Program EVALPLOT
(Version 2018-1)
by
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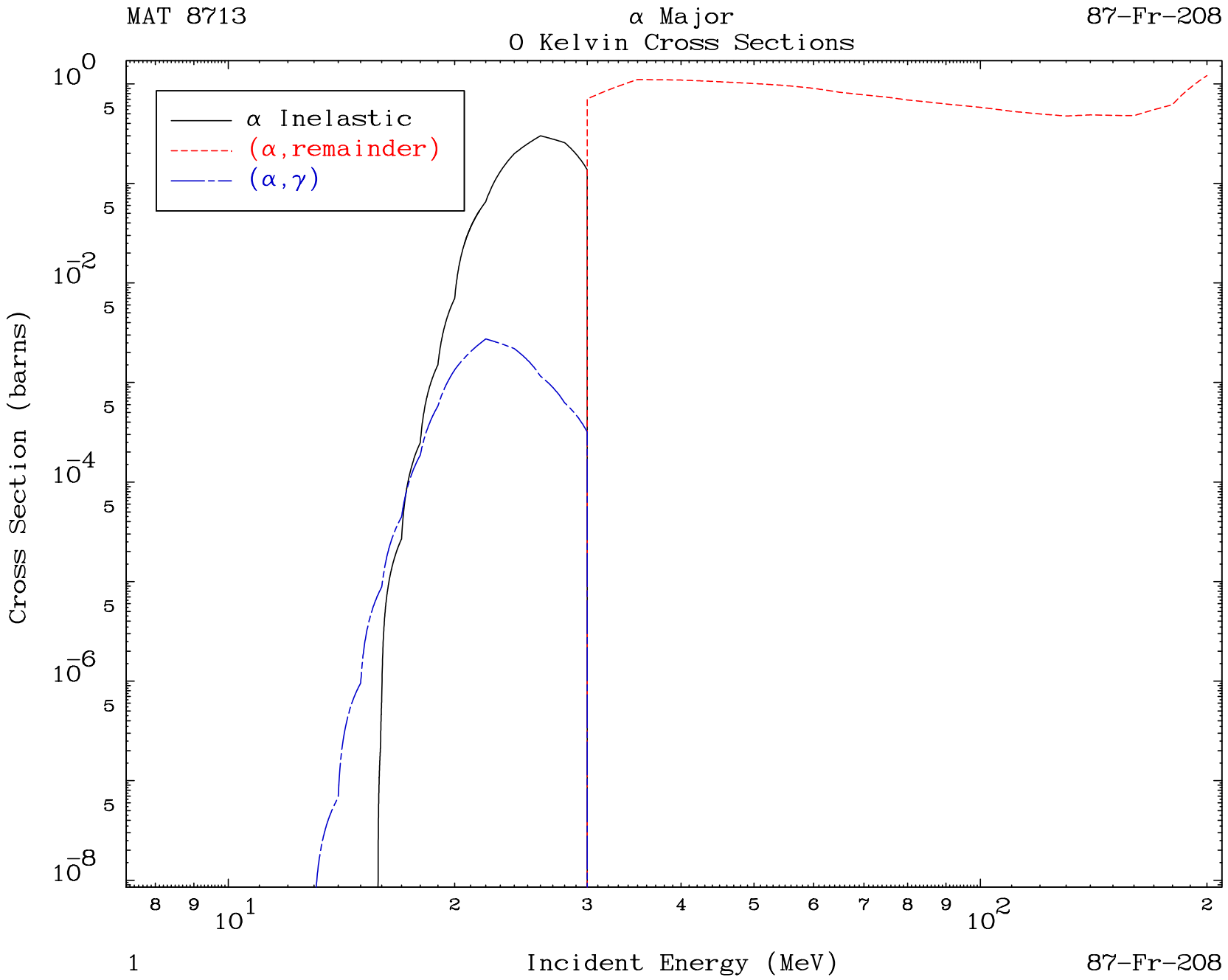
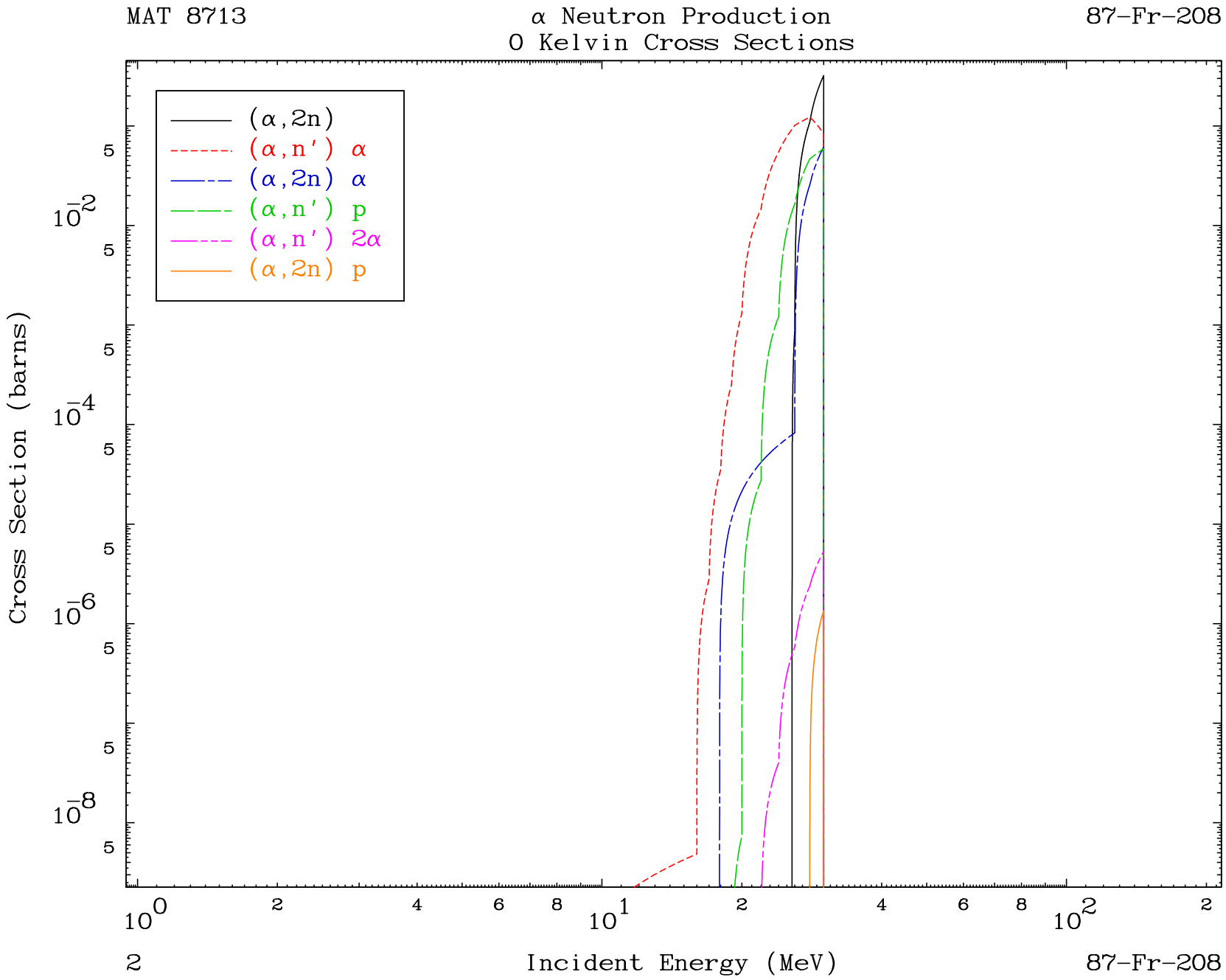
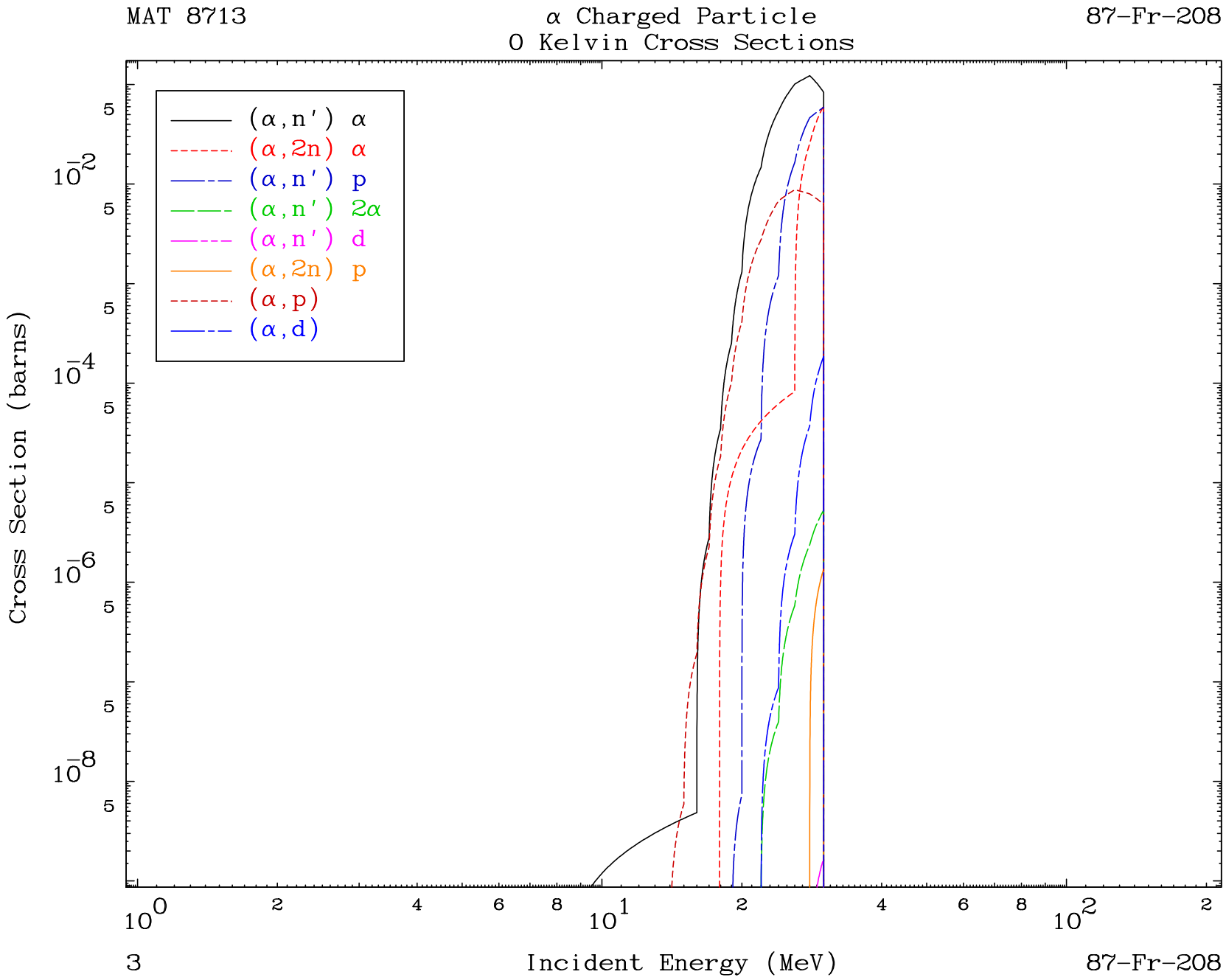
