

Program EVALPLOT  
(Version 2018-1)

by

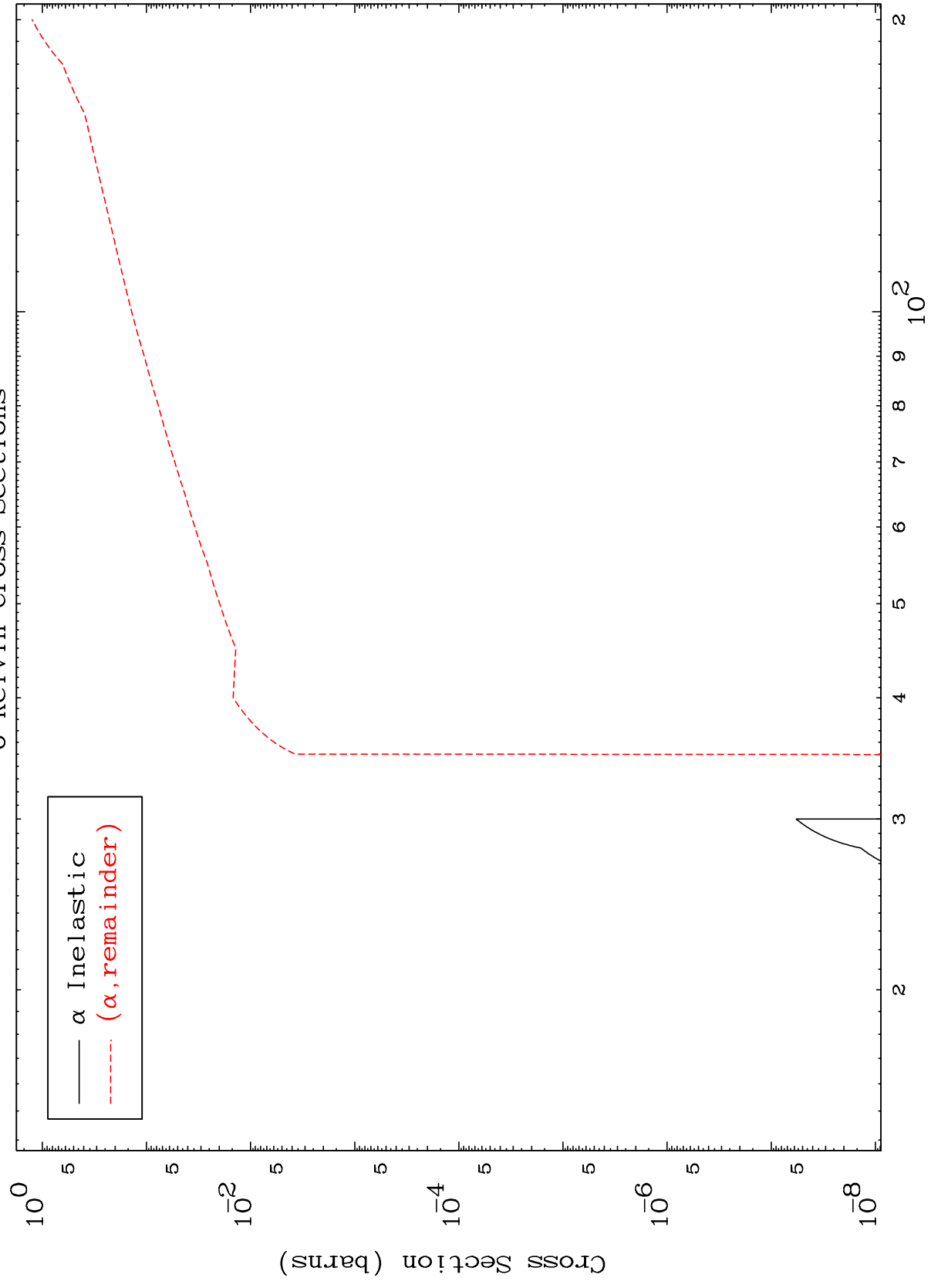
