

Program EVALPLOT  
(Version 2018-1)

by

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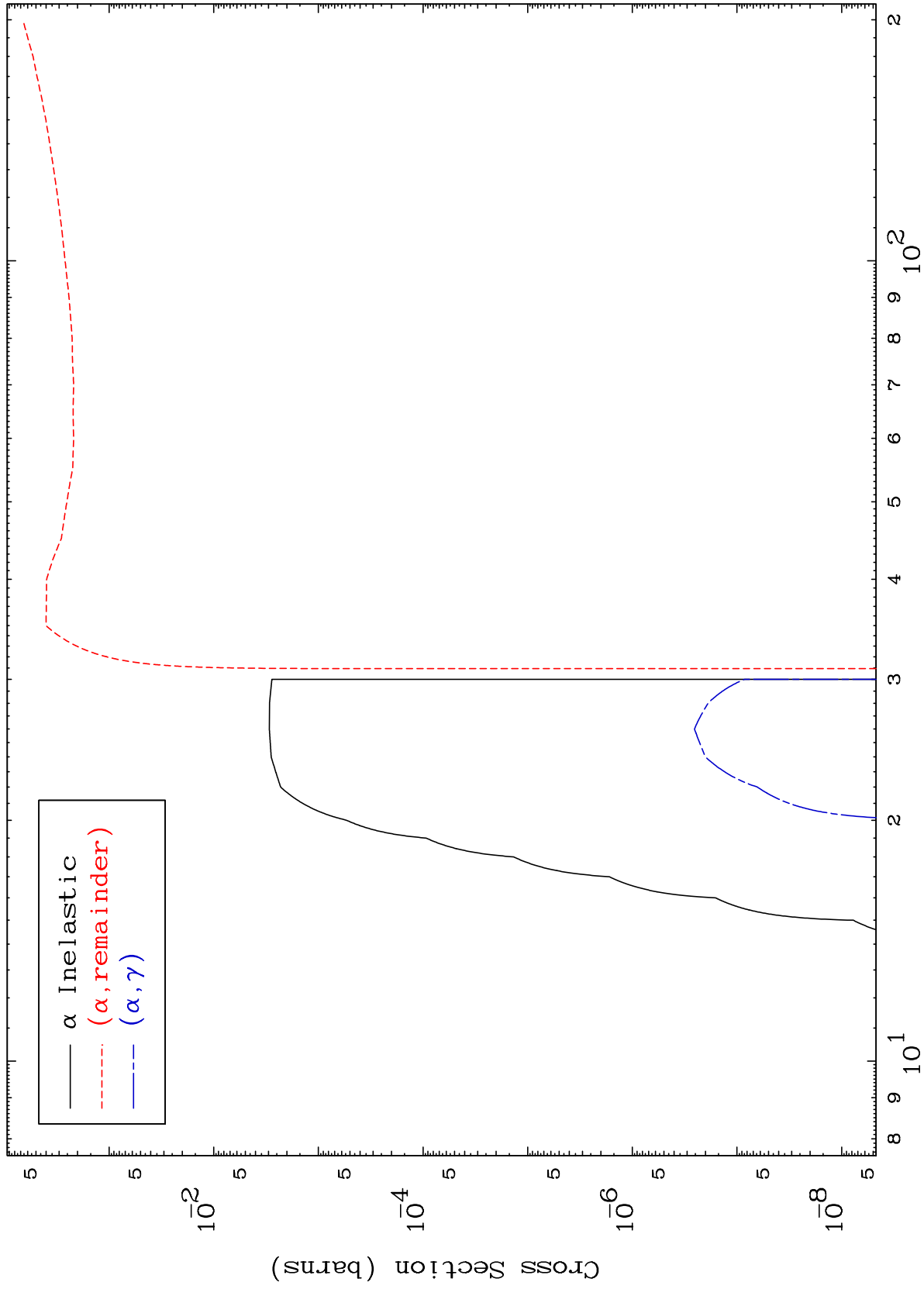
Press Mouse Button to Start

