

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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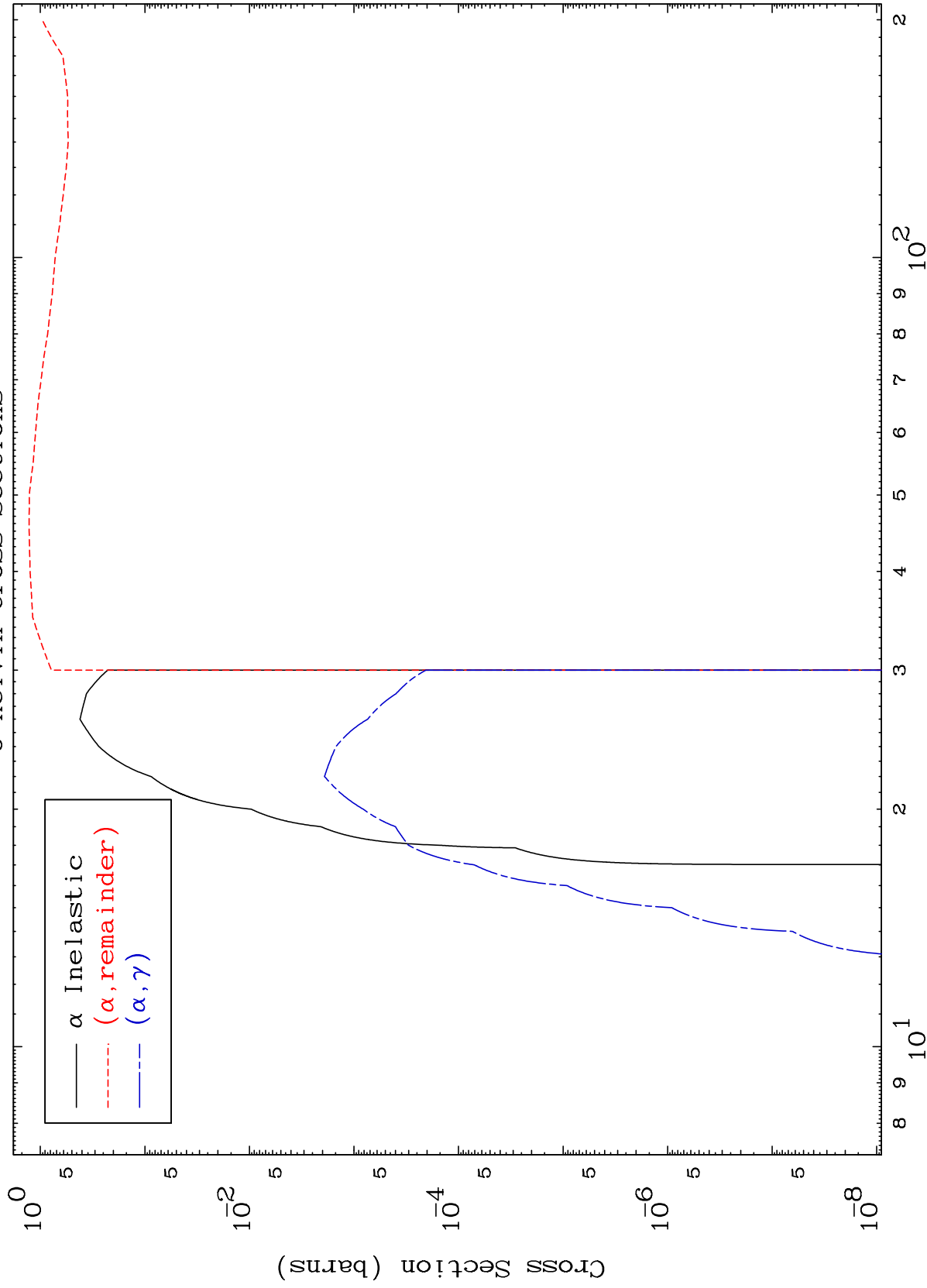
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8716