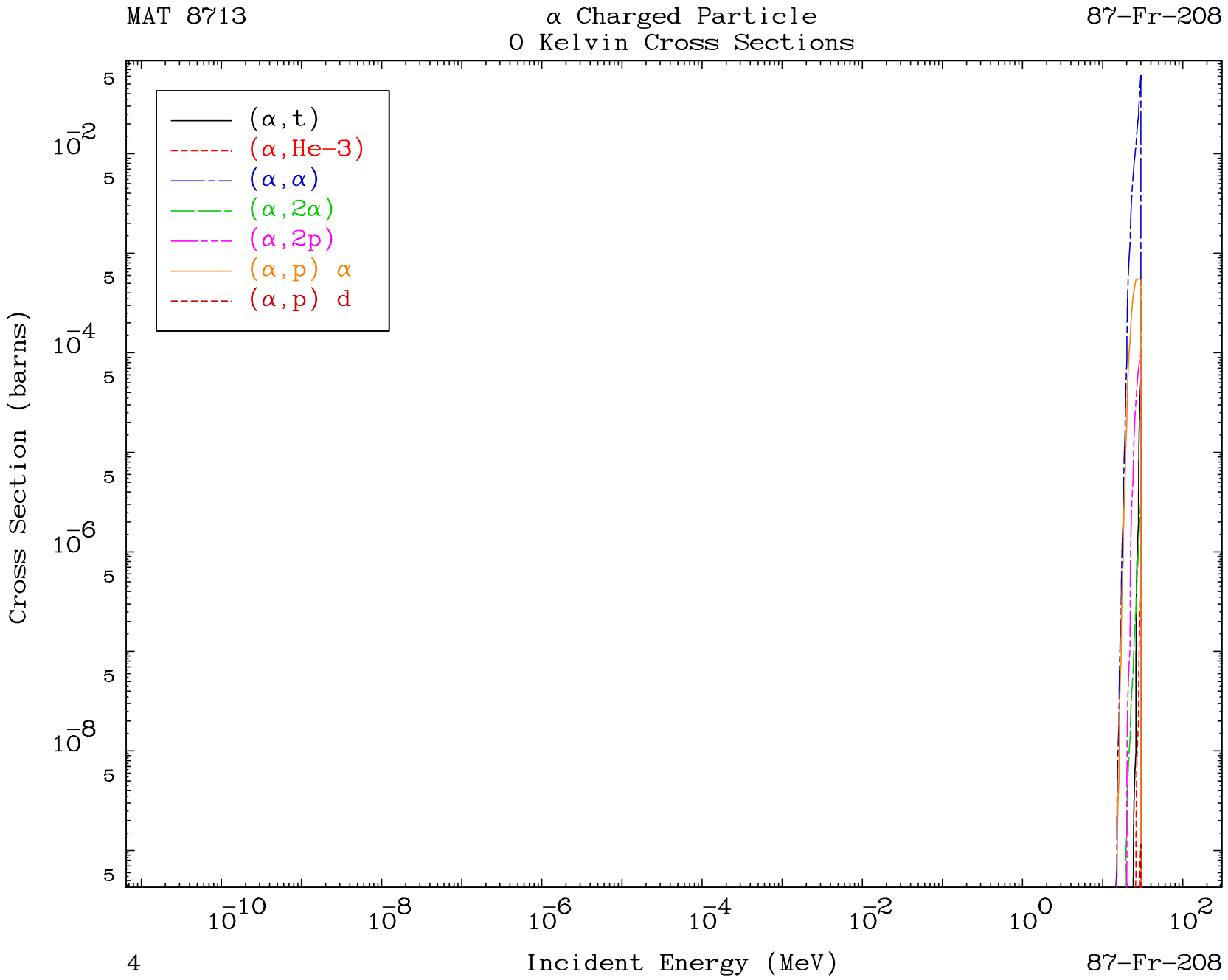
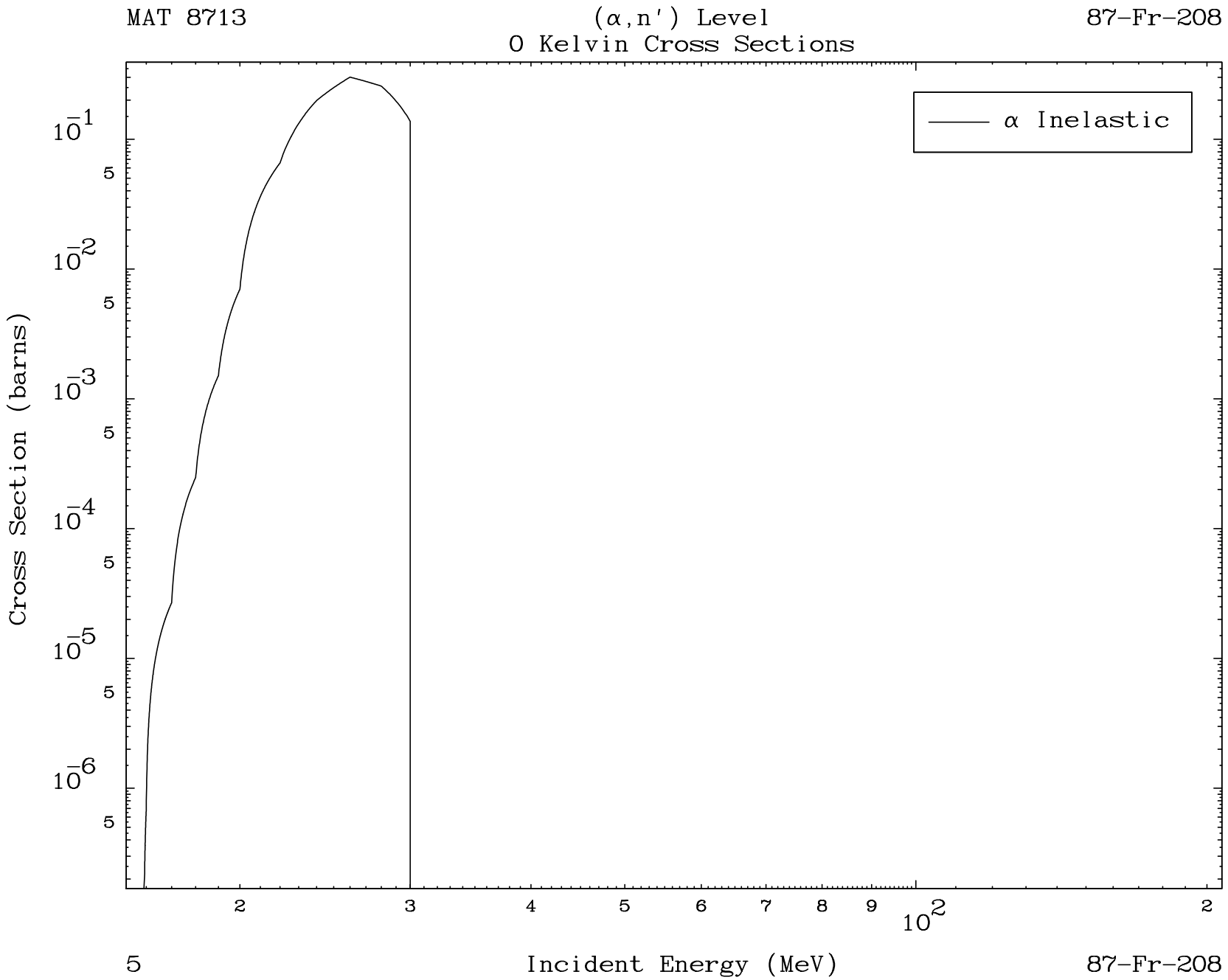
Press Mouse Button to Start