MAT 9936

$\alpha$  Major

100-Fm-255

0 Kelvin Cross Sections



1

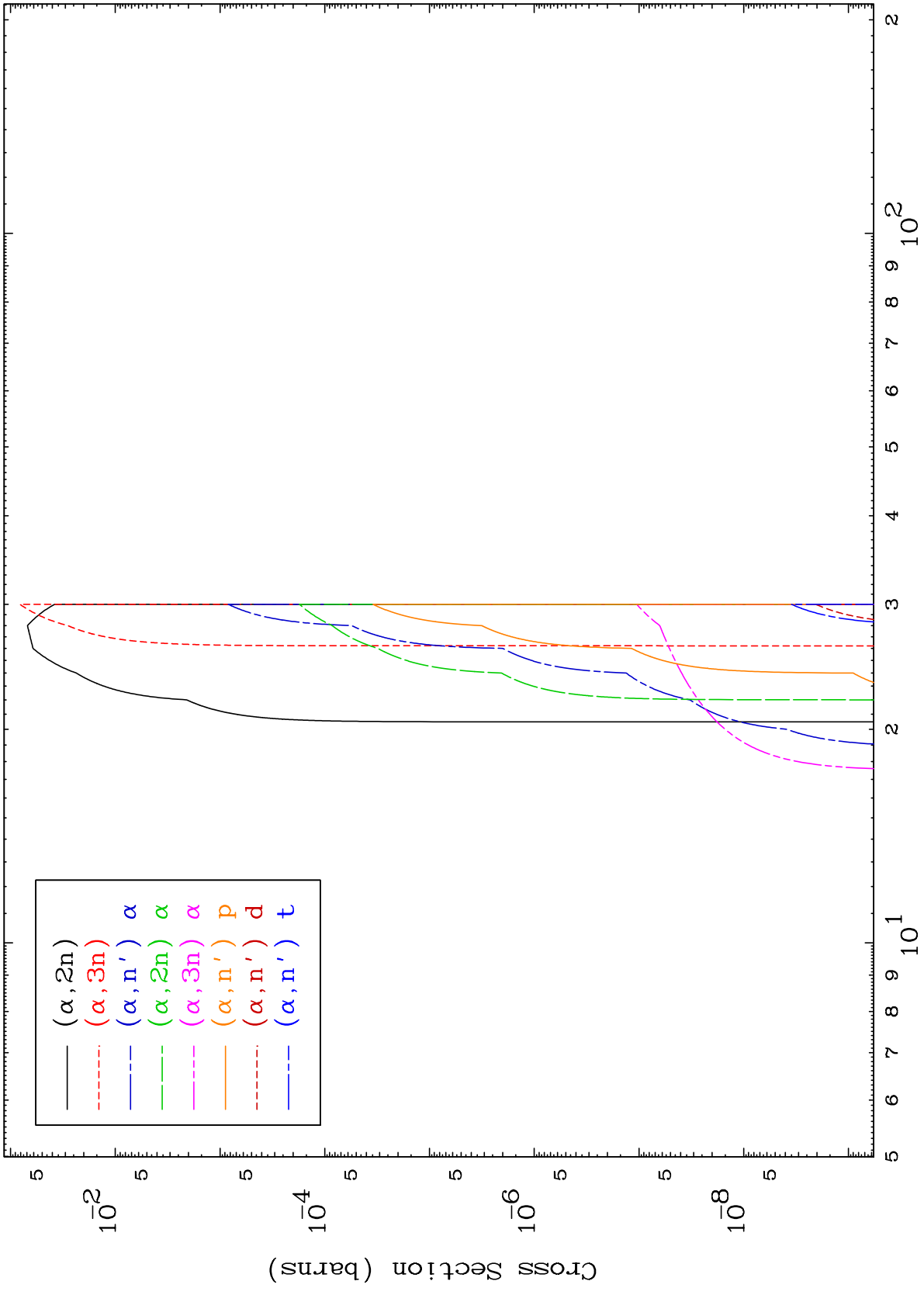
Incident Energy (MeV)

