

Program EVALPLOT  
(Version 2018-1)

by

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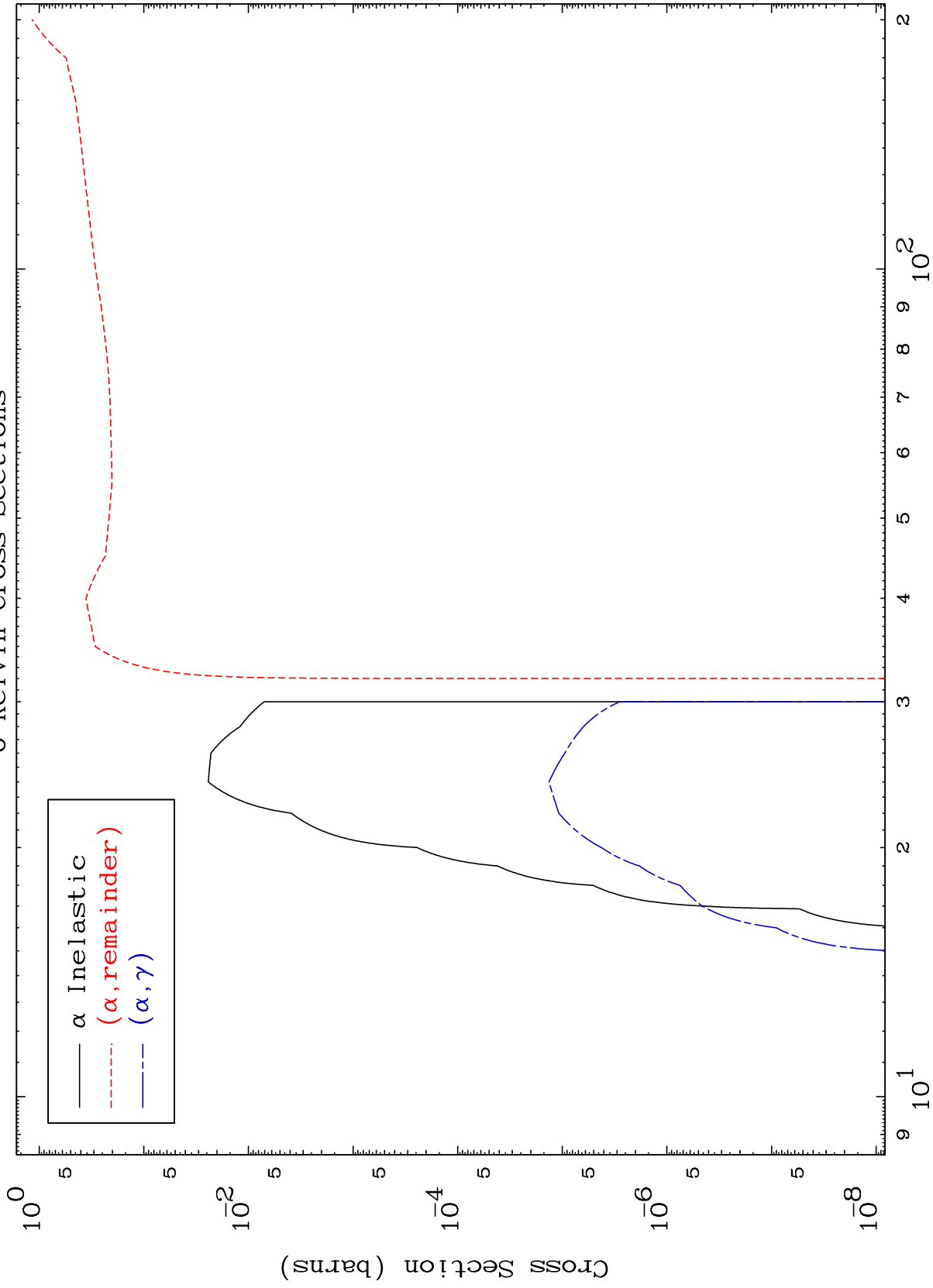
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9928

$\alpha$  Major  
0 Kelvin Cross Sections

100-Fm-247



1

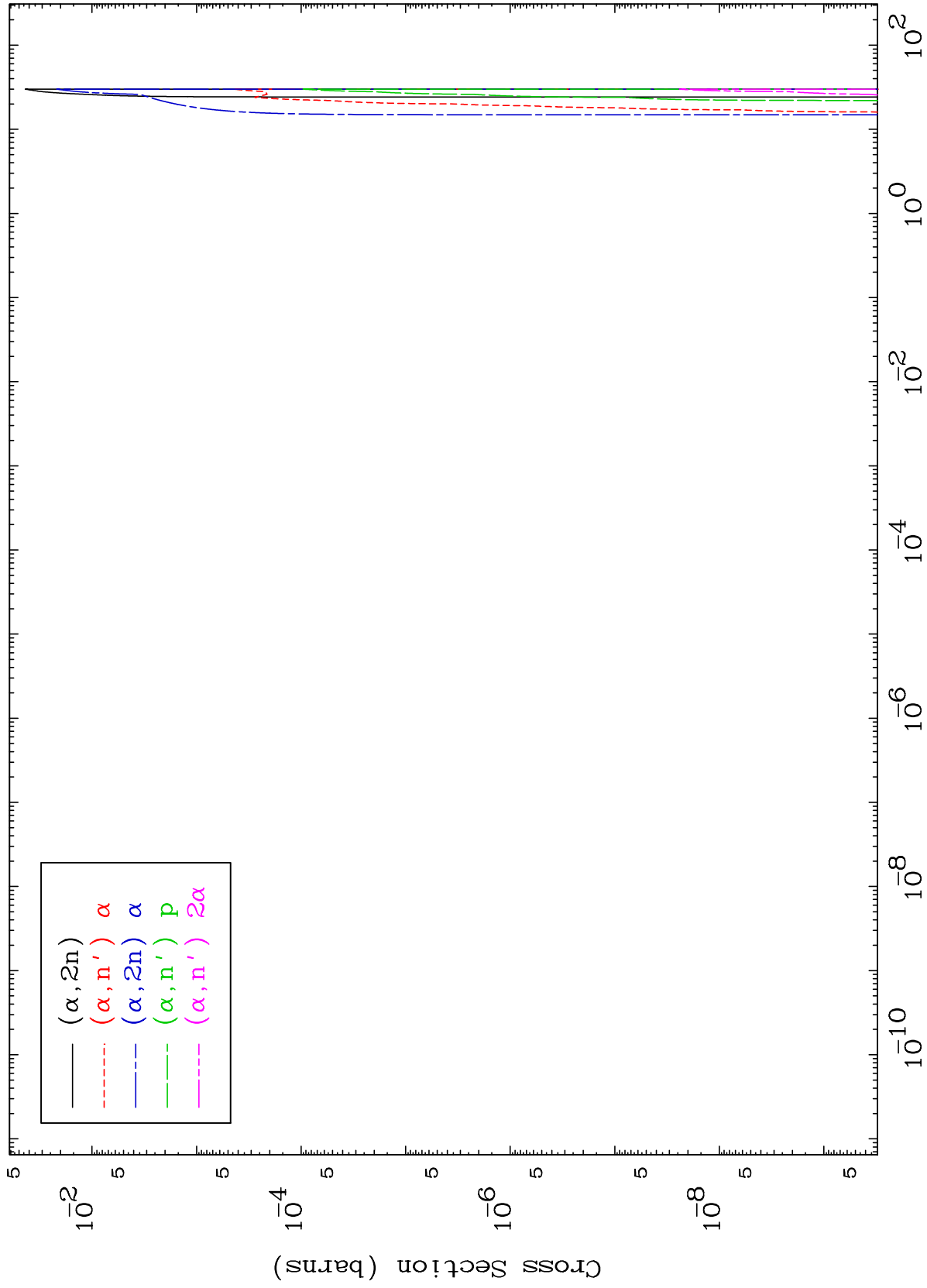
Incident Energy (MeV)