0 Kelvin  $\alpha$  Major  
Cross Sections

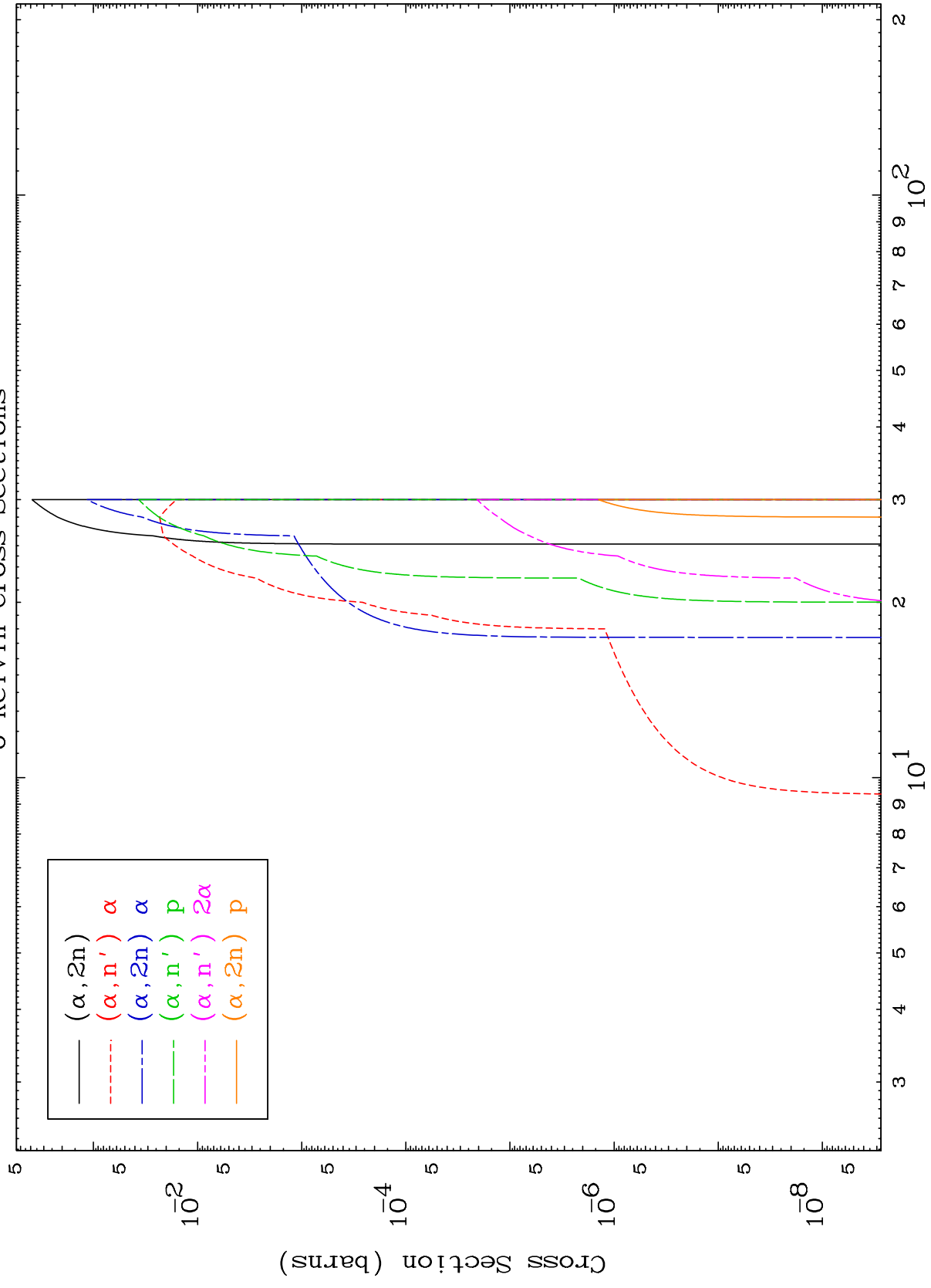
87-Fr-209

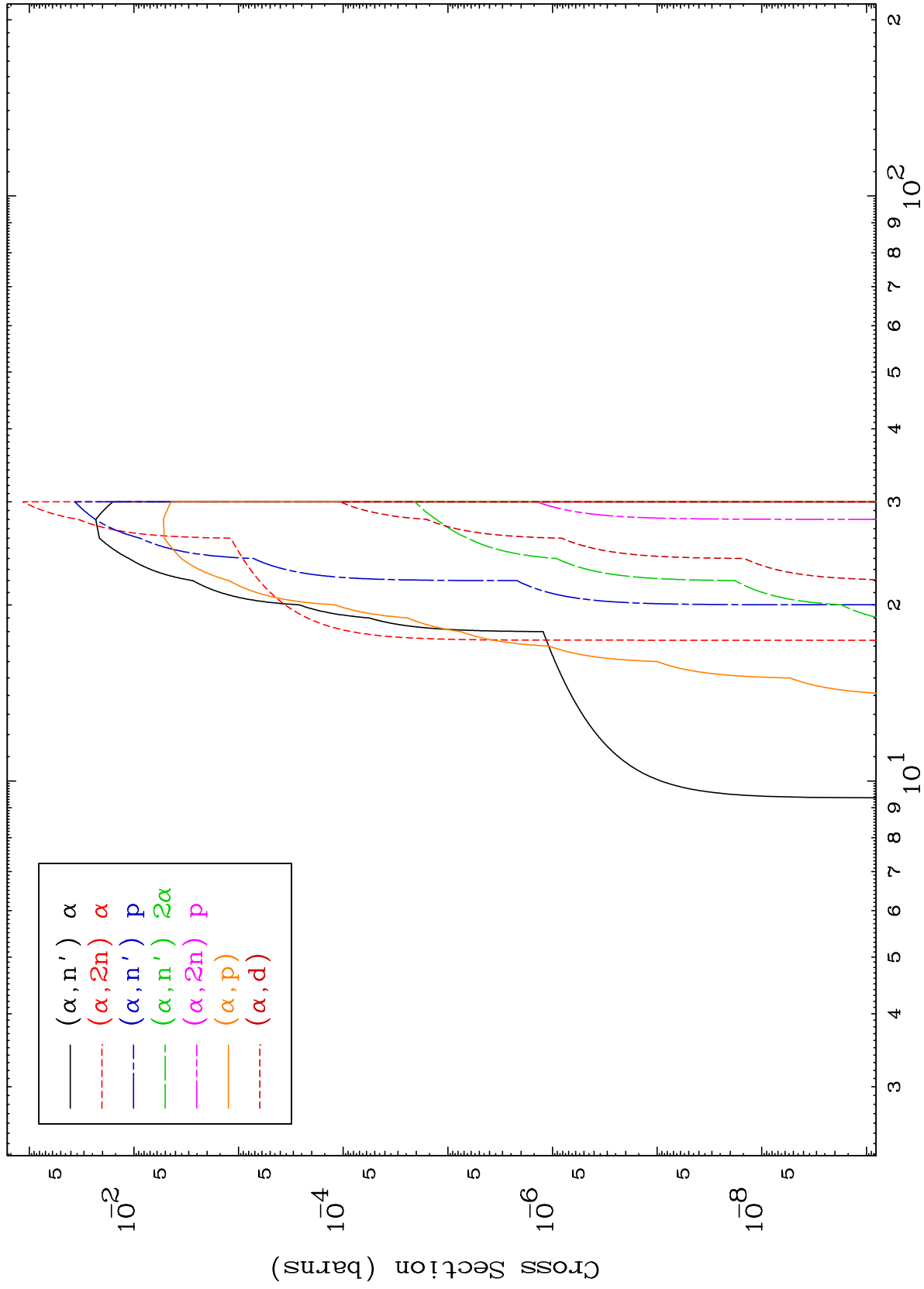


1

Incident Energy (MeV)

