

Program EVALPLOT  
(Version 2018-1)

by

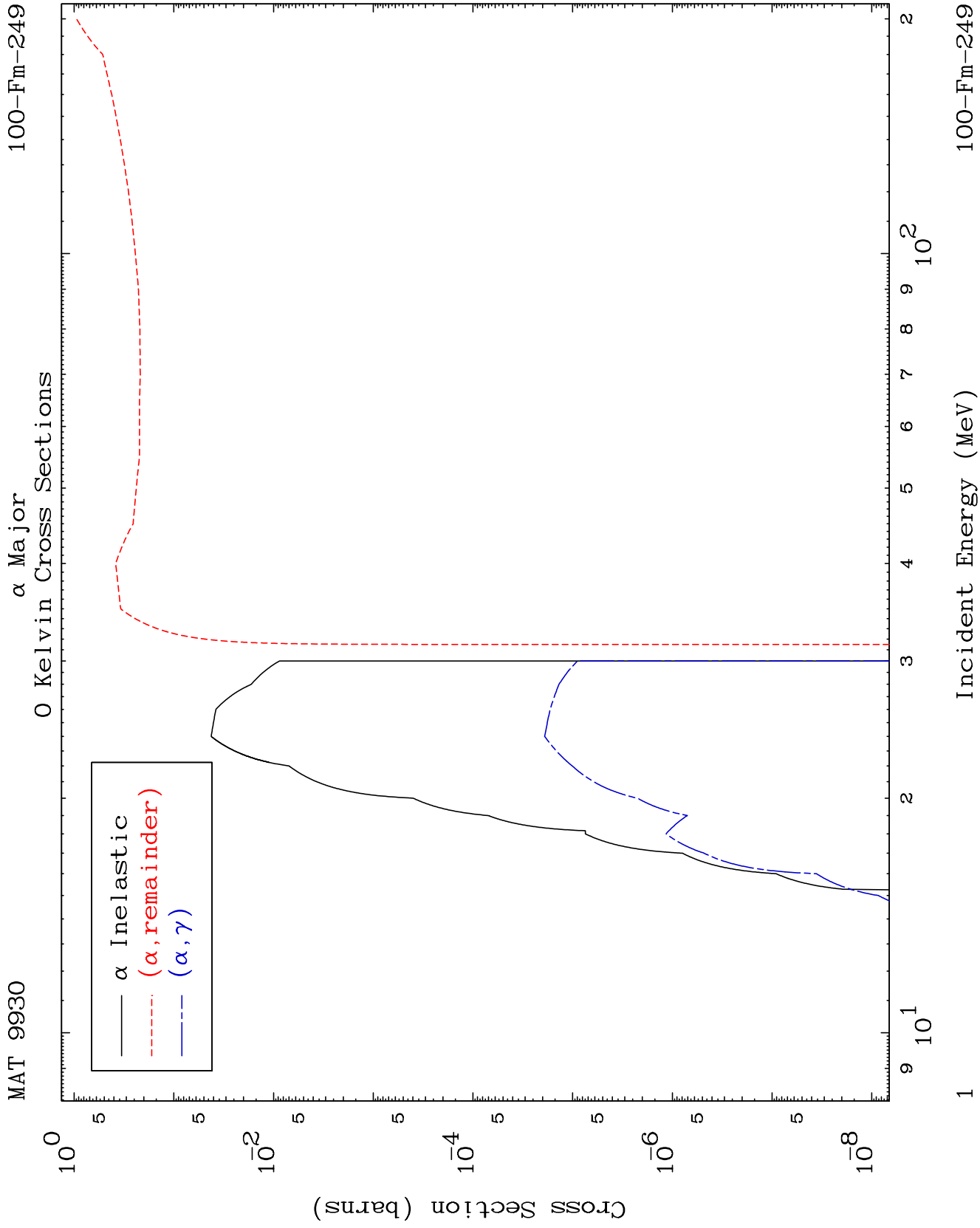
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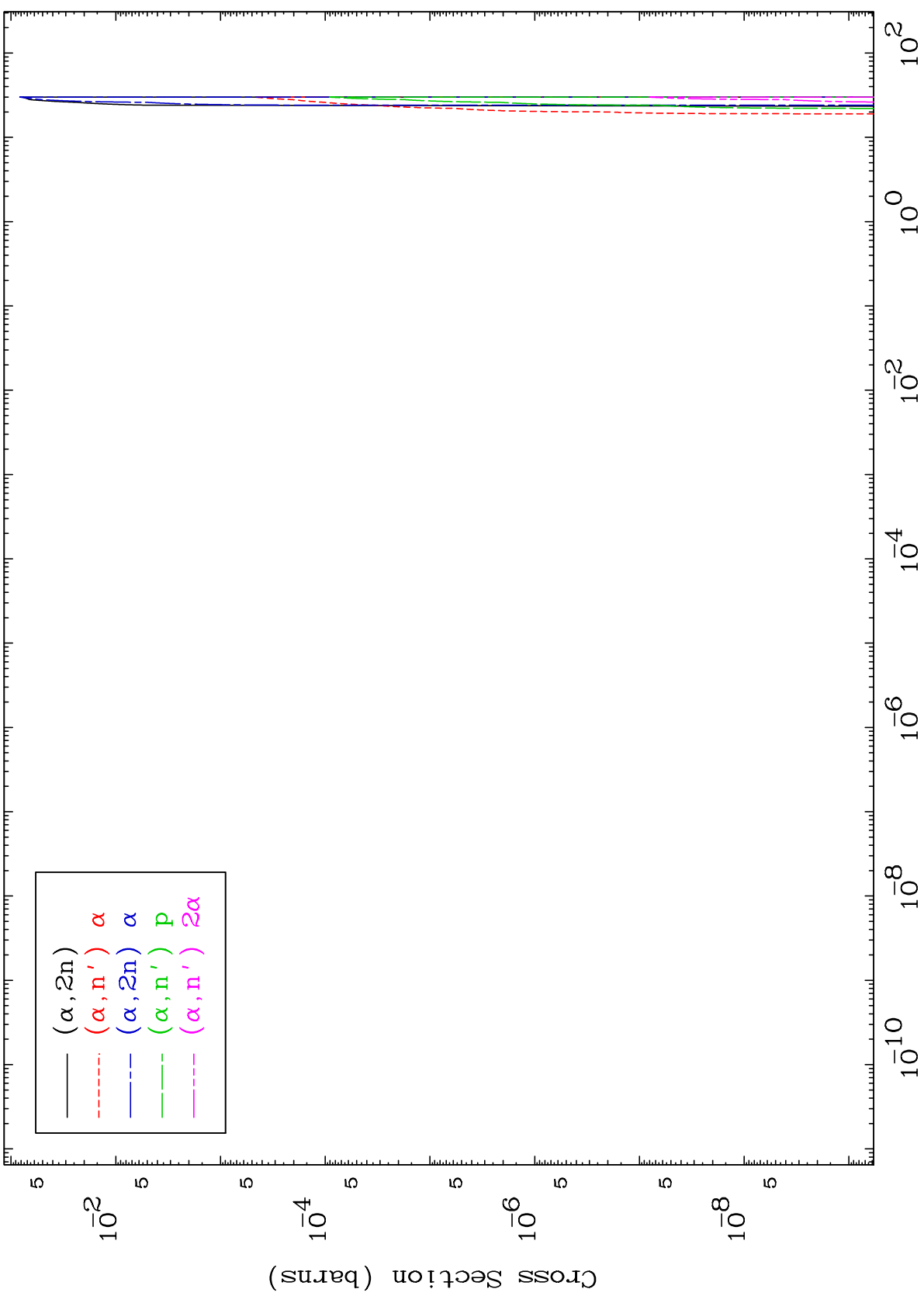
Press Mouse Button to Start



