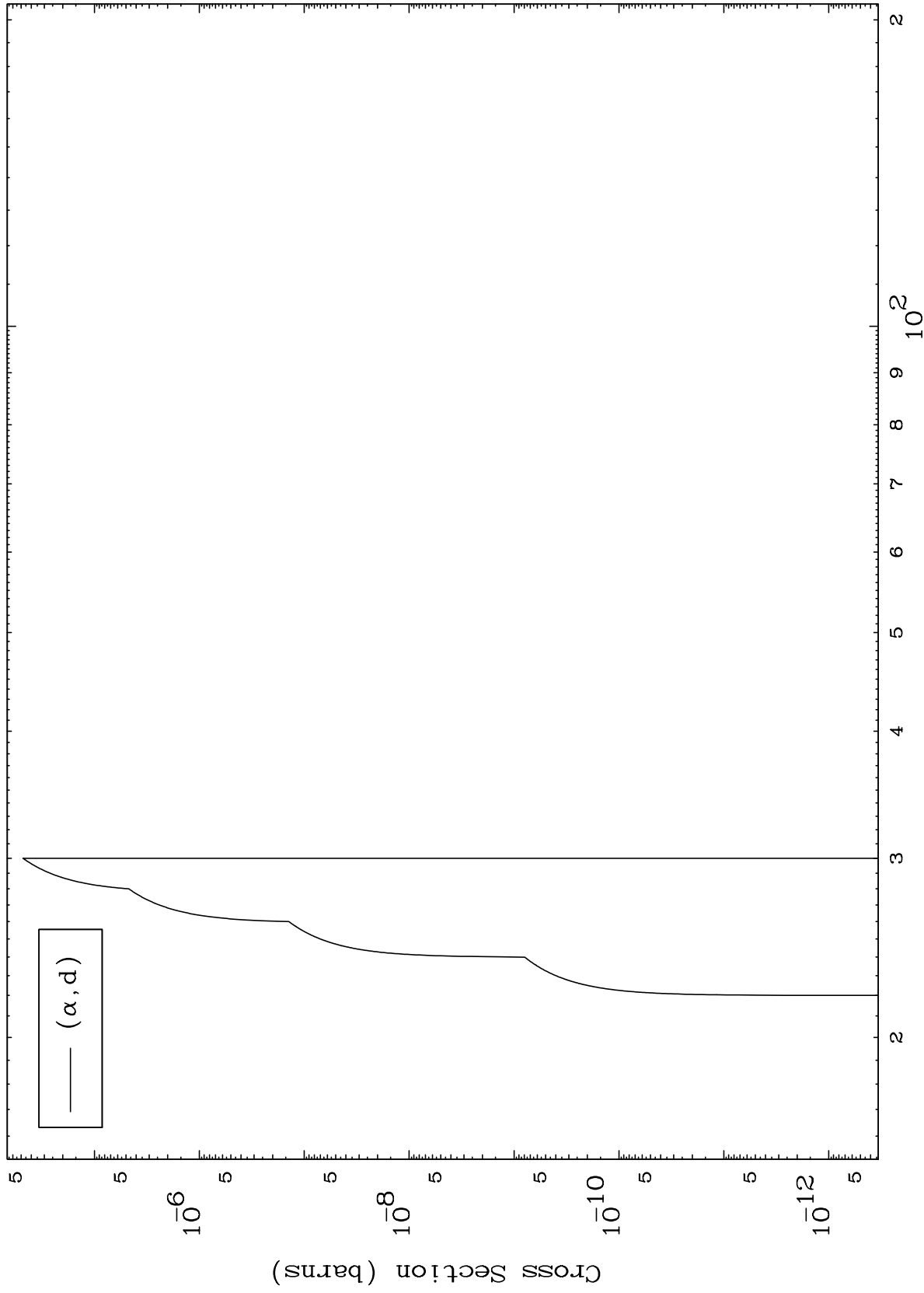
5

98-Cf-253

MAT 9864

(α ,d) Levels
0 Kelvin Cross Sections

