

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

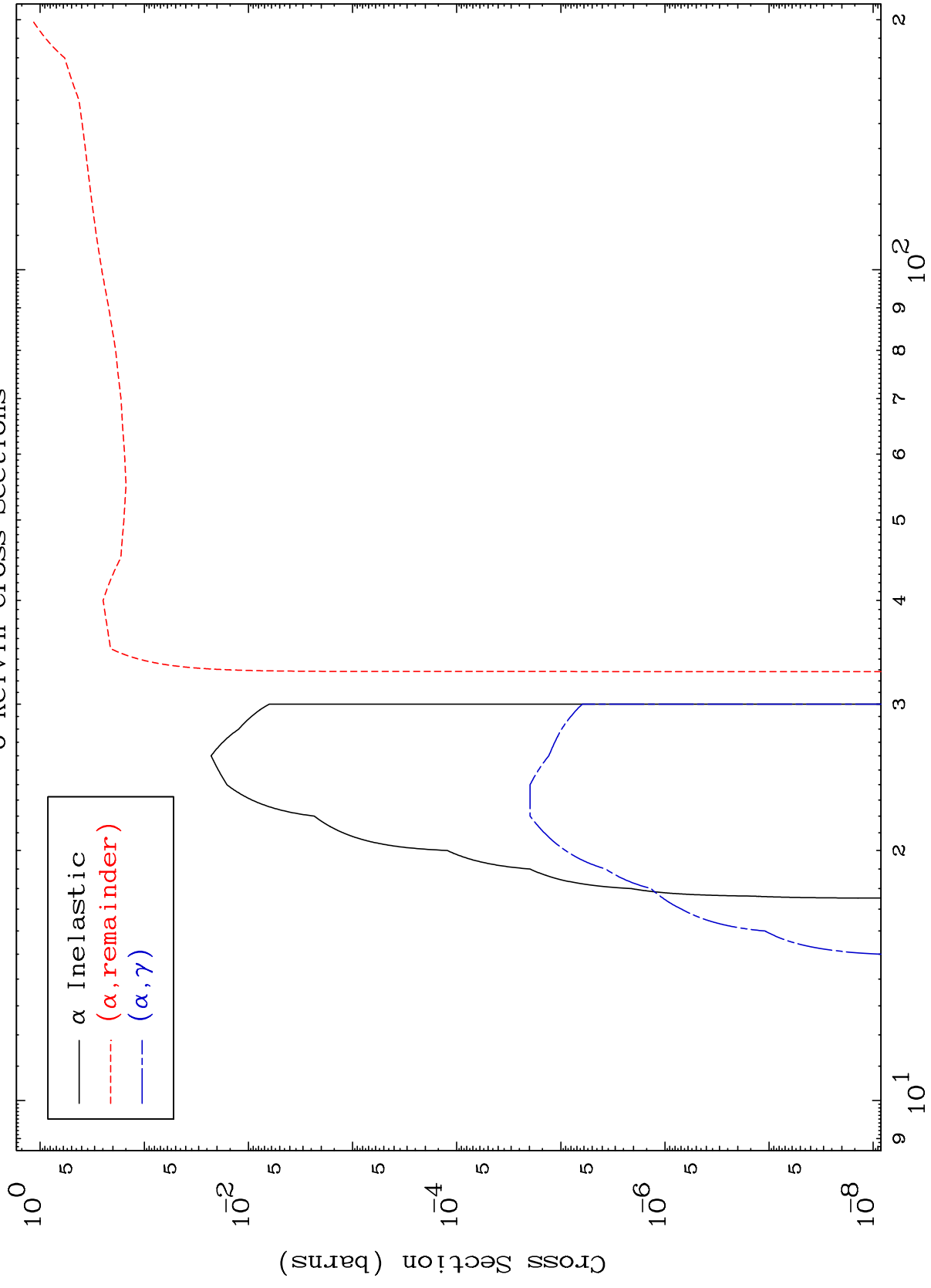
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9927