87-Fr-209

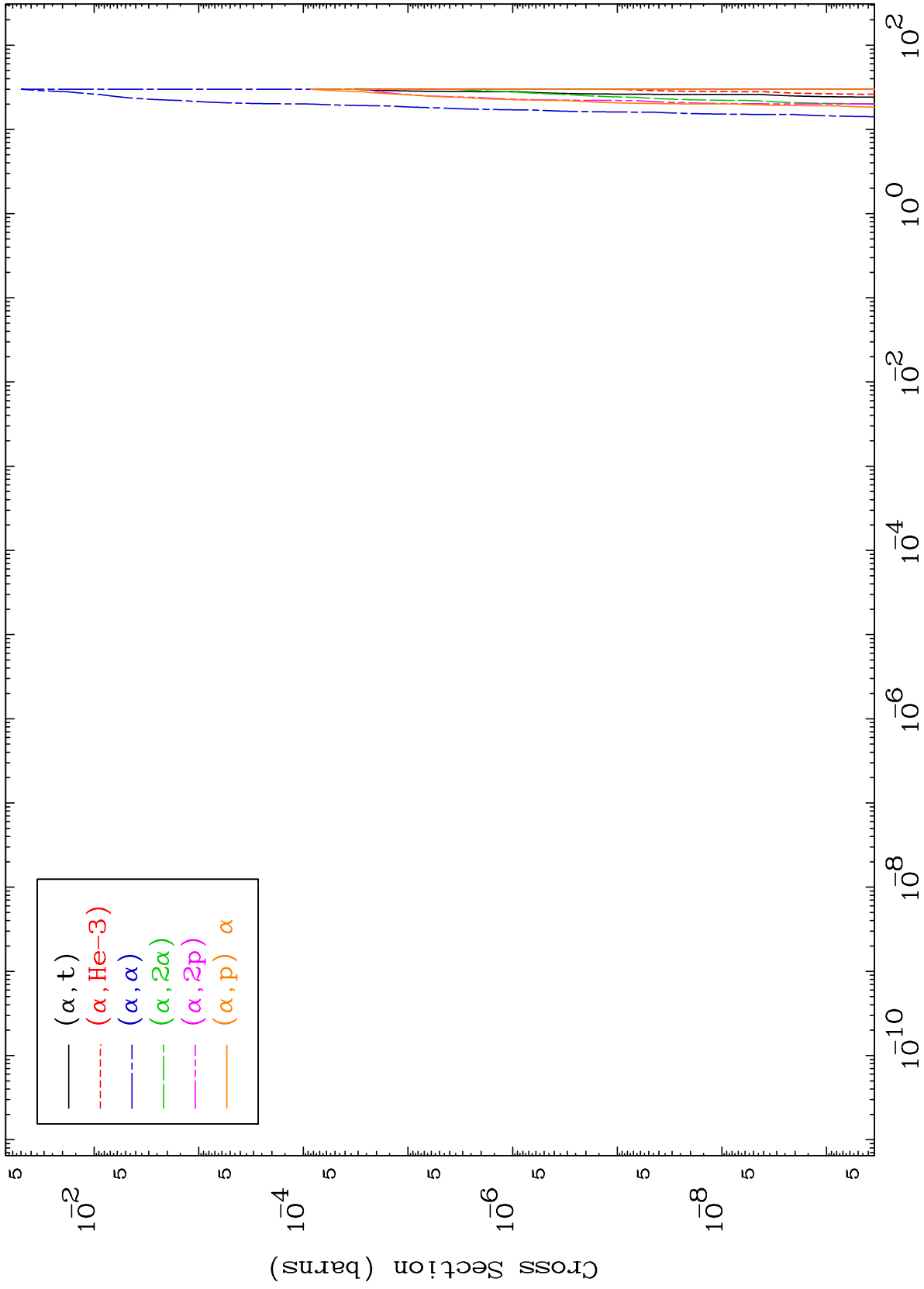




MAT 8716

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

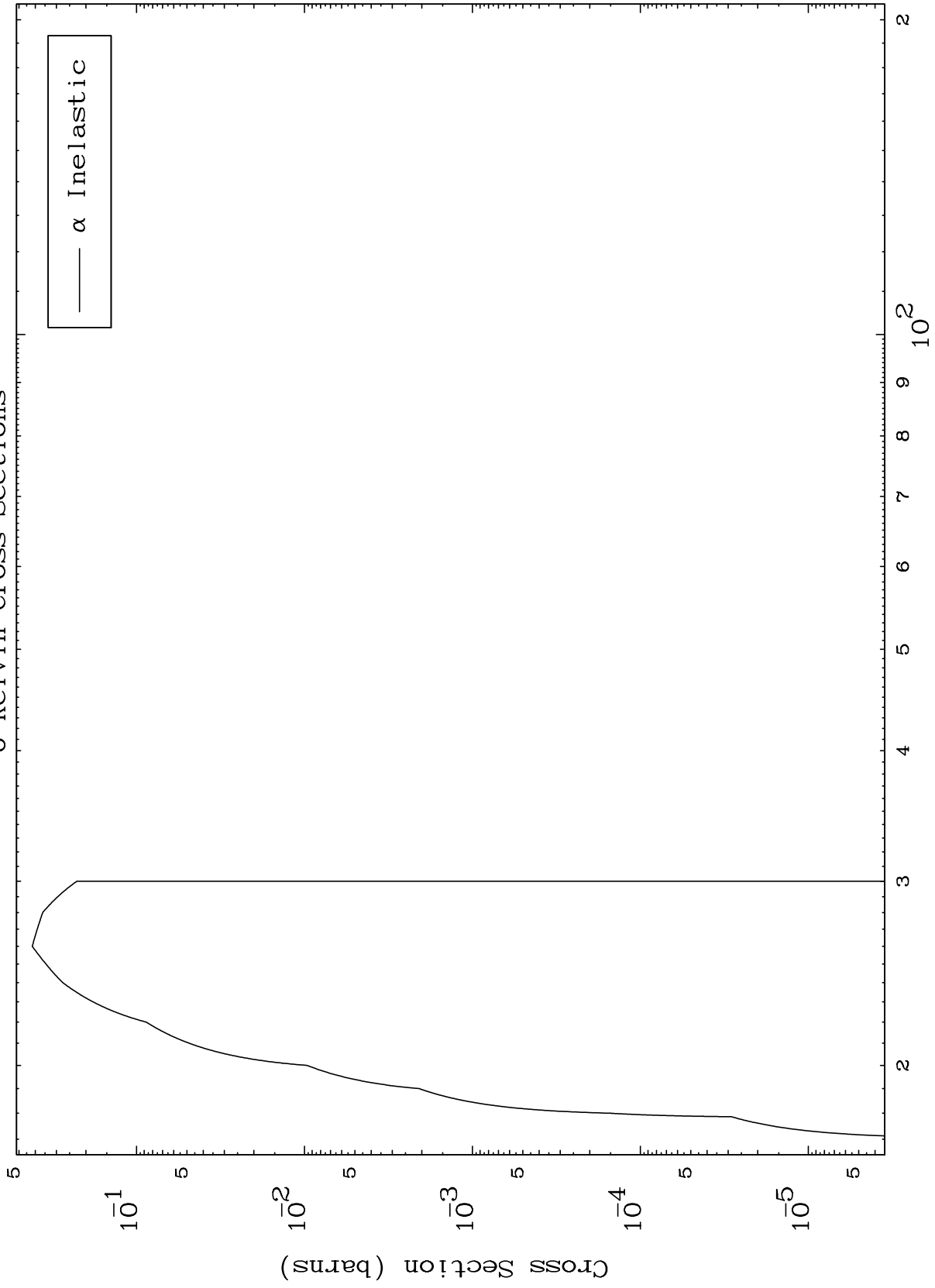
