

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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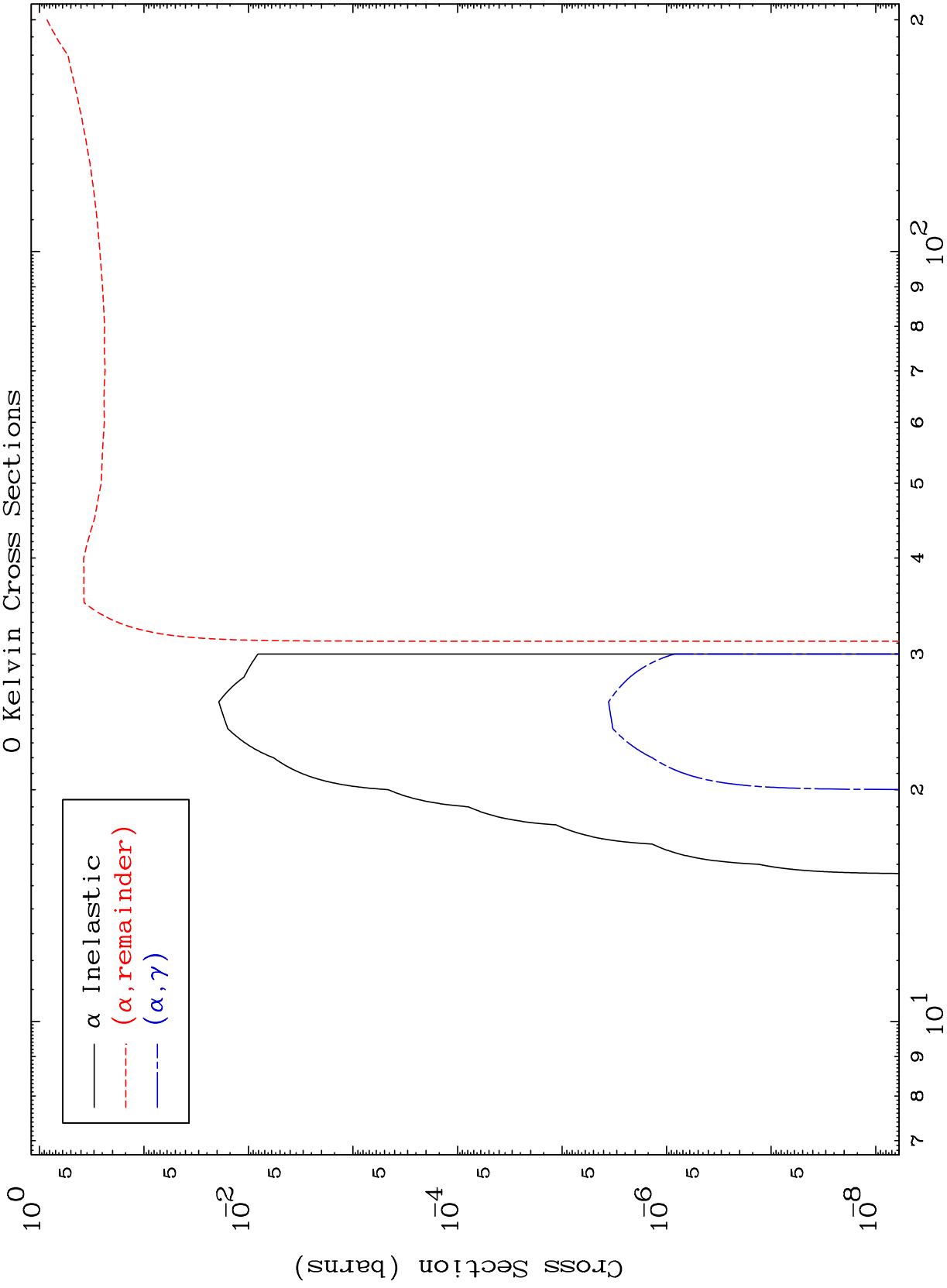
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9936

0 Kelvin  $\alpha$  Major  
Cross Sections

100-Fm-250



$\alpha$  Inelastic  
( $\alpha$ , remainder)  
( $\alpha, \gamma$ )