100-Fm-247

MAT 9928

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

100-Fm-247



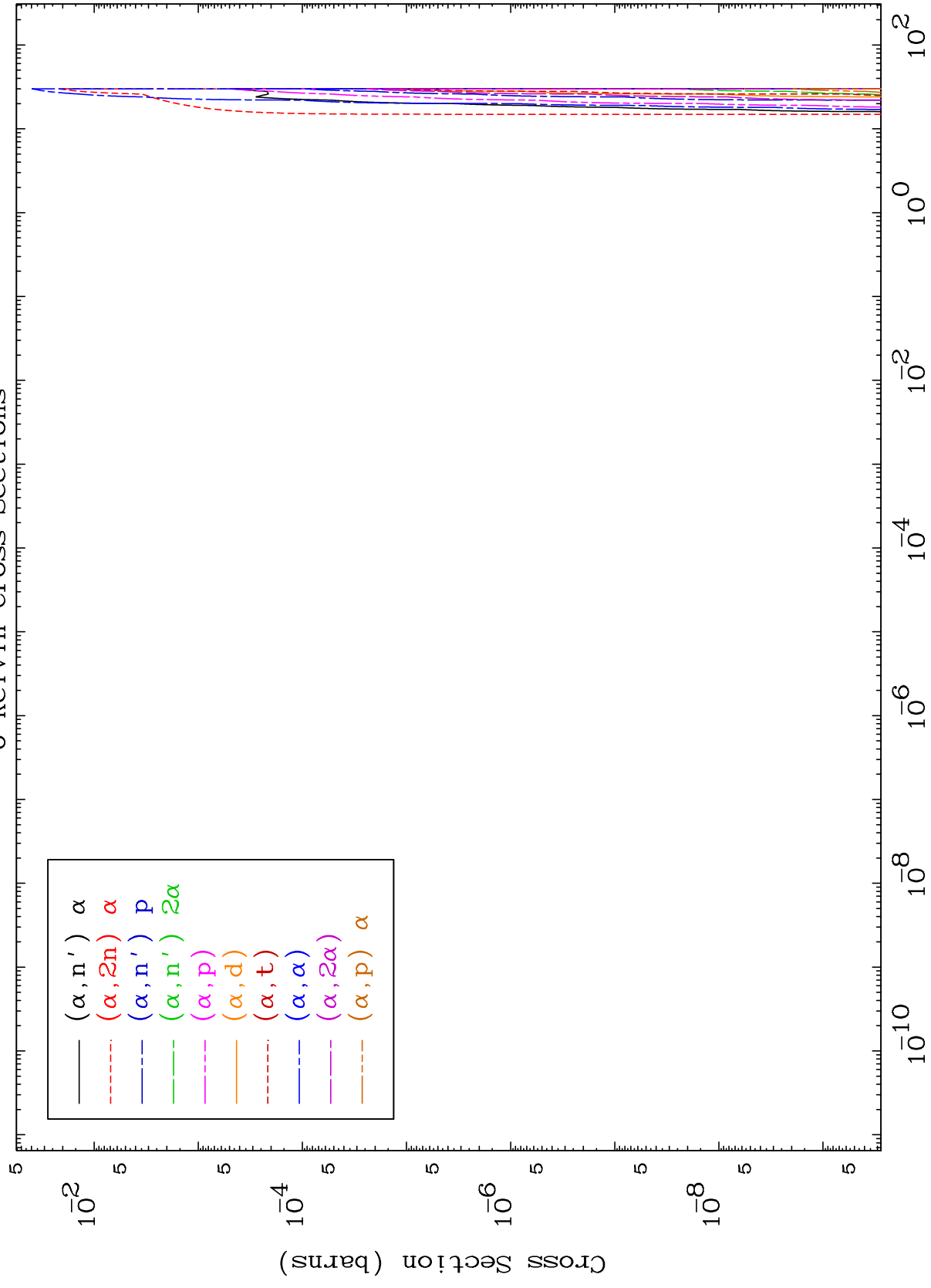
2

100-Fm-247

MAT 9928

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