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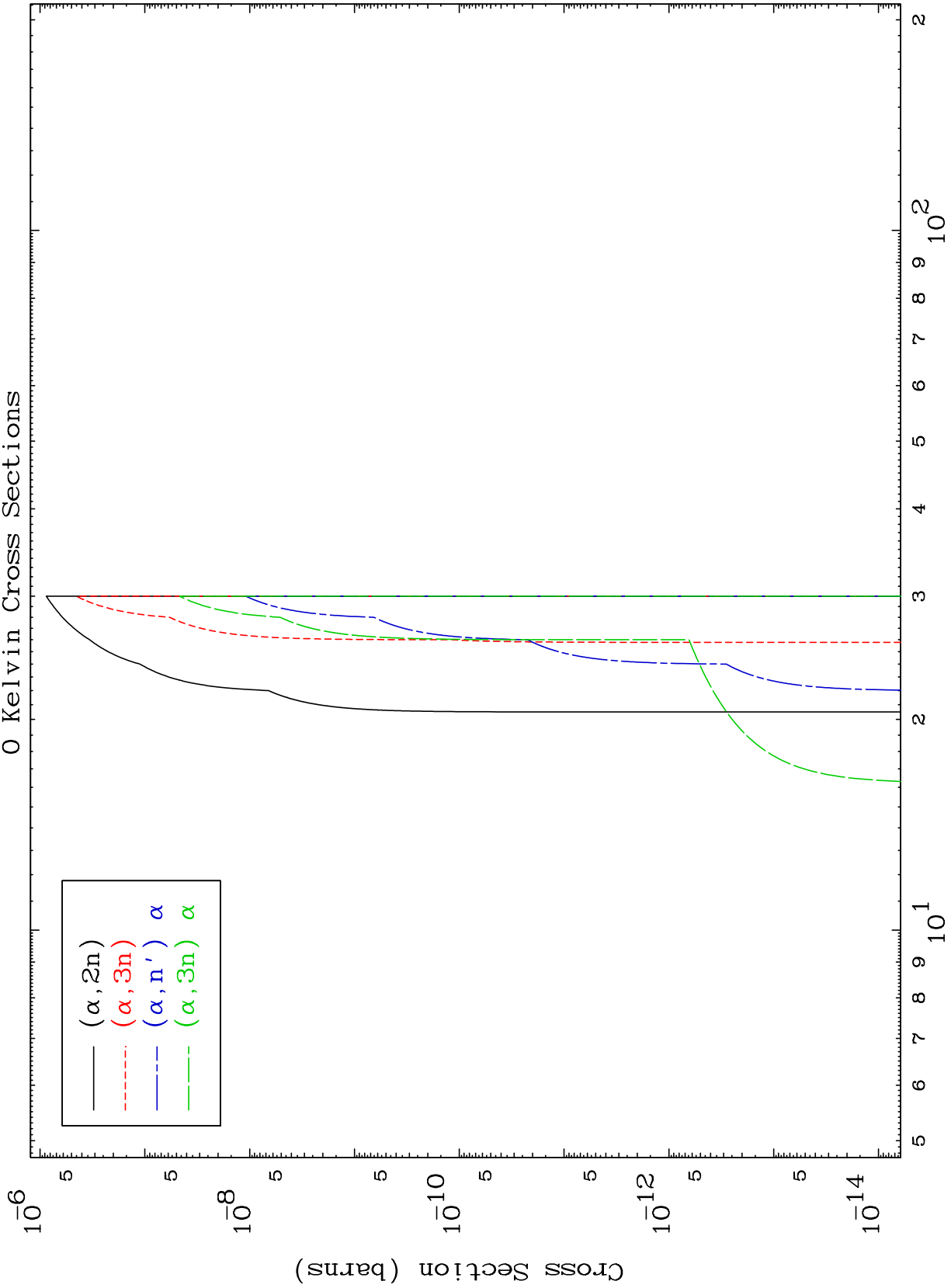
Press Mouse Button to Start



