

Program EVALPLOT  
(Version 2018-1)

by

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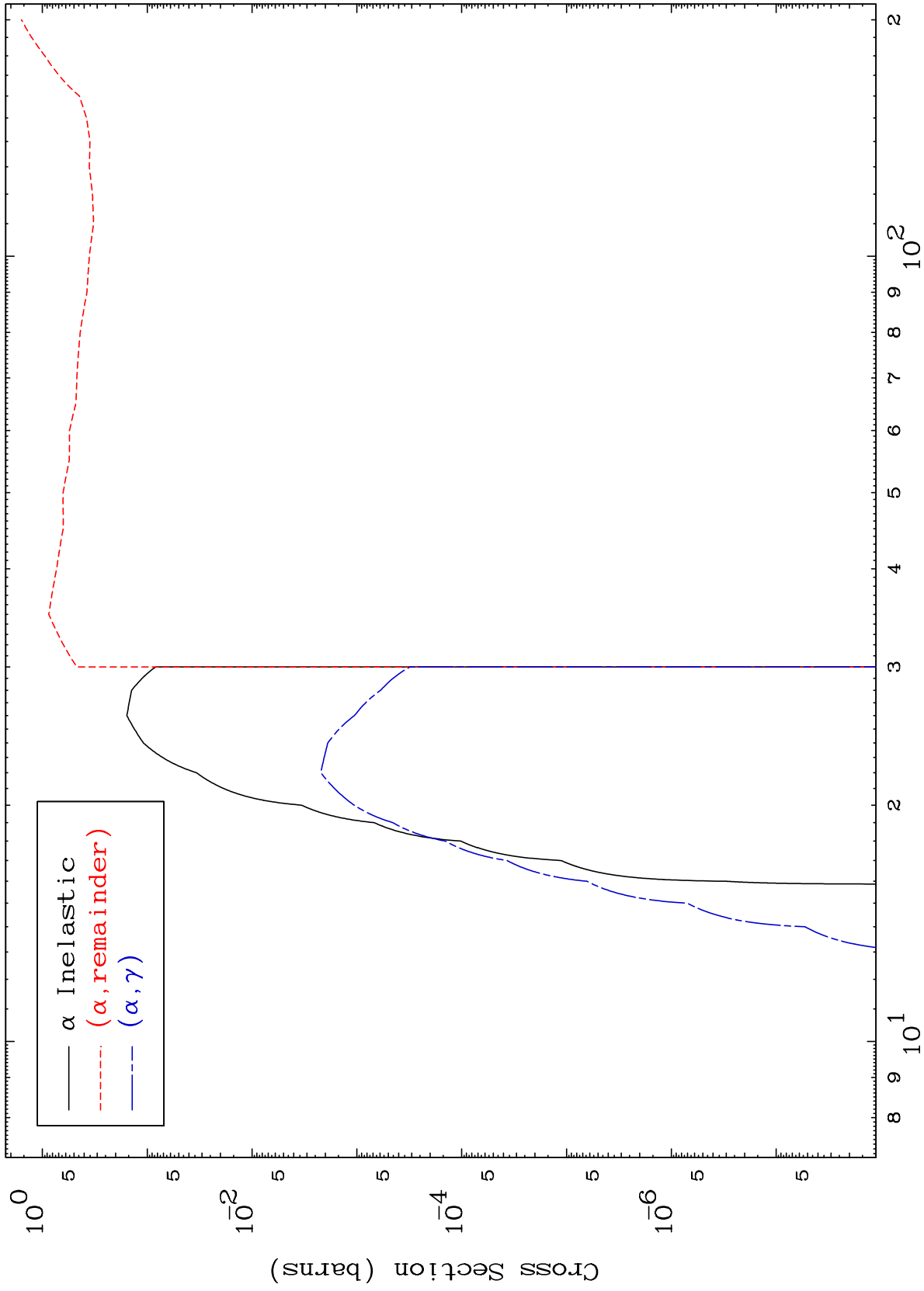
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8708