100-Fm-255

MAT 9936

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

100-Fm-255



2

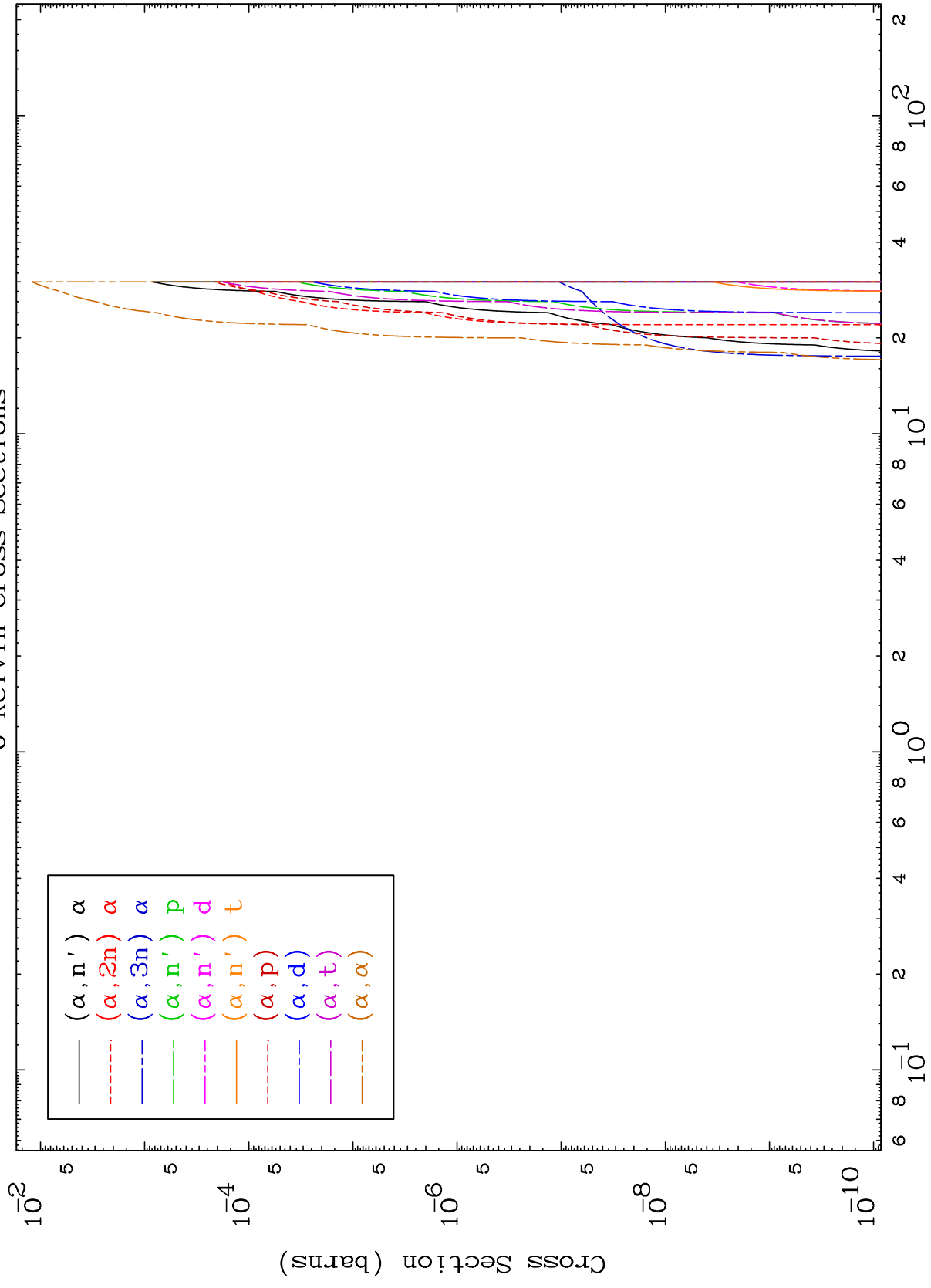
Incident Energy (MeV)