MAT 570

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

105-Db-270



2

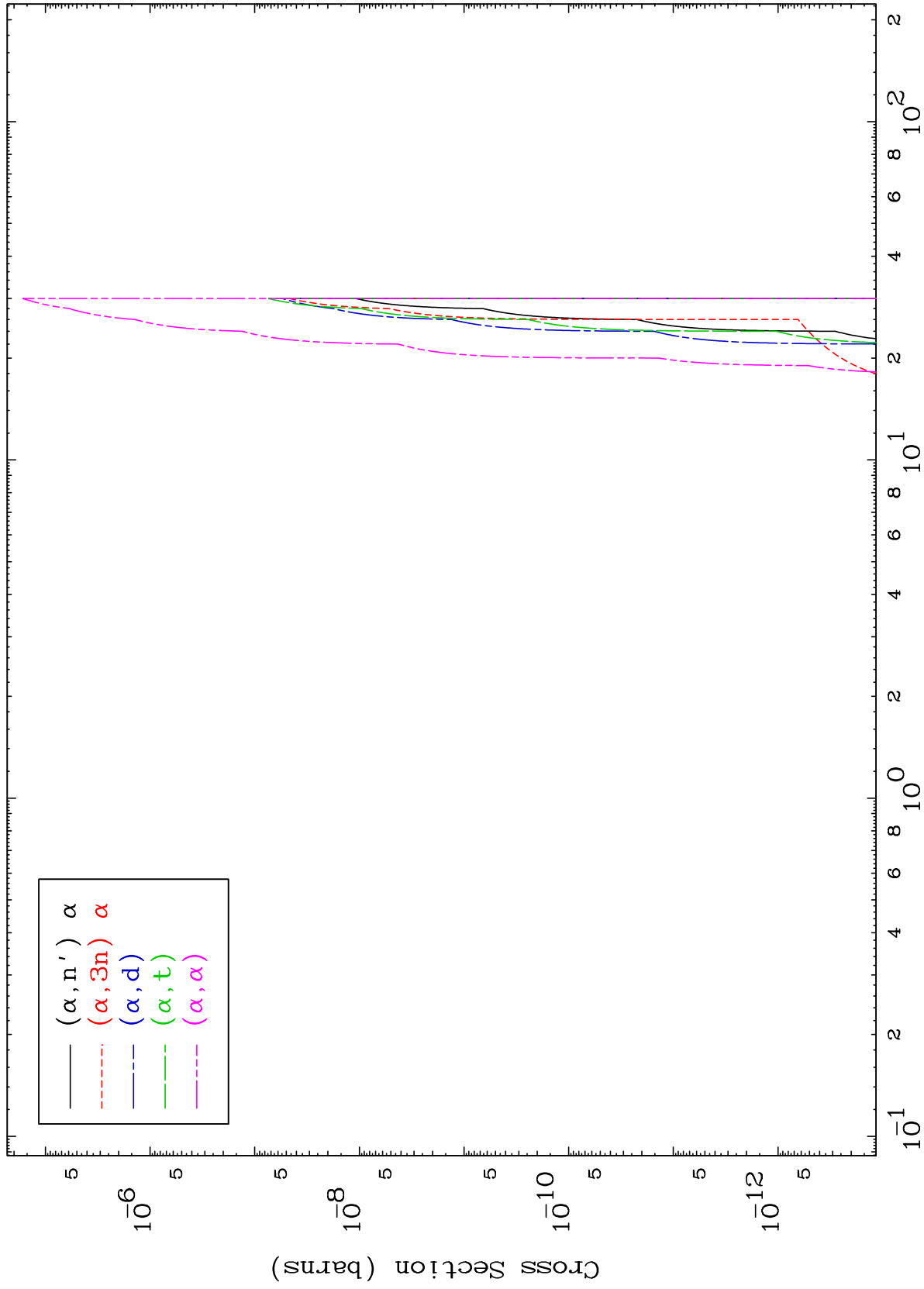
Incident Energy (MeV)