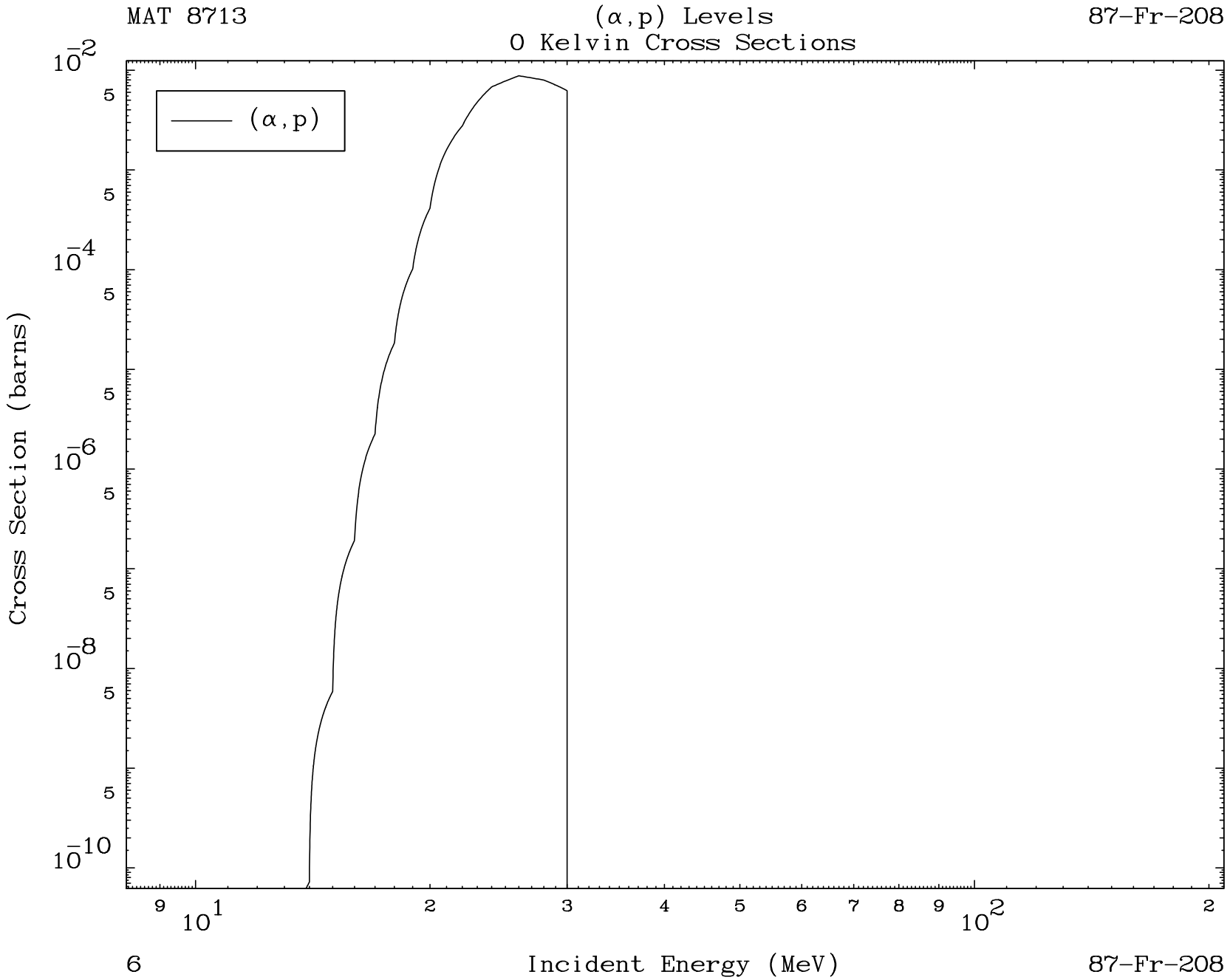

MAT 8713
α Major
87-Fr-208
0 Kelvin Cross Sections
α Inelastic
(α,remainder)
(α,γ)
Cross Section (barns)
Incident Energy (MeV)
87-Fr-208

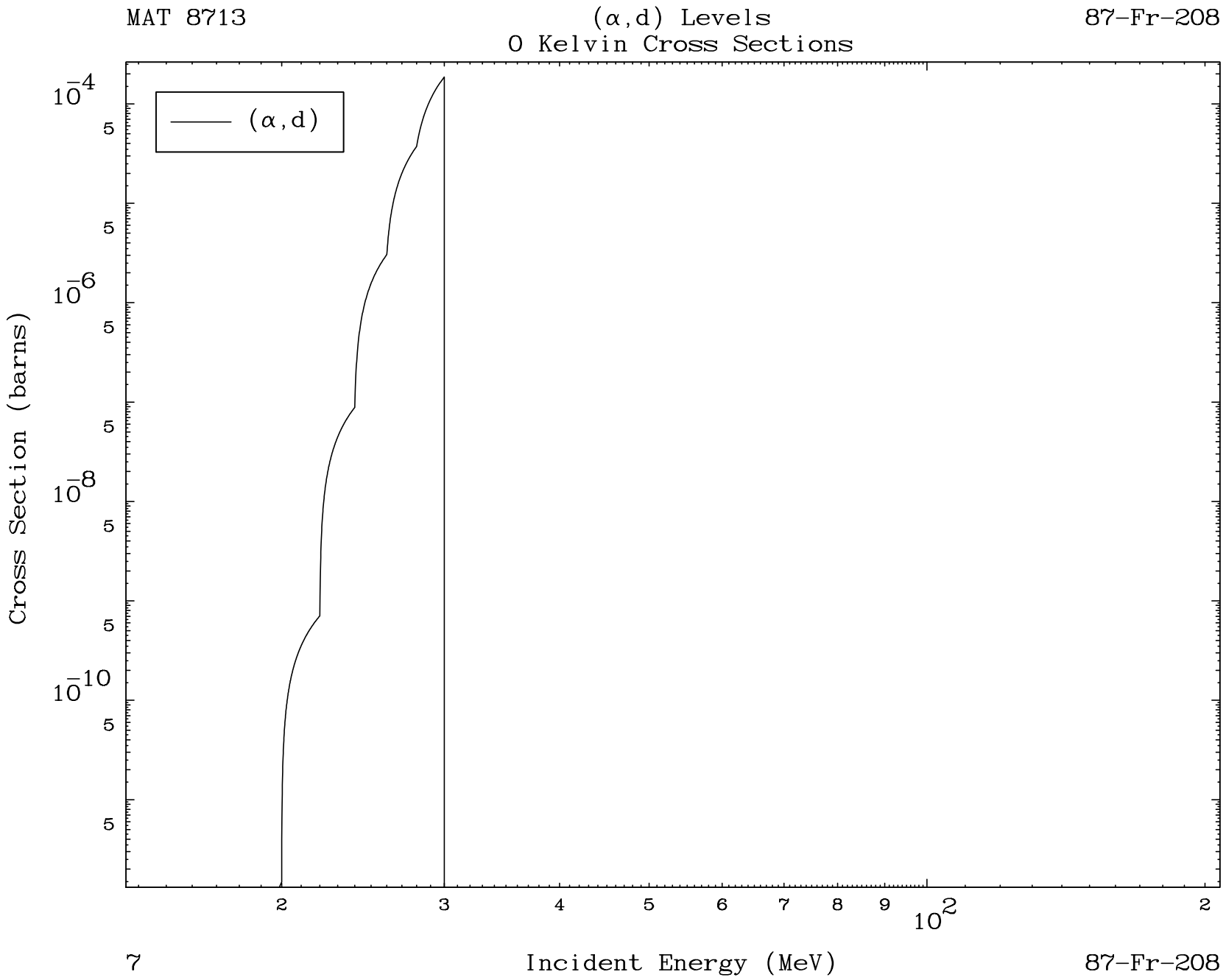

MAT 8713
α Neutron Production
0 Kelvin Cross Sections
87-Fr-208
(α,2n)
(α,n') α
(α,2n) α
(α,n') p
(α,n') 2α
(α,2n) p
Cross Section (barns)
Incident Energy (MeV)
87-Fr-208