0 Kelvin  $\alpha$  Major  
Cross Sections

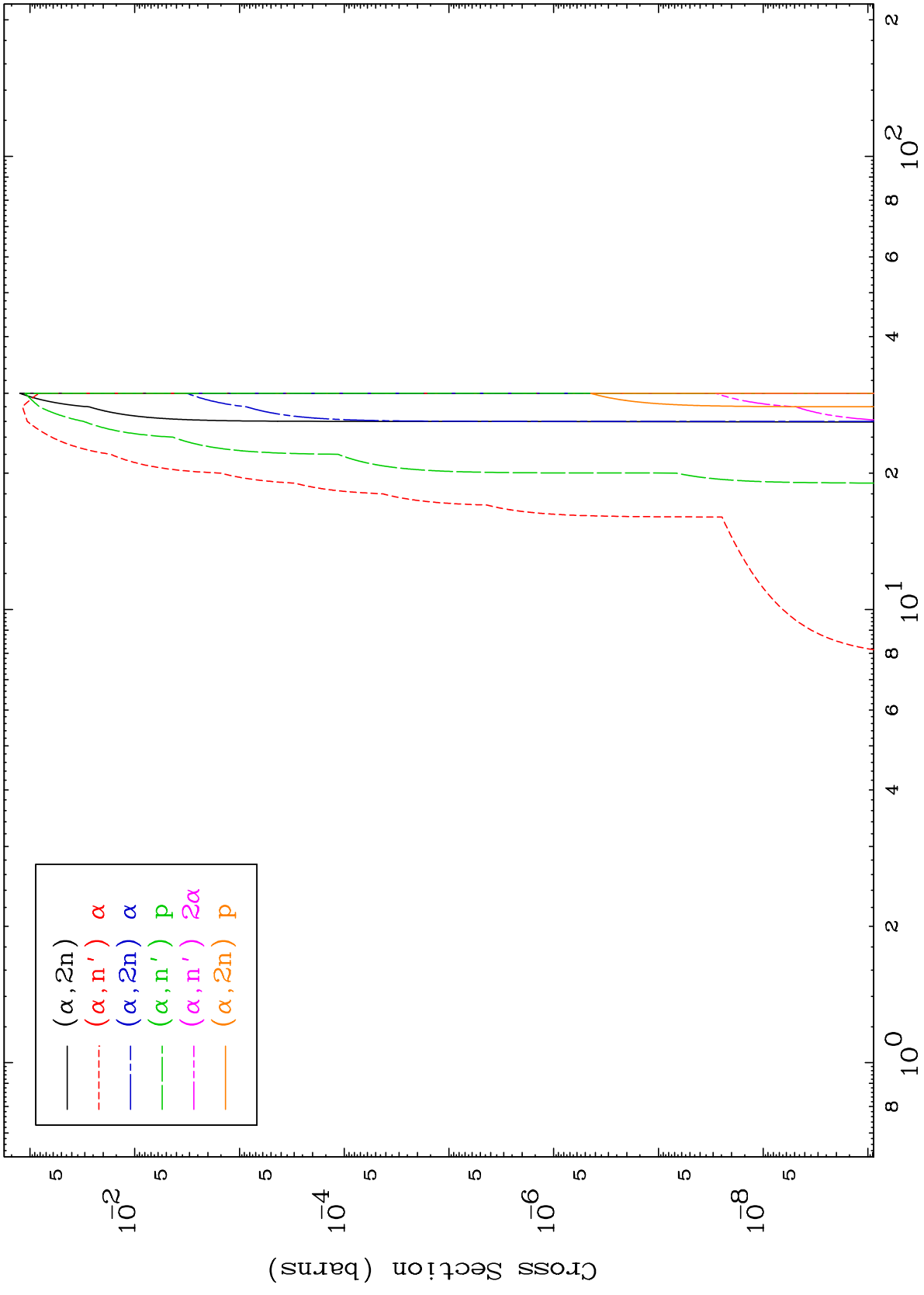
87-Fr-206



MAT 8708

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

87-Fr-206



2

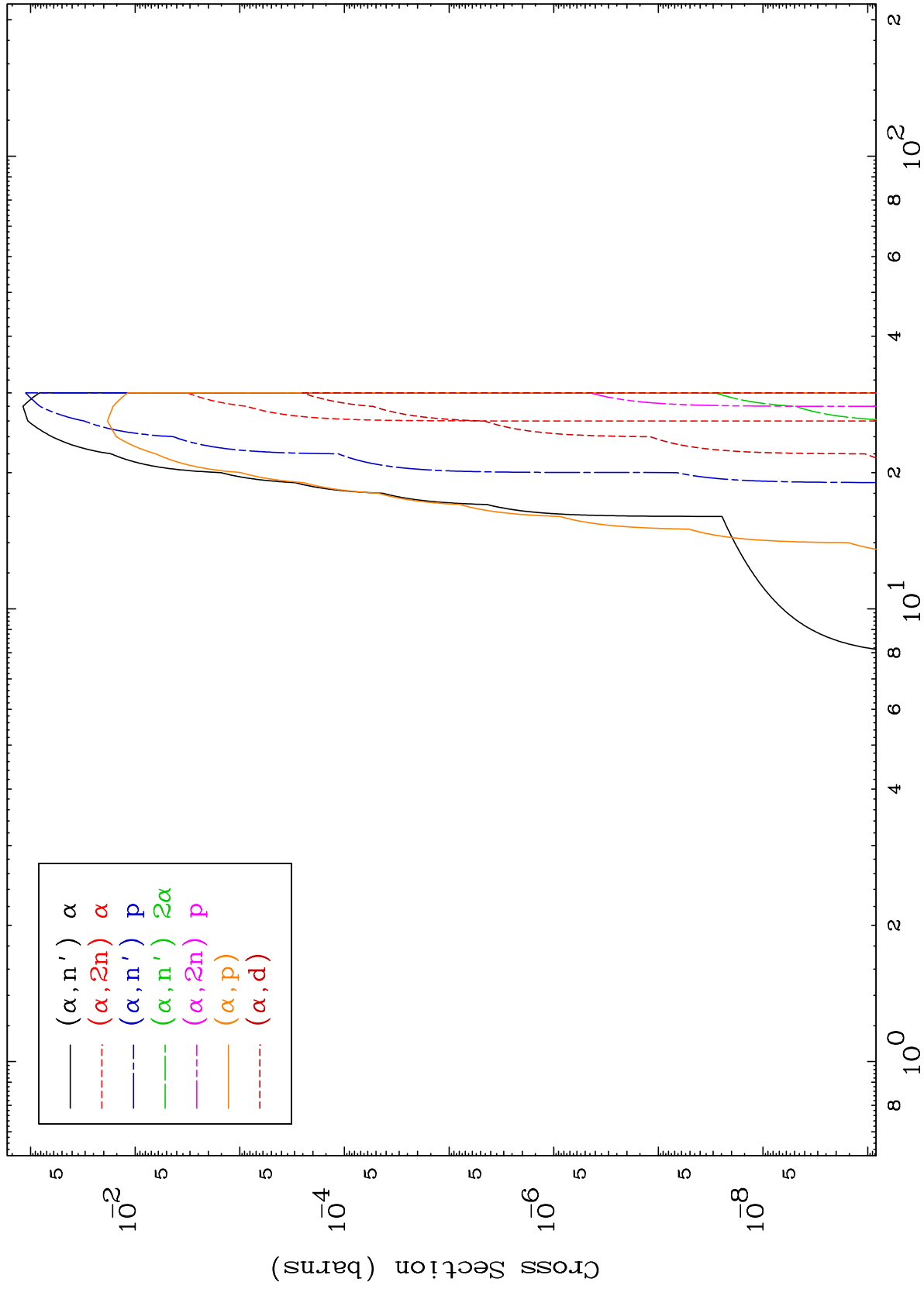
Incident Energy (MeV)

87-Fr-206

MAT 8708

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

87-Fr-206



3

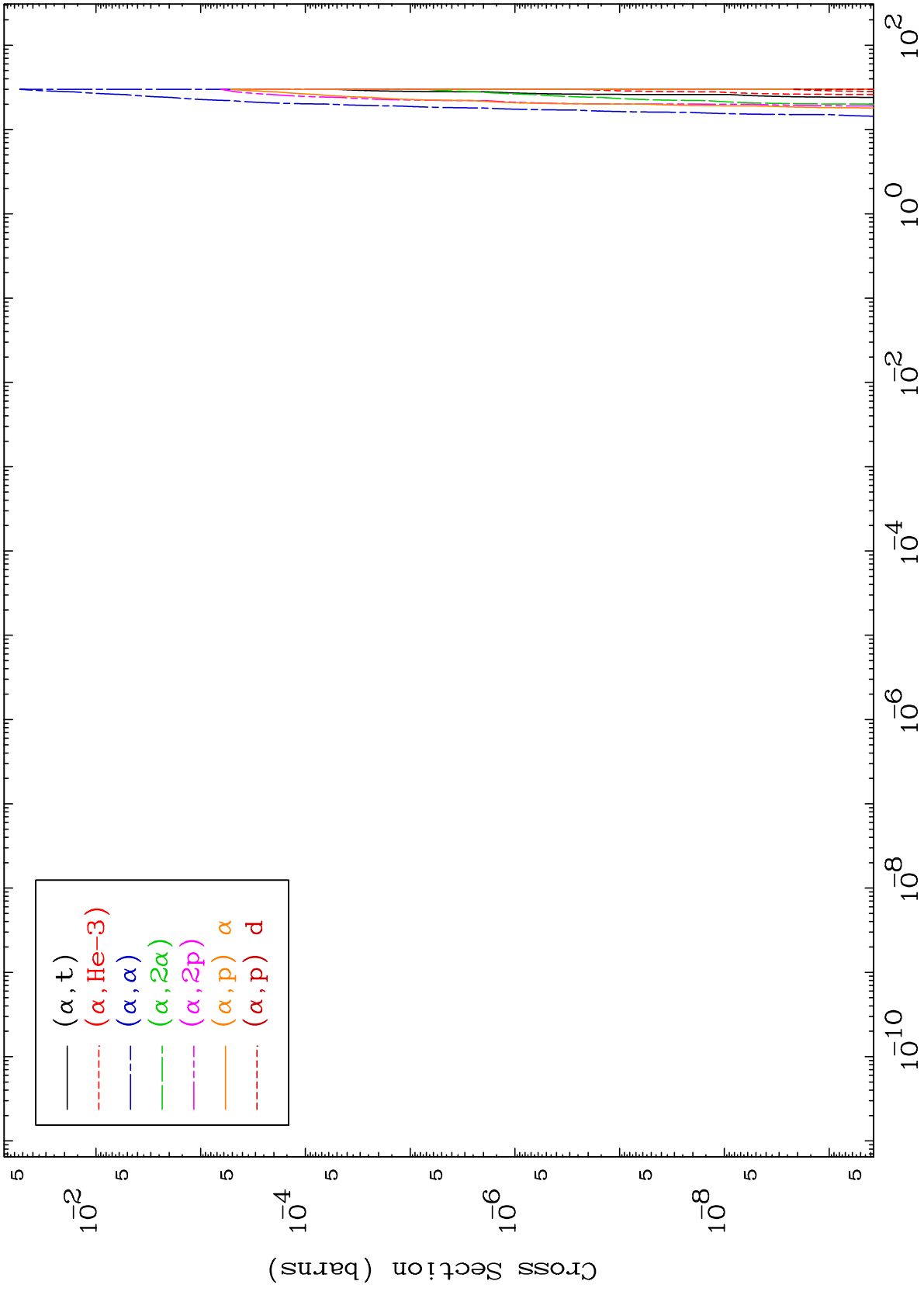
Incident Energy (MeV)