1

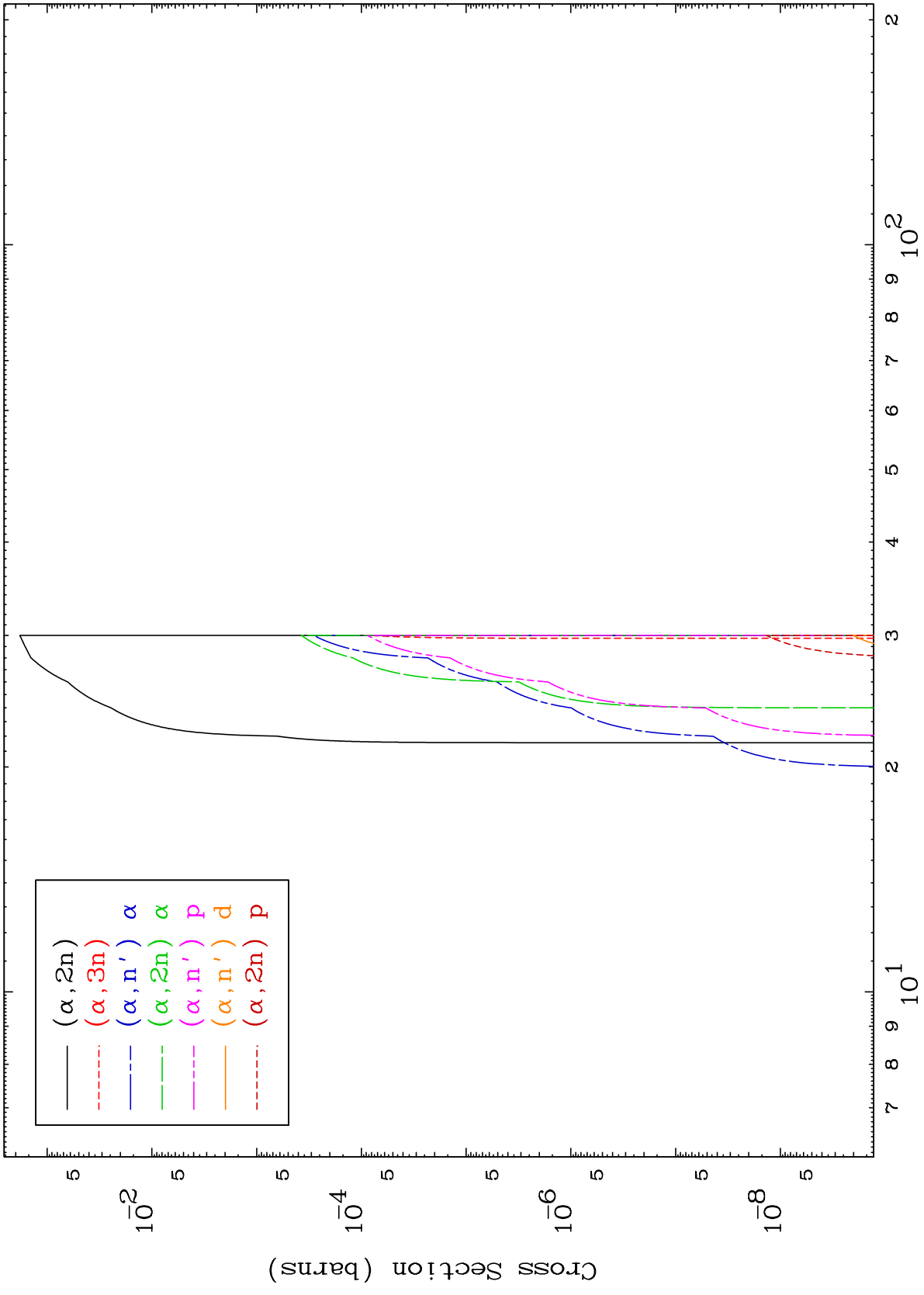
Incident Energy (MeV)

100-Fm-250

MAT 9936

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

100-Fm-250



2

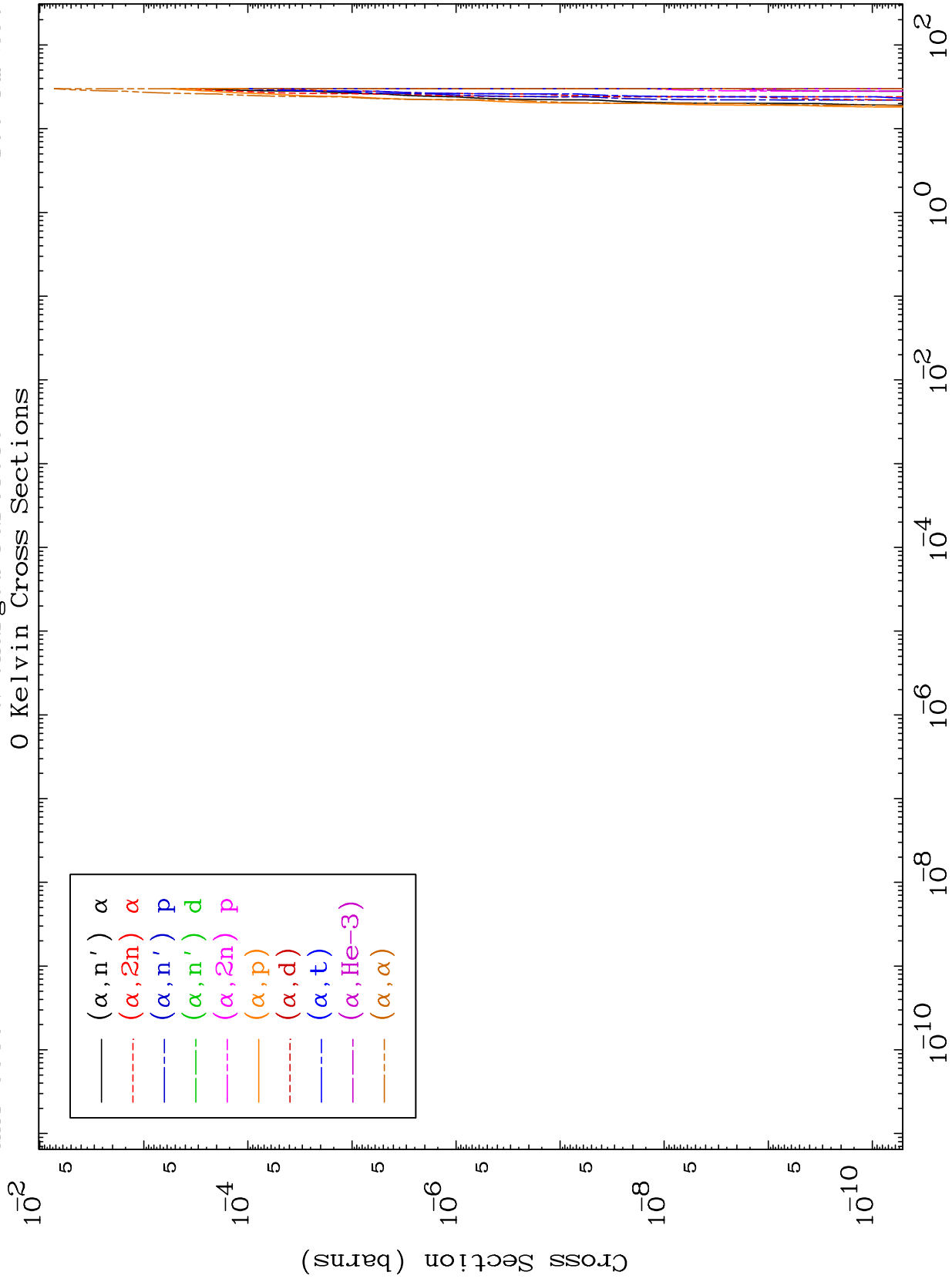
Incident Energy (MeV)