100-Fm-255

MAT 9936

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

100-Fm-255



3

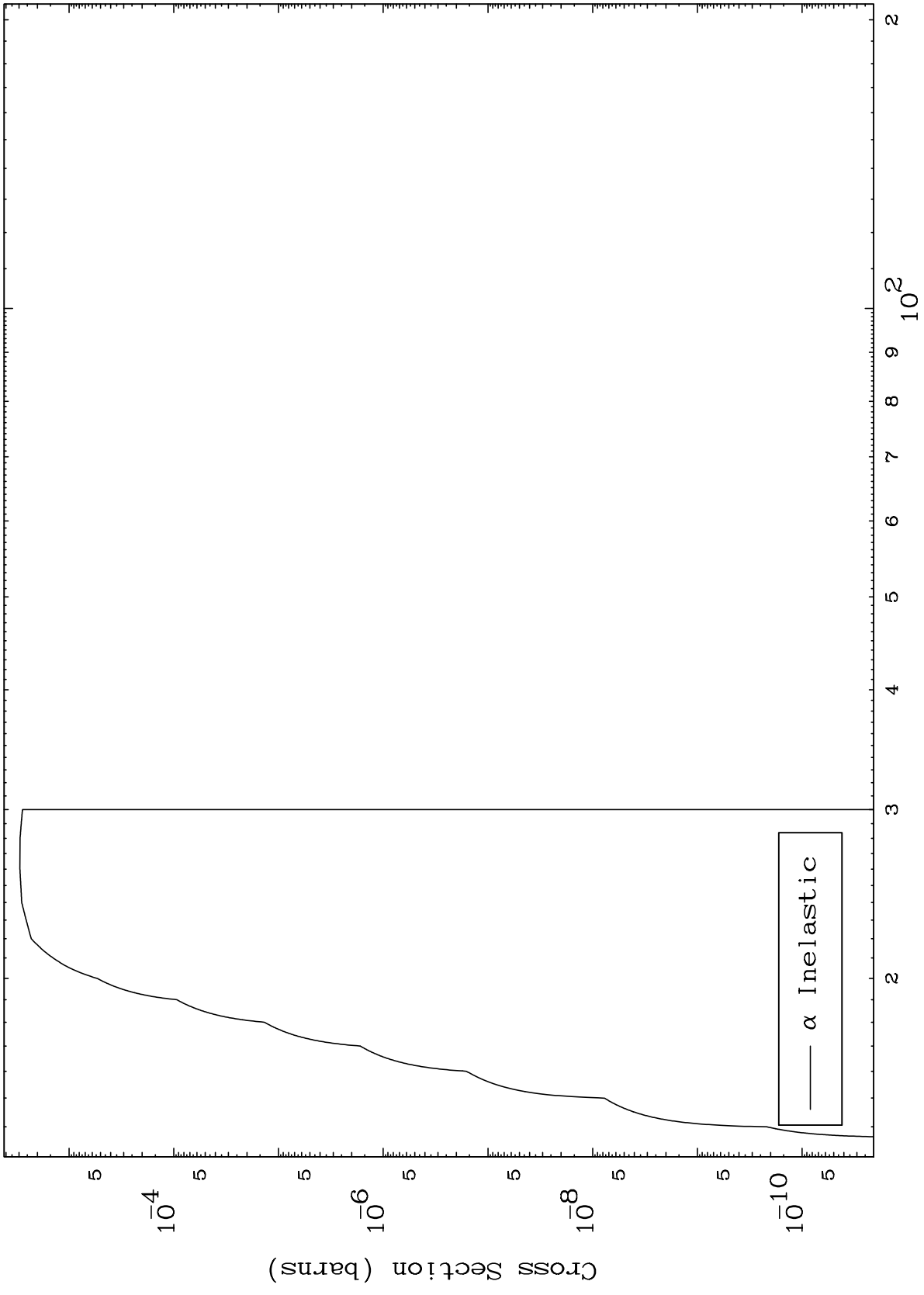
Incident Energy (MeV)