87-Fr-206

MAT 8708

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

87-Fr-206

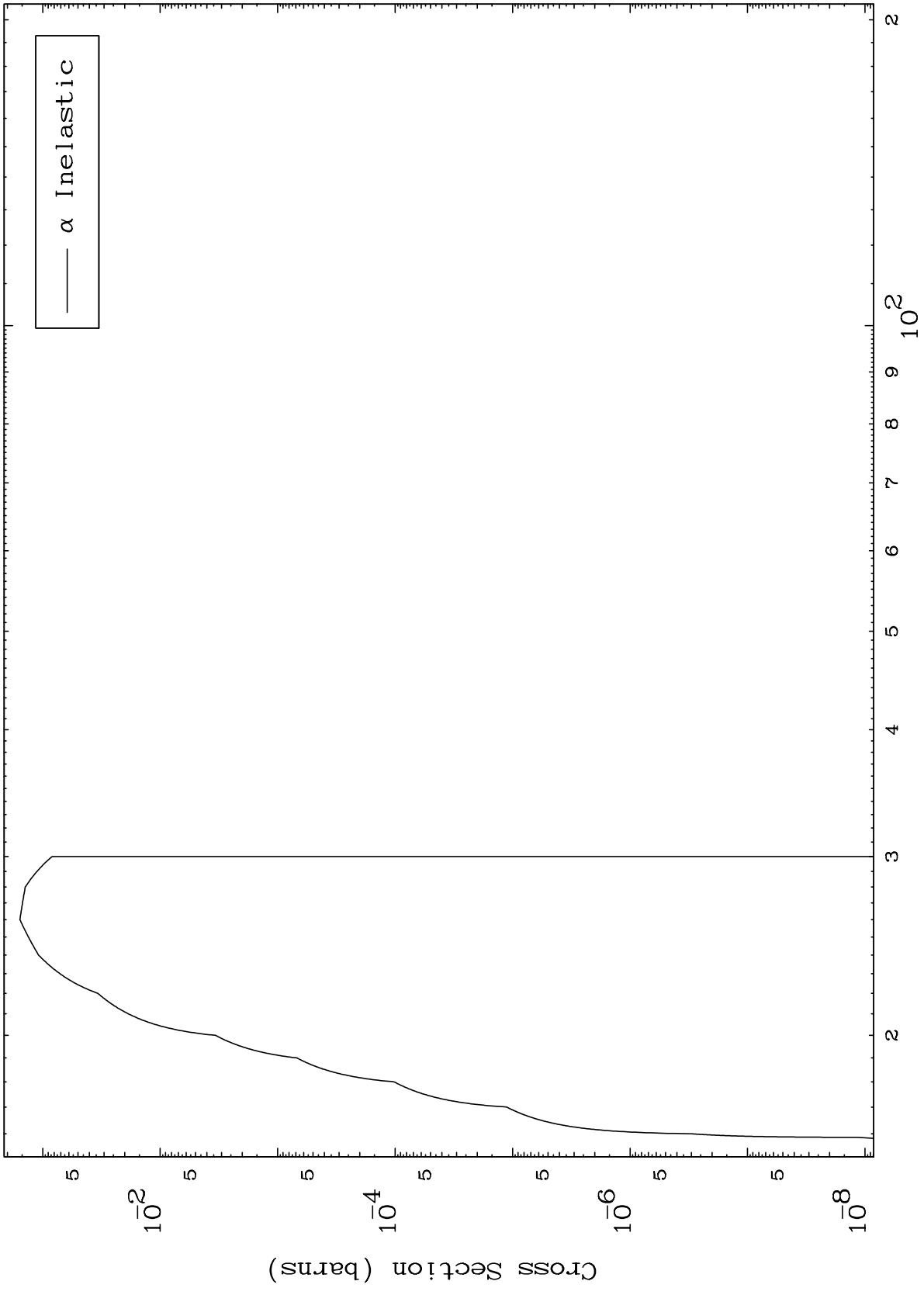


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MAT 8708

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

87-Fr-206



5

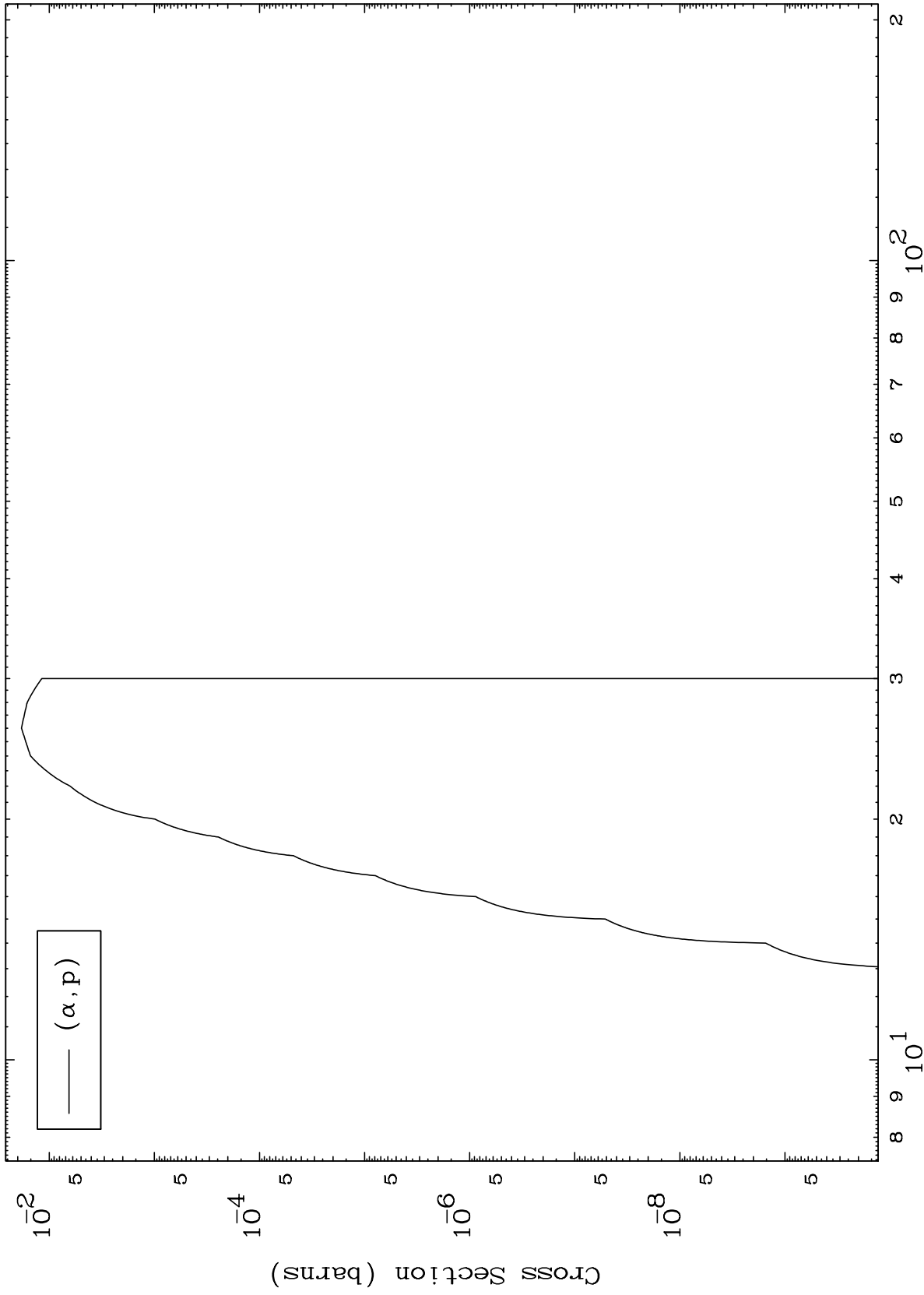
Incident Energy (MeV)