100-Fm-250

MAT 9936

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

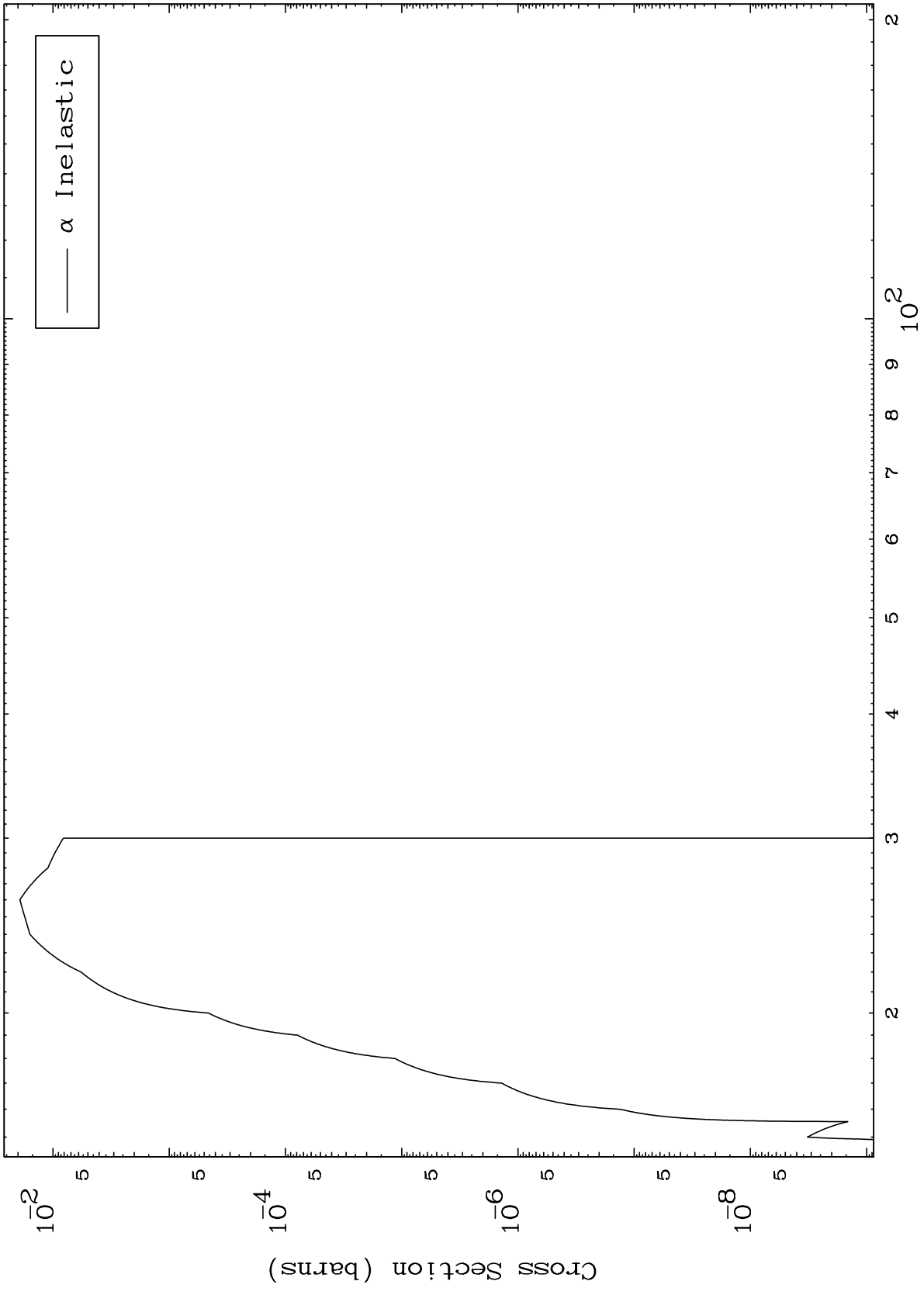
100-Fm-250



MAT 9936

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

100-Fm-250



4