MAT 9930

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

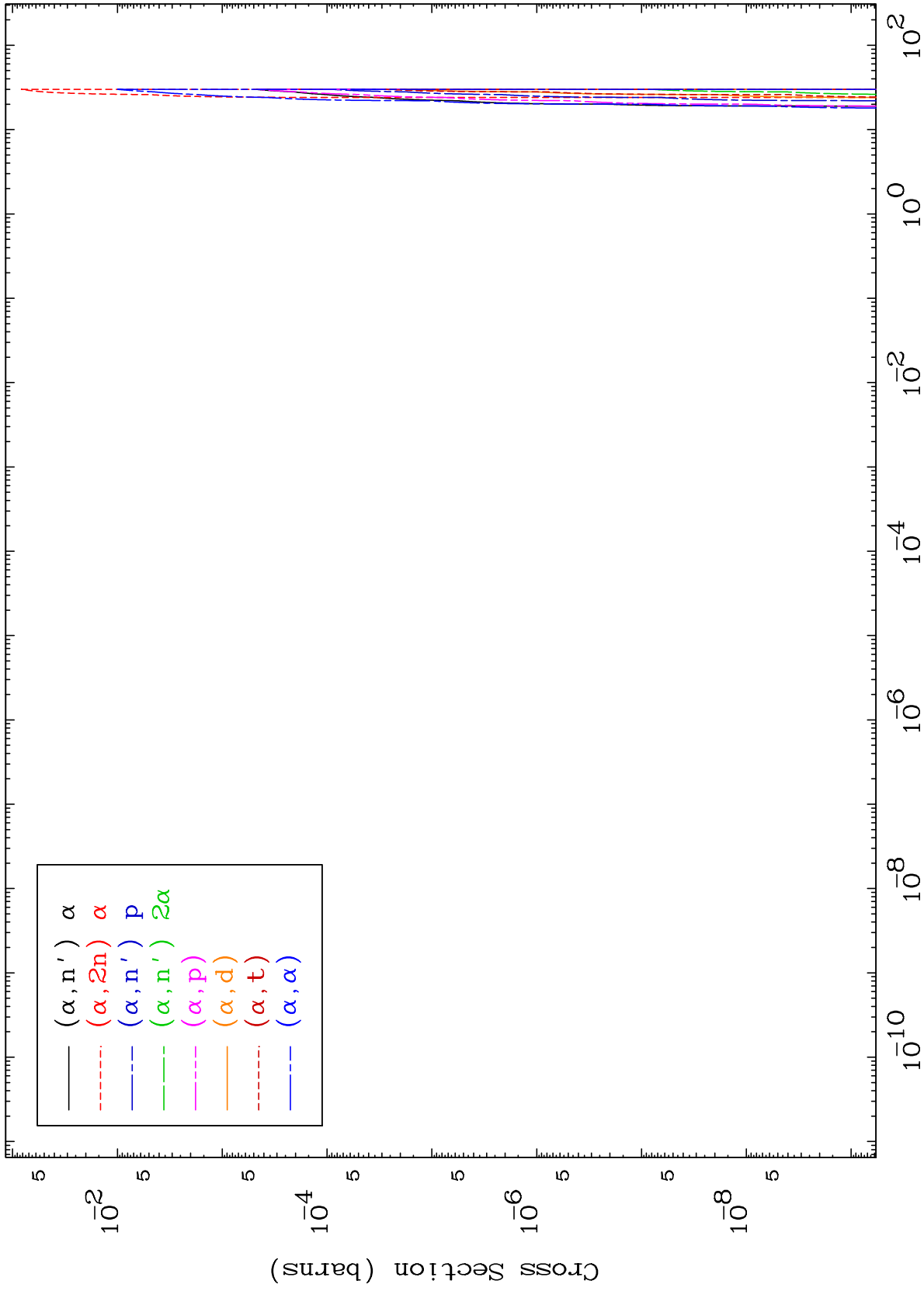
100-Fm-249



MAT 9930

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

100-Fm-249