105-Db-270

MAT 570

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

105-Db-270



105-Db-270

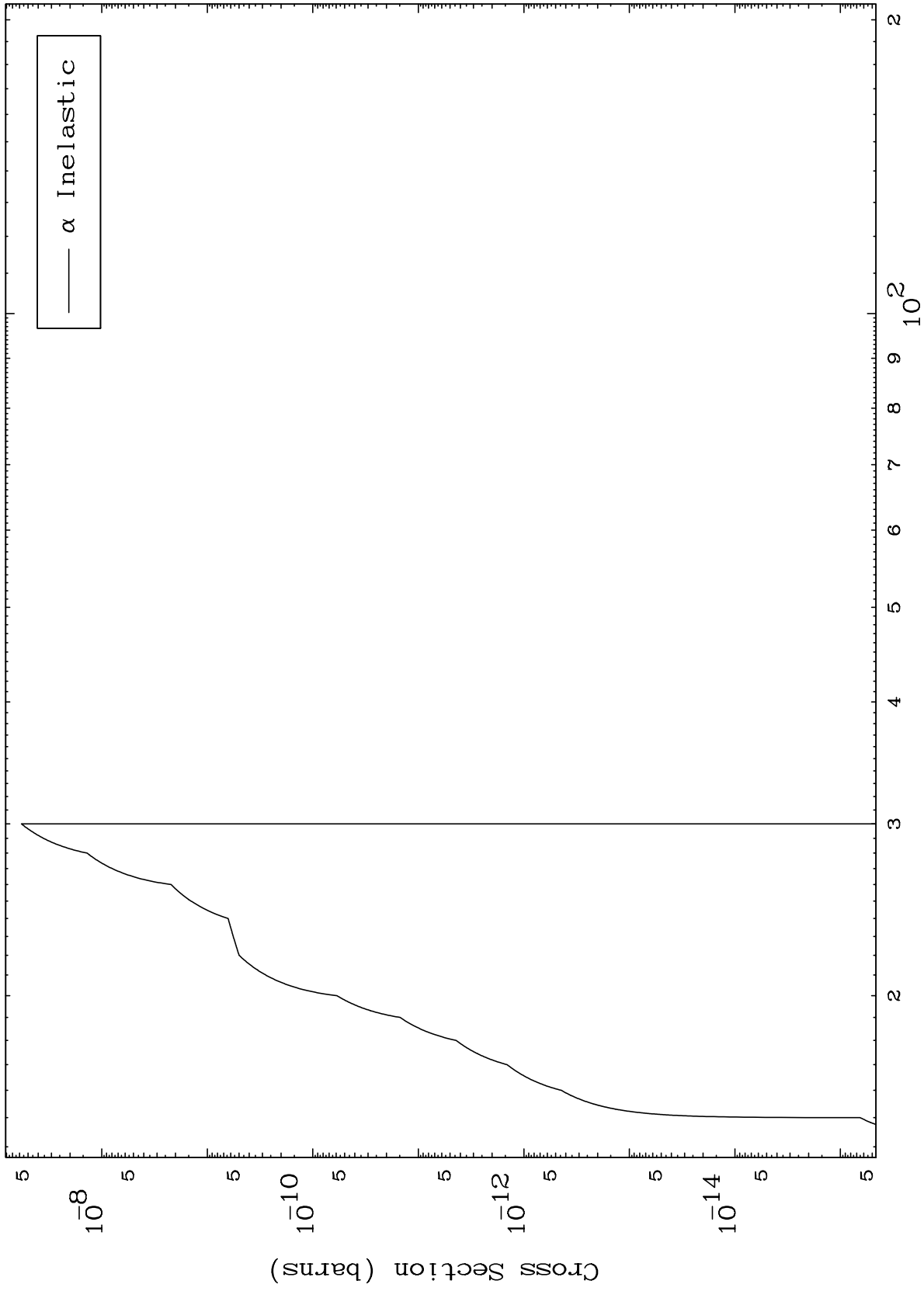
105-Db-270