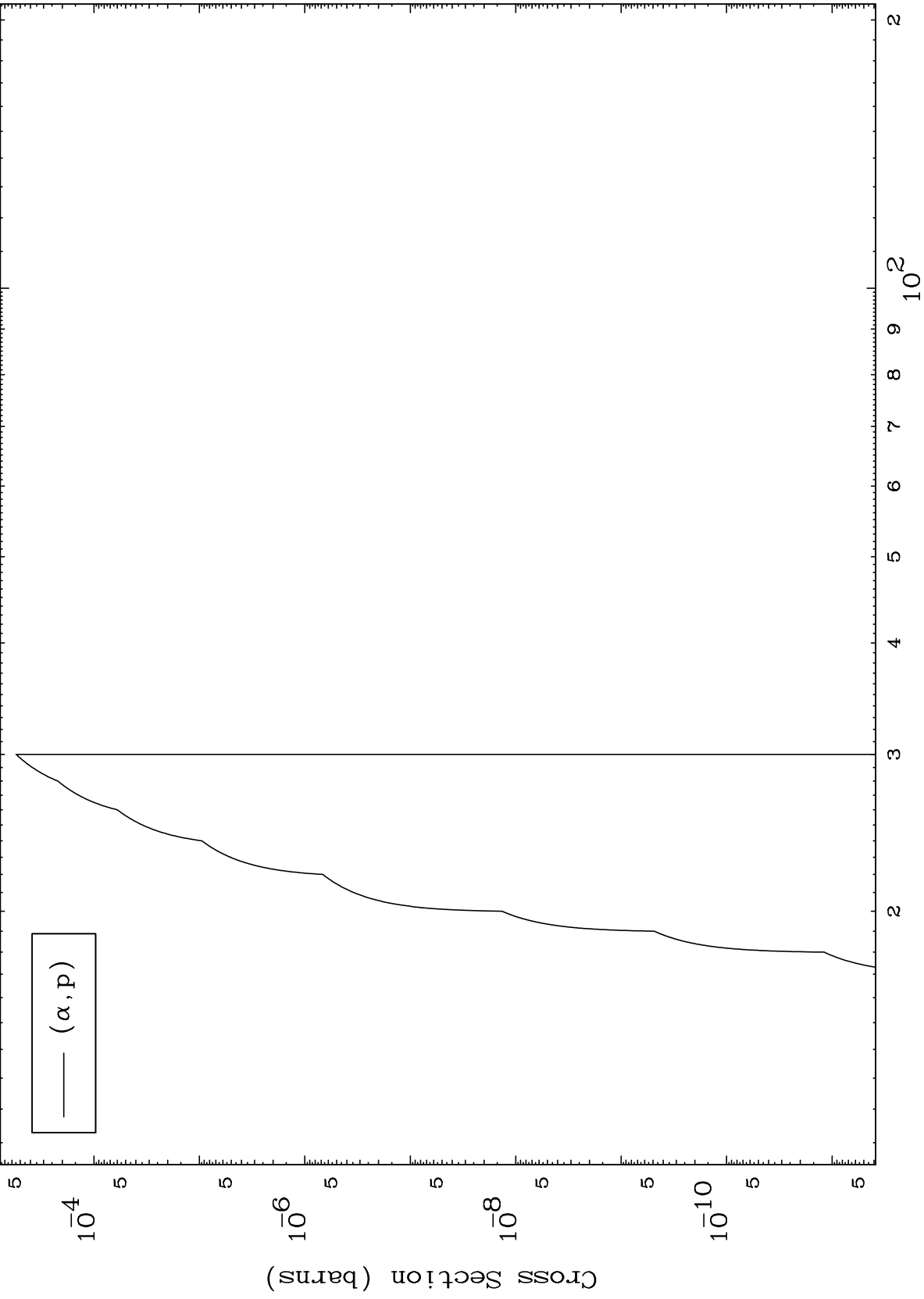
100-Fm-250

Incident Energy (MeV)

MAT 9936

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

100-Fm-250



5

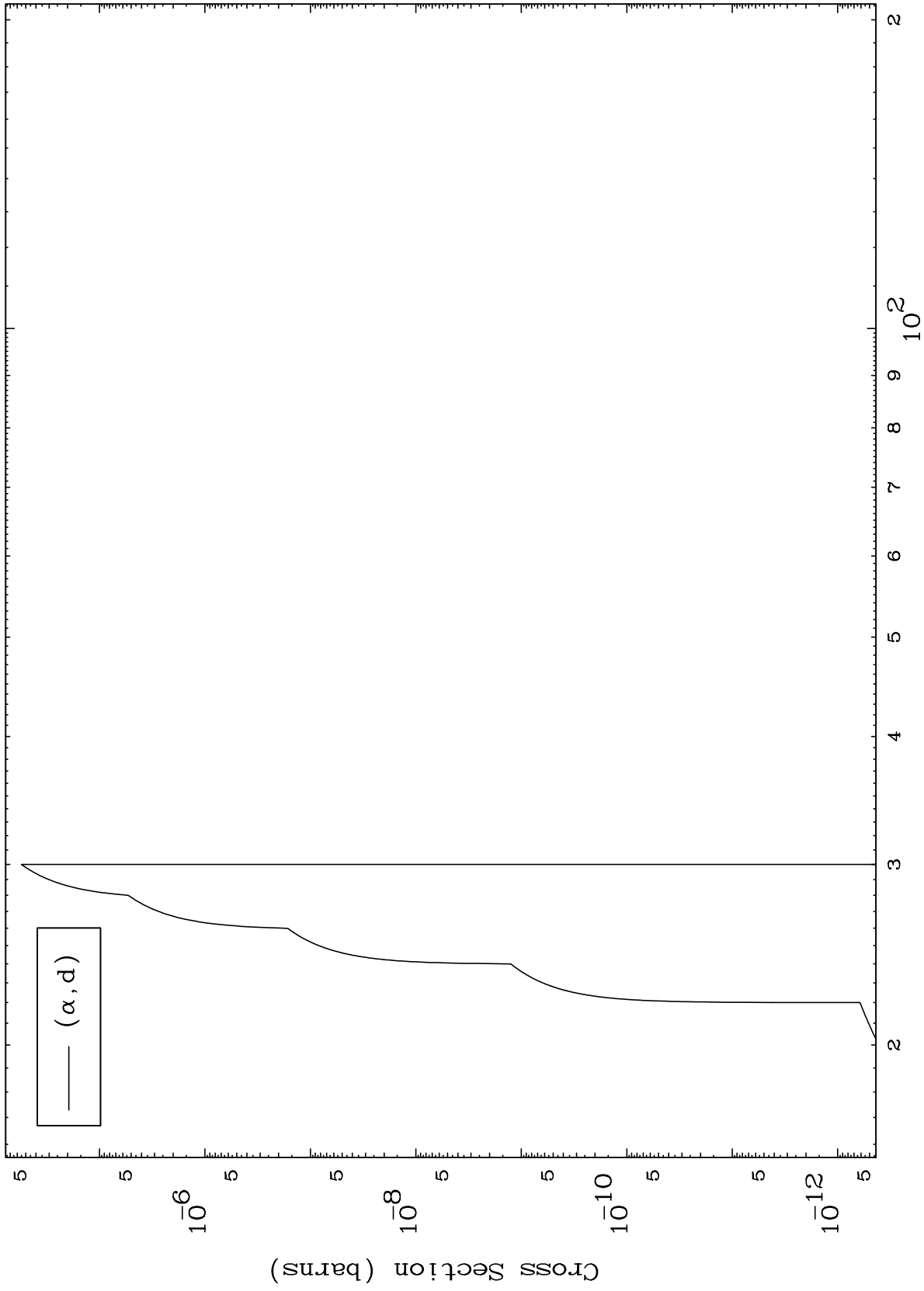
Incident Energy (MeV)