100-Fm-255

MAT 9936

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

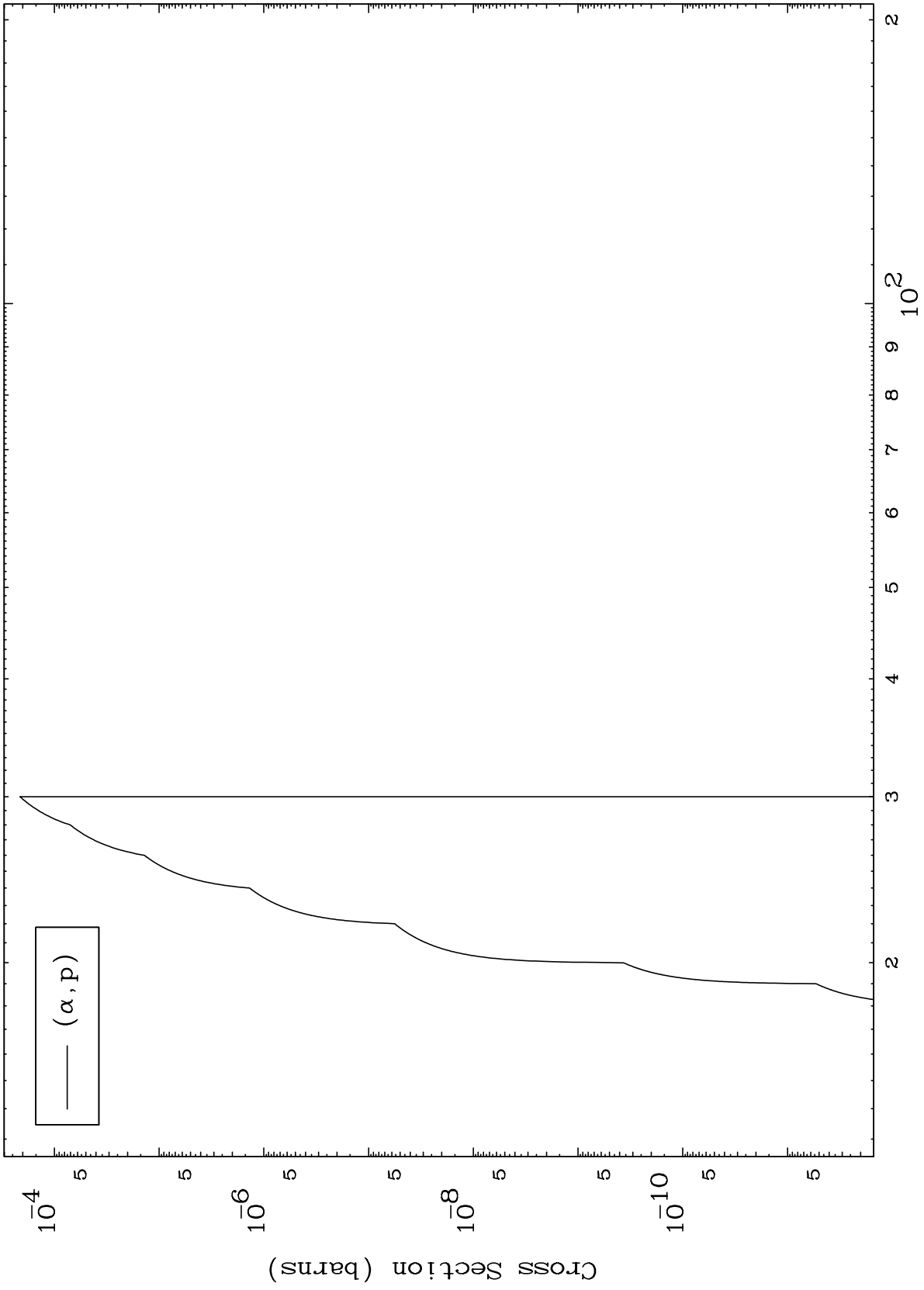
100-Fm-255



MAT 9936

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

100-Fm-255



5

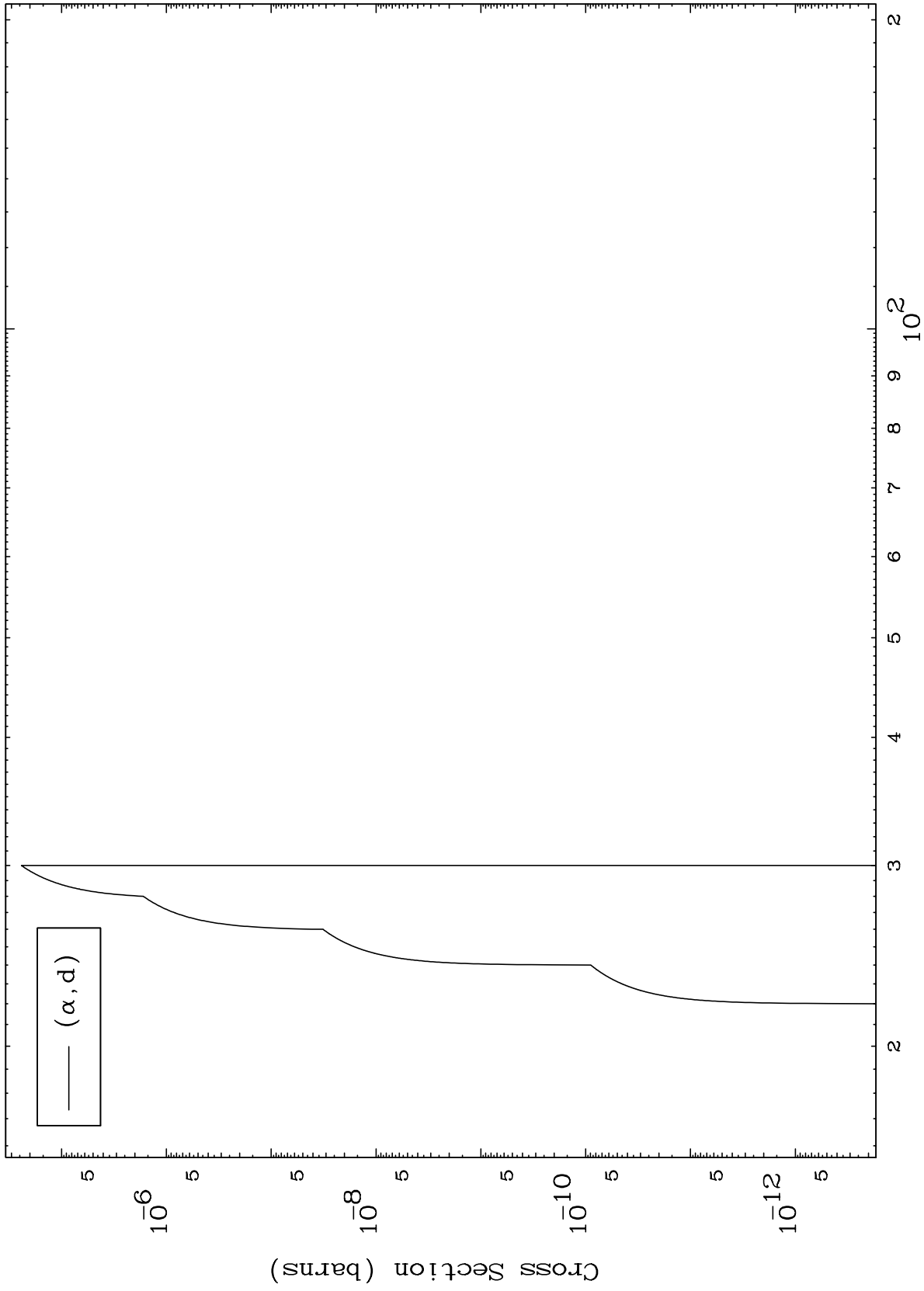
Incident Energy (MeV)