87-Fr-209



MAT 8716

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

87-Fr-209



5

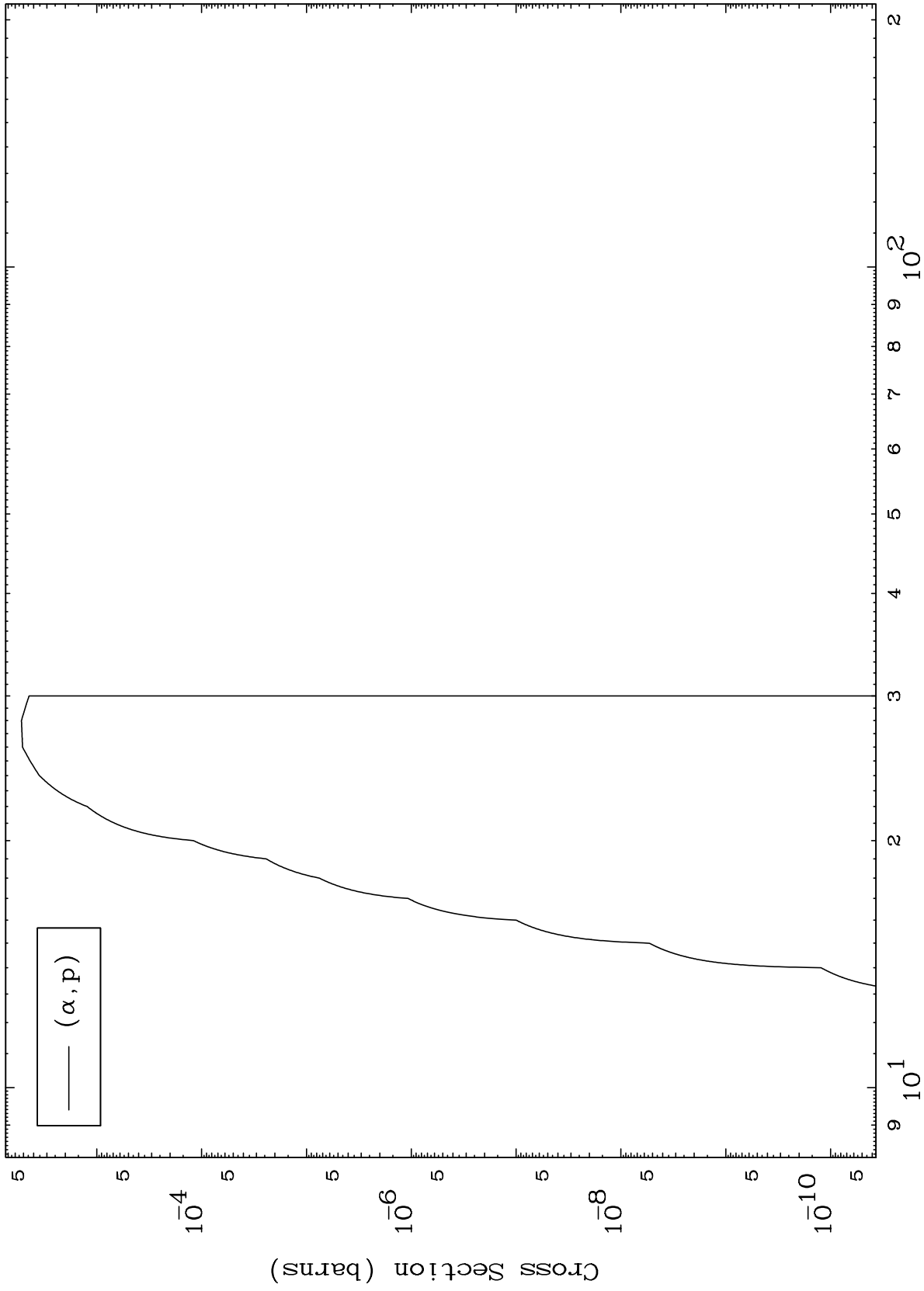
Incident Energy (MeV)