3

MAT 570

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

105-Db-270



4

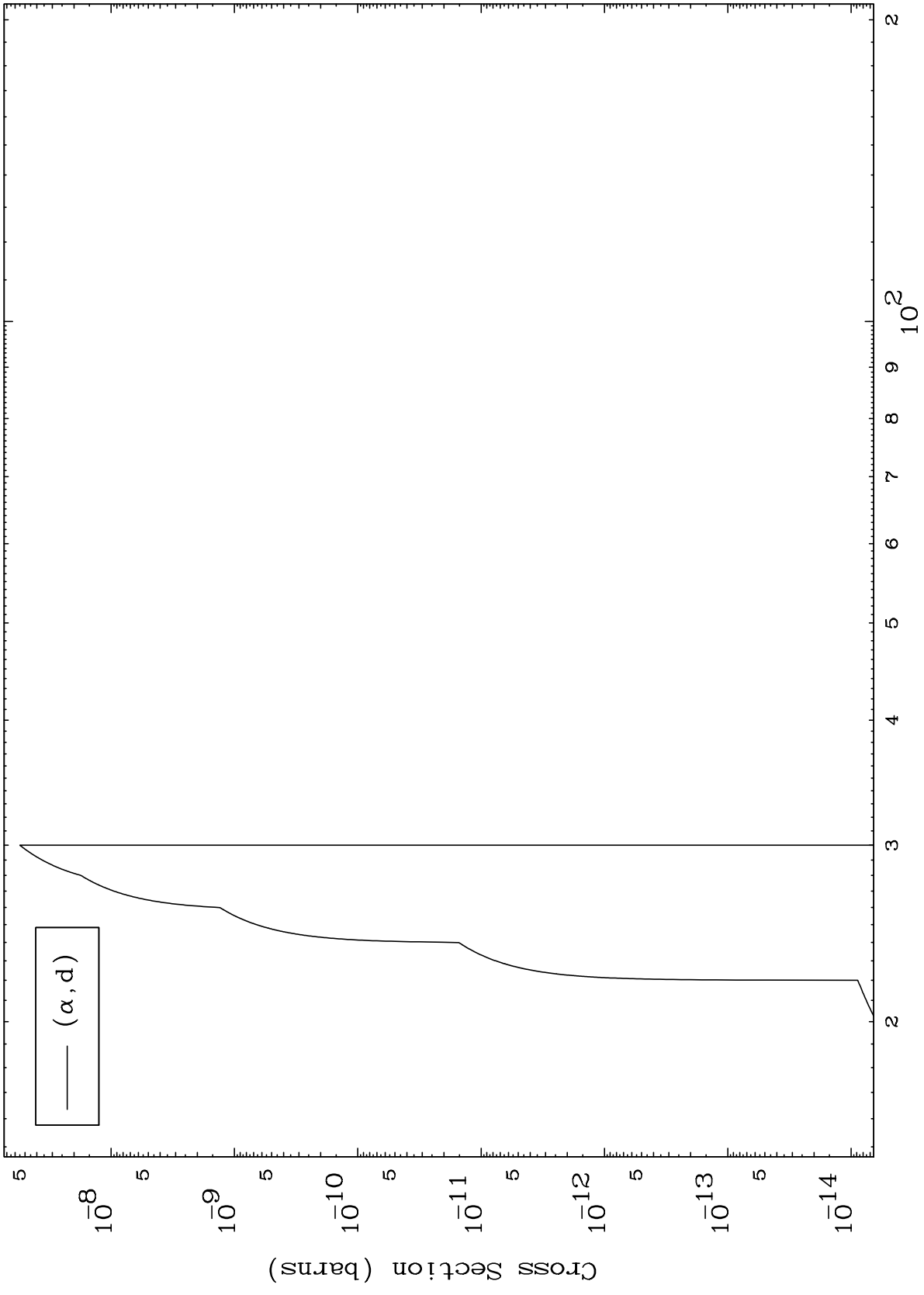
Incident Energy (MeV)