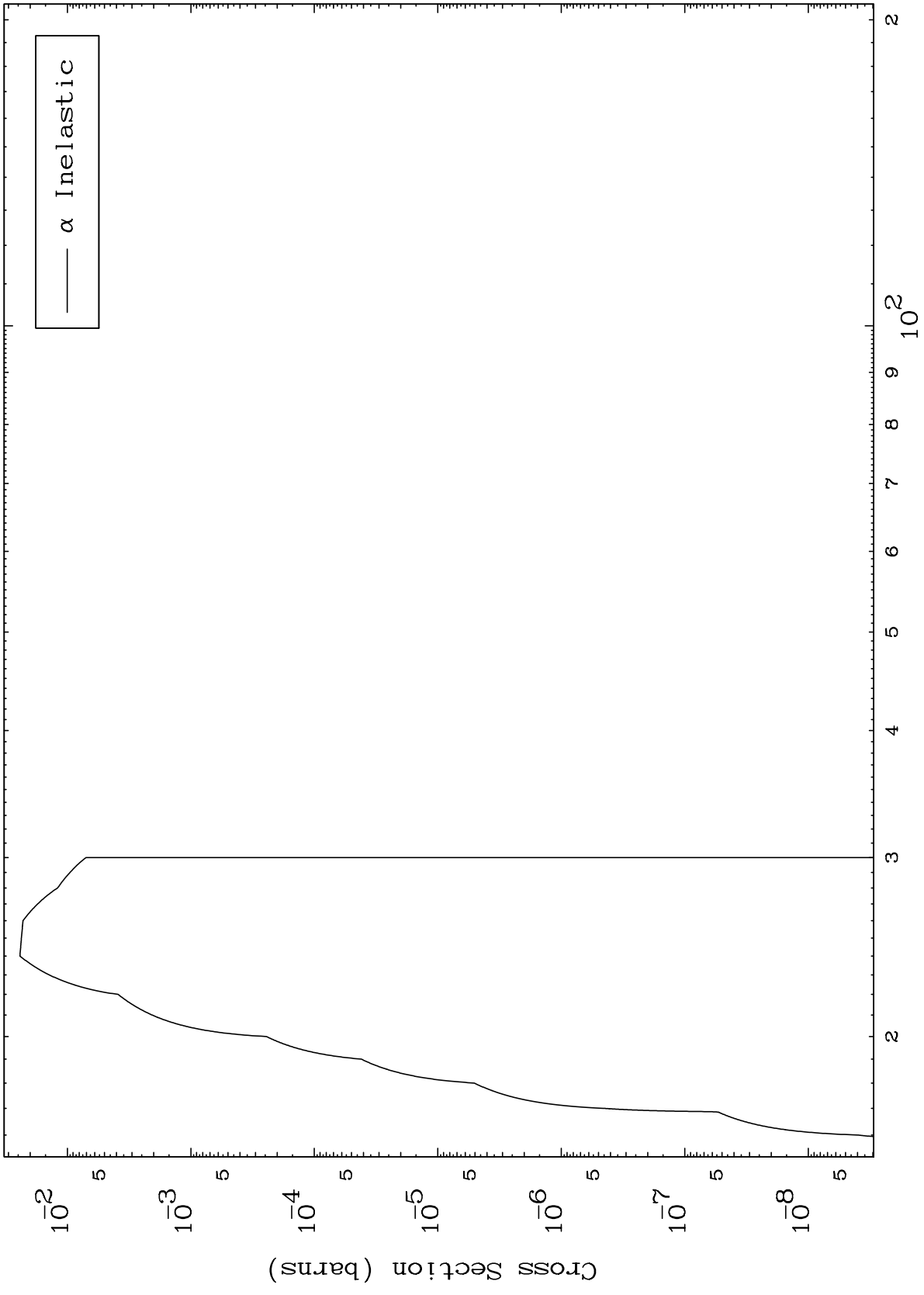
100-Fm-247



MAT 9928

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

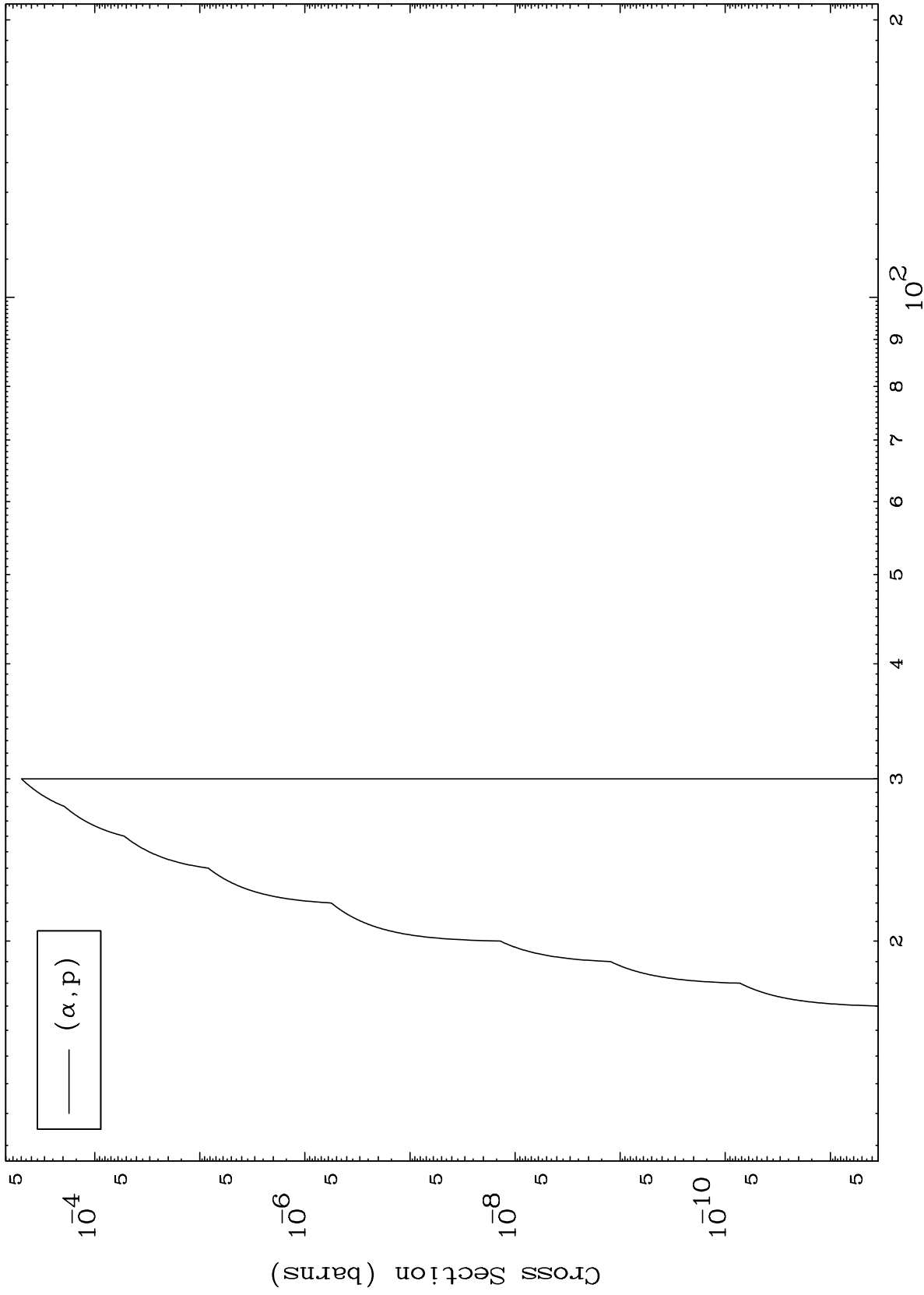
100-Fm-247



4

Incident Energy (MeV)