100-Fm-250

MAT 9936

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

100-Fm-250



6

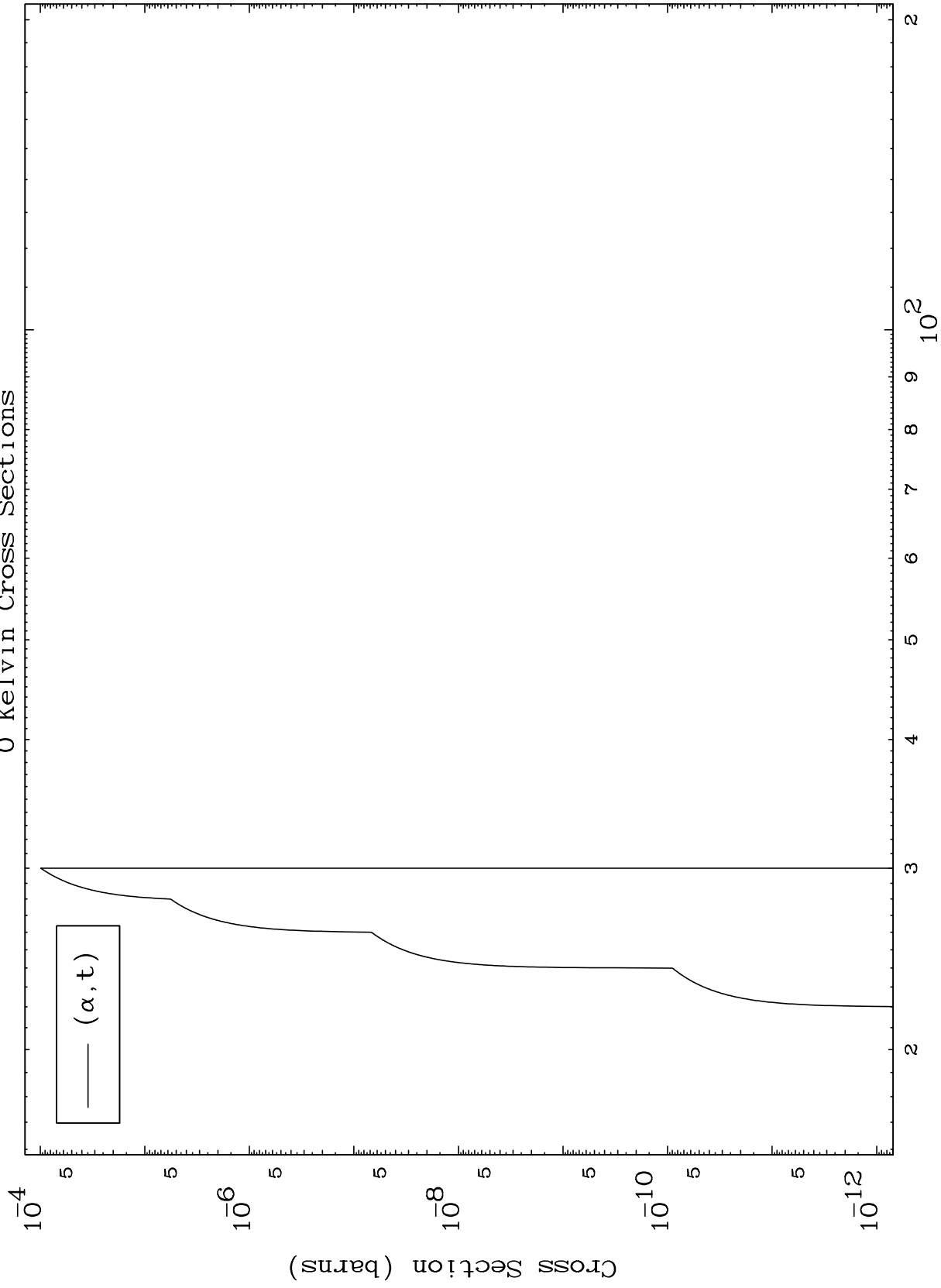
Incident Energy (MeV)