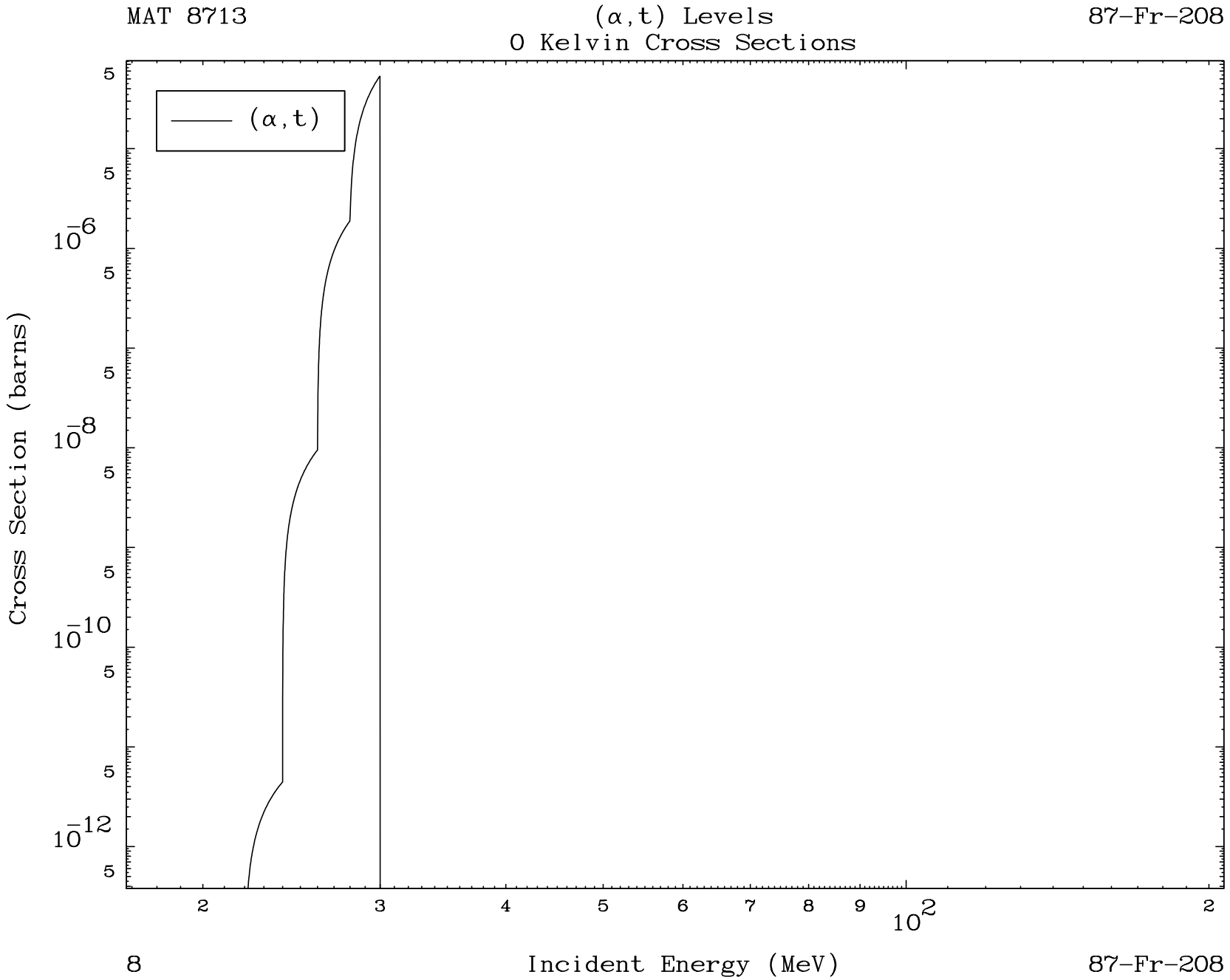

MAT 8713
α Charged Particle
0 Kelvin Cross Sections
87-Fr-208
(α,n') α
(α,2n) α
(α,n') p
(α,n') 2α
(α,n') d
(α,2n) p
(α,p)
(α,d)
Cross Section (barns)
Incident Energy (MeV)
3
87-Fr-208