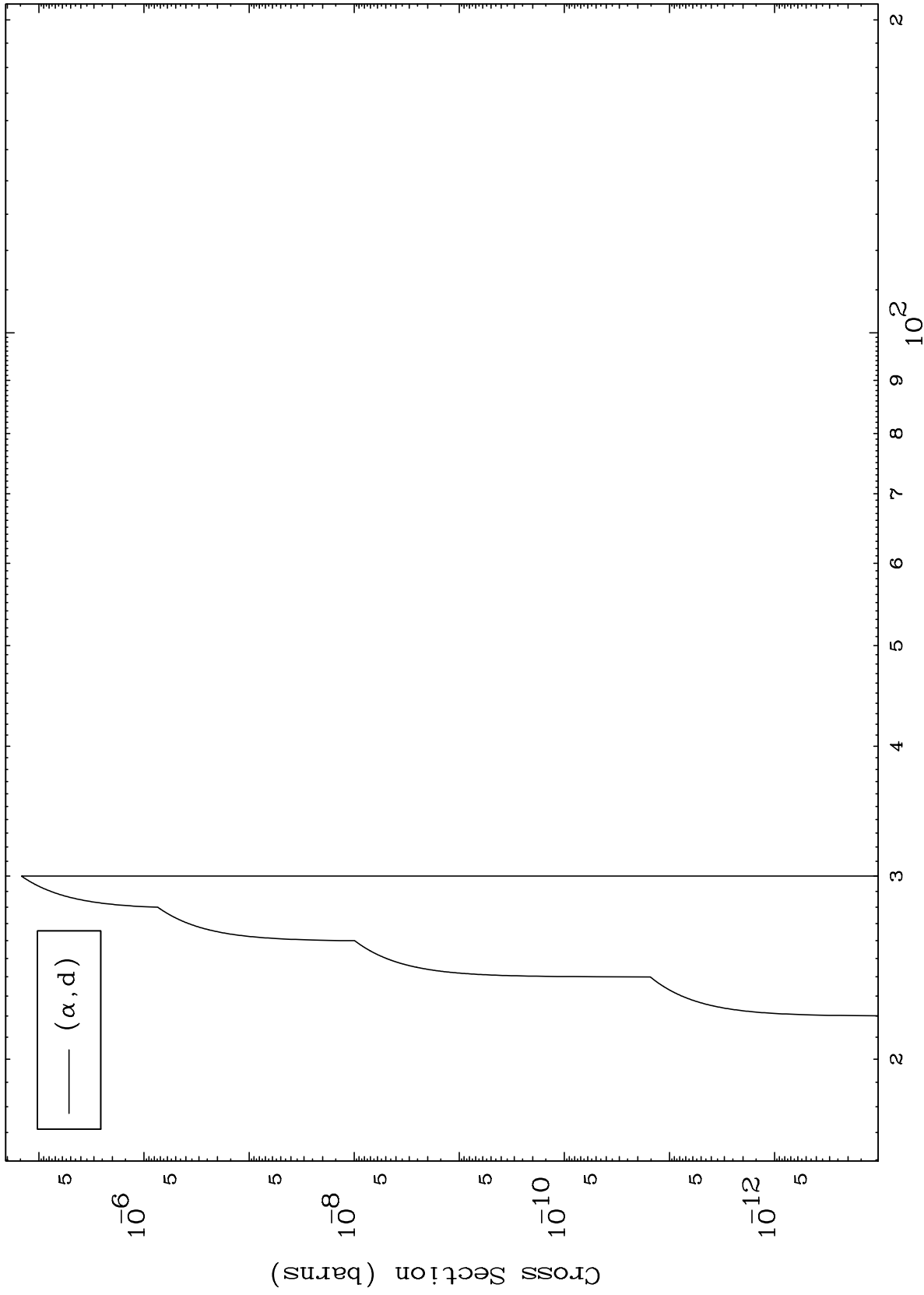
100-Fm-247



MAT 9928

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

100-Fm-247



6

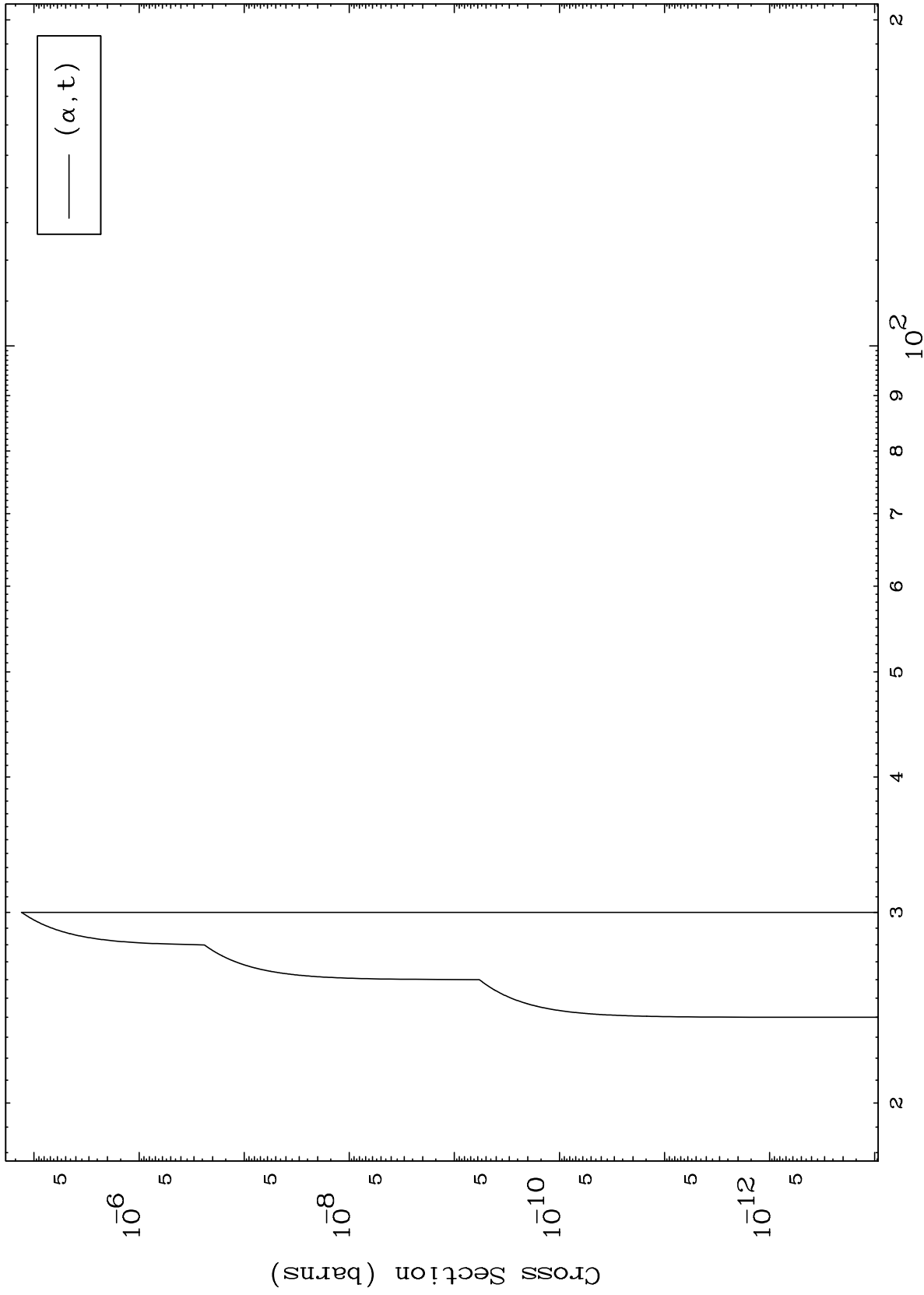
Incident Energy (MeV)