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MAT 8708

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

87-Fr-206



6

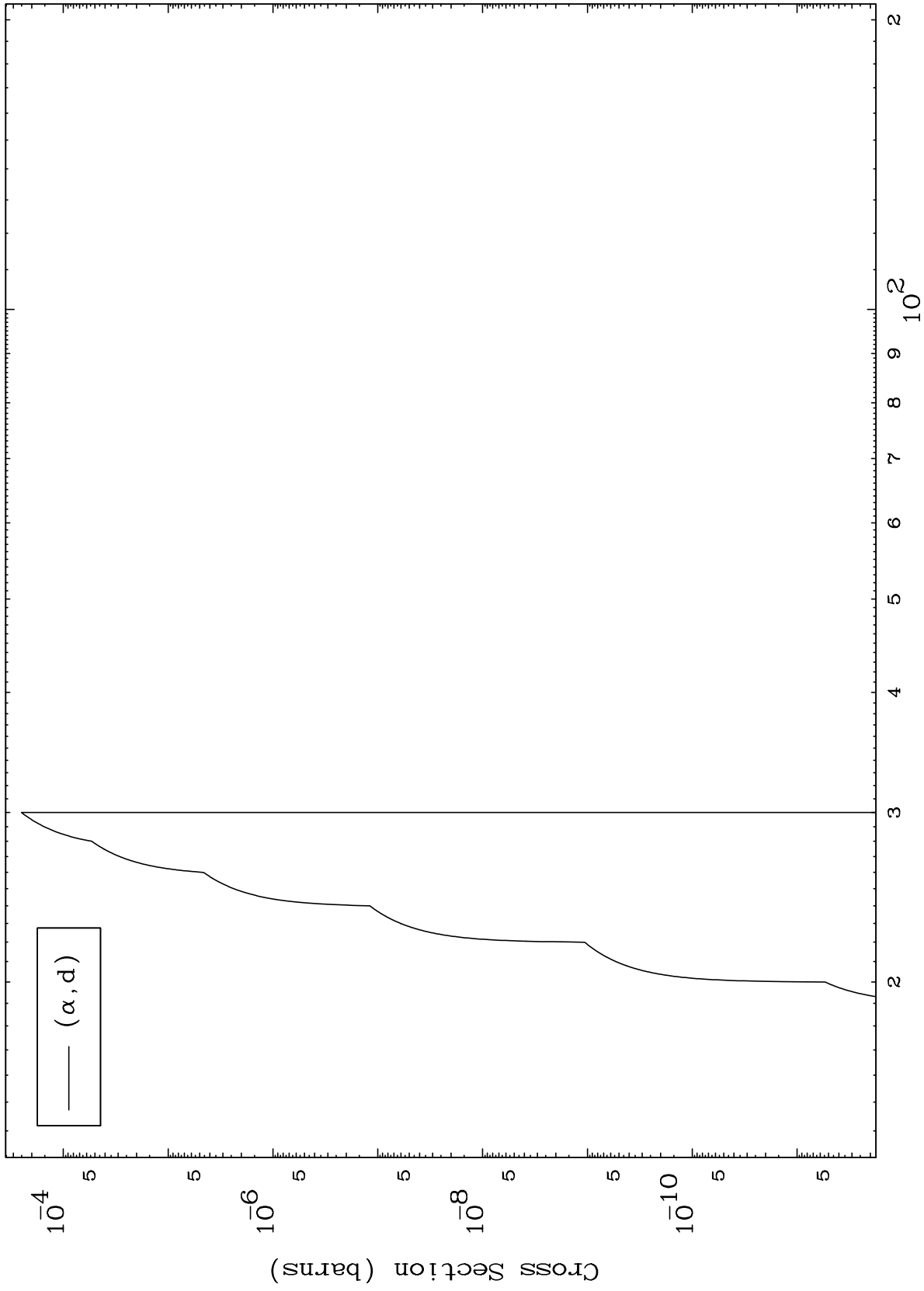
Incident Energy (MeV)

87-Fr-206

MAT 8708

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

87-Fr-206



7

Incident Energy (MeV)