98-Cf-253



6

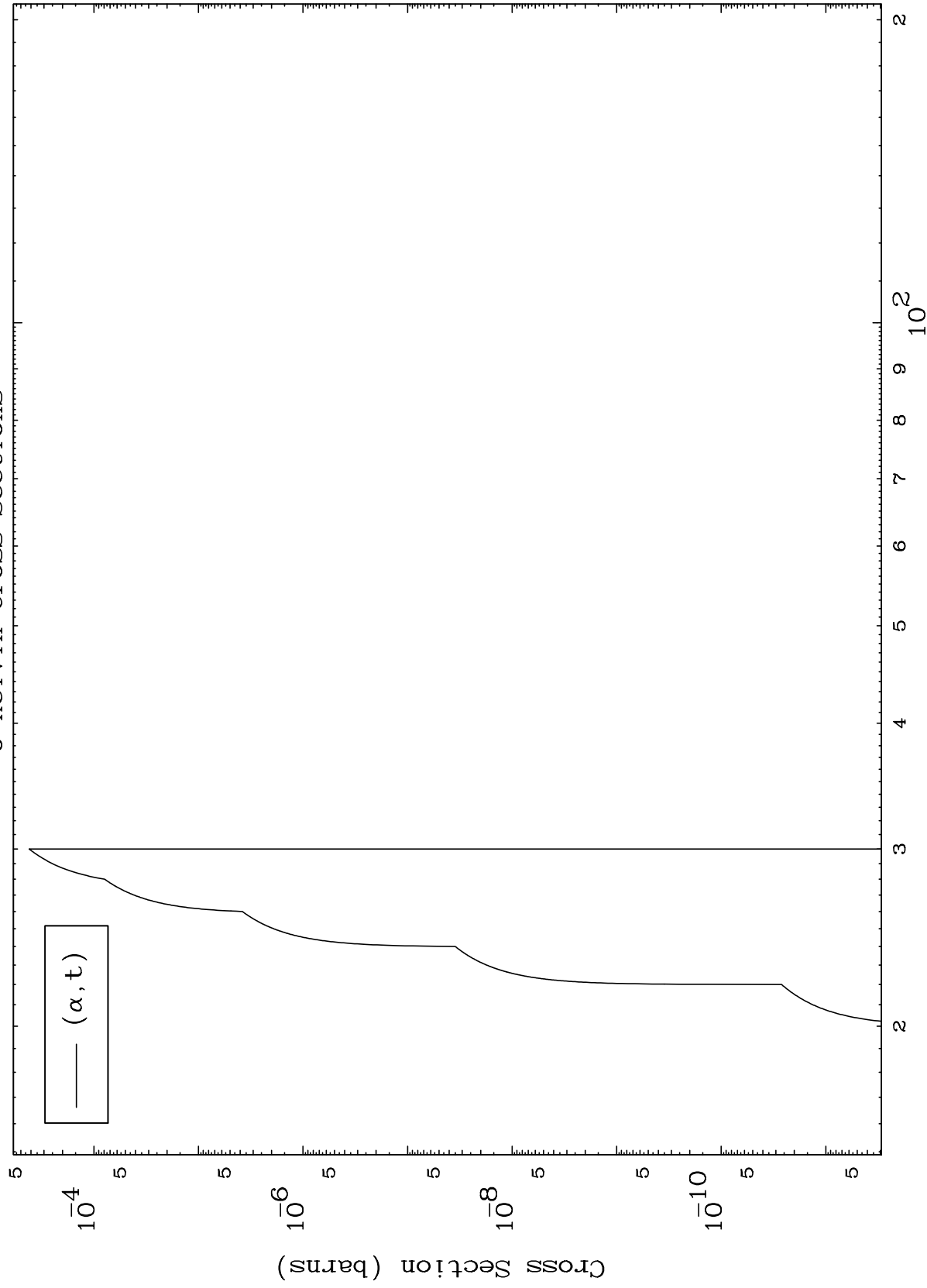
Incident Energy (MeV)