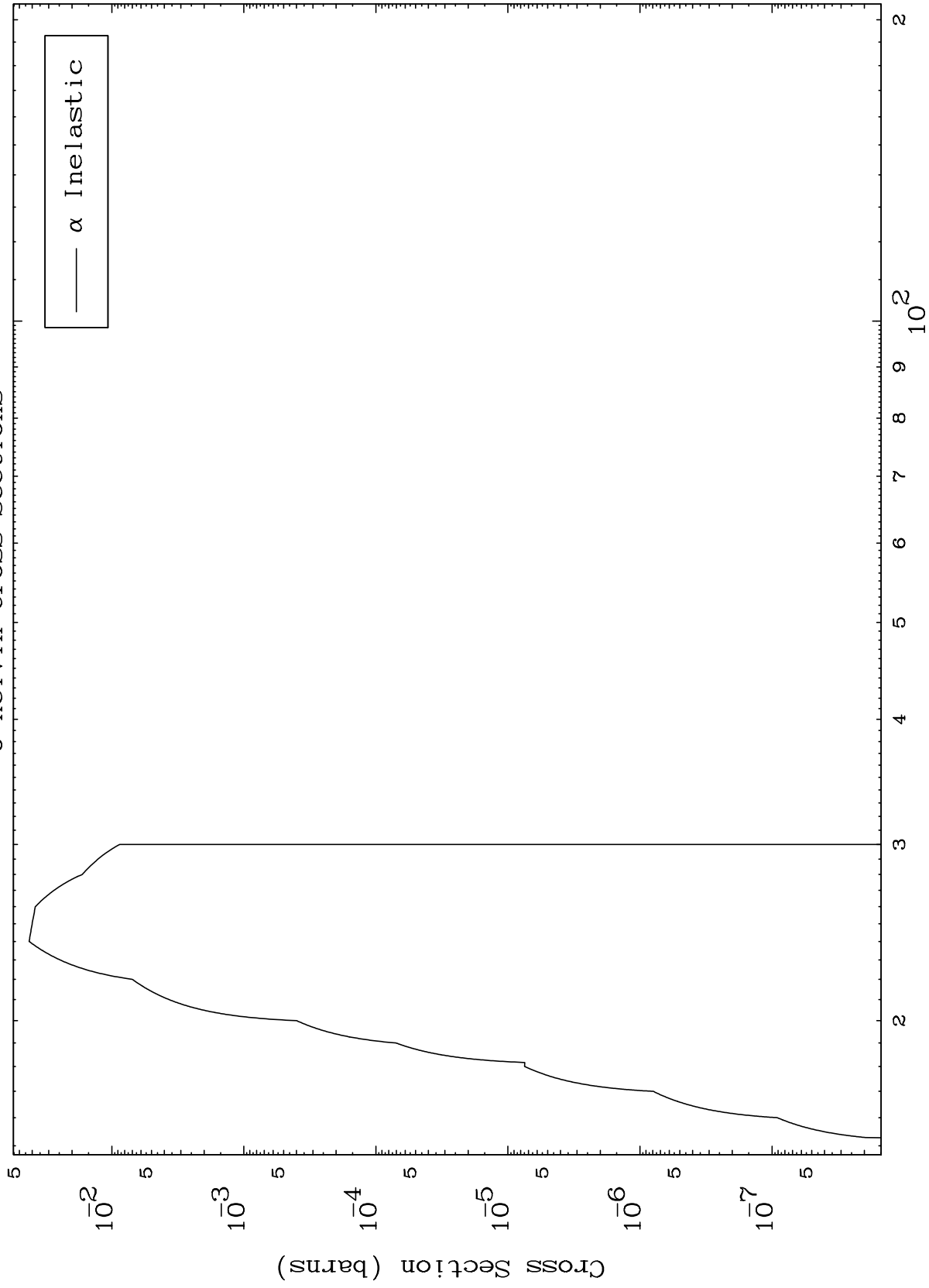
3

100-Fm-249

MAT 9930

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

100-Fm-249



4

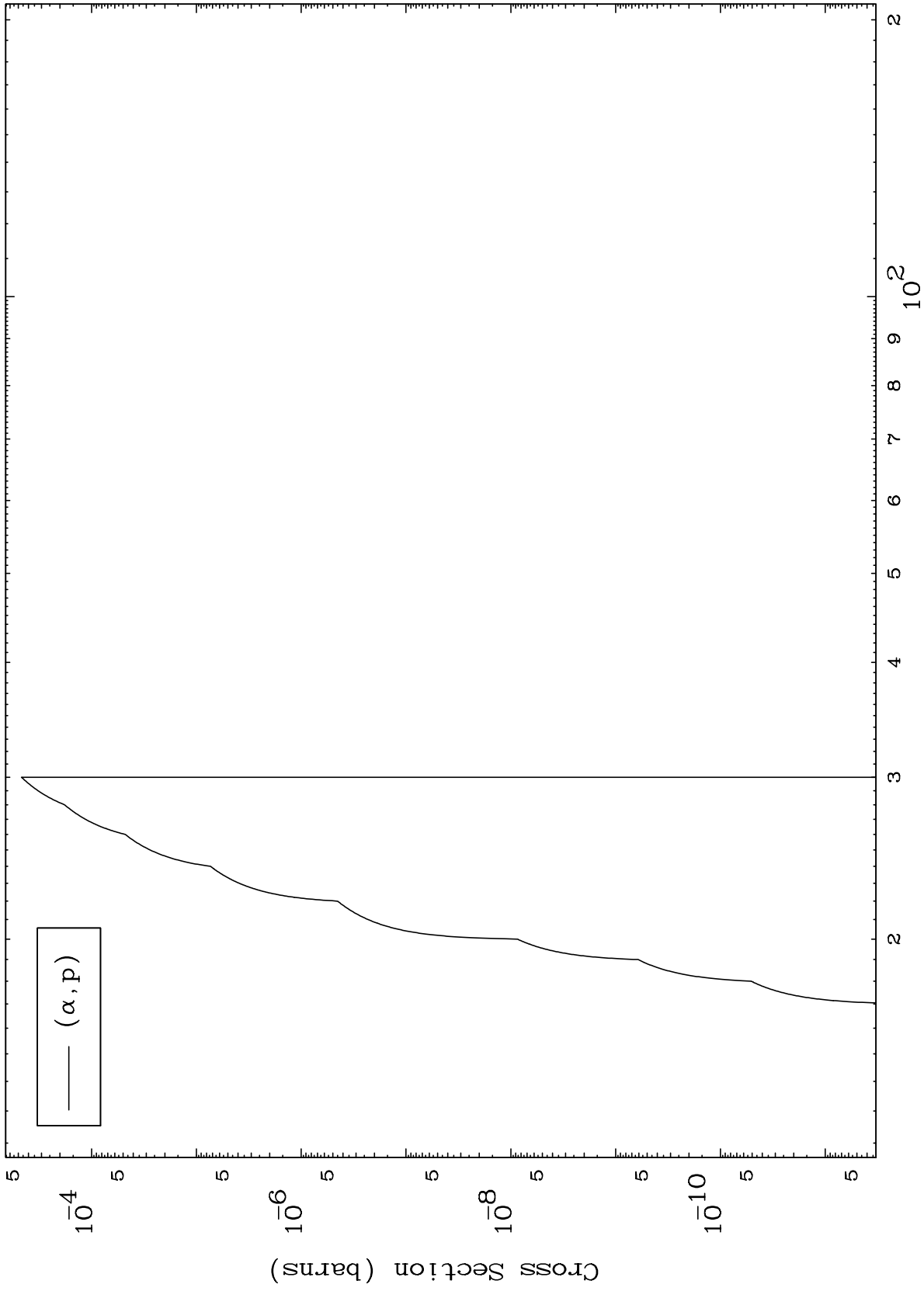
Incident Energy (MeV)