105-Db-270

MAT 570

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

105-Db-270



5

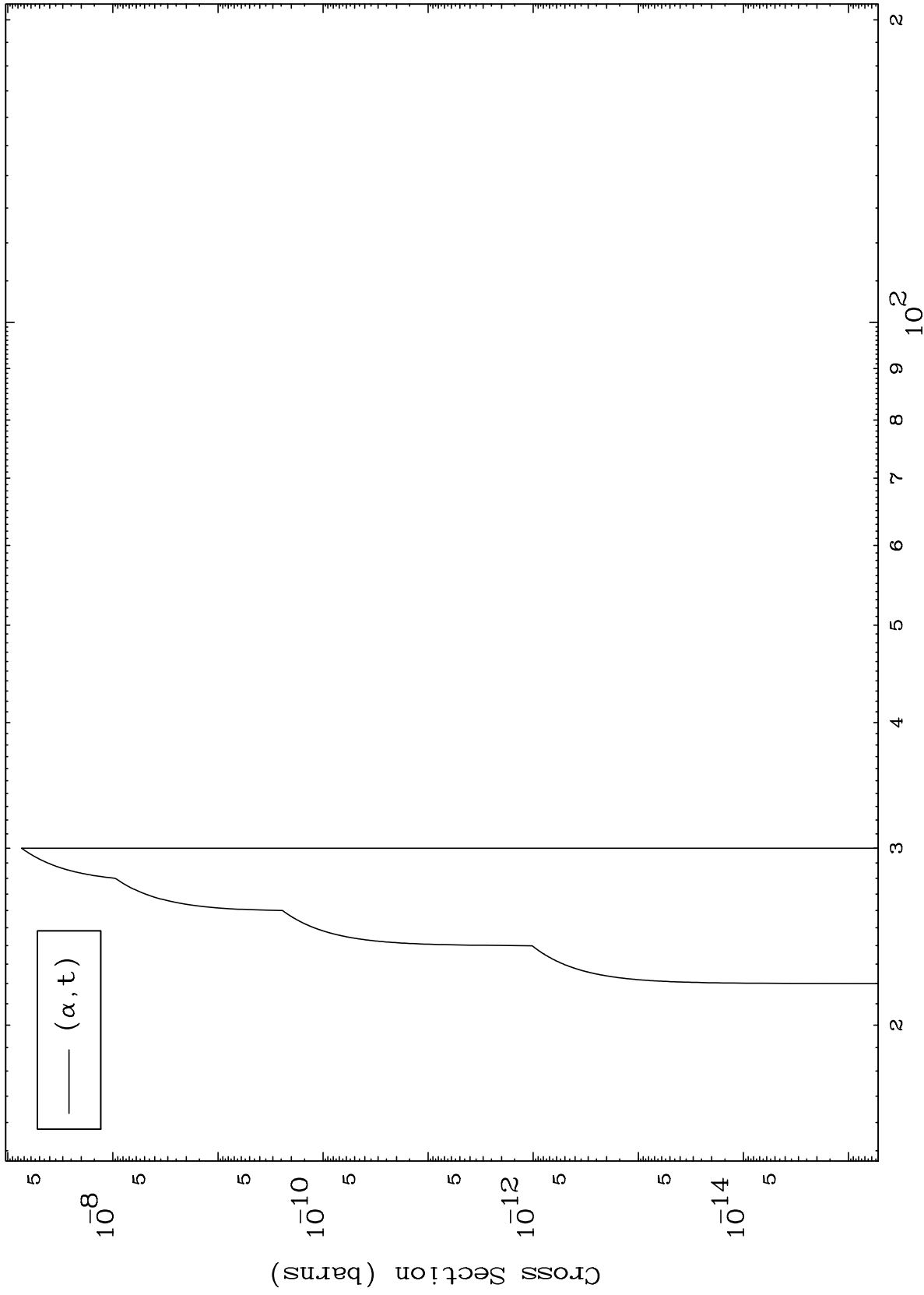
Incident Energy (MeV)