100-Fm-255

MAT 9936

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

100-Fm-255



6

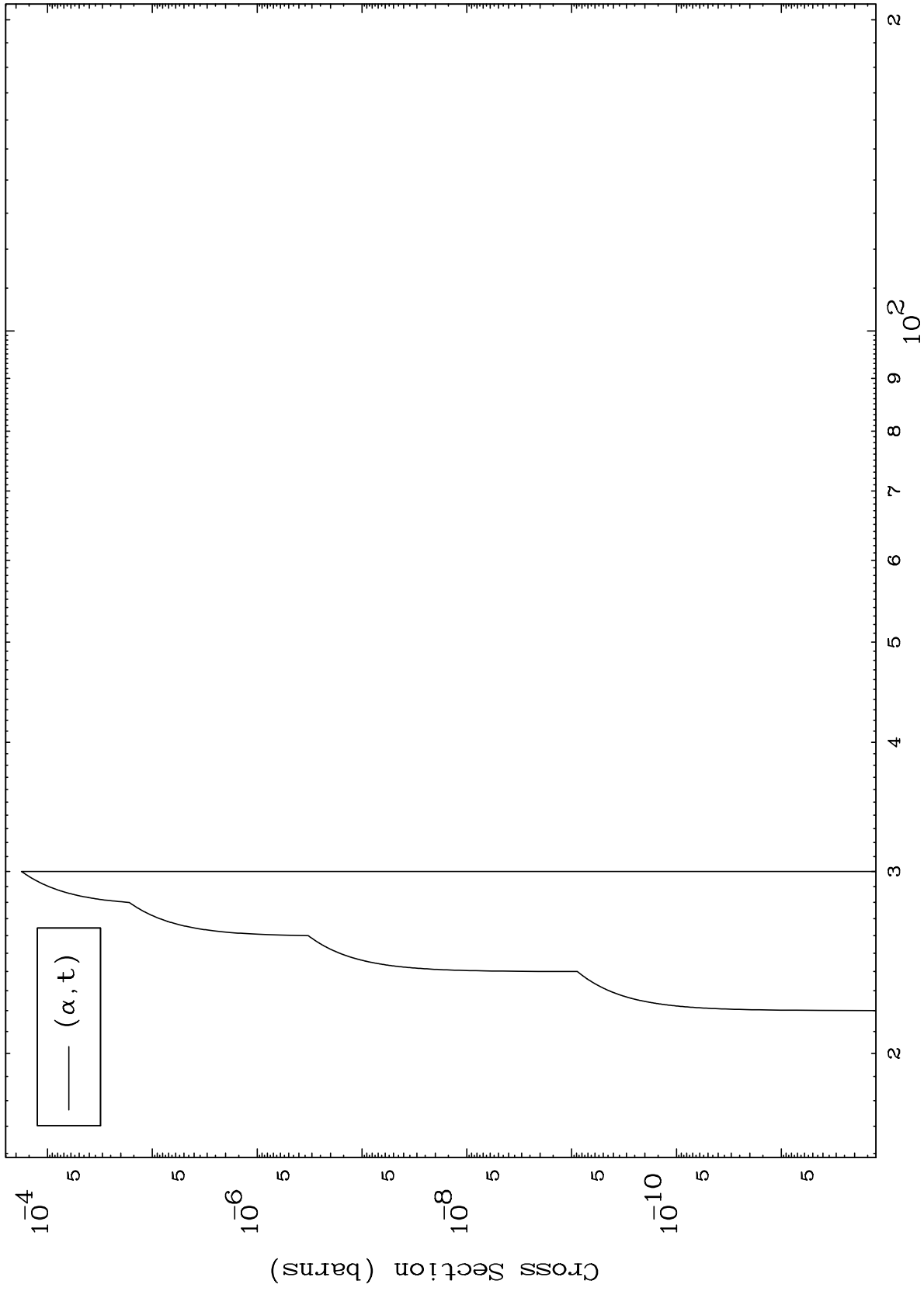
Incident Energy (MeV)

100-Fm-255

MAT 9936

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

100-Fm-255



7

Incident Energy (MeV)