87-Fr-209

MAT 8716

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

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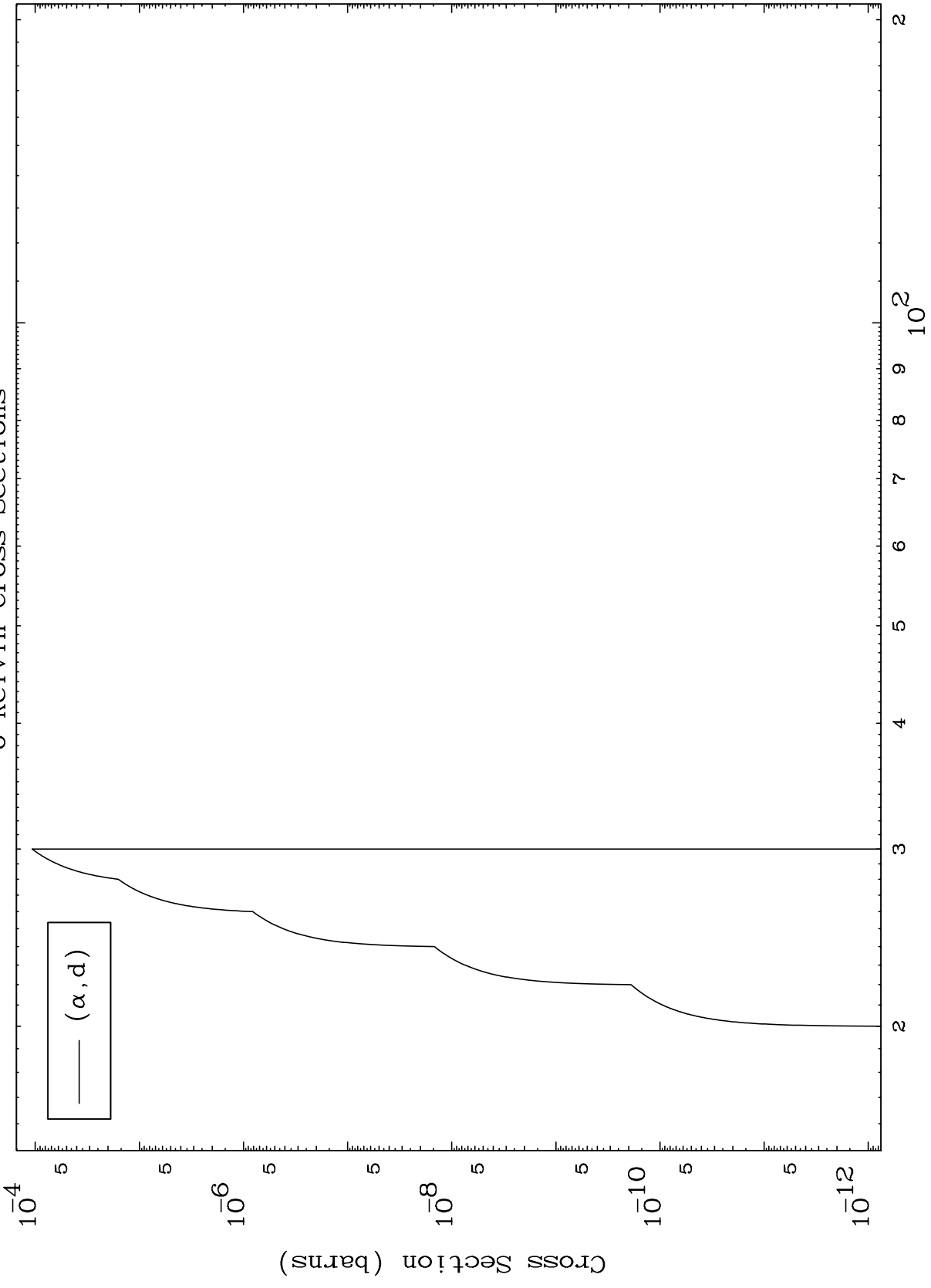
Incident Energy (MeV)