105-Db-270

MAT 570

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

105-Db-270



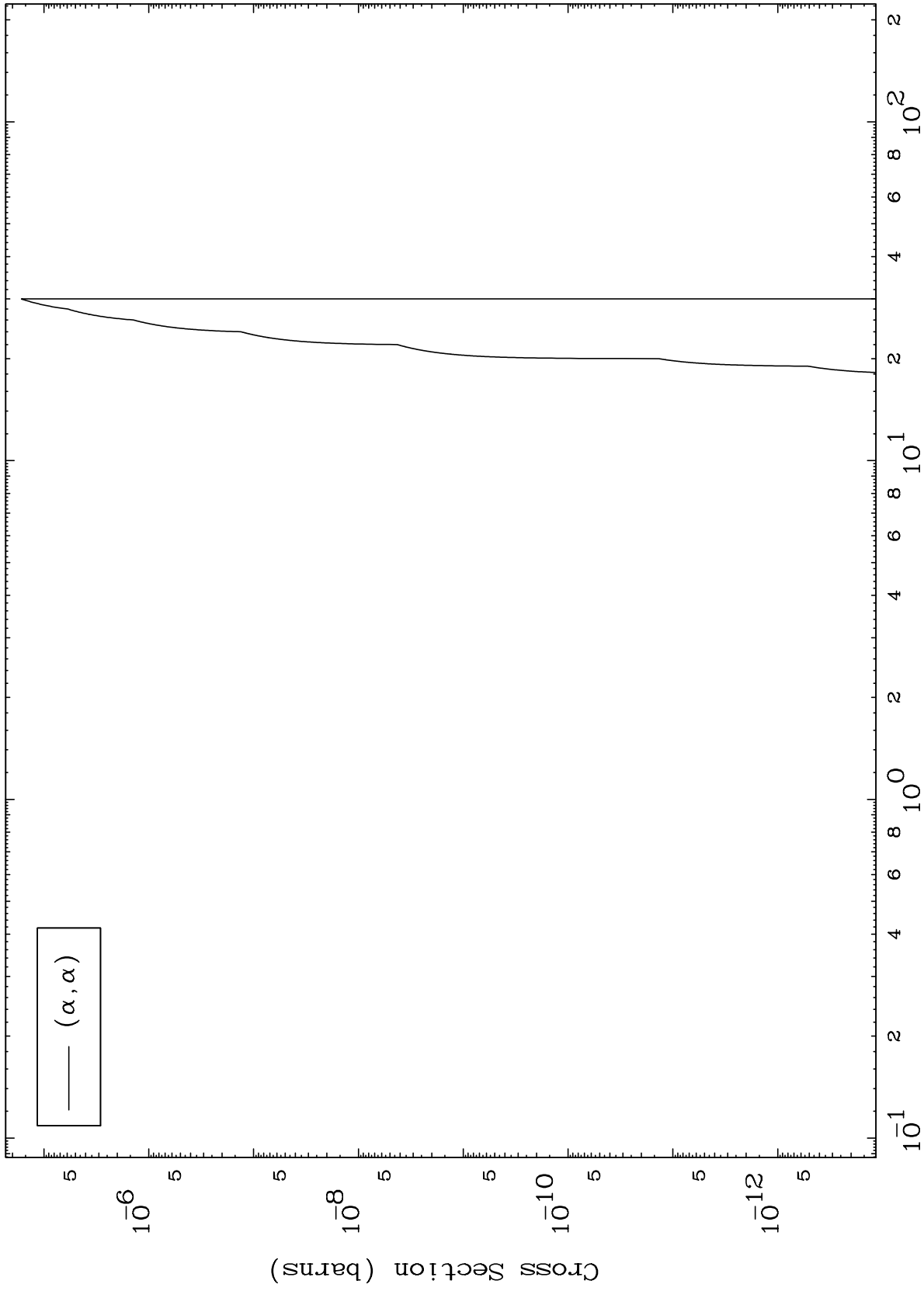
6

105-Db-270

MAT 570

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

105-Db-270



Incident Energy (MeV)