98-Cf-253

MAT 9864

98-Cf-253

(α, t) Levels
0 Kelvin Cross Sections



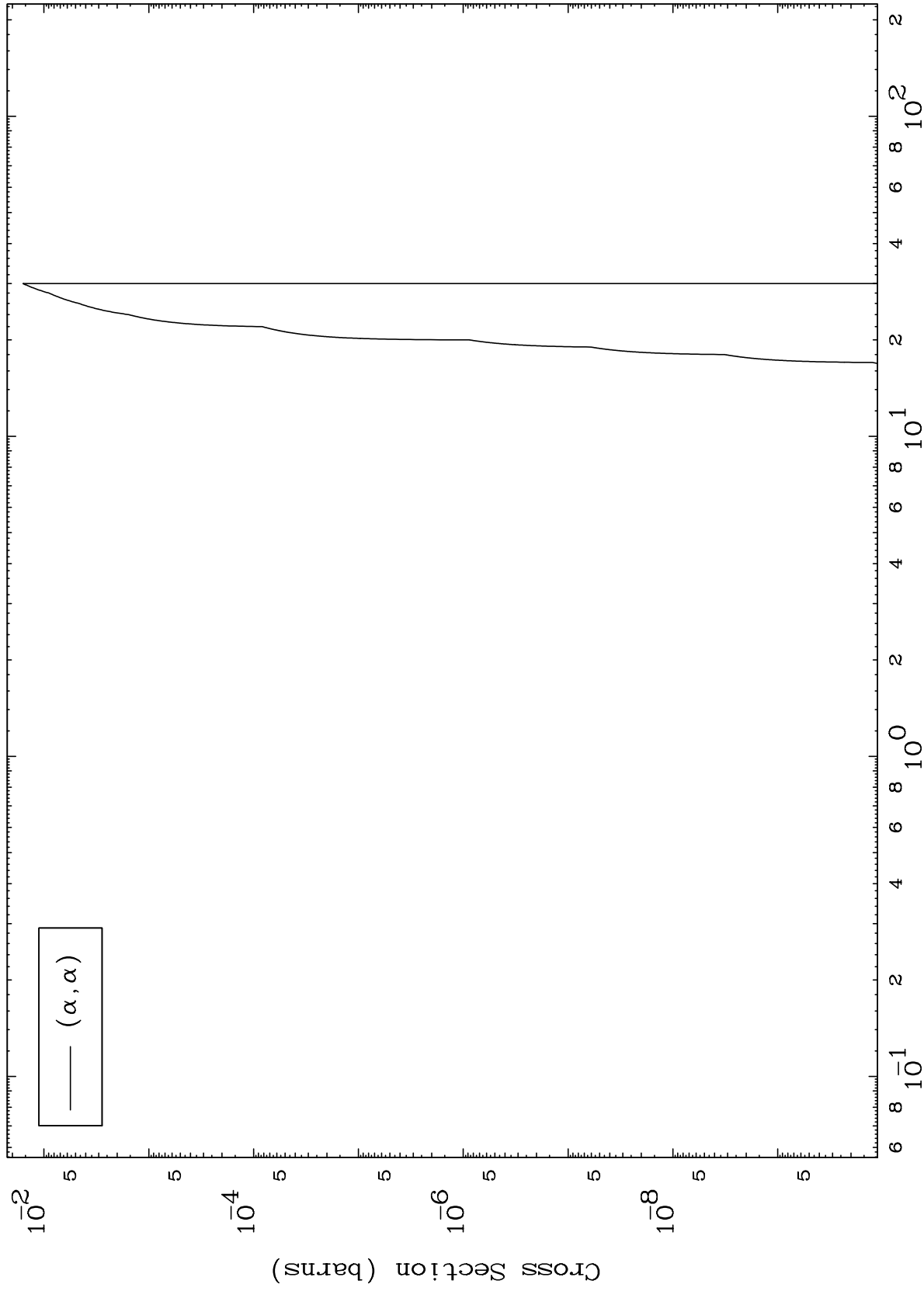
7

98-Cf-253