$\alpha$  Major  
0 Kelvin Cross Sections

100-Fm-246



1

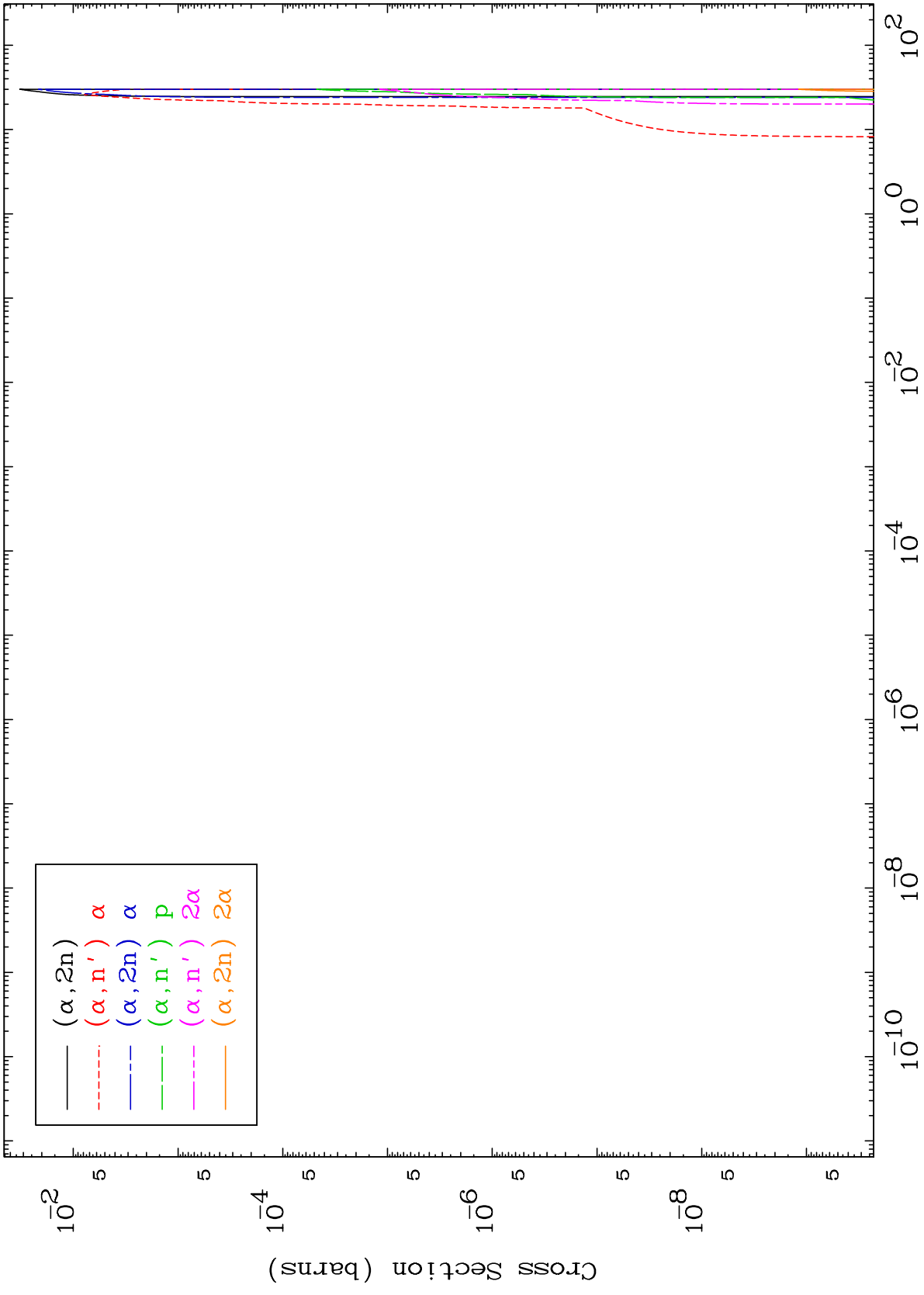
Incident Energy (MeV)