87-Fr-209

MAT 8716

( $\alpha$ ,d) Levels  
0 Kelvin Cross Sections

87-Fr-209



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Incident Energy (MeV)

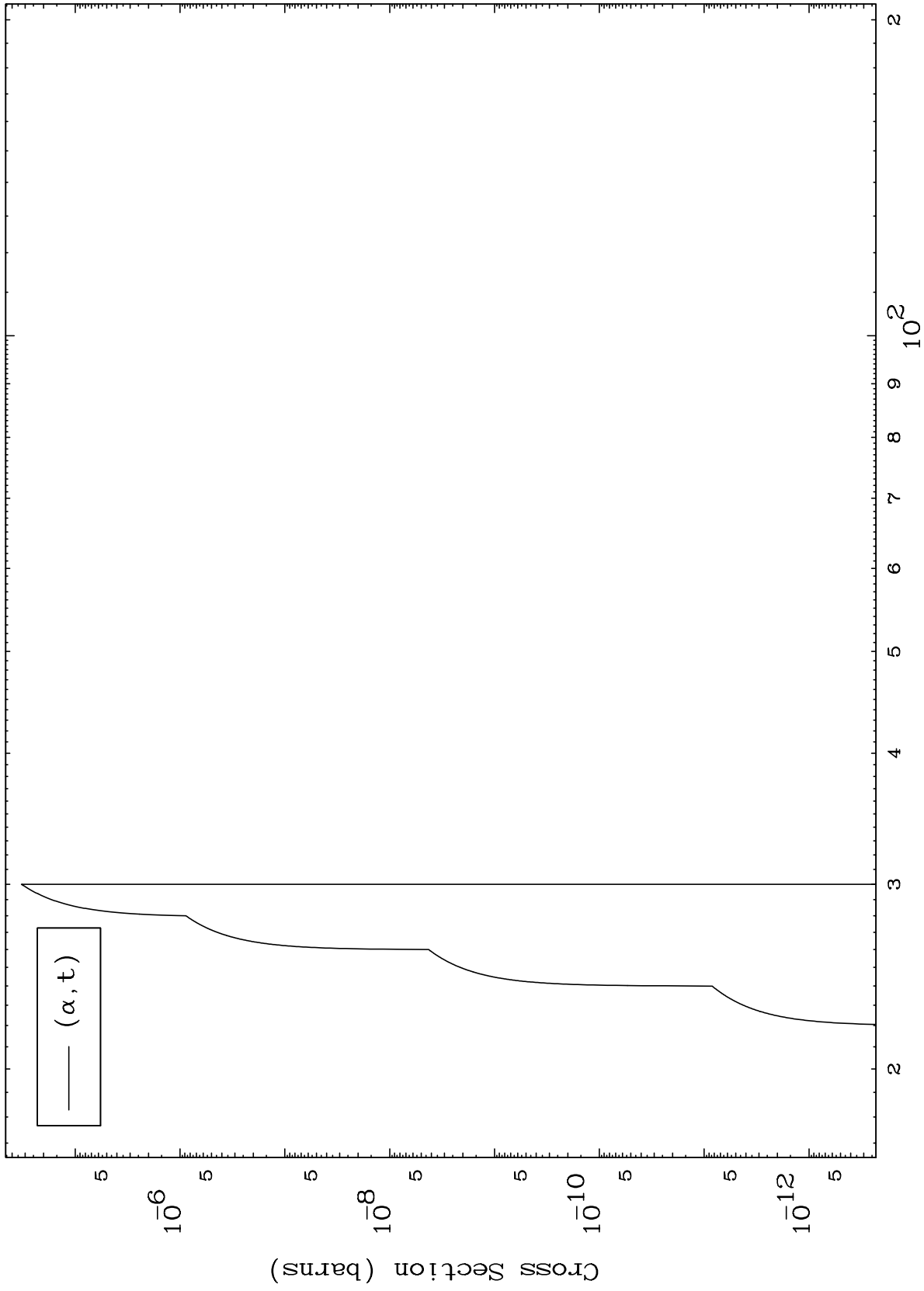
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MAT 8716