100-Fm-249

MAT 9930

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

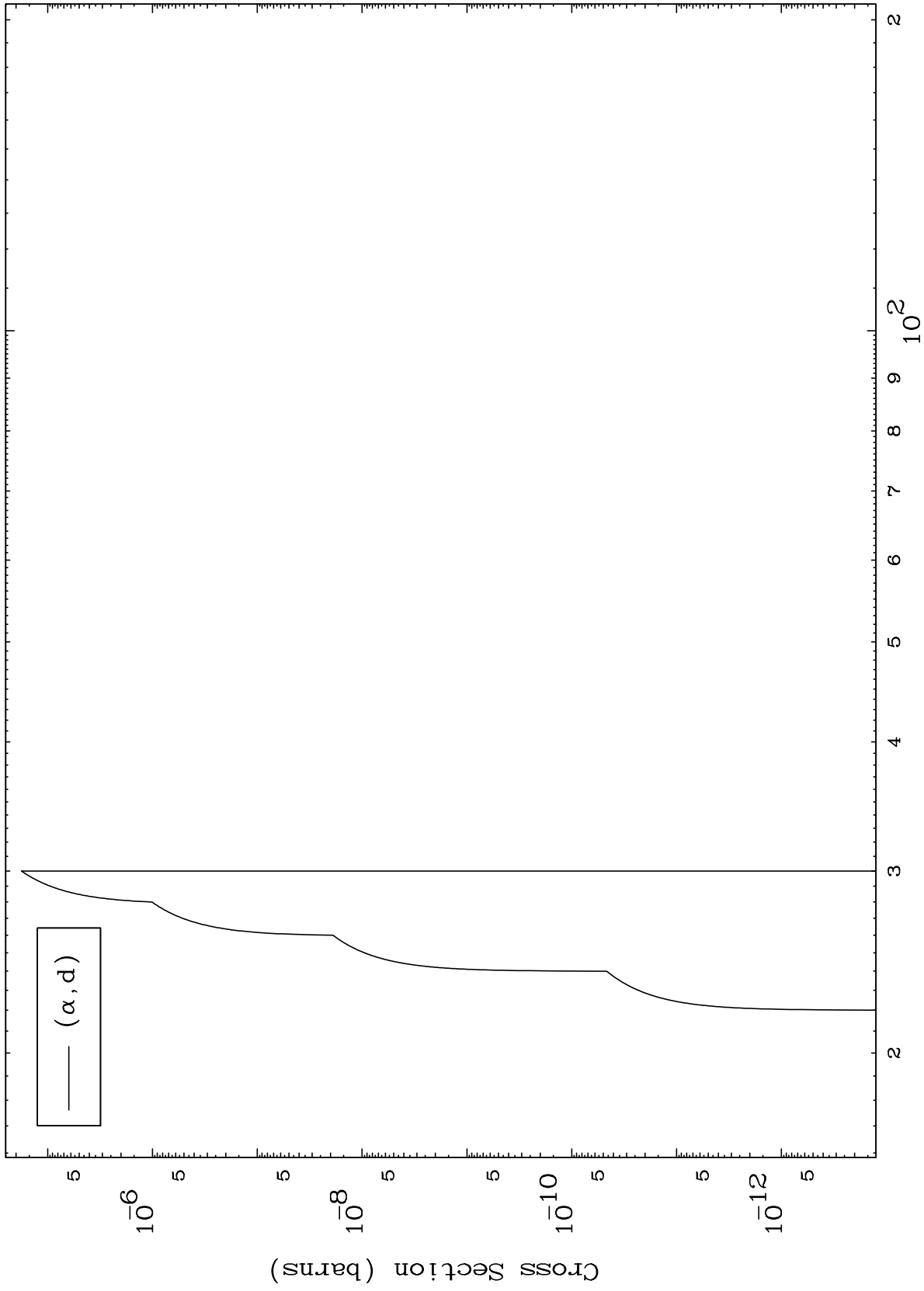
100-Fm-249



MAT 9930

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

100-Fm-249



6

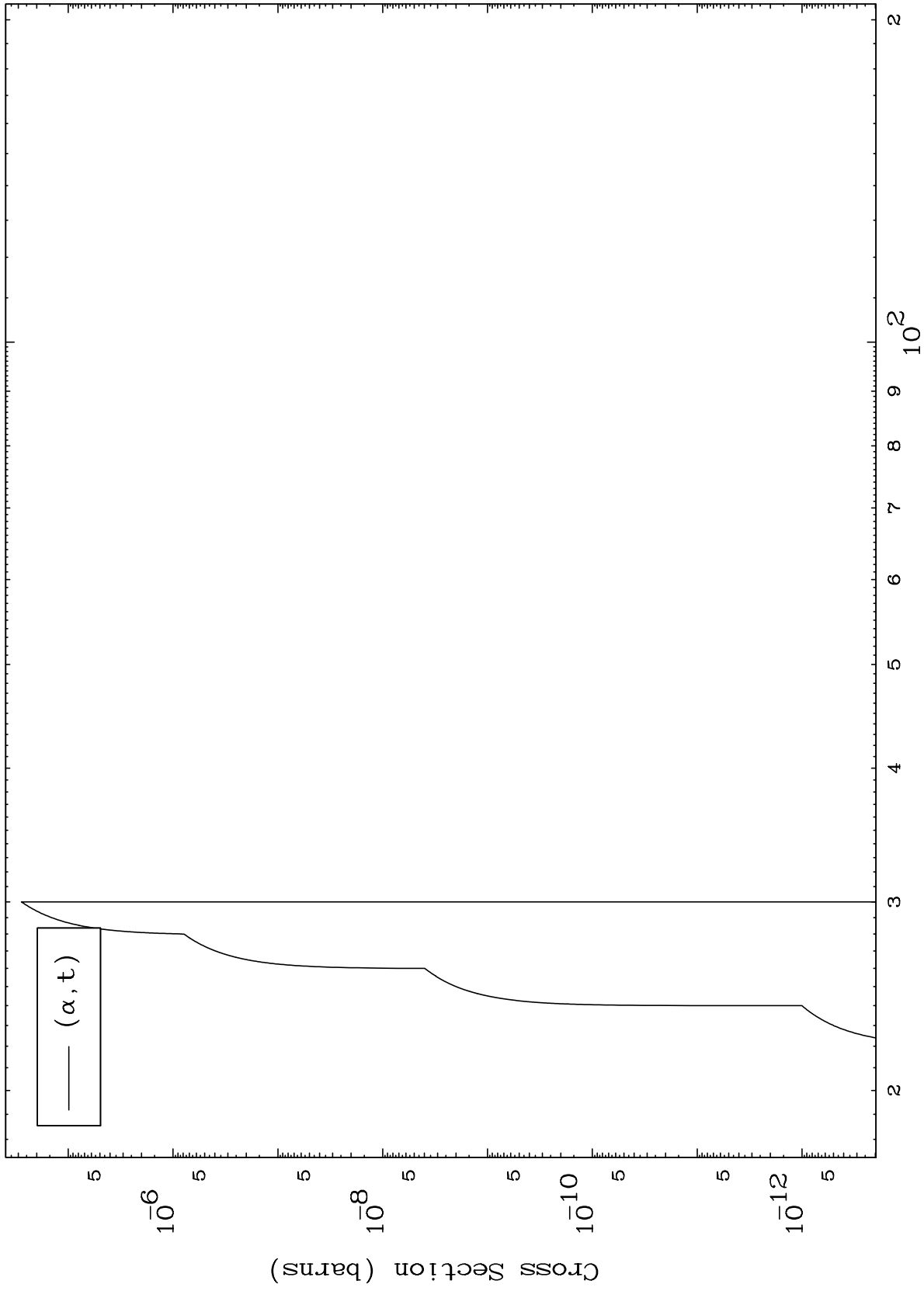
Incident Energy (MeV)