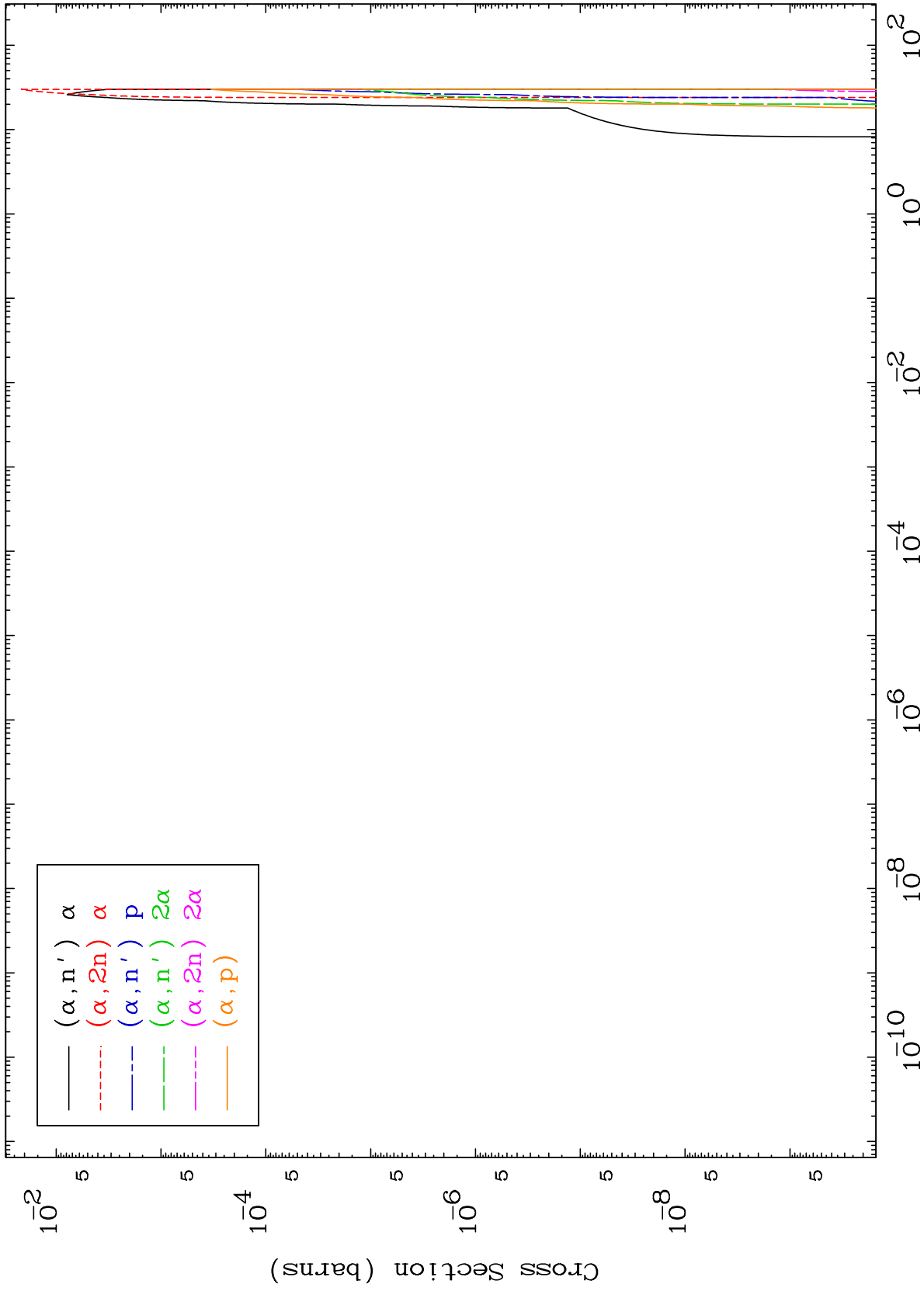
100-Fm-246

MAT 9927

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

100-Fm-246

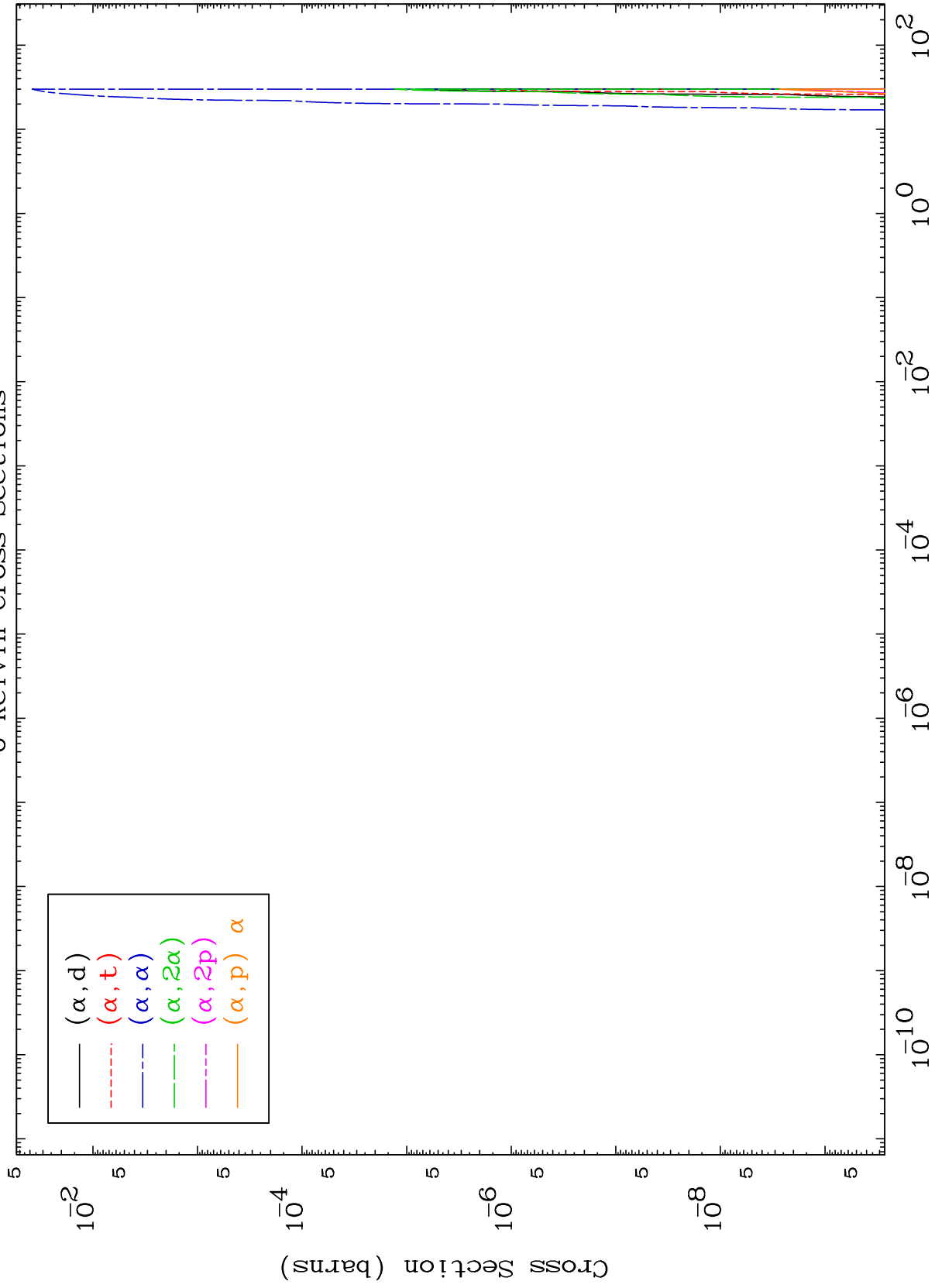




MAT 9927

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