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

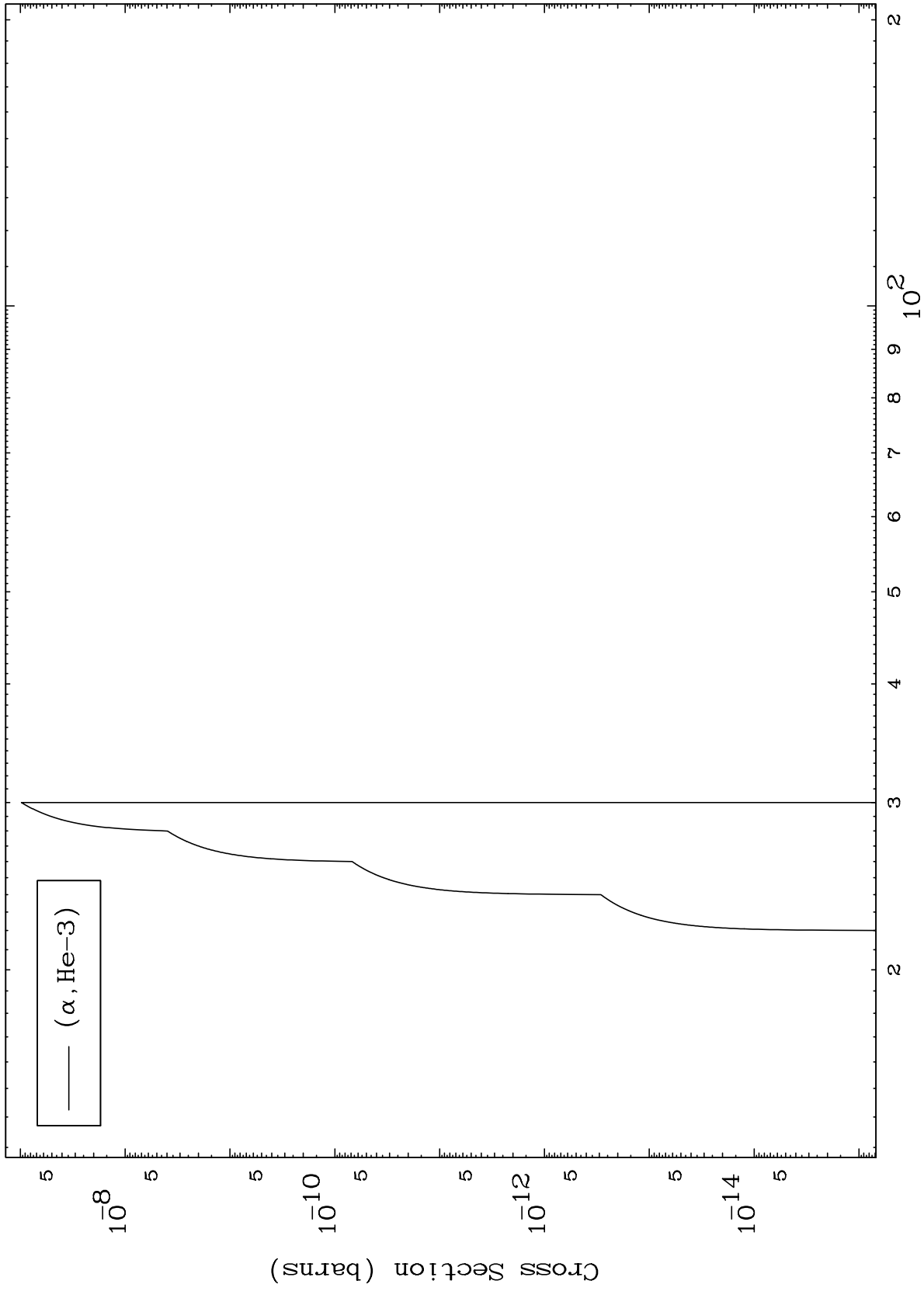
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Incident Energy (MeV)