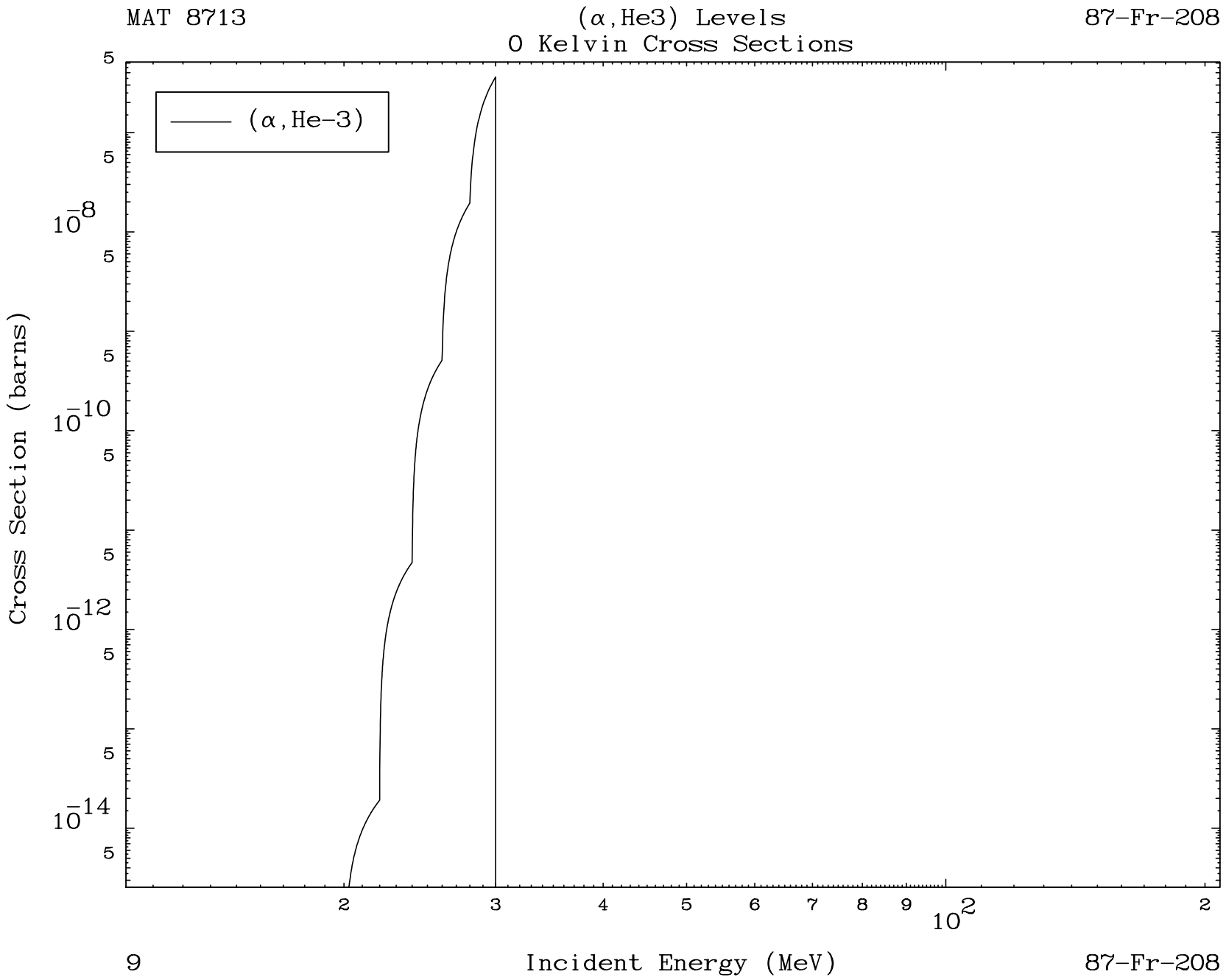

MAT 8713
α Charged Particle
0 Kelvin Cross Sections
87-Fr-208
(α,t)
(α,He-3)
(α,α)
(α,2α)
(α,2p)
(α,p) α
(α,p) d
Cross Section (barns)
Incident Energy (MeV)
4
87-Fr-208

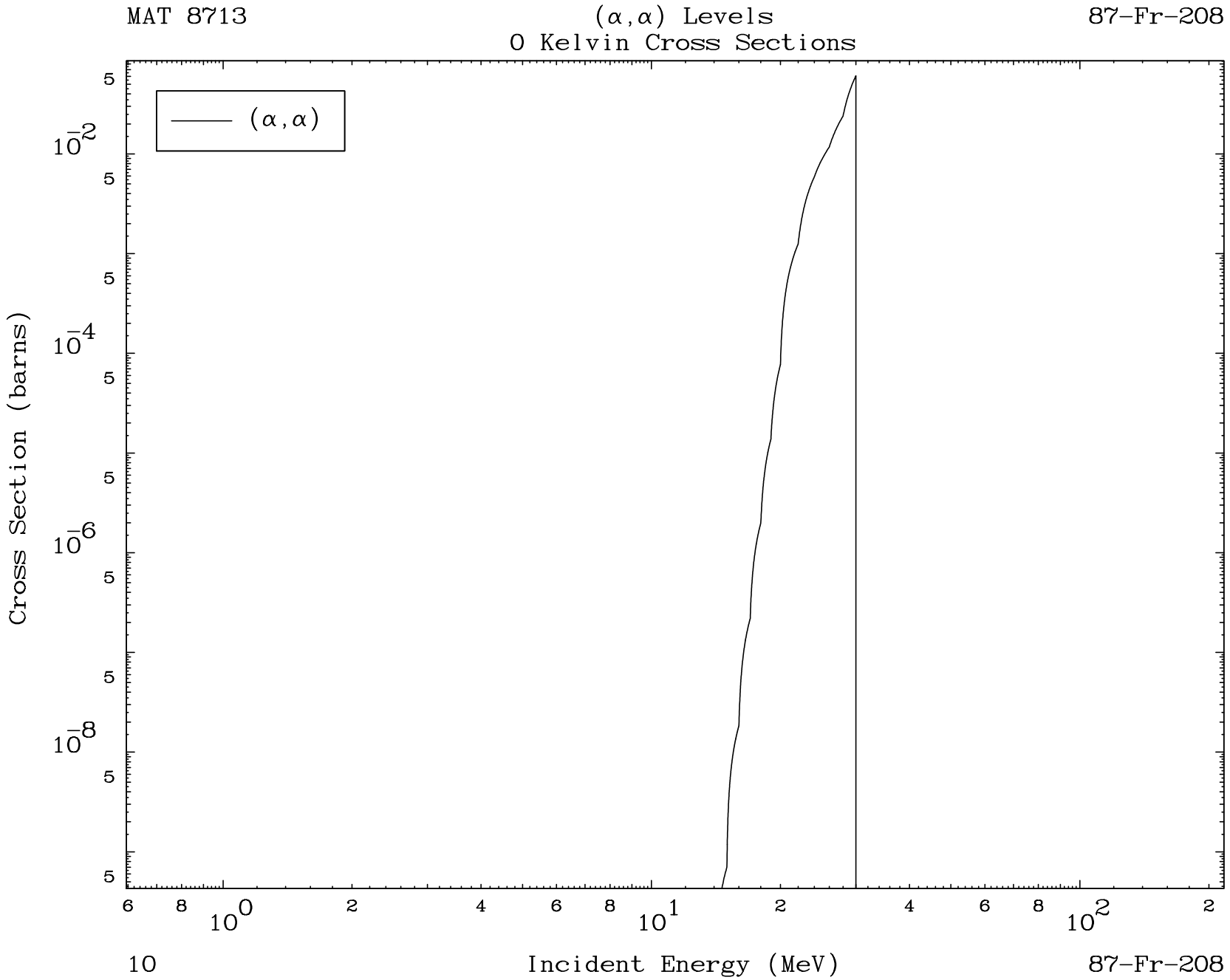

MAT 8713
(α,n') Level
87-Fr-208
0 Kelvin Cross Sections
α Inelastic
Cross Section (barns)
10-1
10-2
10-3
10-4
10-5
10-6
5
2
3
4
5
6
7
8
9
10 2
2
Incident Energy (MeV)
87-Fr-208