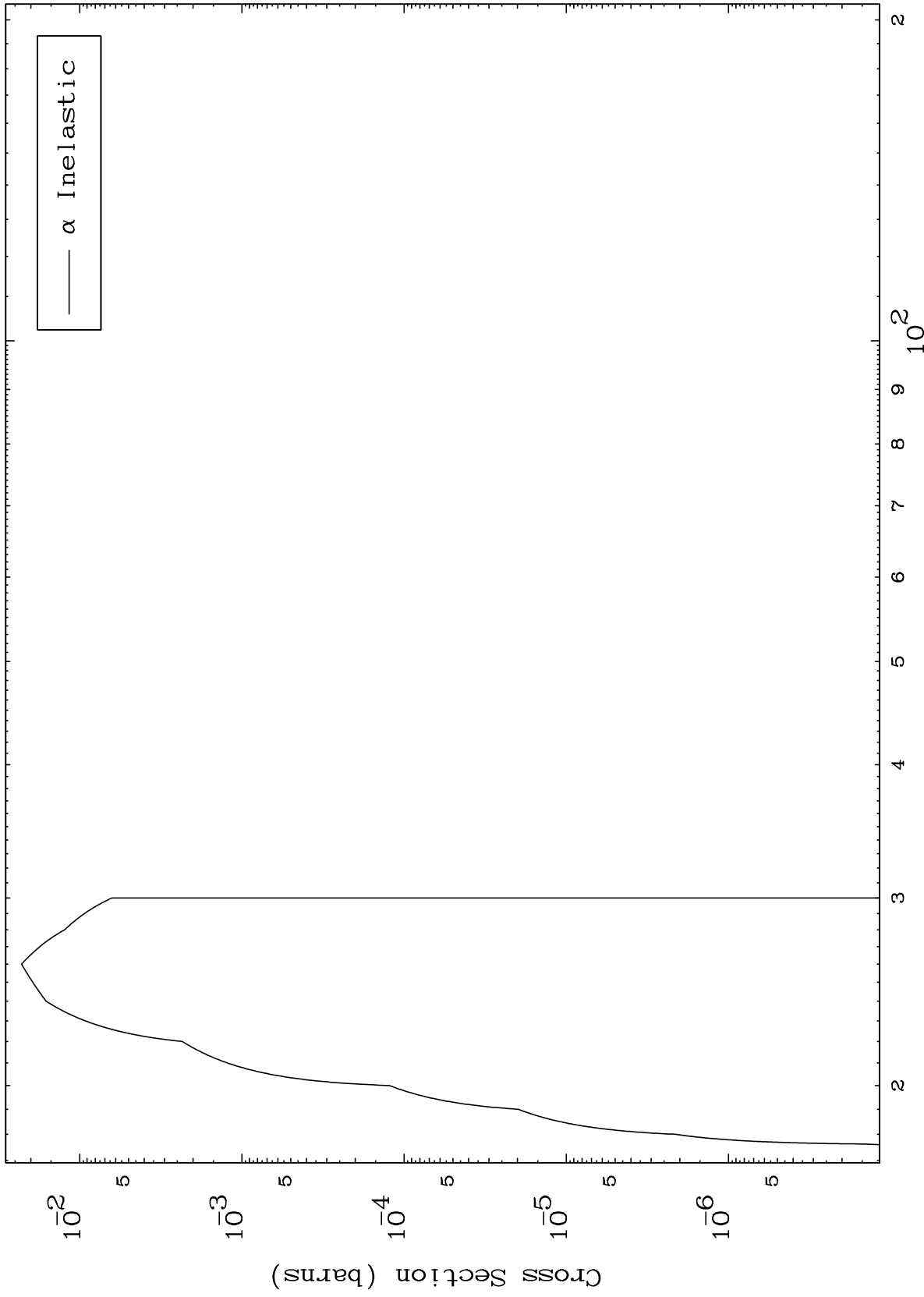
100-Fm-246



MAT 9927

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

100-Fm-246



5

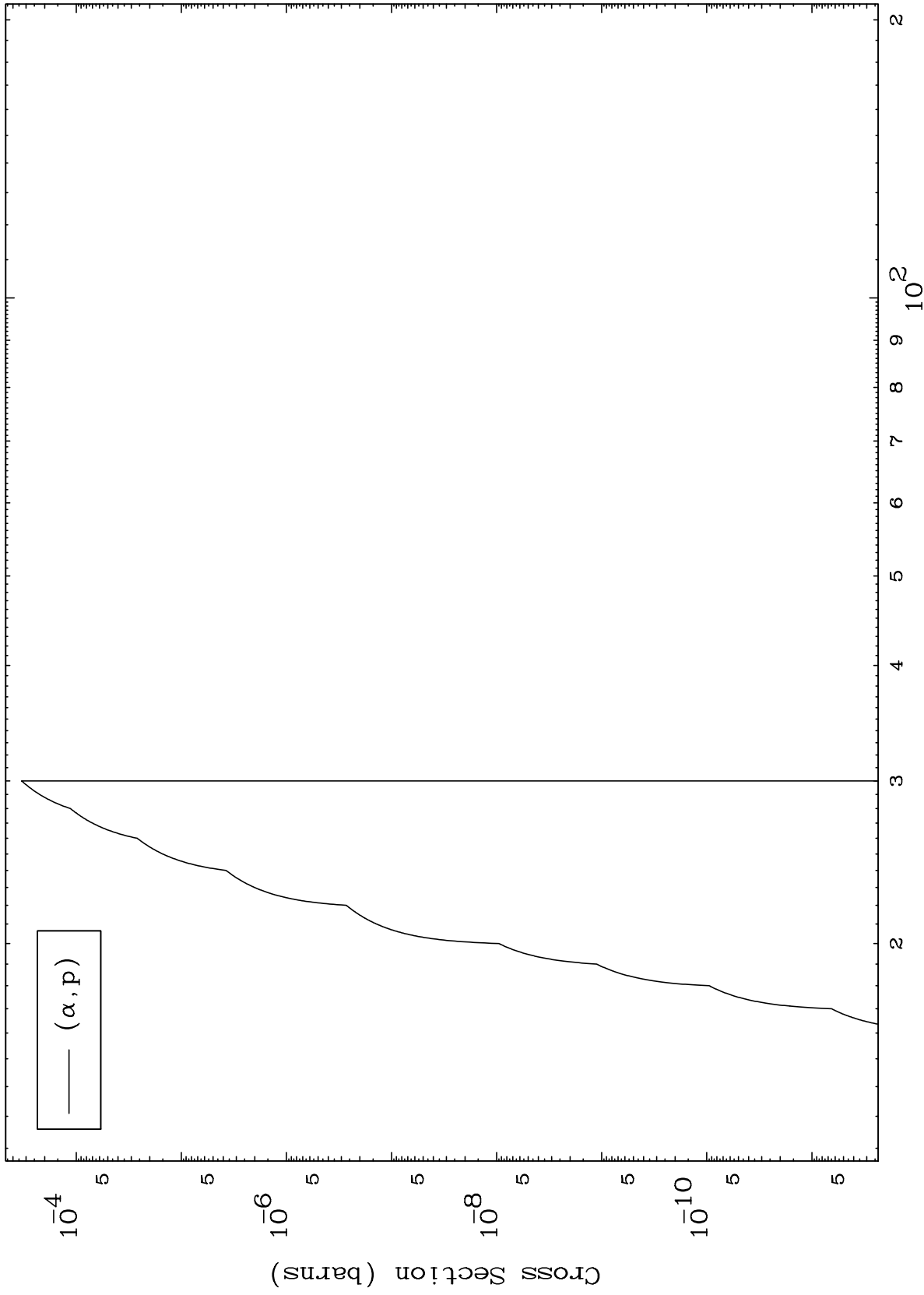
Incident Energy (MeV)