100-Fm-249

MAT 9930

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

100-Fm-249



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Incident Energy (MeV)

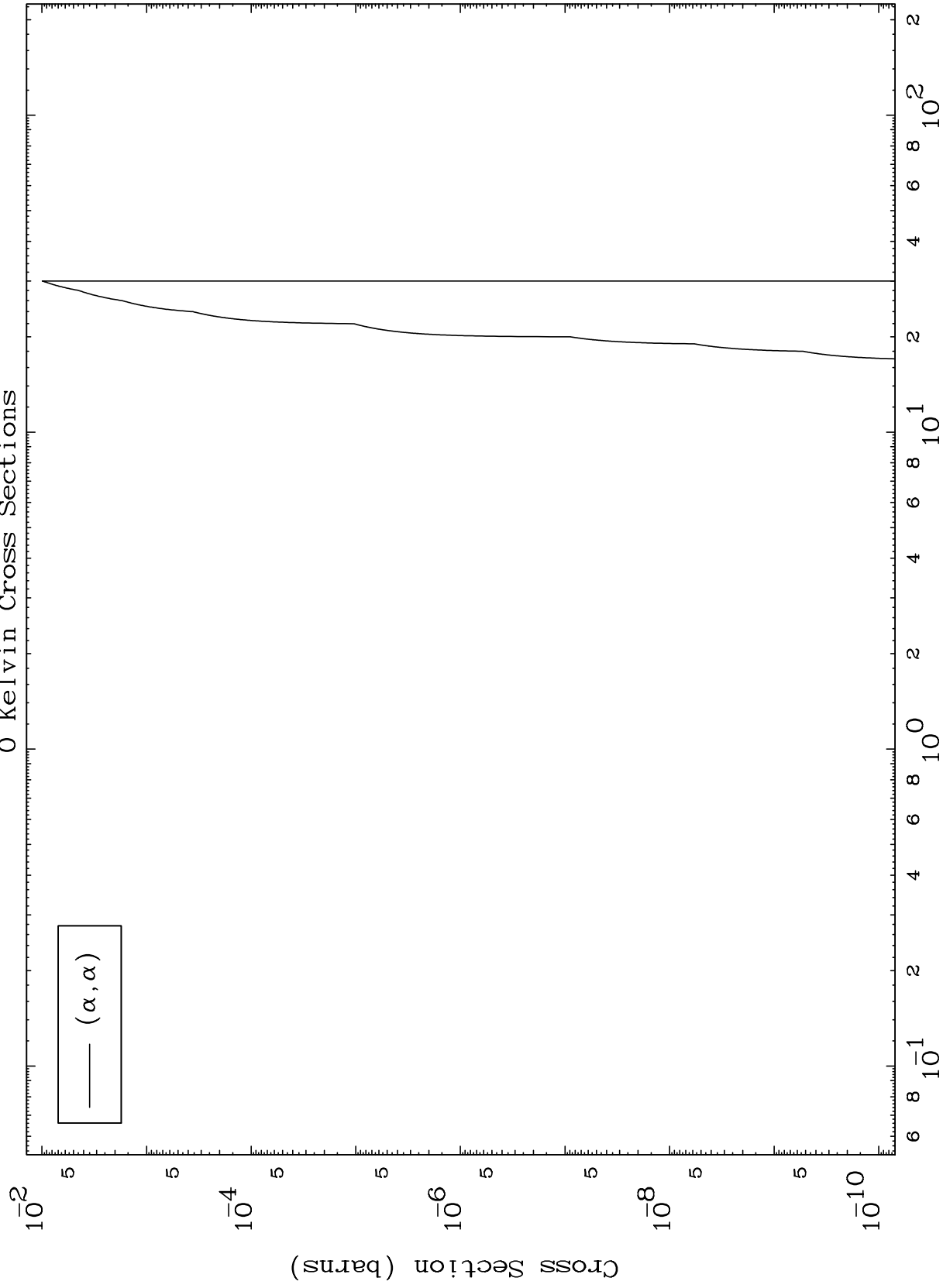