100-Fm-250

MAT 9936

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

100-Fm-250



7

Incident Energy (MeV)

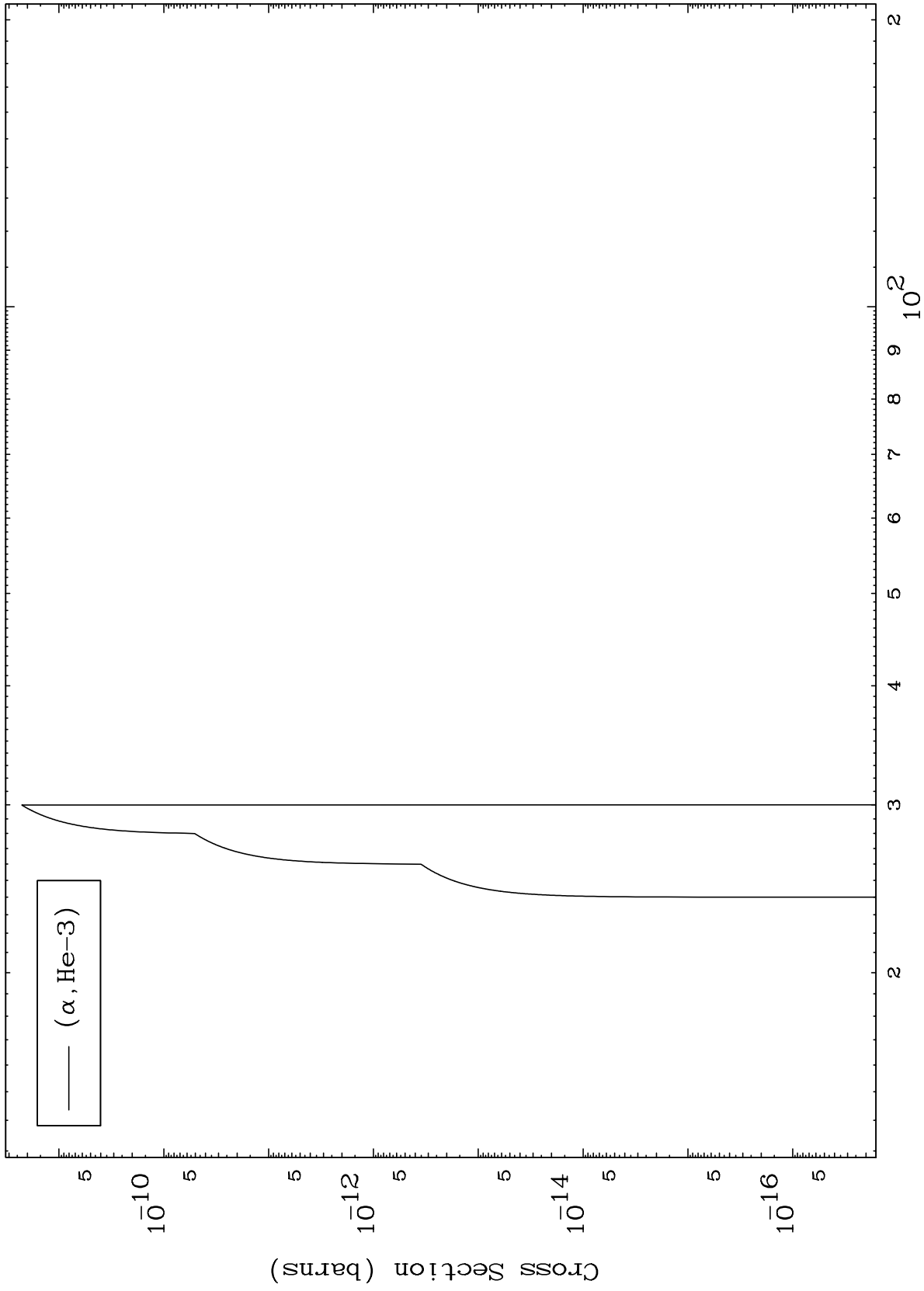
100-Fm-250



MAT 9936

( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections

100-Fm-250



8

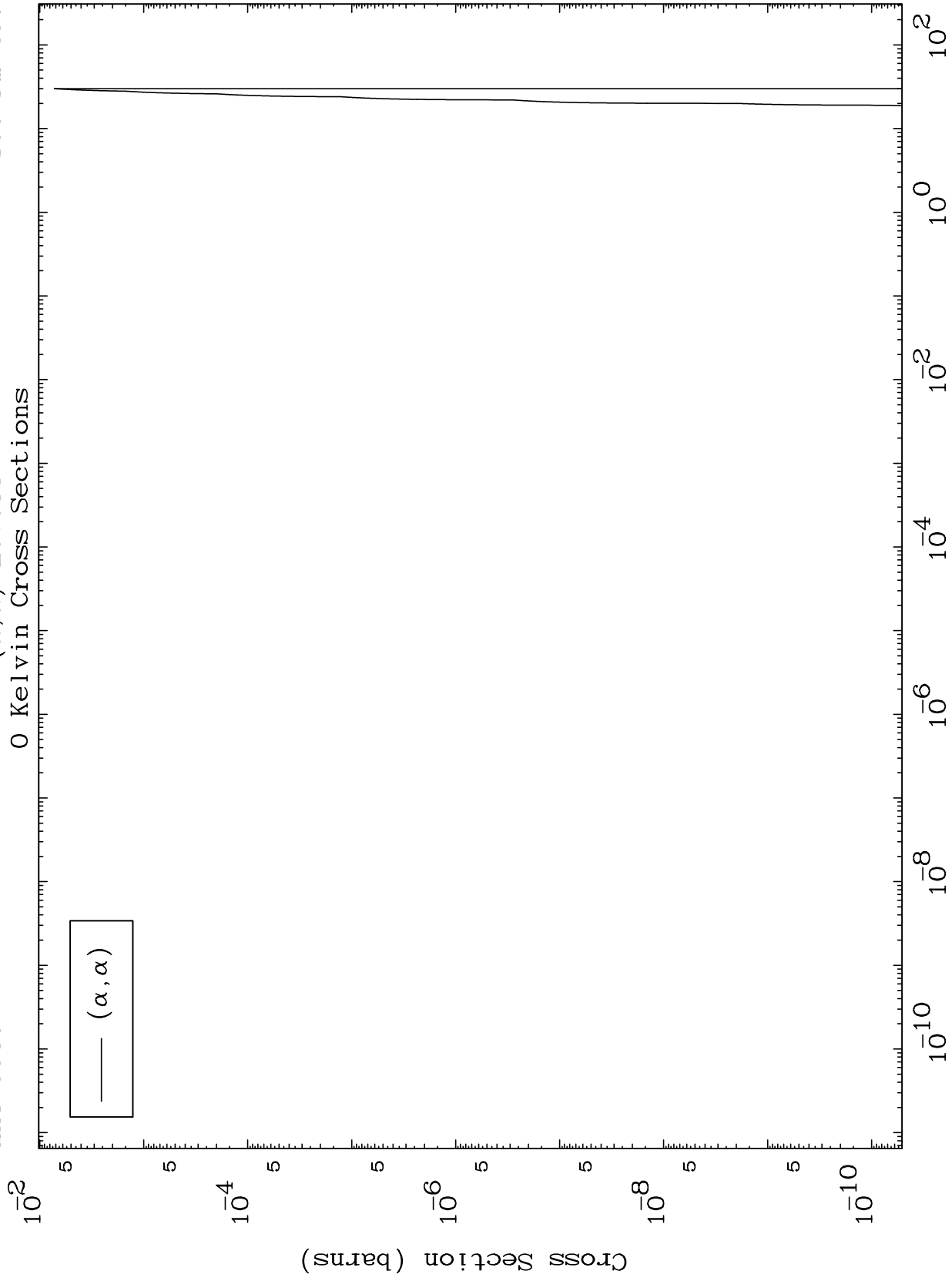
Incident Energy (MeV)