100-Fm-246

MAT 9927

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

100-Fm-246



6

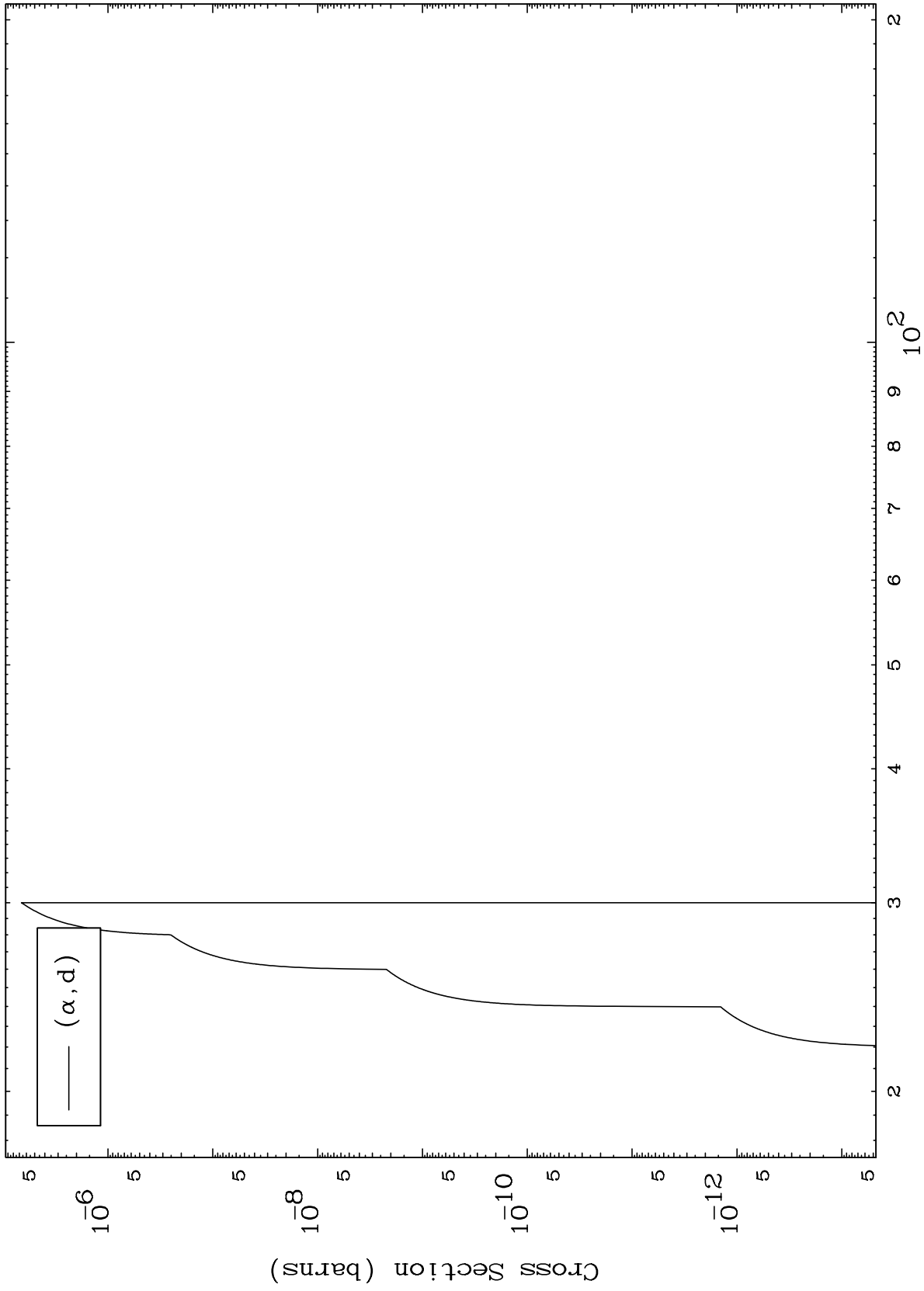
Incident Energy (MeV)