100-Fm-249



MAT 9930

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

100-Fm-249



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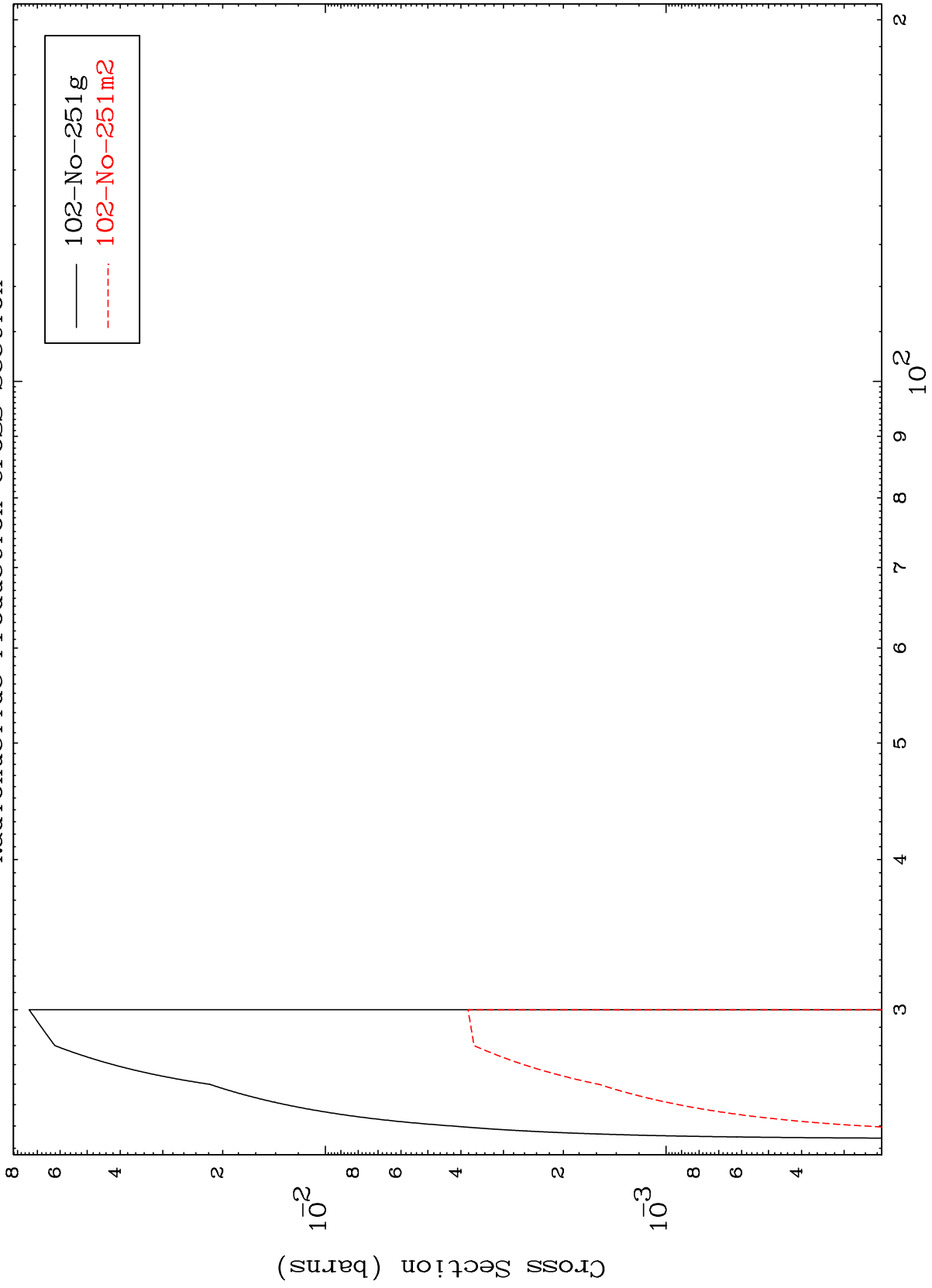
Incident Energy (MeV)