100-Fm-247

MAT 9928

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

100-Fm-247



7

Incident Energy (MeV)

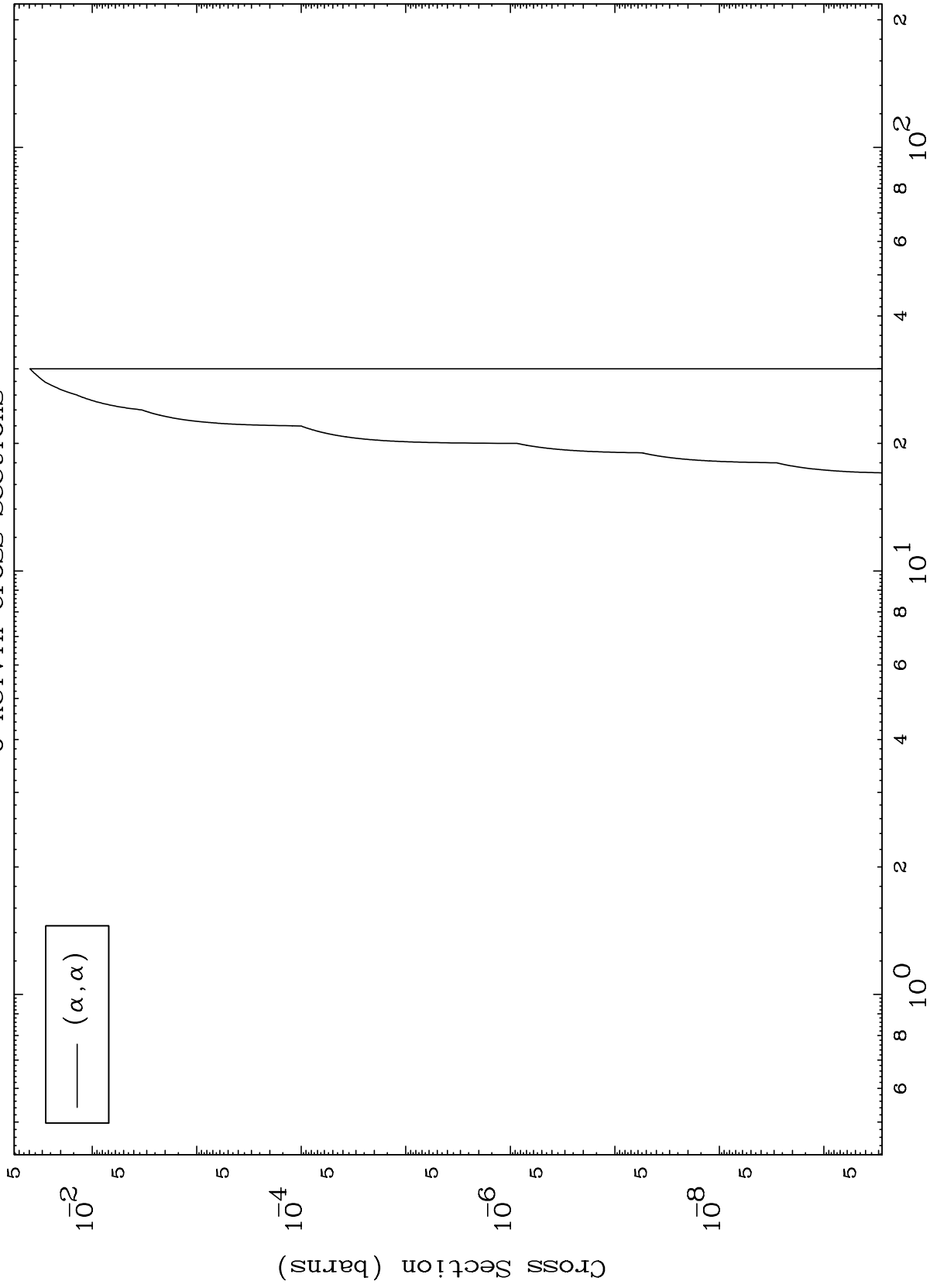
100-Fm-247



MAT 9928

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

100-Fm-247



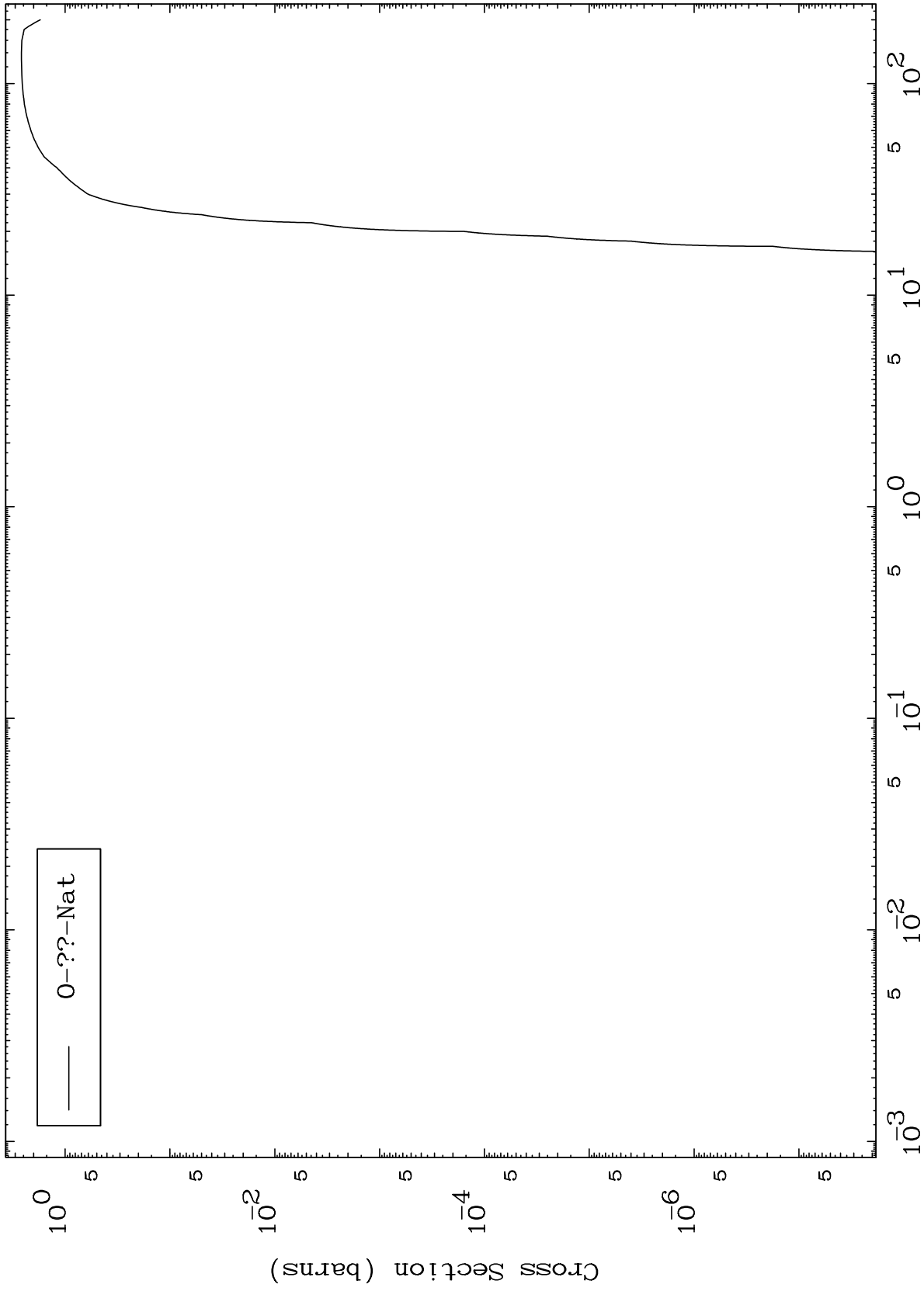
8

100-Fm-247

MAT 9928

$\alpha$  Fission  
Radionuclide Production Cross Section

100-Fm-247



9

Incident Energy (MeV)