100-Fm-246

MAT 9927

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

100-Fm-246



7

Incident Energy (MeV)

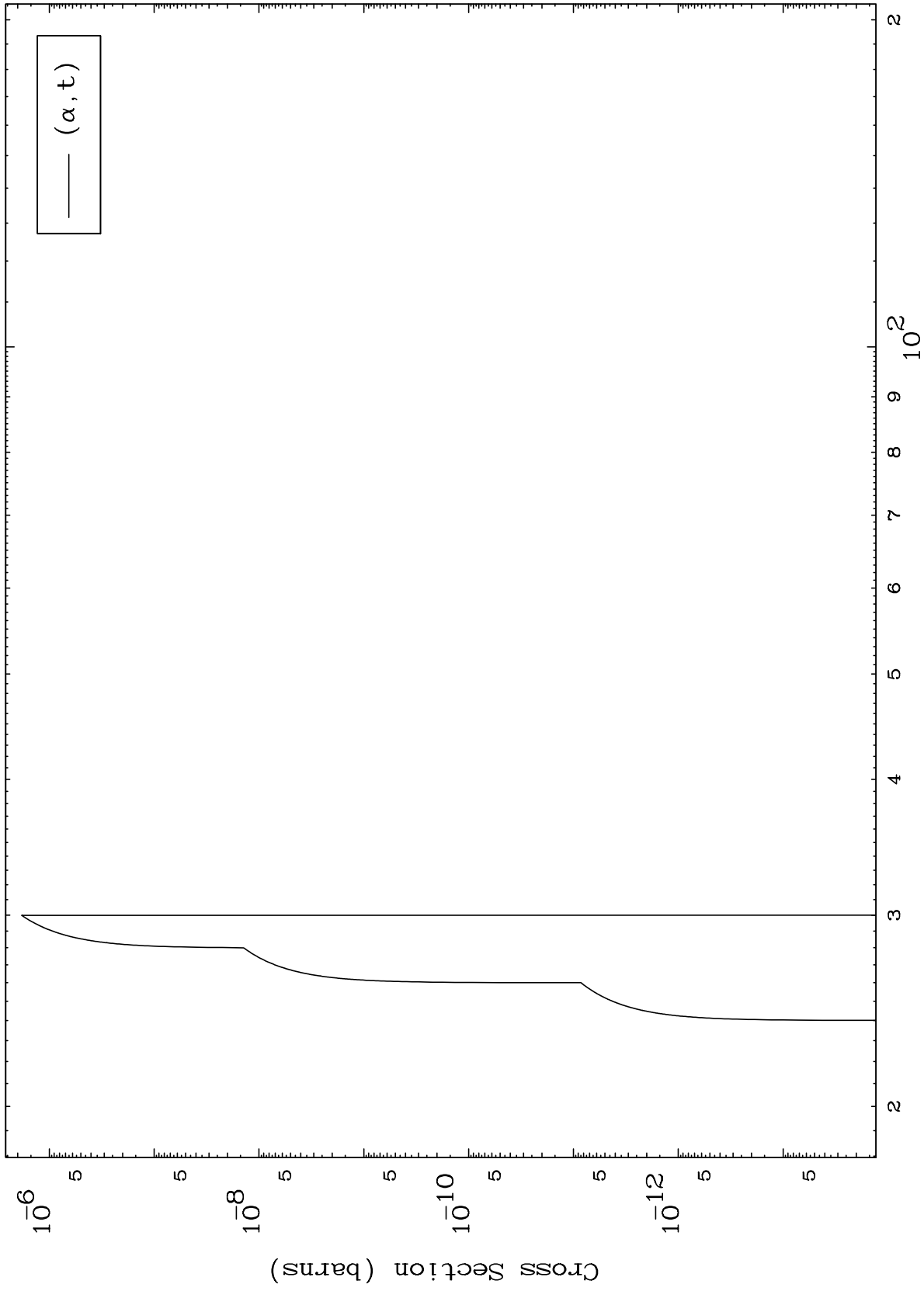
100-Fm-246



MAT 9927

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