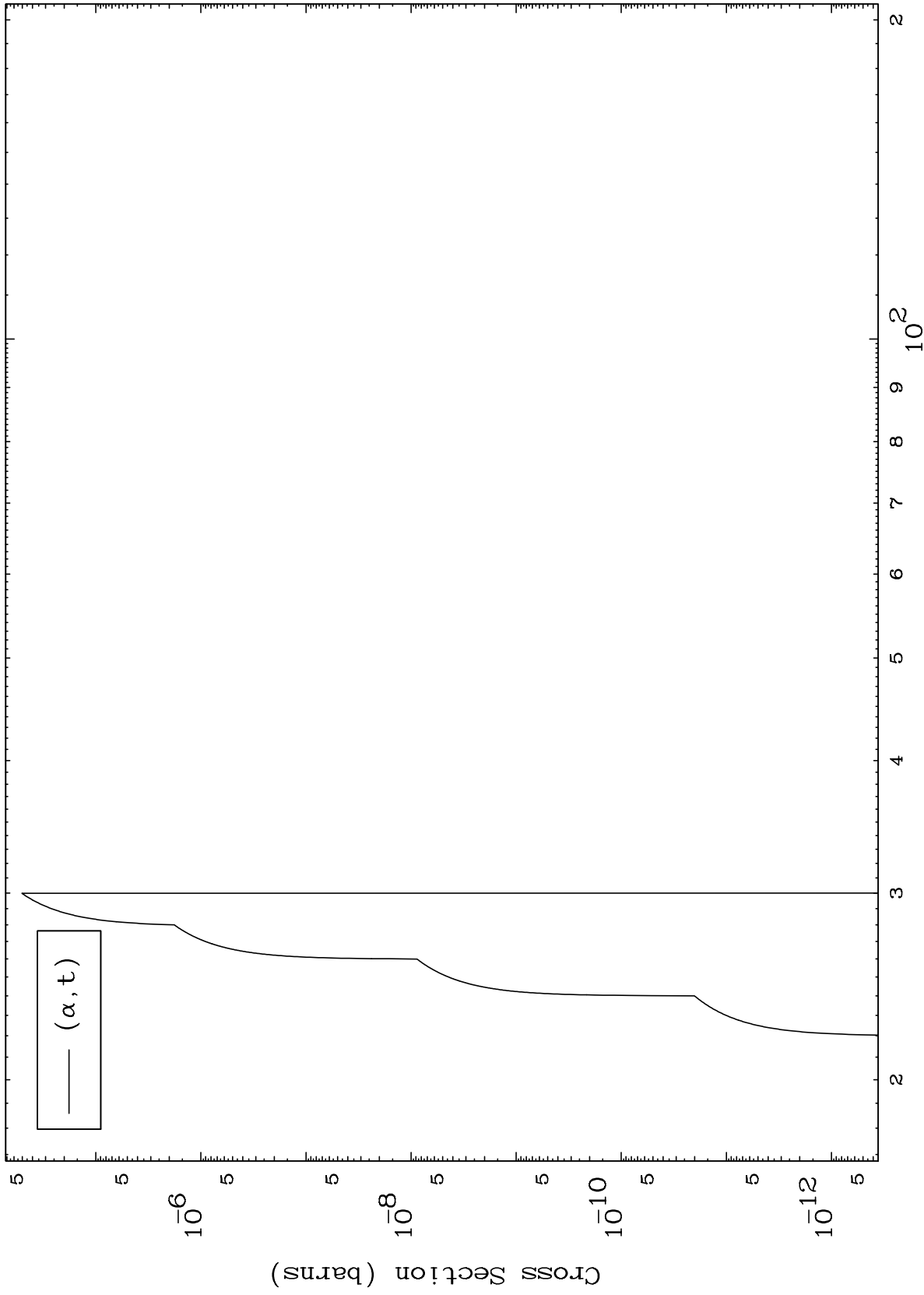
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MAT 8708

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

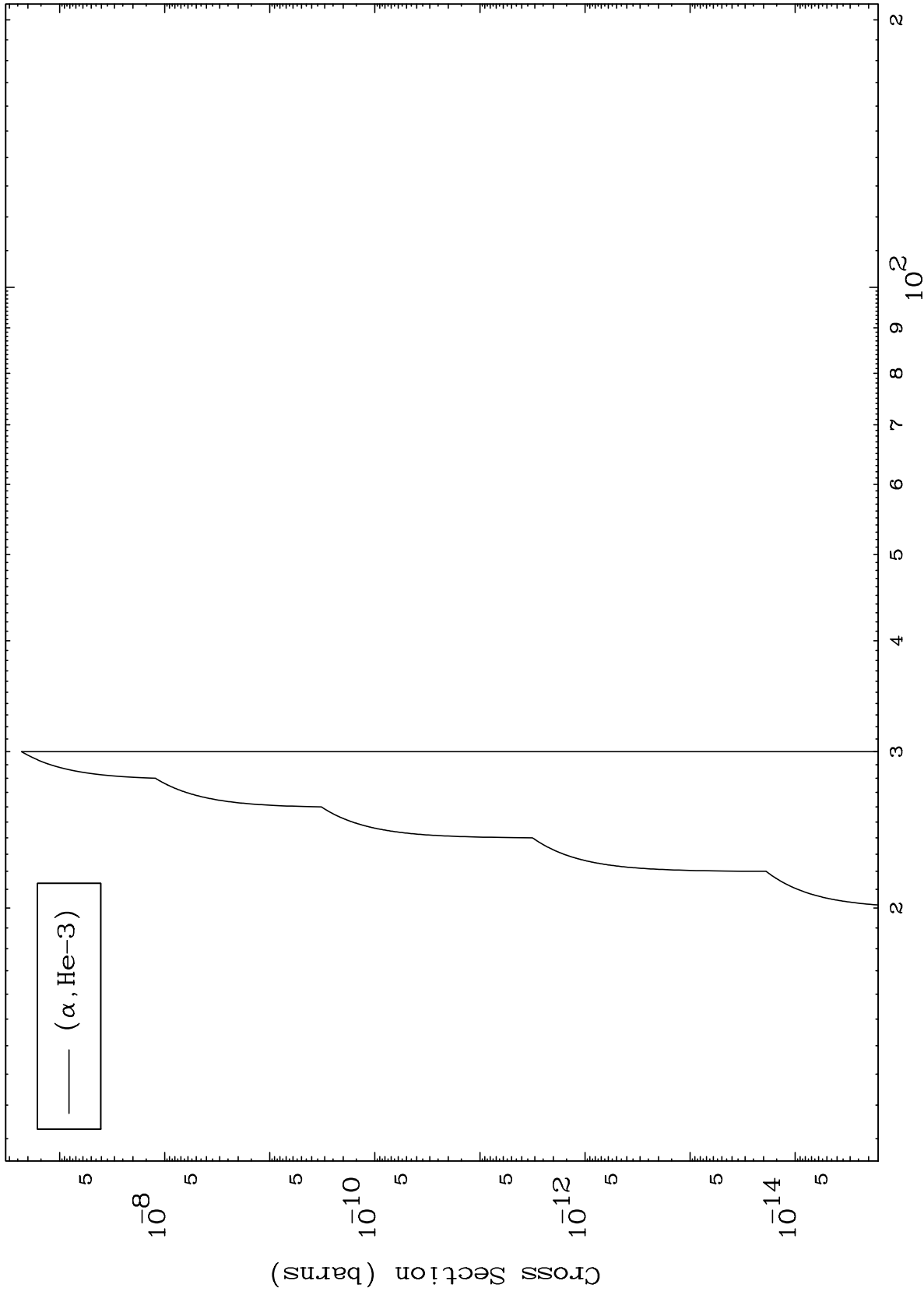
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8