MAT 9864

98-Cf-253

(α, α) Levels
0 Kelvin Cross Sections



8

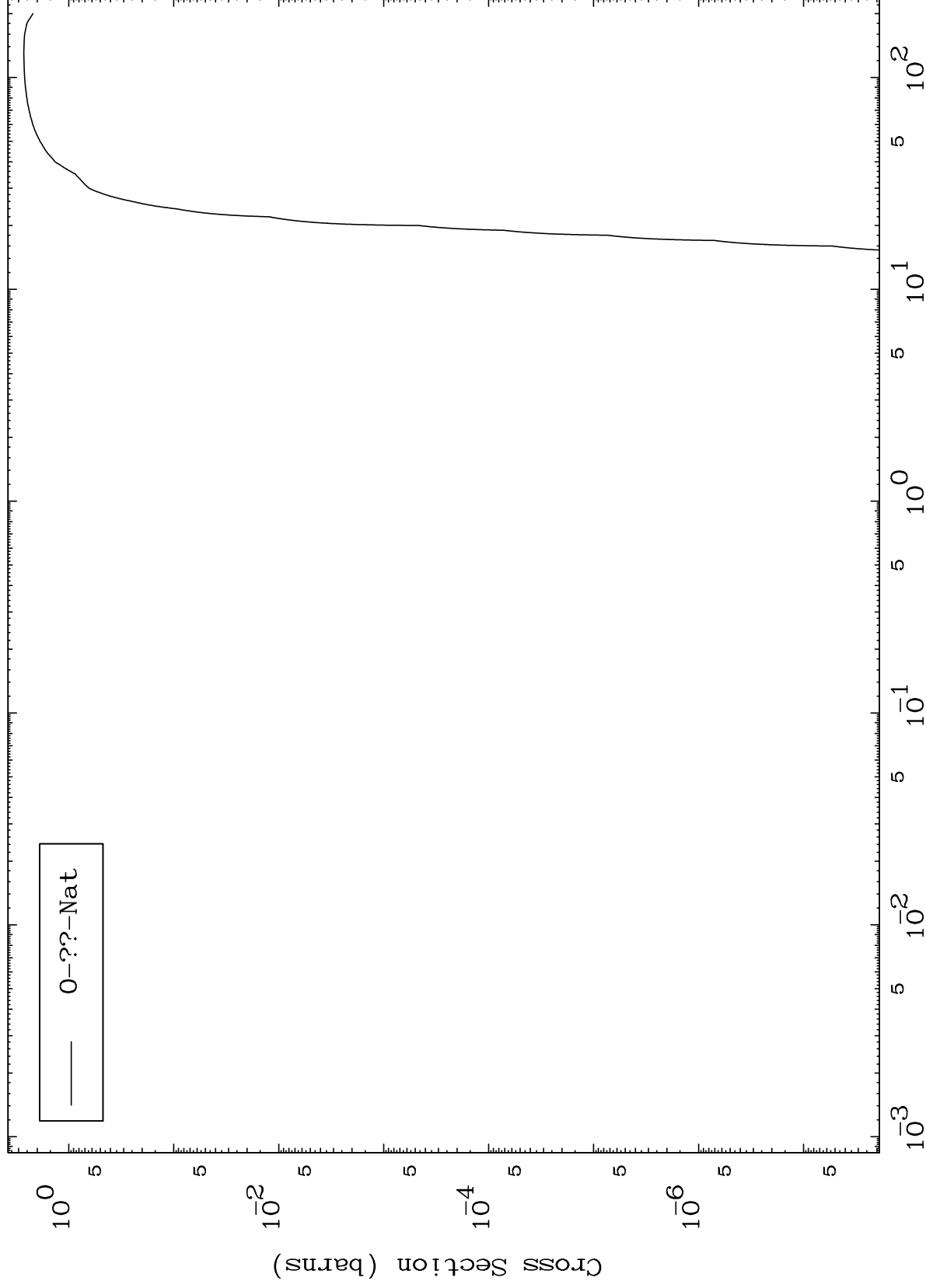
Incident Energy (MeV)