100-Fm-246



8

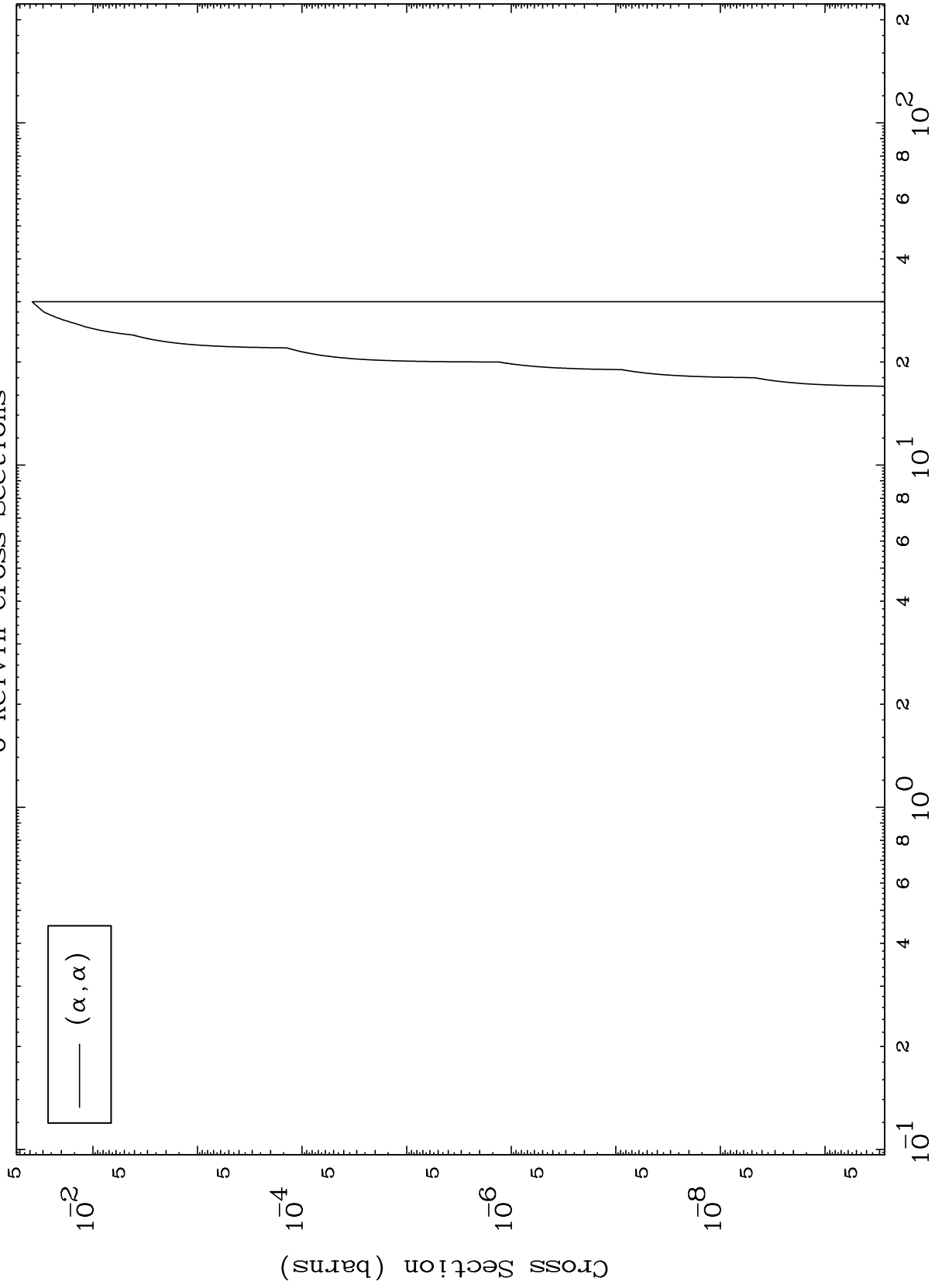
Incident Energy (MeV)

100-Fm-246

MAT 9927

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

100-Fm-246



9

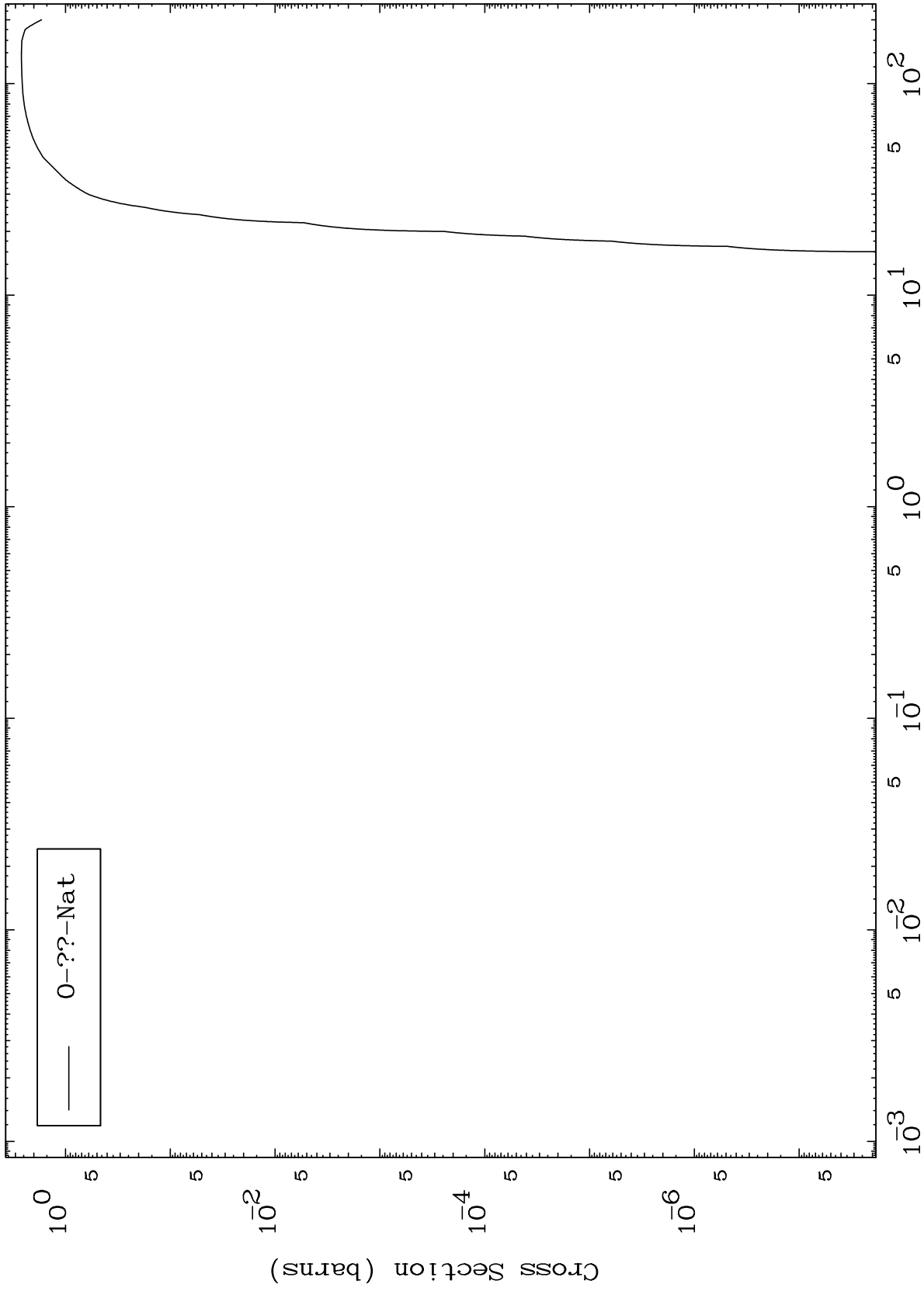
Incident Energy (MeV)