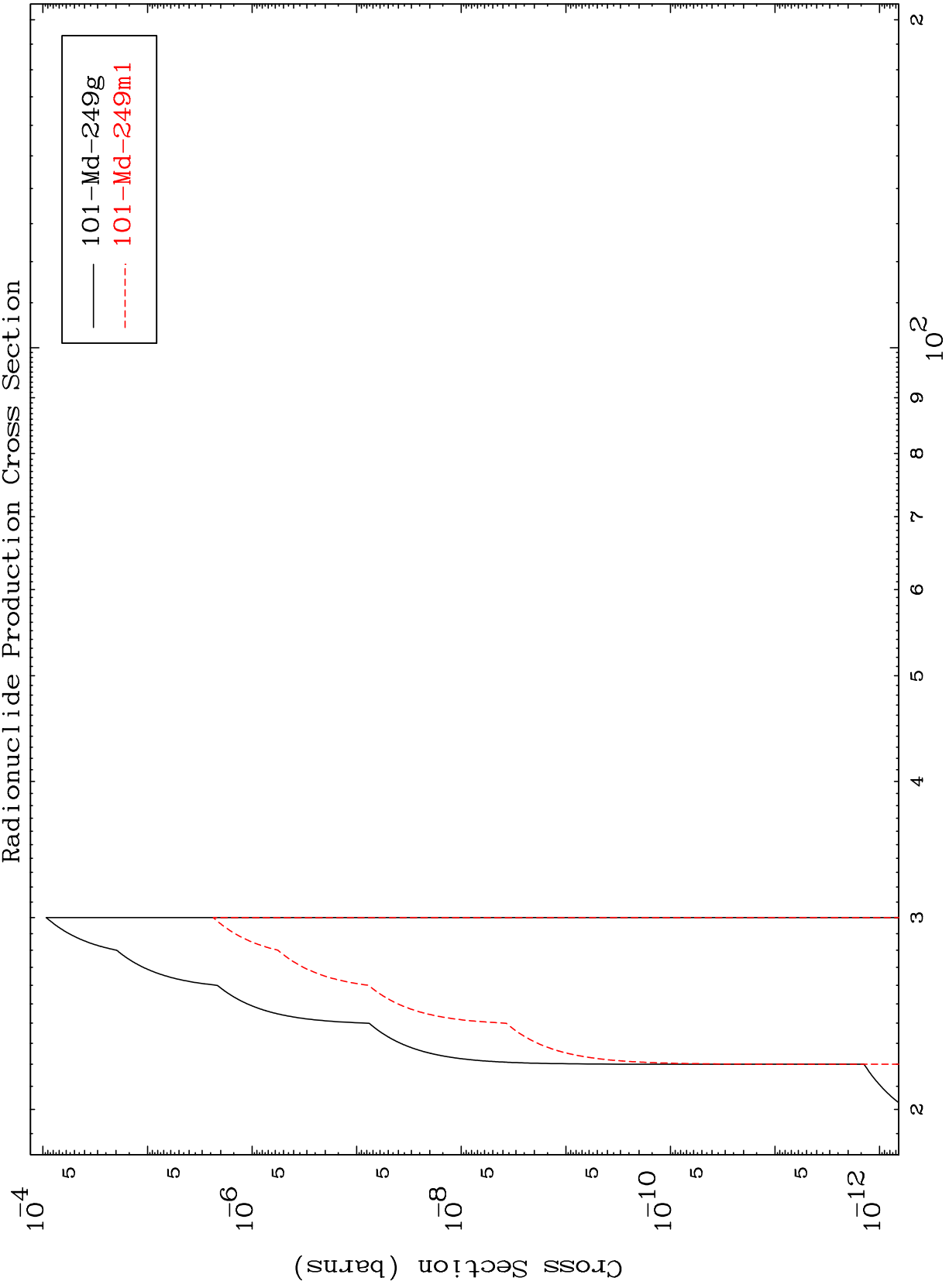
100-Fm-247

MAT 9928

( $\alpha, n'$ ) p

100-Fm-247

Radionuclide Production Cross Section



10

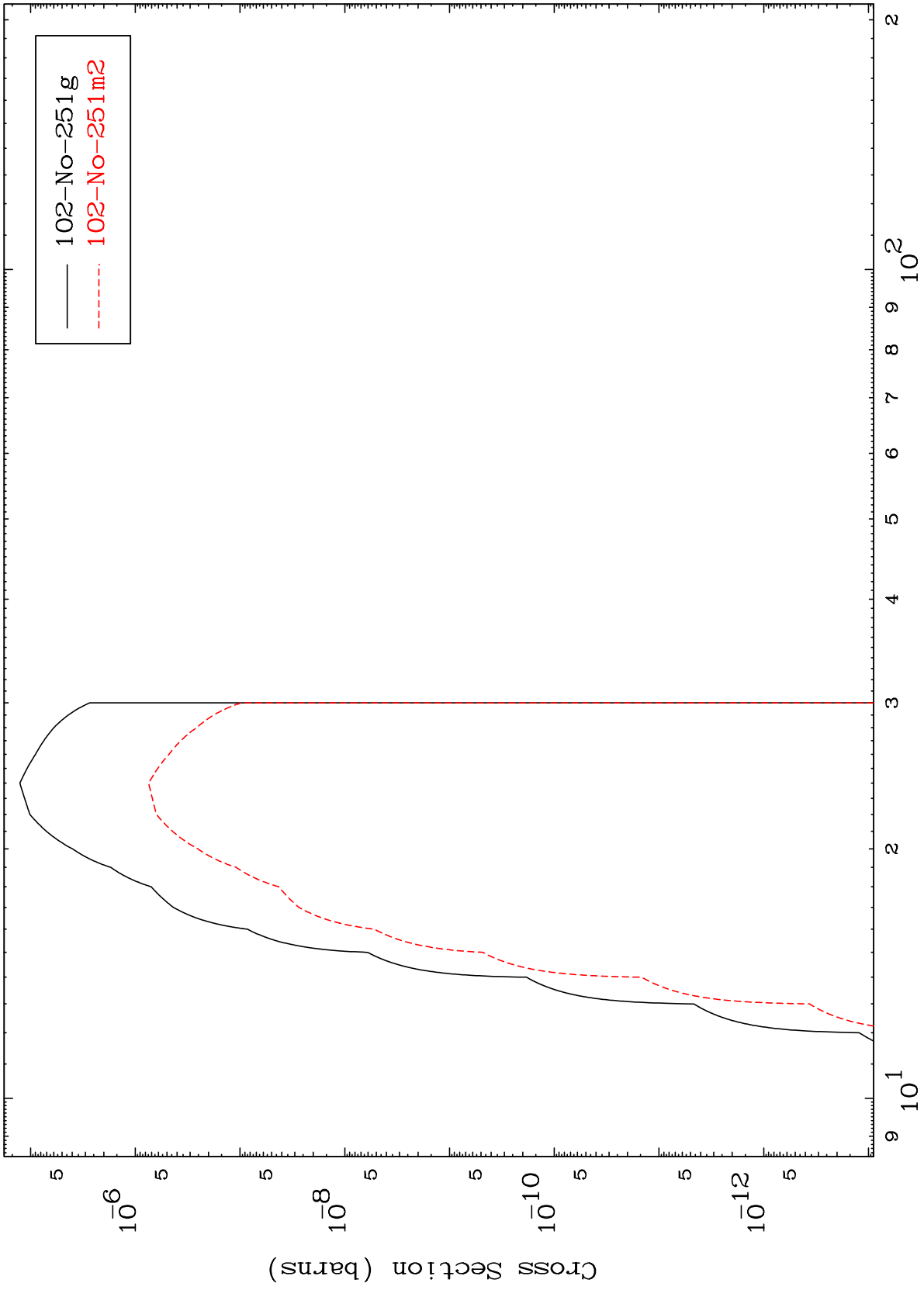
Incident Energy (MeV)