98-Cf-253

MAT 9864

α Fission
Radionuclide Production Cross Section

98-Cf-253



Incident Energy (MeV)

98-Cf-253

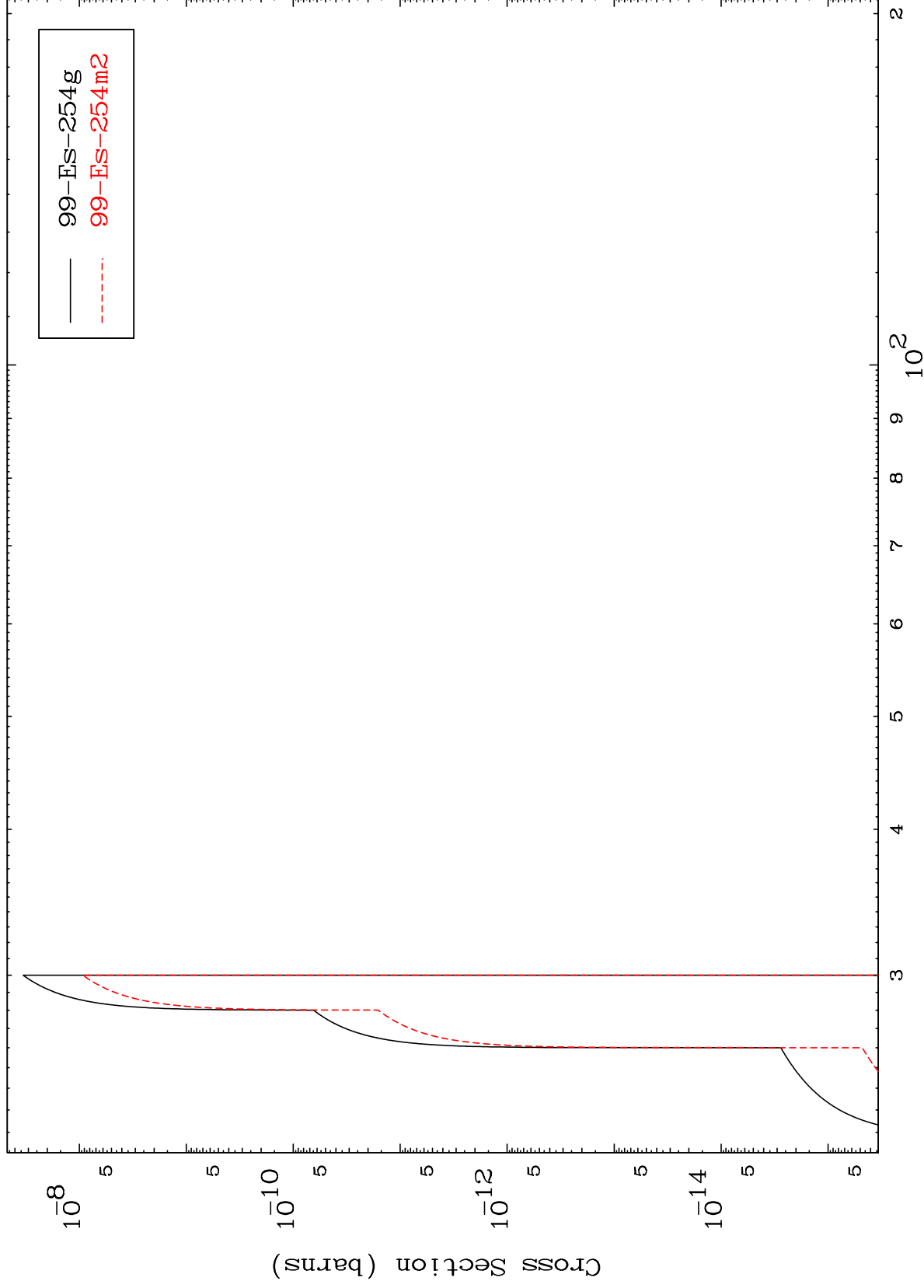