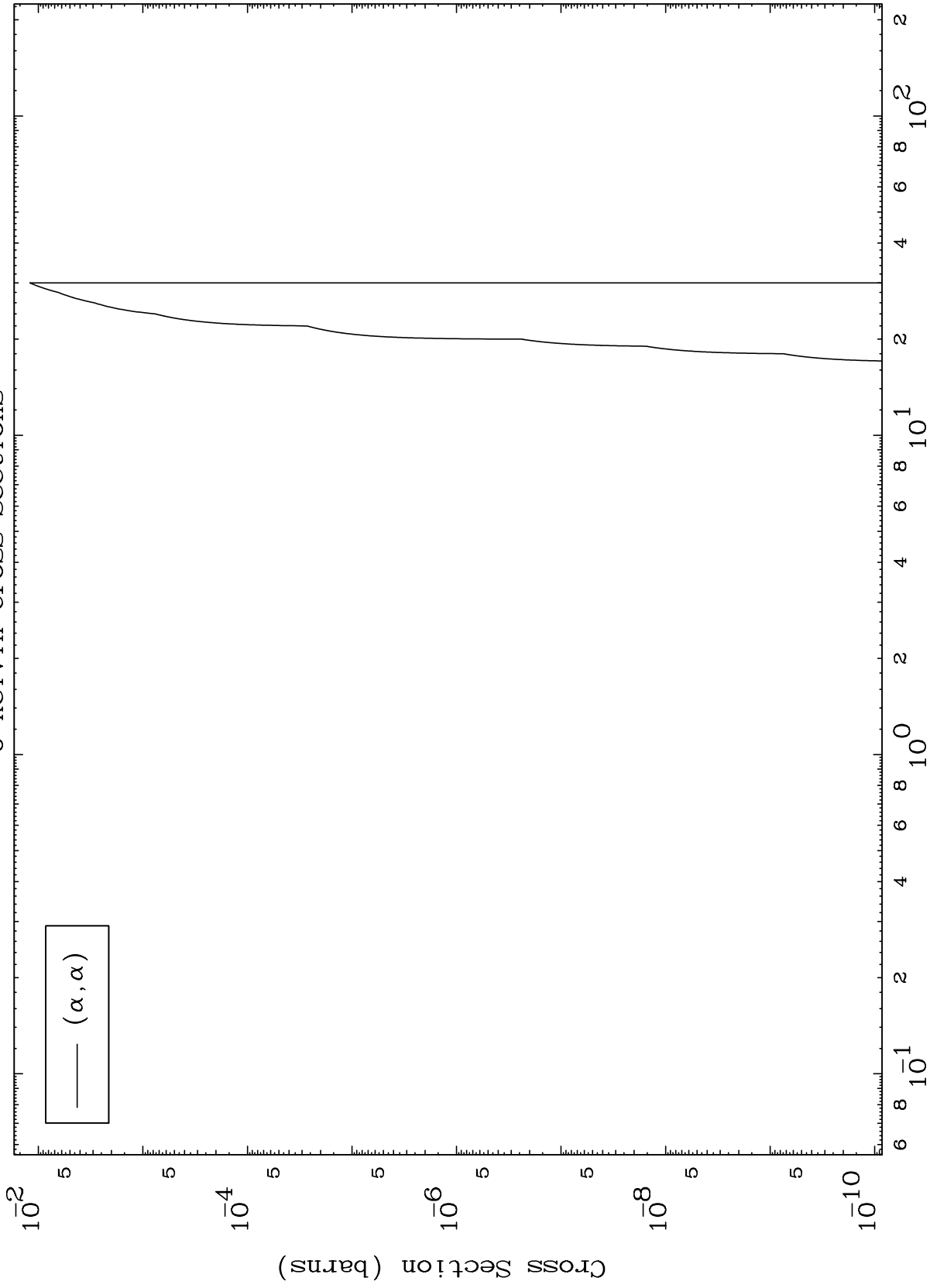
100-Fm-255



MAT 9936

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

100-Fm-255

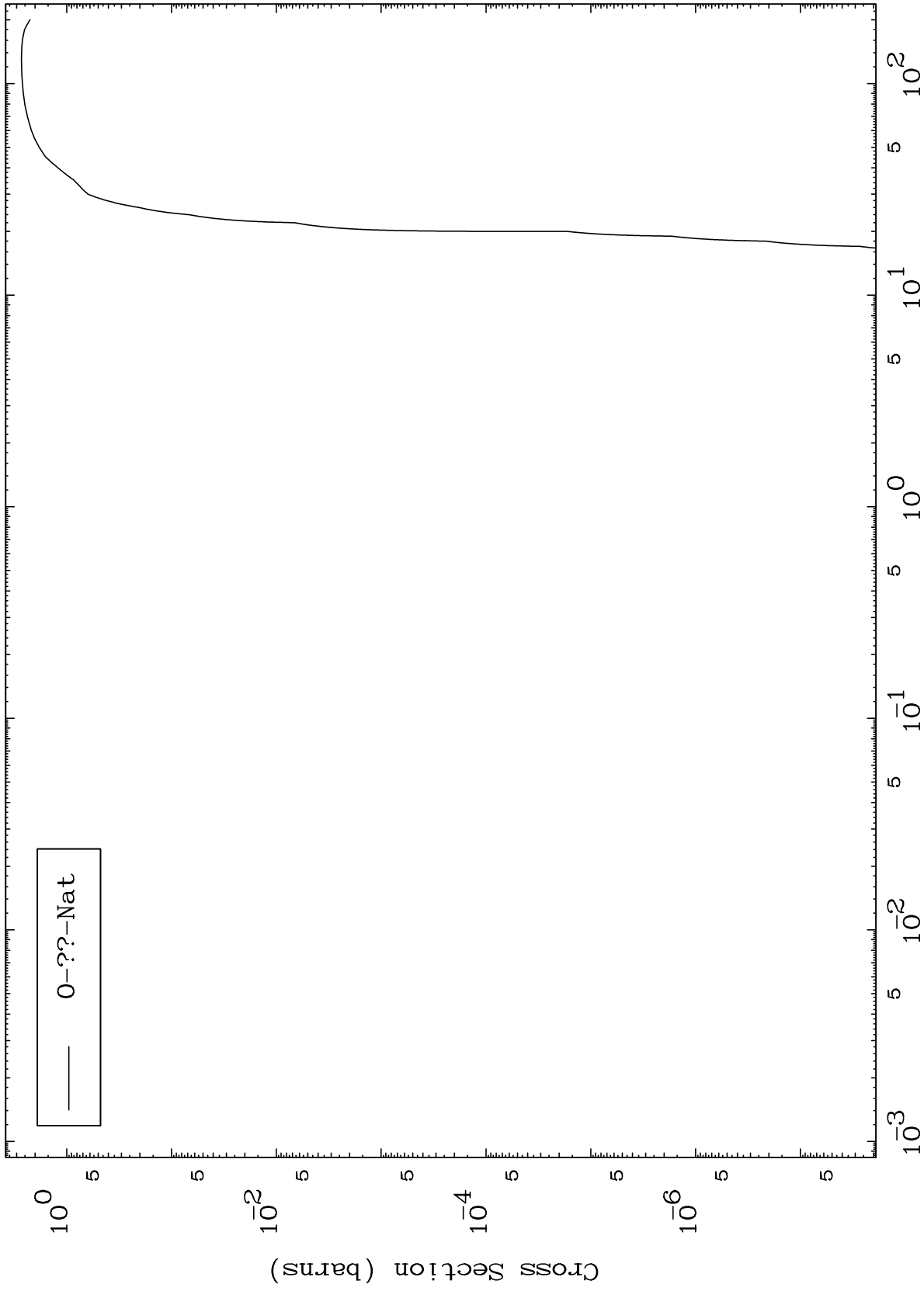


( $\alpha, \alpha$ )

MAT 9936

$\alpha$  Fission  