Incident Energy (MeV)

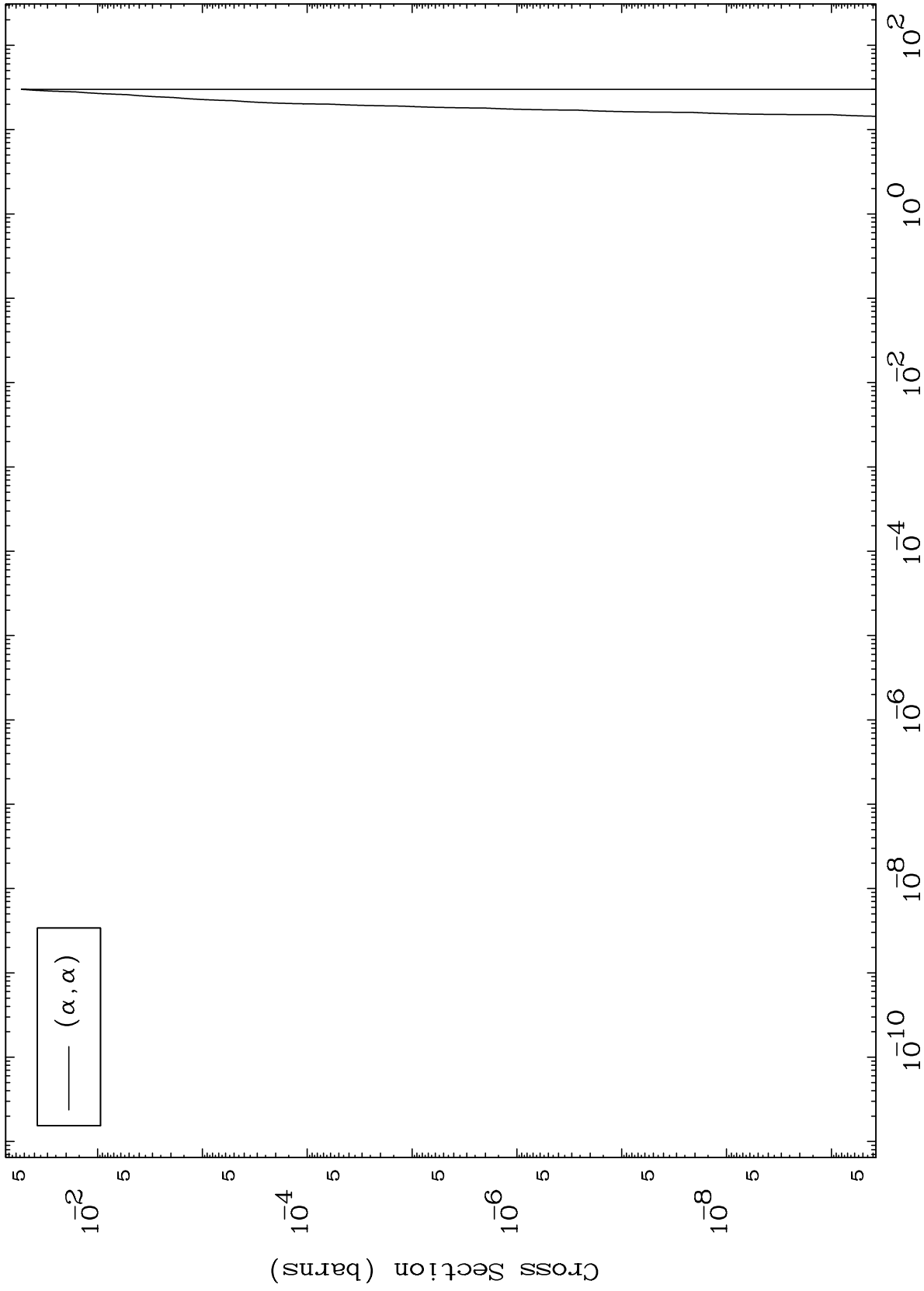
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MAT 8708

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

87-Fr-206



10

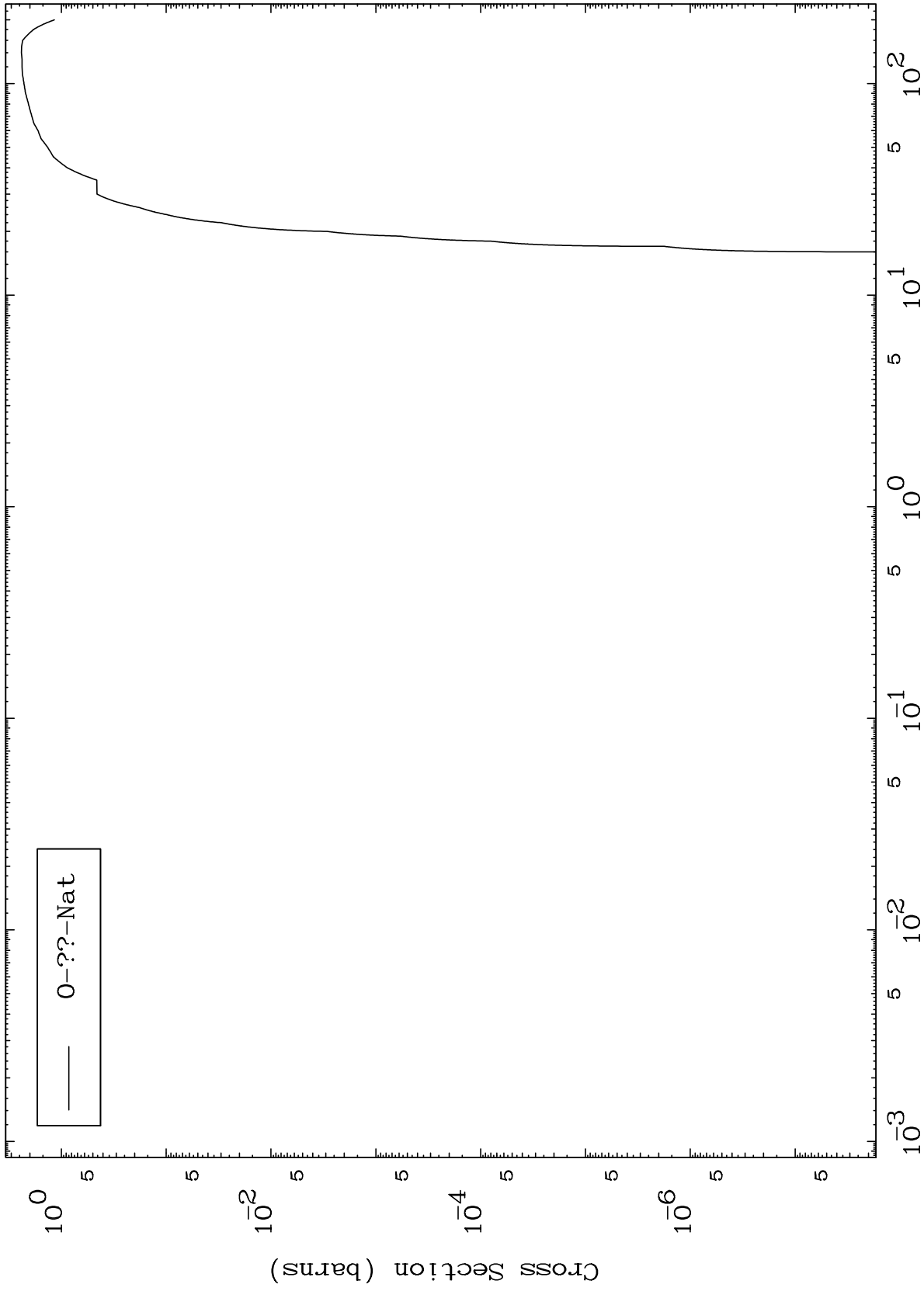
Incident Energy (MeV)