100-Fm-246

MAT 9927

$\alpha$  Fission  
Radionuclide Production Cross Section

100-Fm-246



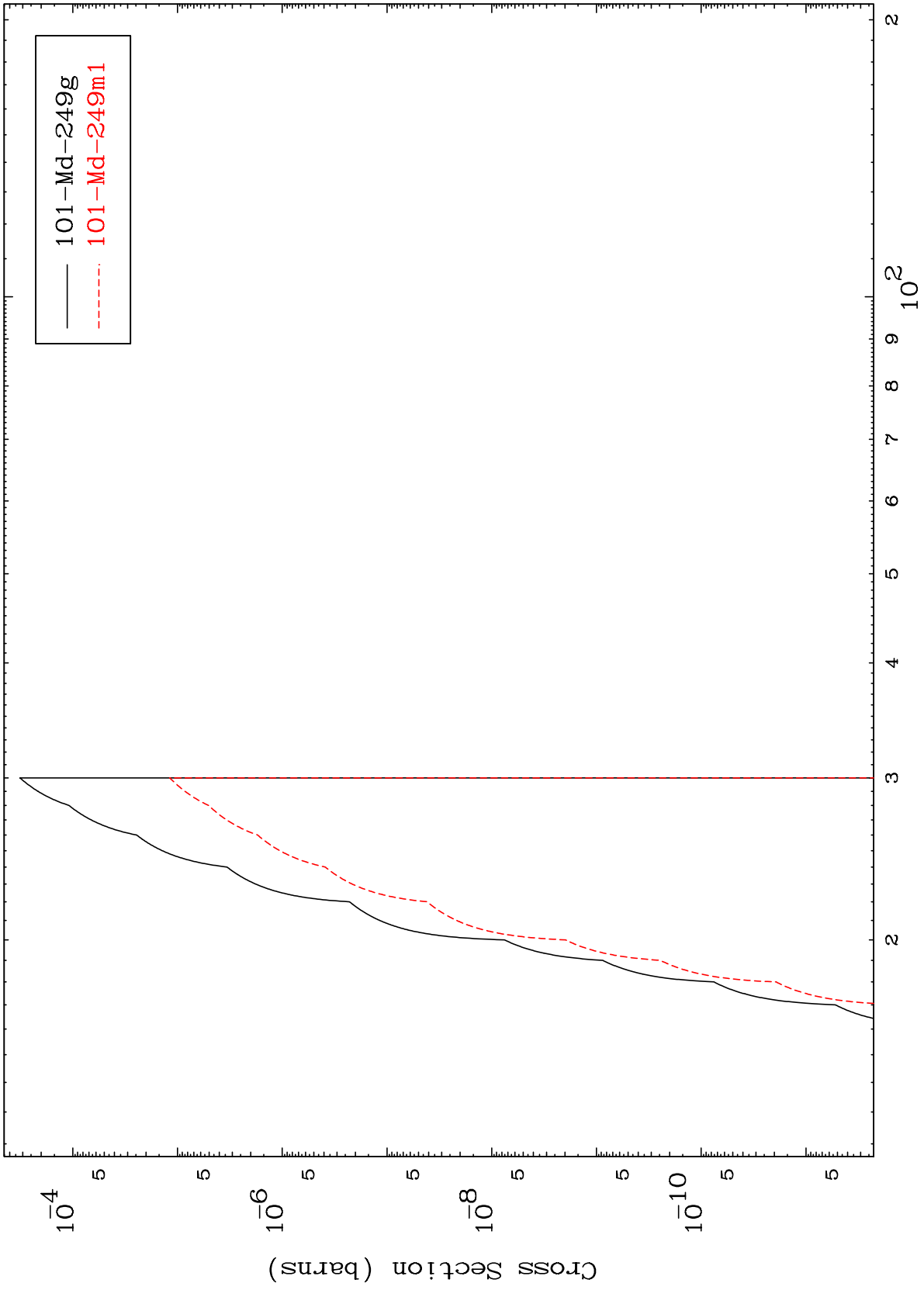
Incident Energy (MeV)

100-Fm-246

MAT 9927

100-Fm-246

( $\alpha, p$ )  
Radionuclide Production Cross Section



11

100-Fm-246

Incident Energy (MeV)