100-Fm-247

MAT 9928

100-Fm-247

$(\alpha, \gamma)$   
Radionuclide Production Cross Section



11

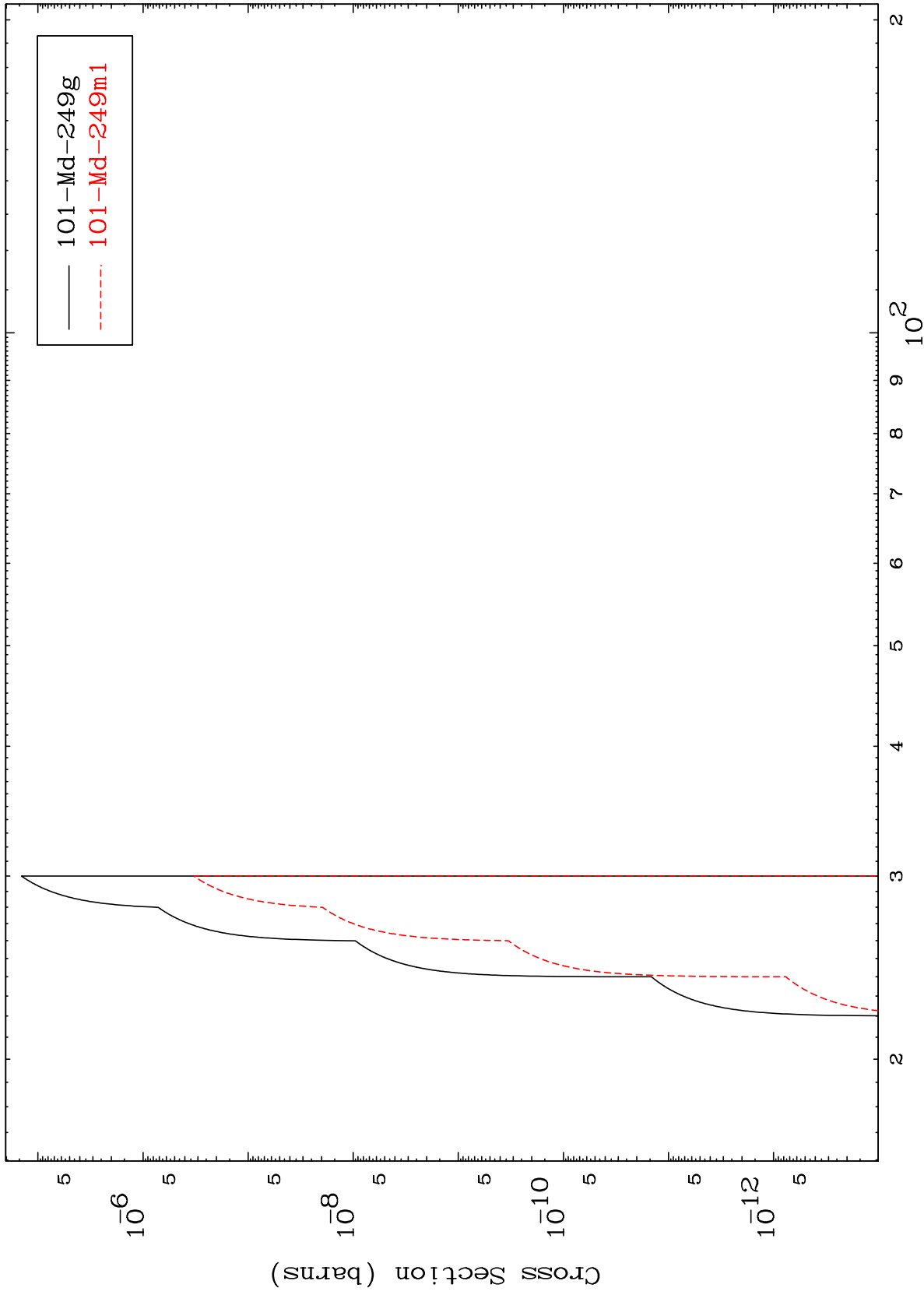
100-Fm-247

MAT 9928

( $\alpha, d$ )

100-Fm-247

Radionuclide Production Cross Section