9

MAT 9864

(α, n') d

98-Cf-253

Radionuclide Production Cross Section

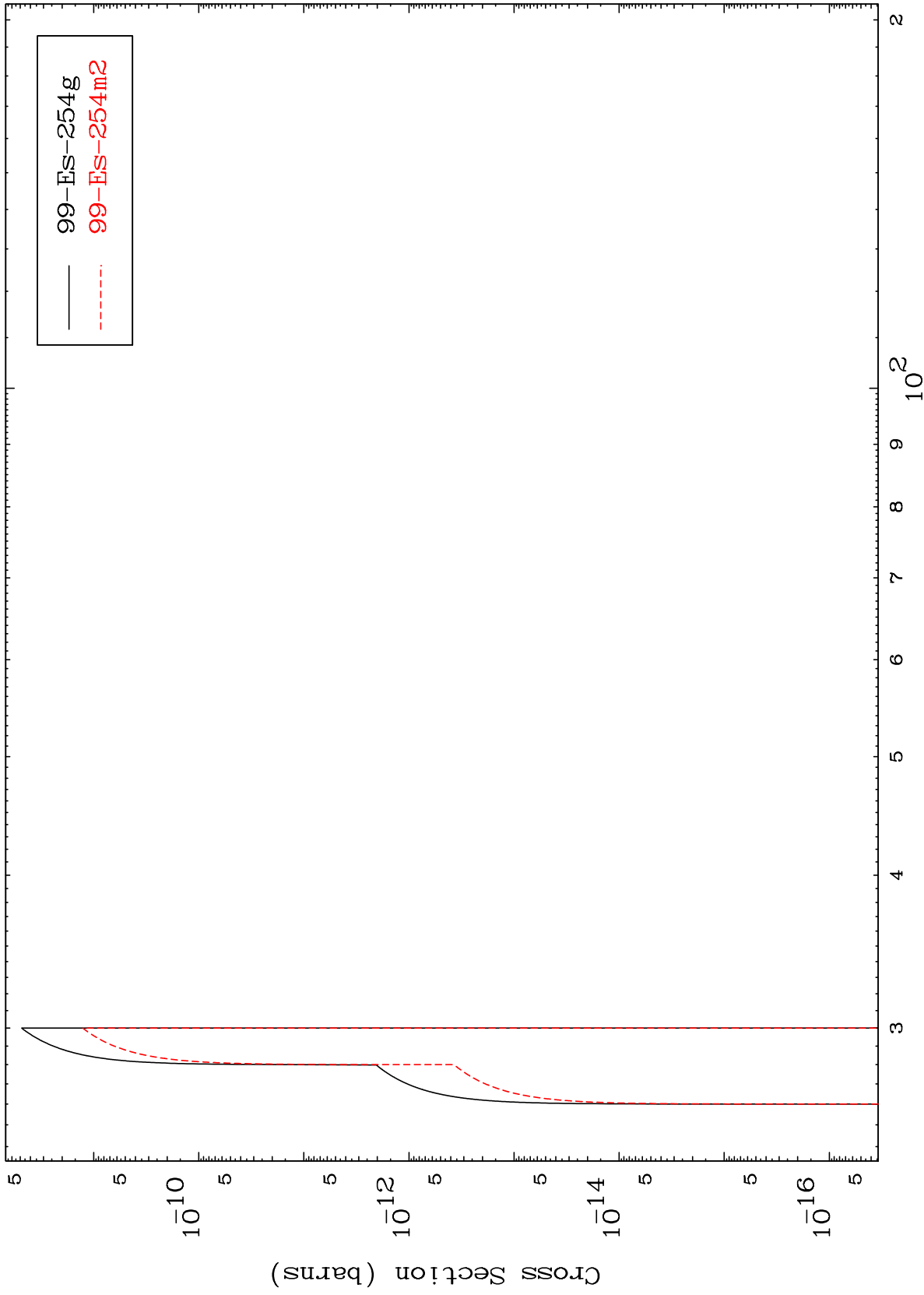


10

Incident Energy (MeV)