100-Fm-250

MAT 9936

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

100-Fm-250

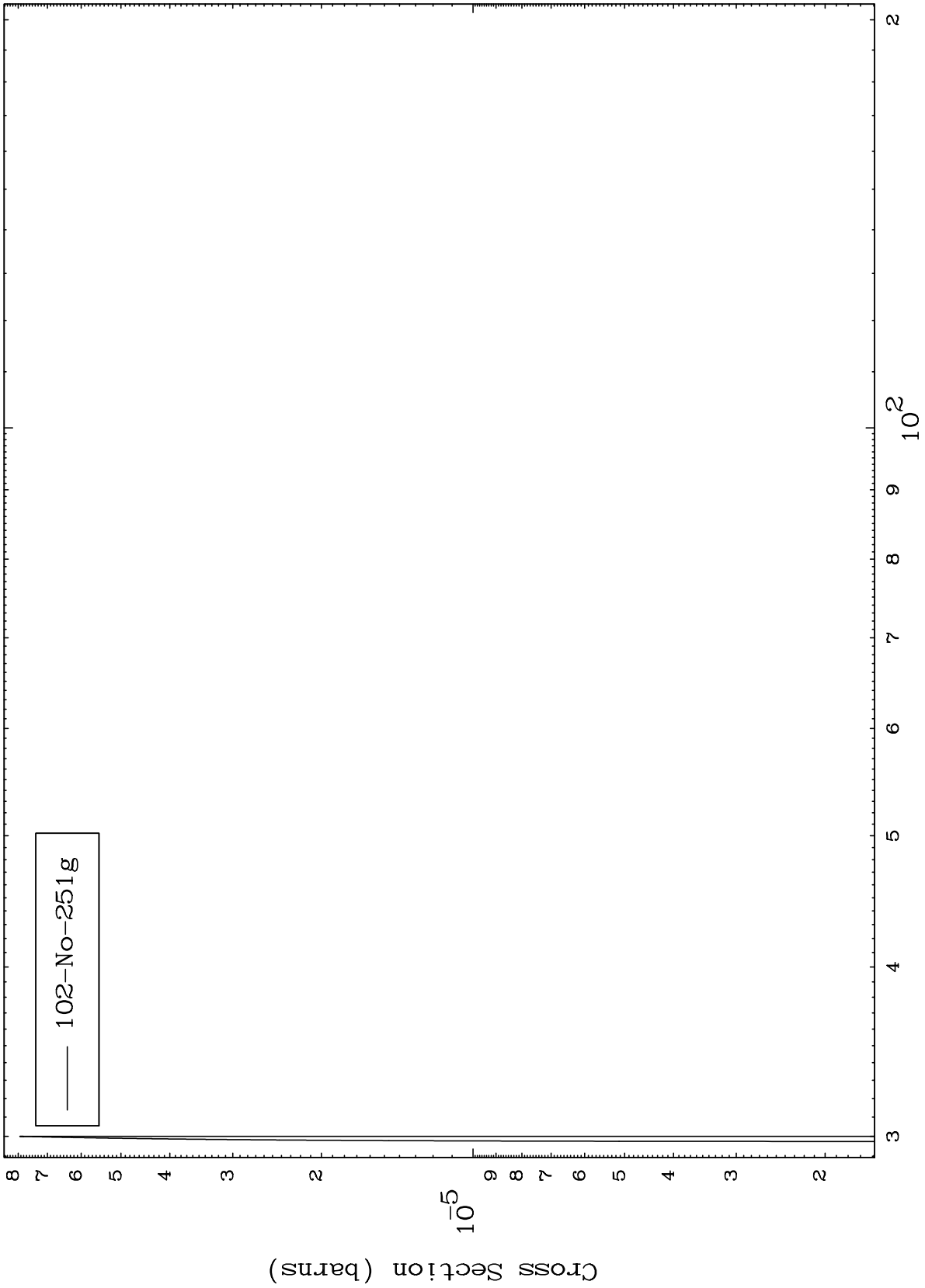


MAT 9936

( $\alpha, 3n$ )

100-Fm-250

Radionuclide Production Cross Section



10