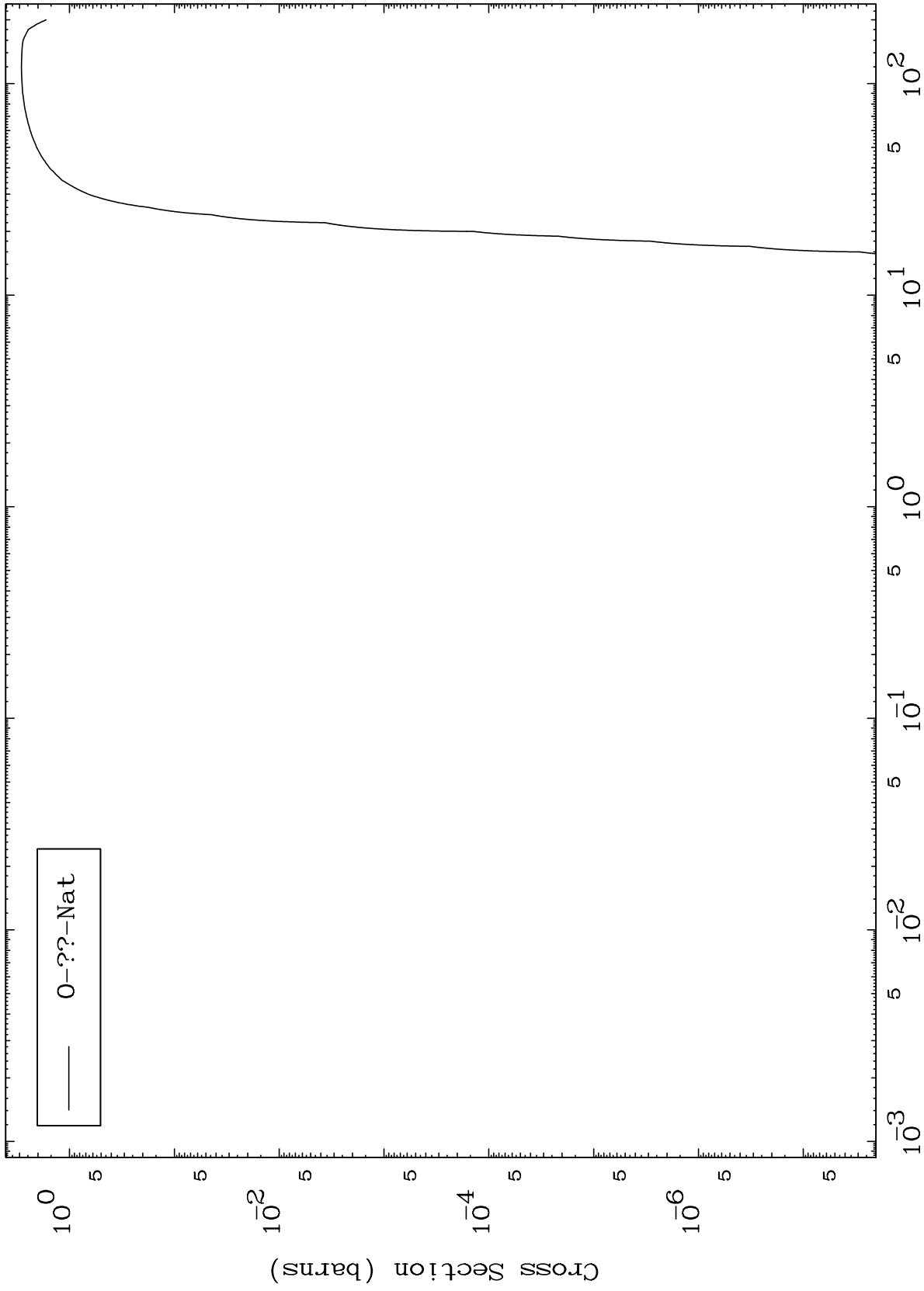
105-Db-270



MAT 570

$\alpha$  Fission  
Radionuclide Production Cross Section

105-Db-270



8

Incident Energy (MeV)

105-Db-270