98-Cf-253

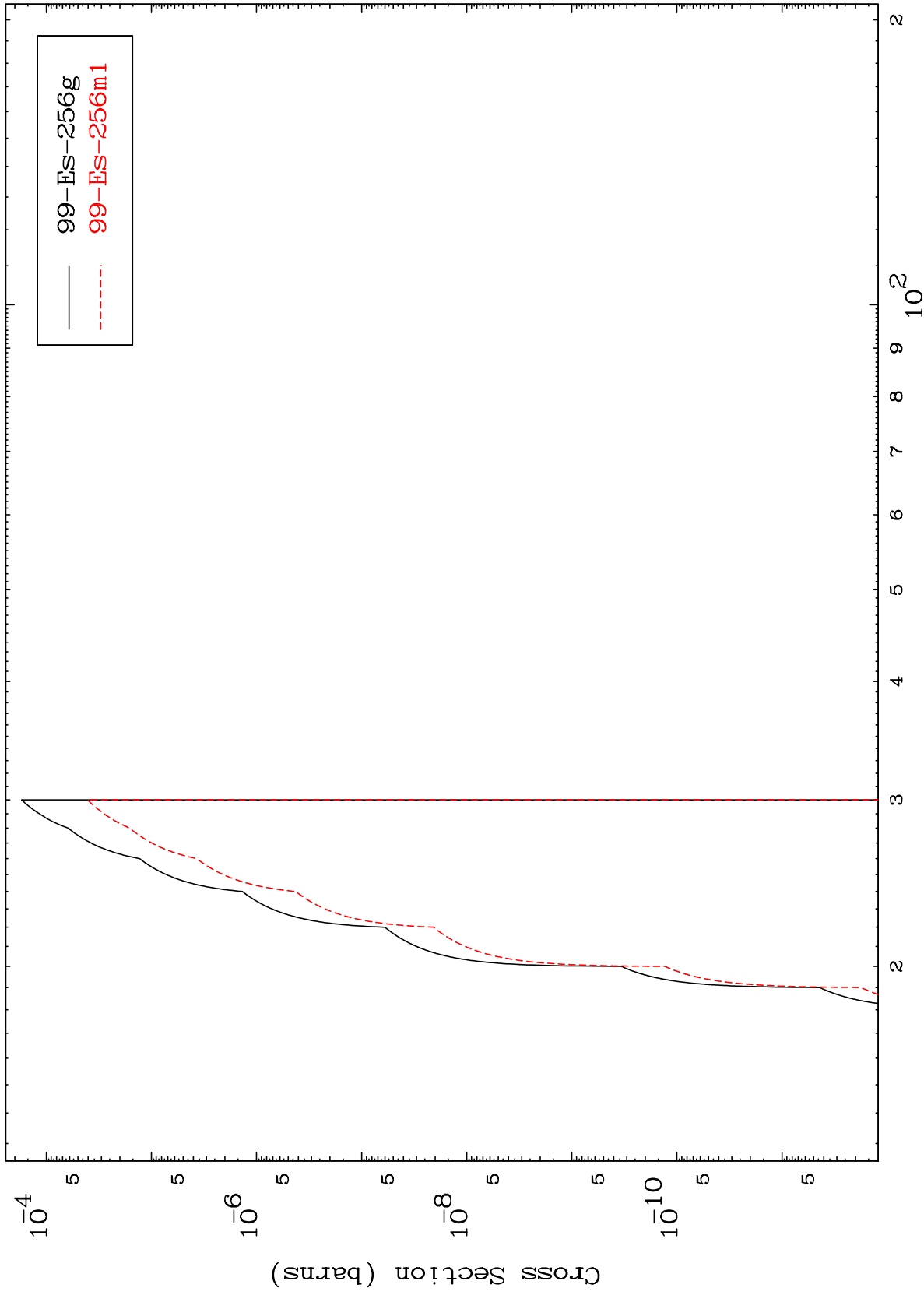
$(\alpha, 2n)$ p
Radionuclide Production Cross Section



MAT 9864

98-Cf-253

(α, p)
Radionuclide Production Cross Section



12

Incident Energy (MeV)