12

Incident Energy (MeV)

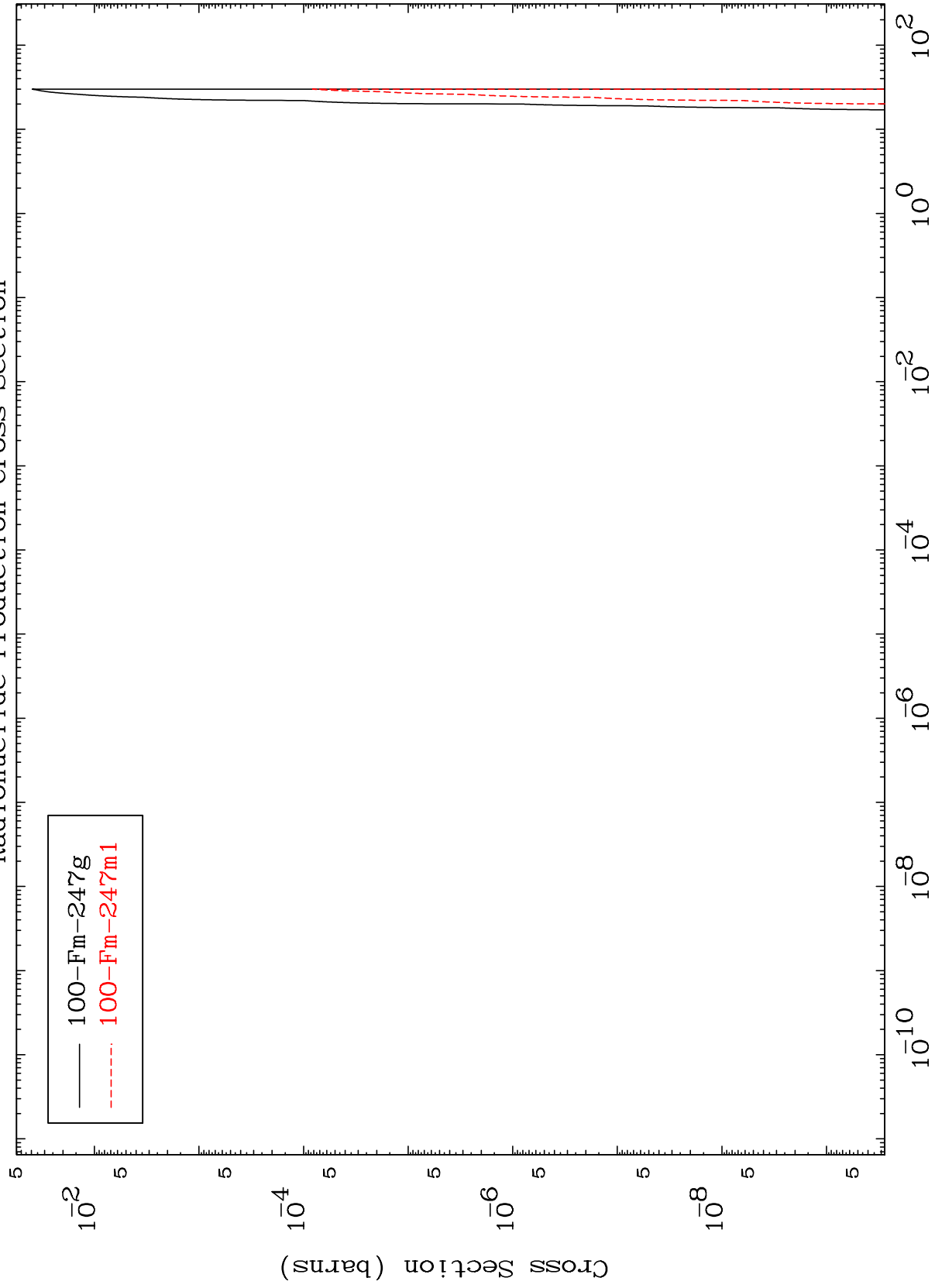
100-Fm-247

MAT 9928

( $\alpha, \alpha$ )

100-Fm-247

Radionuclide Production Cross Section



13

Incident Energy (MeV)

100-Fm-247