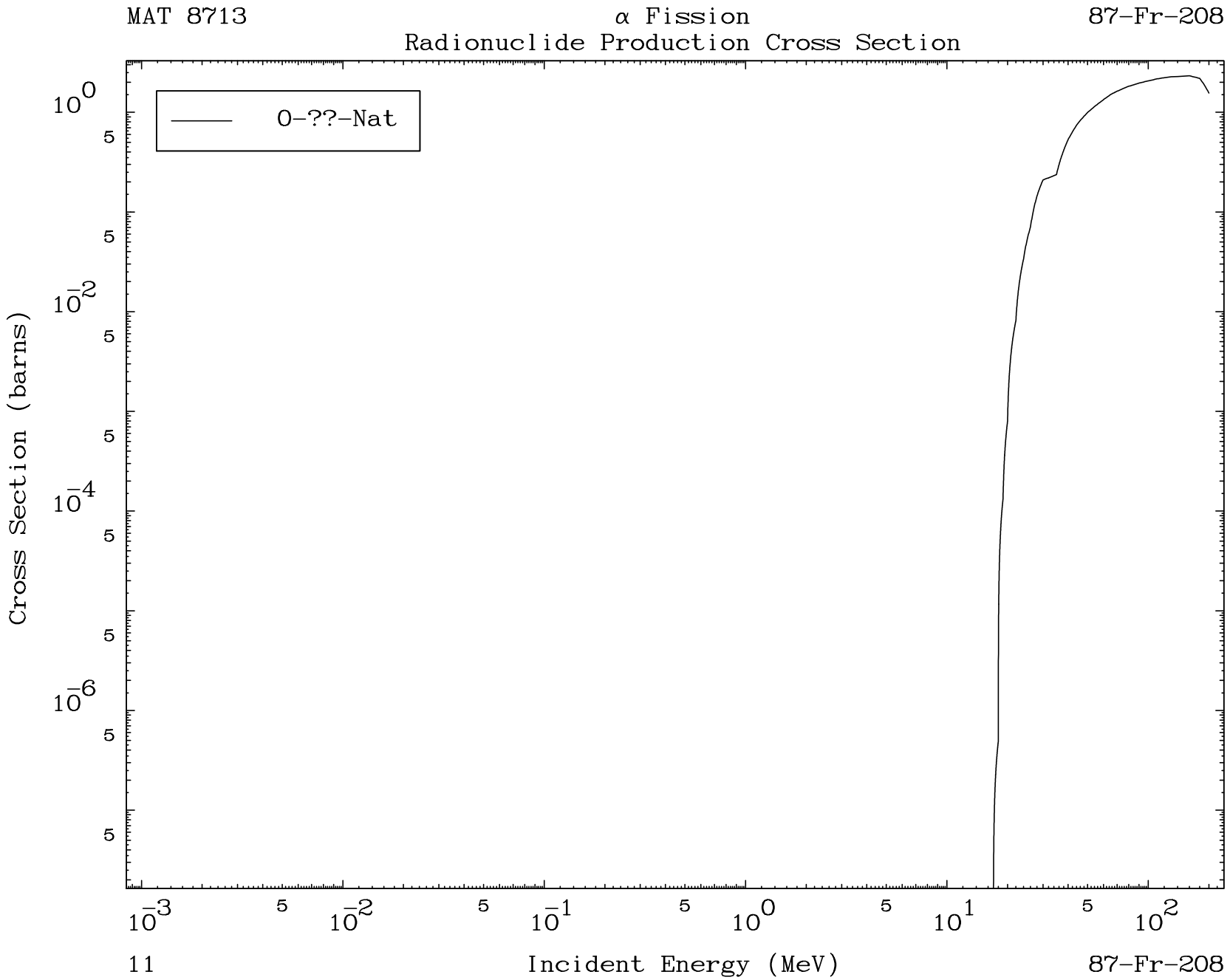

MAT 8713
(α,p) Levels
87-Fr-208
0 Kelvin Cross Sections
(α,p)
Cross Section (barns)
Incident Energy (MeV)
6
87-Fr-208