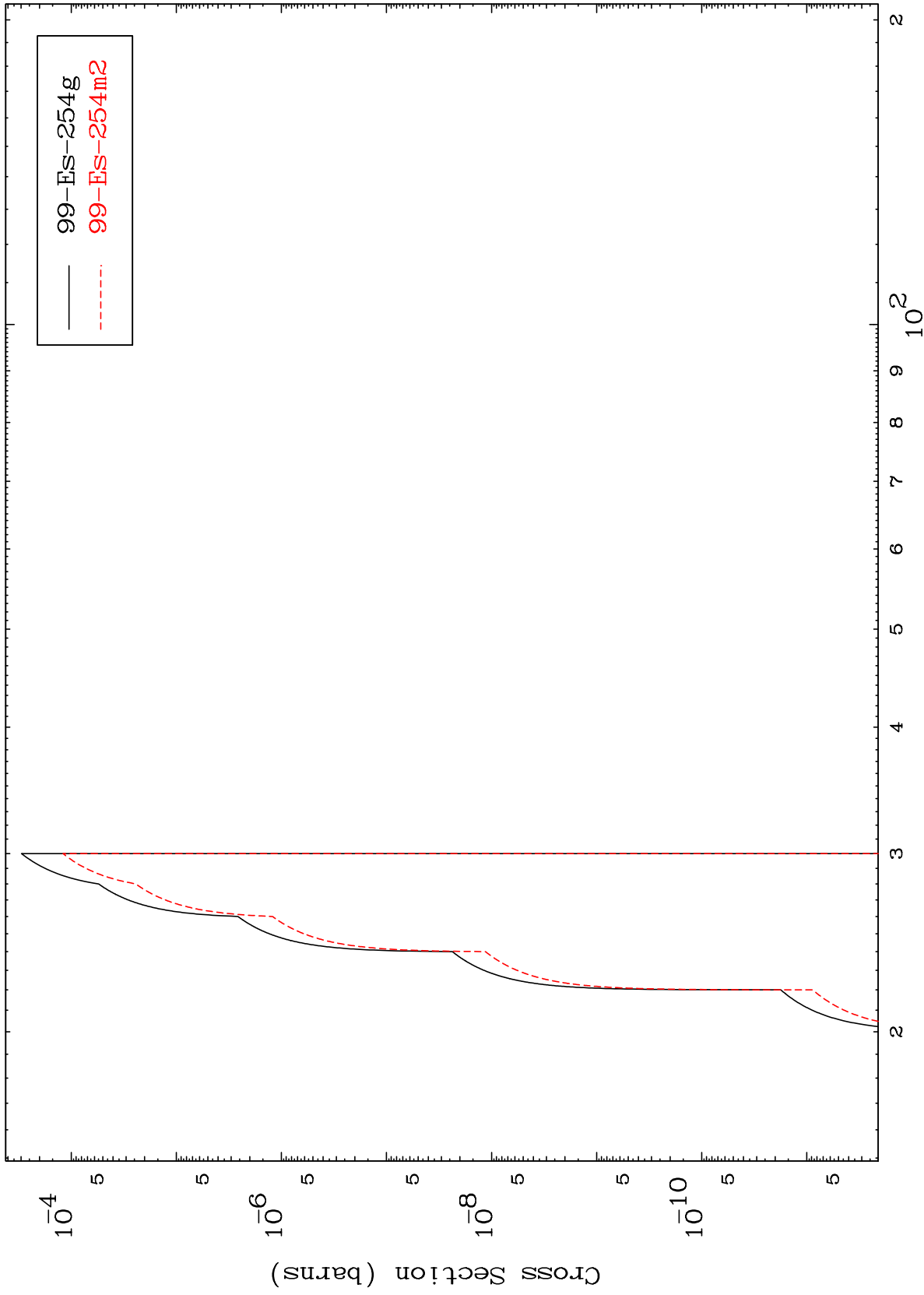
98-Cf-253

MAT 9864

(α, t)

98-Cf-253

Radionuclide Production Cross Section



13

Incident Energy (MeV)

98-Cf-253