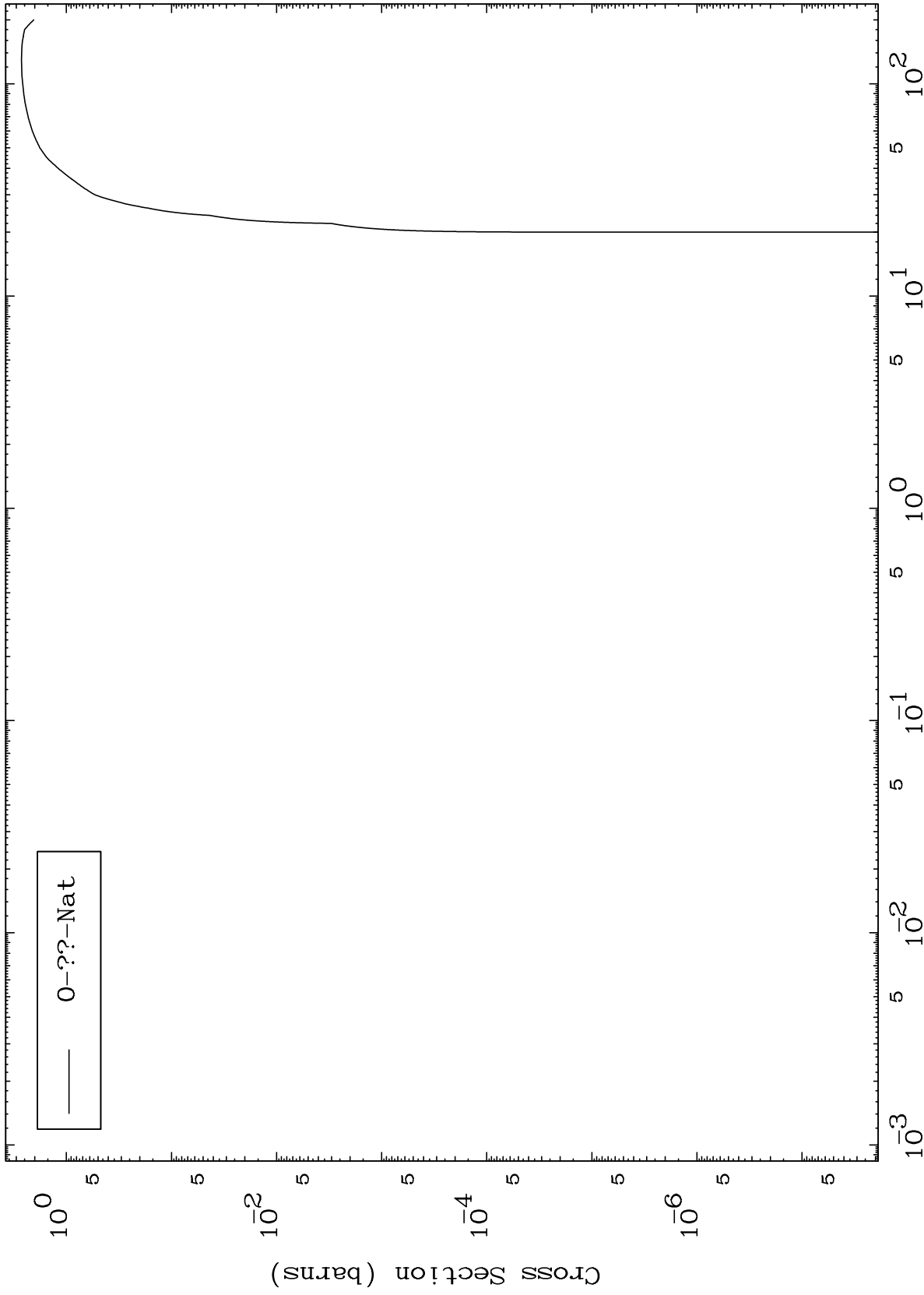
Incident Energy (MeV)

100-Fm-250

MAT 9936

$\alpha$  Fission  
Radionuclide Production Cross Section

100-Fm-250



11

Incident Energy (MeV)

100-Fm-250

MAT 9936

( $\alpha, \alpha$ )