100-Fm-249

MAT 9930

100-Fm-249

( $\alpha, 2n$ )  
Radionuclide Production Cross Section



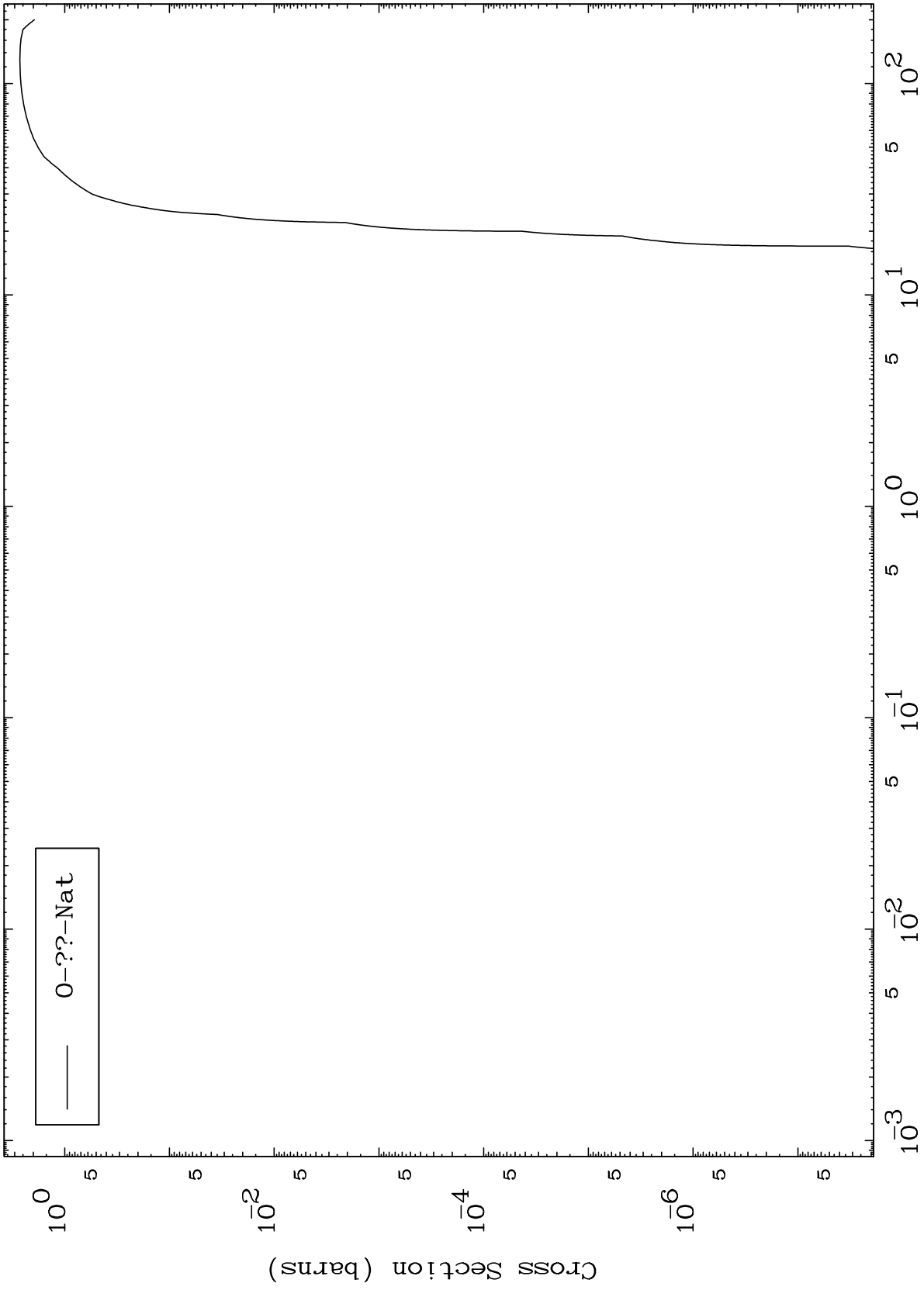
9

100-Fm-249

MAT 9930

$\alpha$  Fission  
Radionuclide Production Cross Section

100-Fm-249