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MAT 8708

$\alpha$  Fission  
Radionuclide Production Cross Section

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11

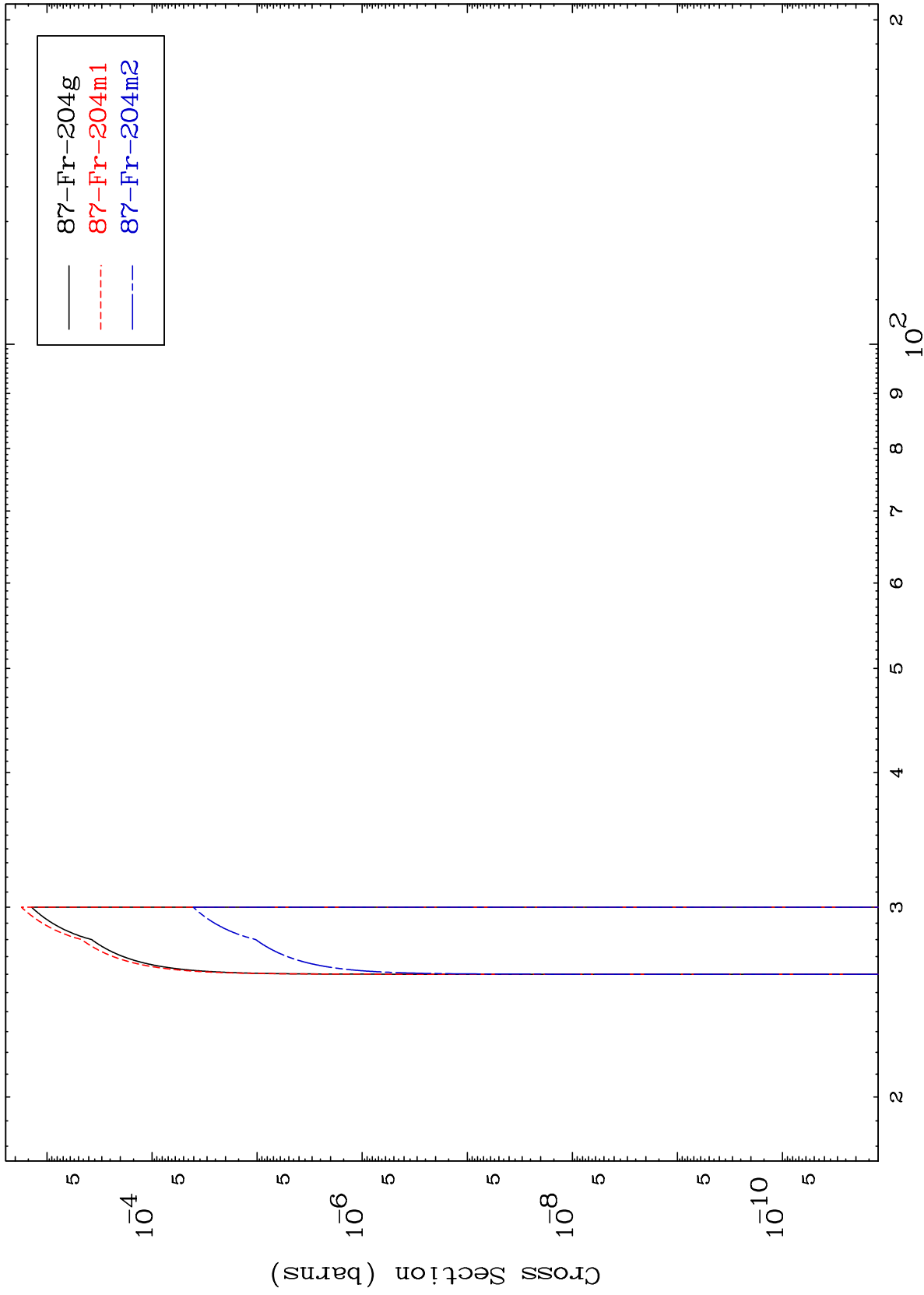
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MAT 8708

$(\alpha, 2n) \alpha$

87-Fr-206

Radionuclide Production Cross Section



12

Incident Energy (MeV)