Radionuclide Production Cross Section

100-Fm-255



9

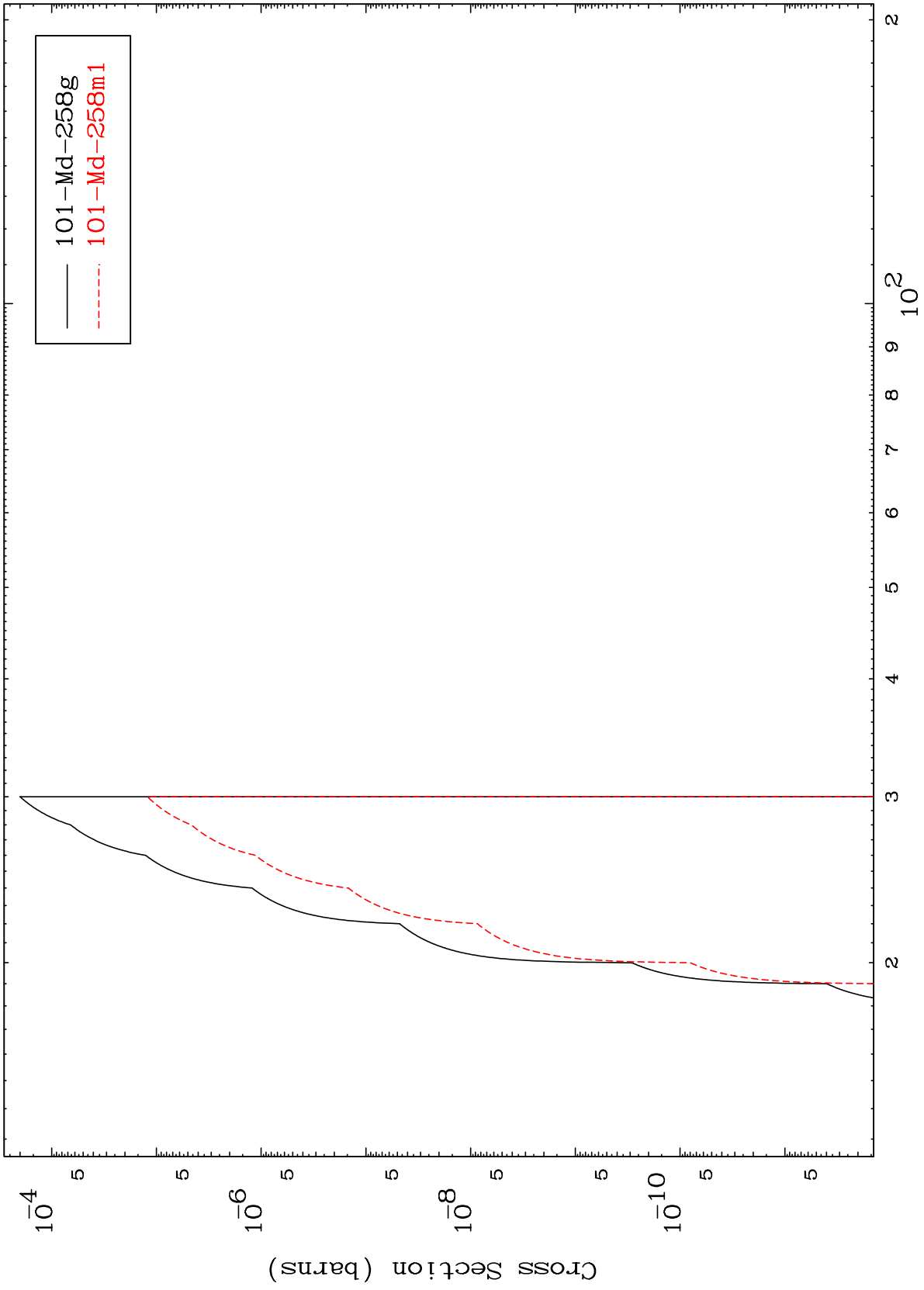
Incident Energy (MeV)

100-Fm-255

MAT 9936

100-Fm-255

( $\alpha, p$ )  
Radionuclide Production Cross Section



10

Incident Energy (MeV)

100-Fm-255