Incident Energy (MeV)

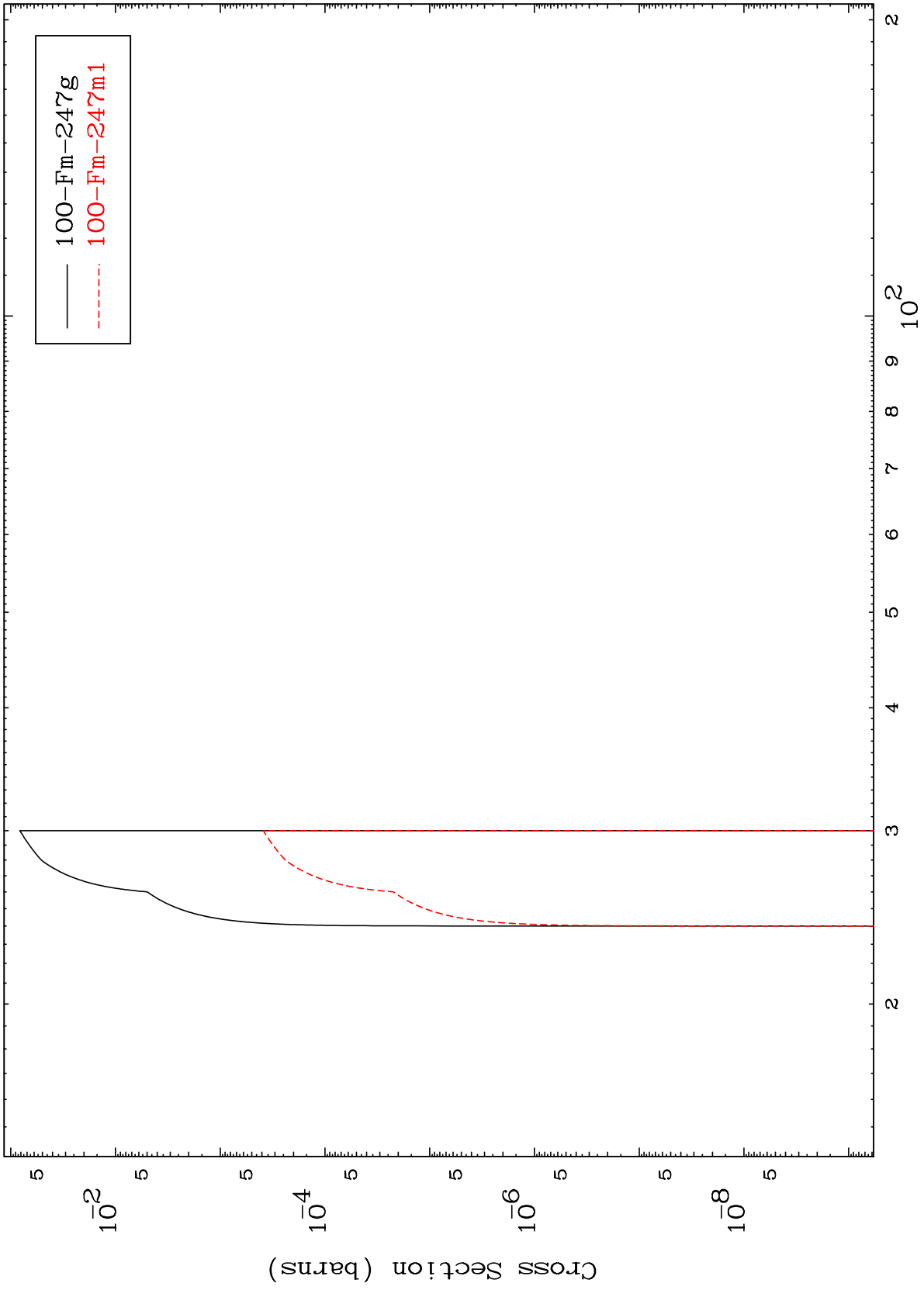
100-Fm-249

MAT 9930

( $\alpha, 2n$ )  $\alpha$

100-Fm-249

Radionuclide Production Cross Section



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Incident Energy (MeV)

100-Fm-249