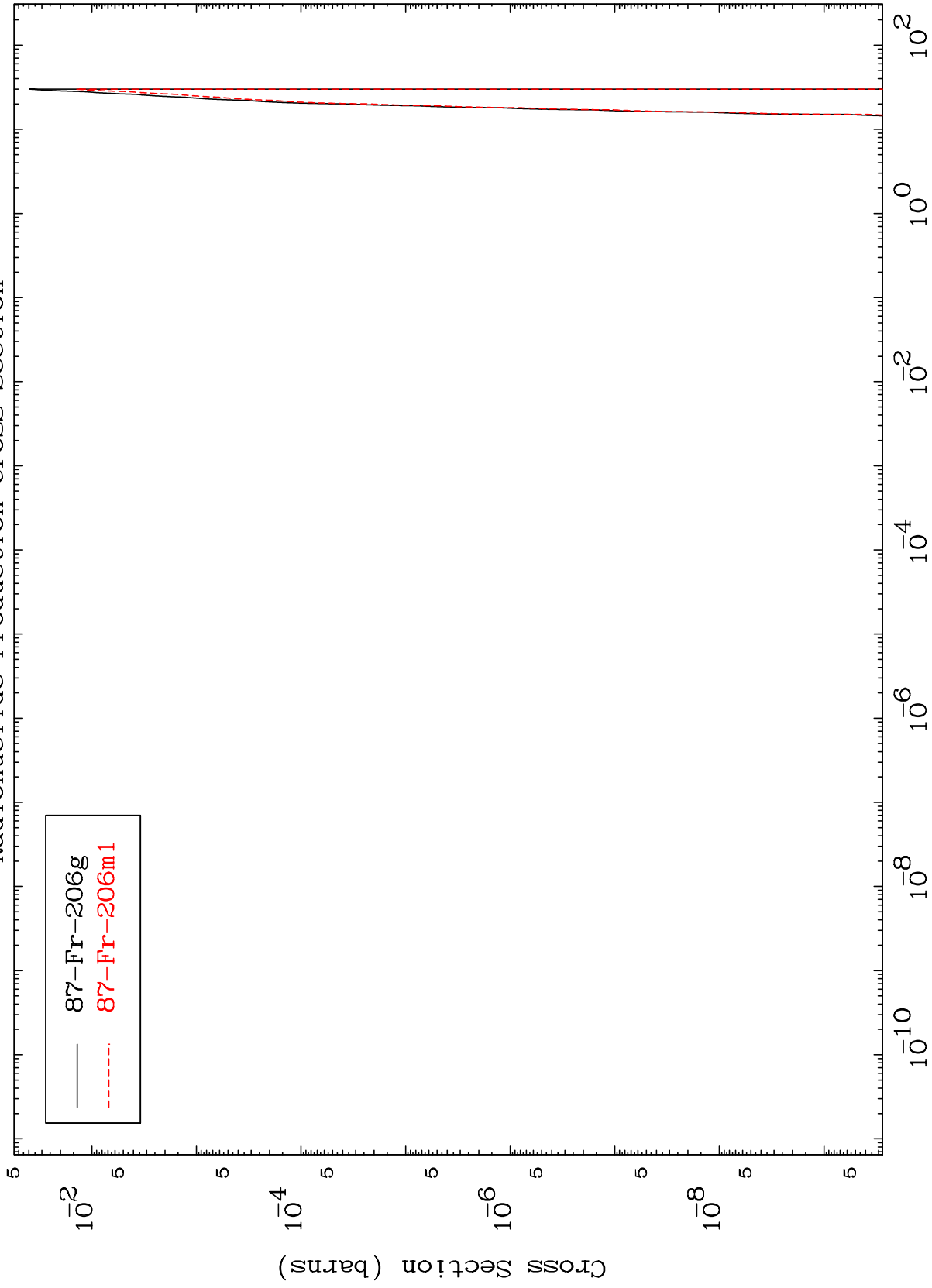
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MAT 8708

( $\alpha, \alpha$ )

87-Fr-206

Radionuclide Production Cross Section



13

Incident Energy (MeV)

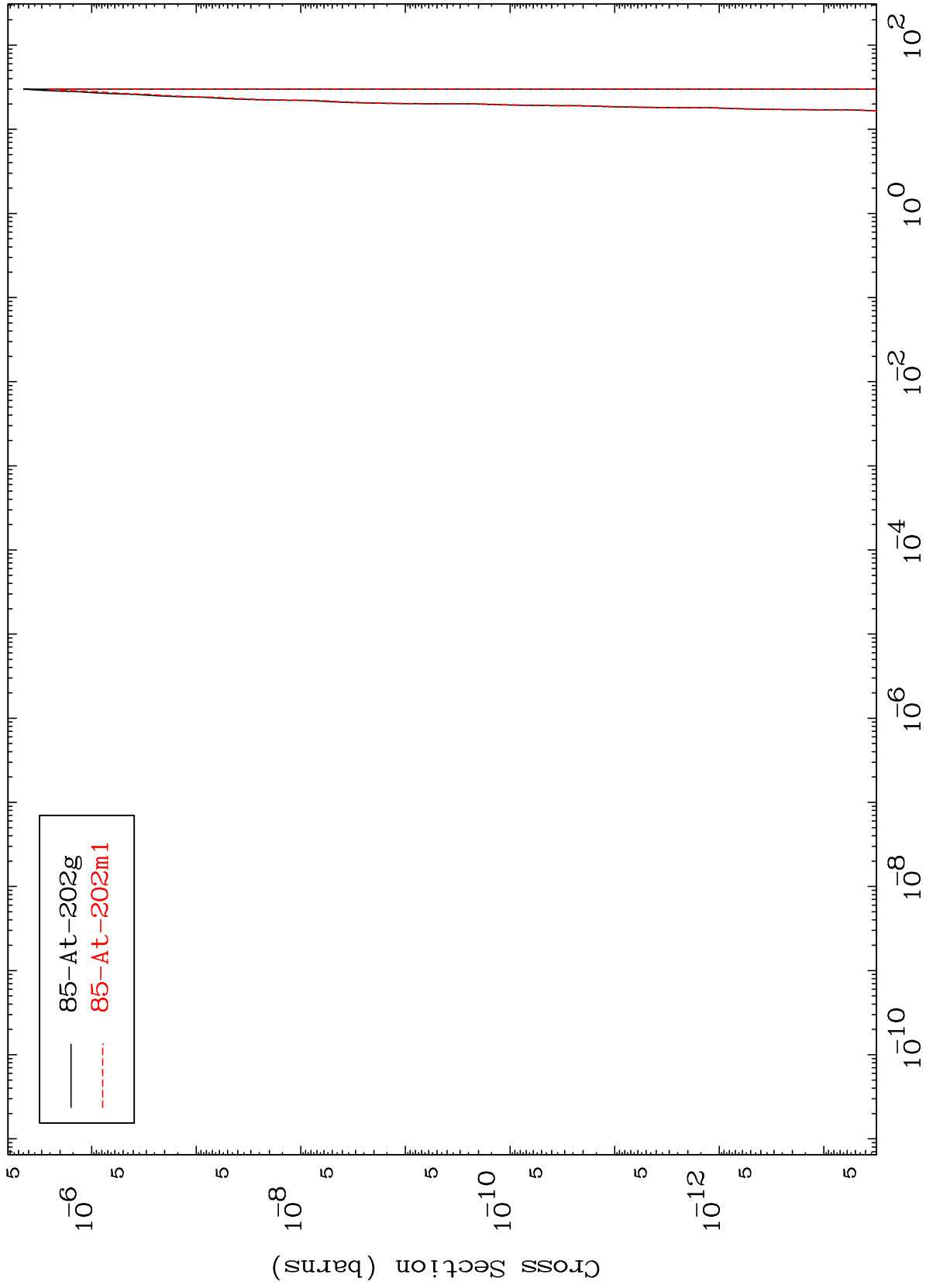
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MAT 8708

( $\alpha, 2\alpha$ )

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Radionuclide Production Cross Section



14

Incident Energy (MeV)

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