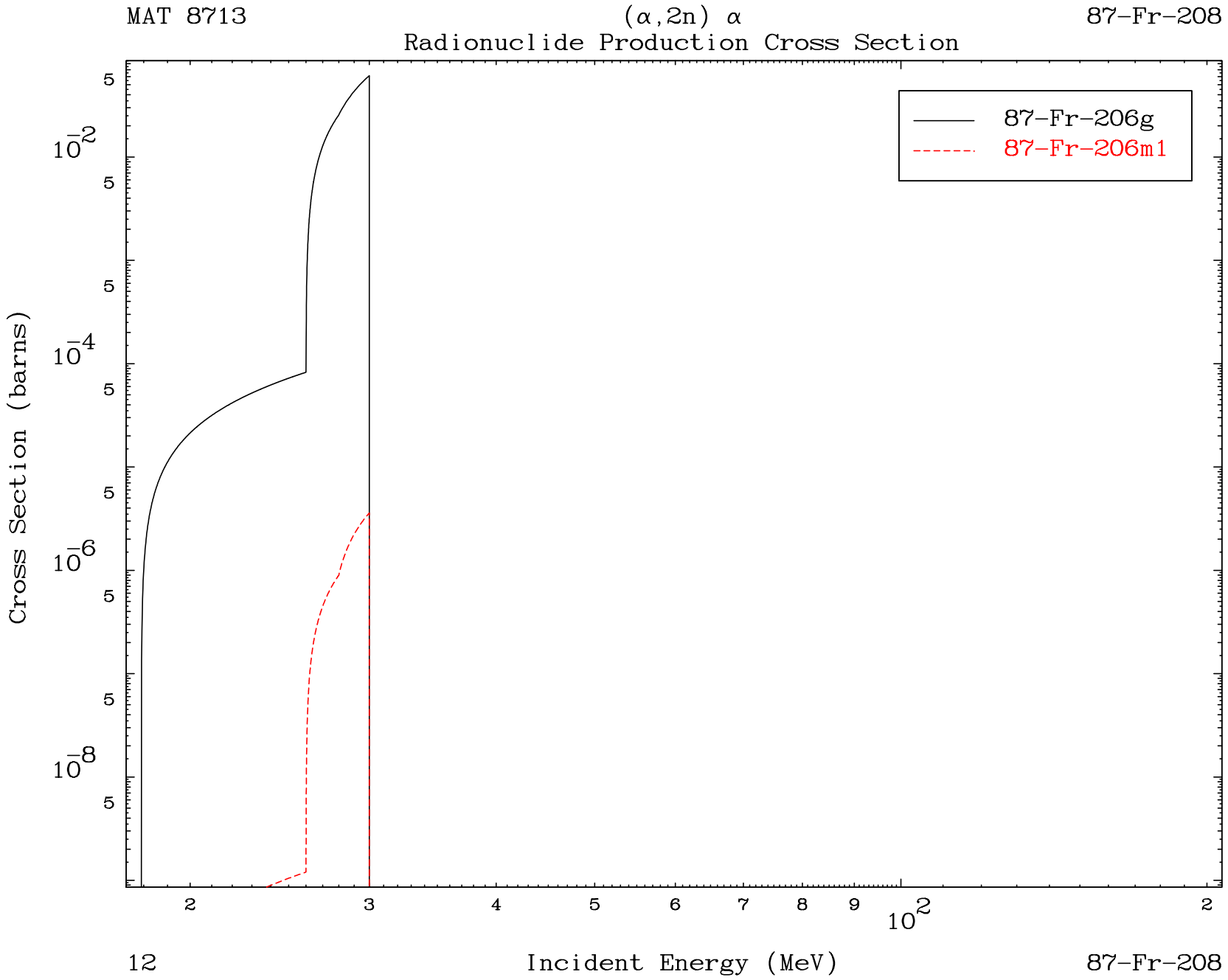

MAT 8713
(α,d) Levels
87-Fr-208
0 Kelvin Cross Sections
(α,d)
Cross Section (barns)
10^-4
10^-6
10^-8
10^-10
Incident Energy (MeV)
10^2
7
87-Fr-208

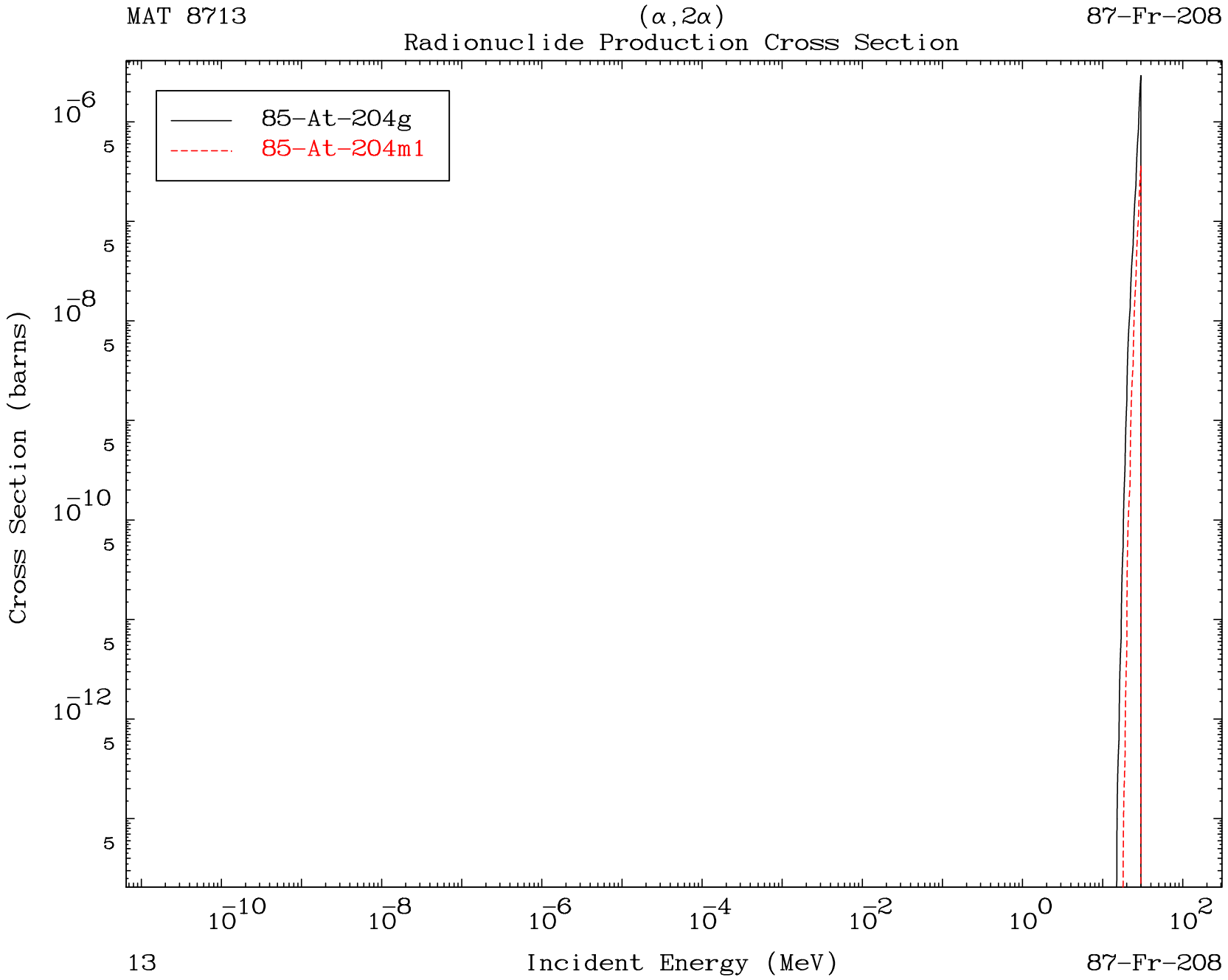

MAT 8713
(α,t) Levels
87-Fr-208
0 Kelvin Cross Sections
(α,t)
Cross Section (barns)
10-6
10-8
10-10
10-12
5
2
3
4
5
6
7
8
9
10 2
Incident Energy (MeV)
8
87-Fr-208


MAT 8713
(α,He3) Levels
87-Fr-208
0 Kelvin Cross Sections
(α,He-3)
Cross Section (barns)
Incident Energy (MeV)
9
87-Fr-208


MAT 8713
(α,α) Levels
87-Fr-208
0 Kelvin Cross Sections
(α,α)
Cross Section (barns)
Incident Energy (MeV)
87-Fr-208


MAT 8713
α Fission
87-Fr-208
Radionuclide Production Cross Section
0-??-Nat
Cross Section (barns)
Incident Energy (MeV)
11
87-Fr-208


MAT 8713
(α,2n) α
87-Fr-208
Radionuclide Production Cross Section
87-Fr-206g
87-Fr-206m1
Cross Section (barns)
10-2
10-4
10-6
10-8
2
3
4
5
6
7
8
9
10 2
2
Incident Energy (MeV)
12
87-Fr-208


MAT 8713
(α,2α)
87-Fr-208
Radionuclide Production Cross Section
85-At-204g
85-At-204m1
Cross Section (barns)
10^-6
10^-8
10^-10
10^-12
5
10^-10
10^-8
10^-6
10^-4
10^-2
10^0
10^2
13
Incident Energy (MeV)
87-Fr-208