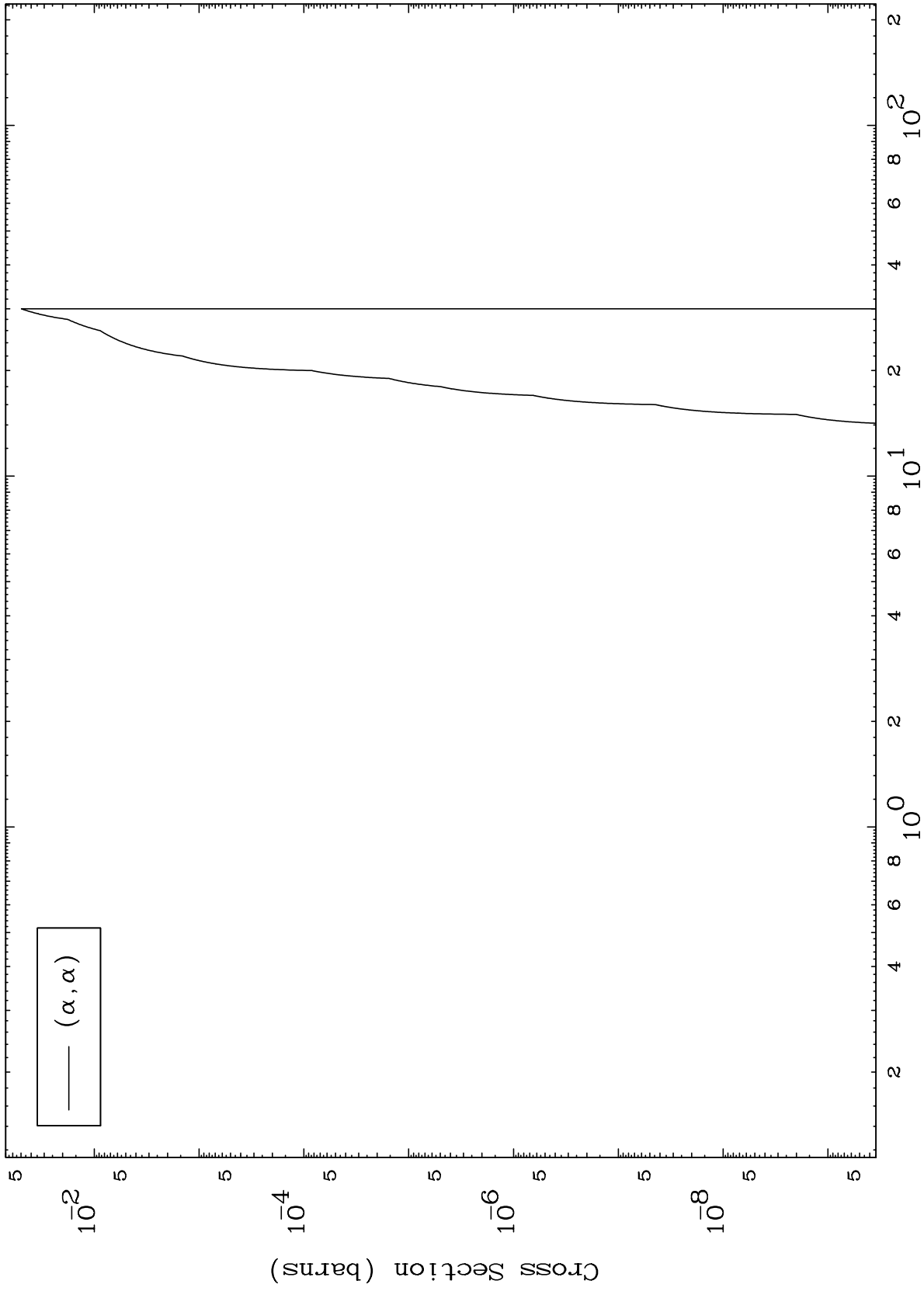
87-Fr-209



MAT 8716

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

87-Fr-209



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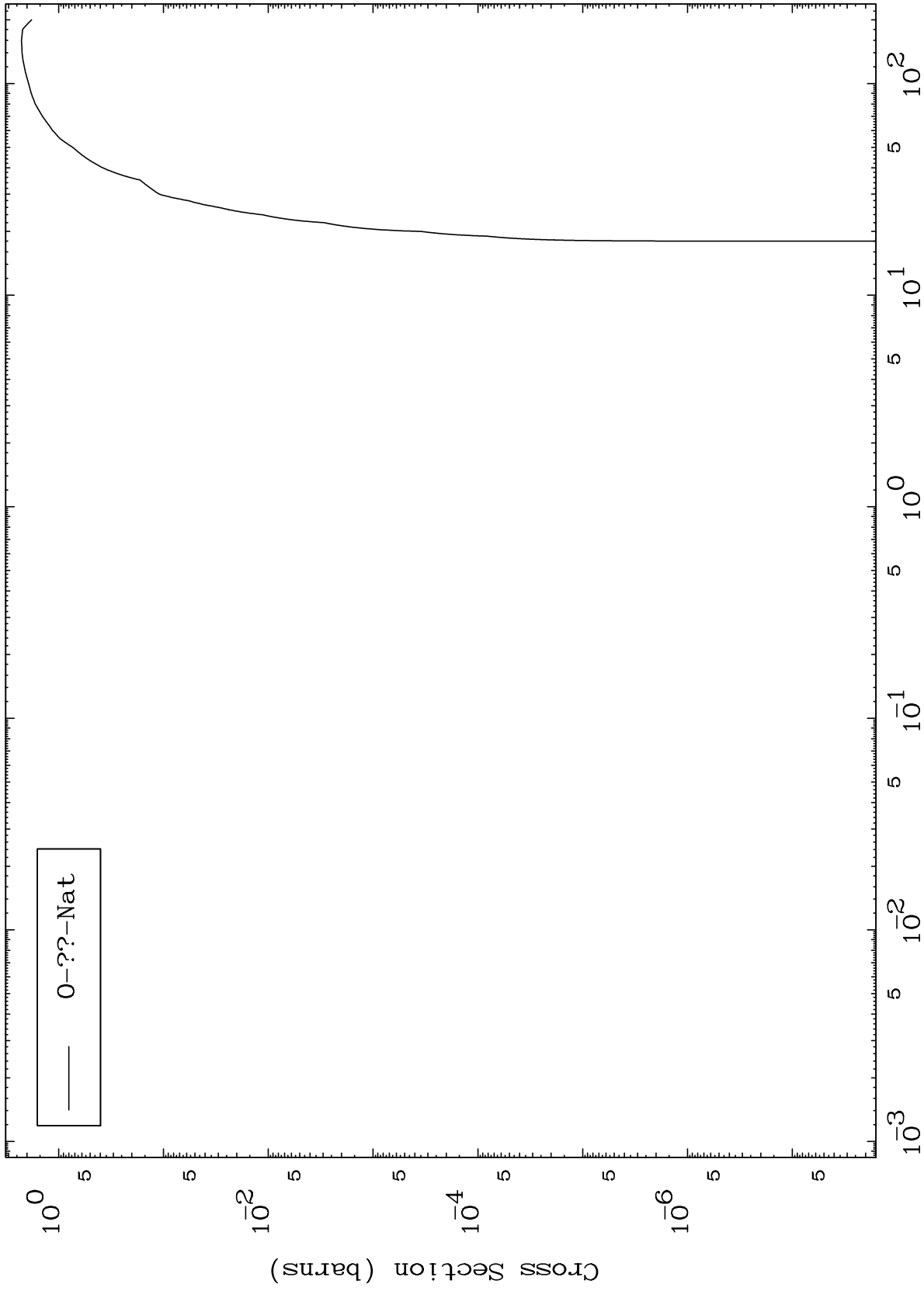
Incident Energy (MeV)

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MAT 8716

$\alpha$  Fission  
Radionuclide Production Cross Section

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Incident Energy (MeV)

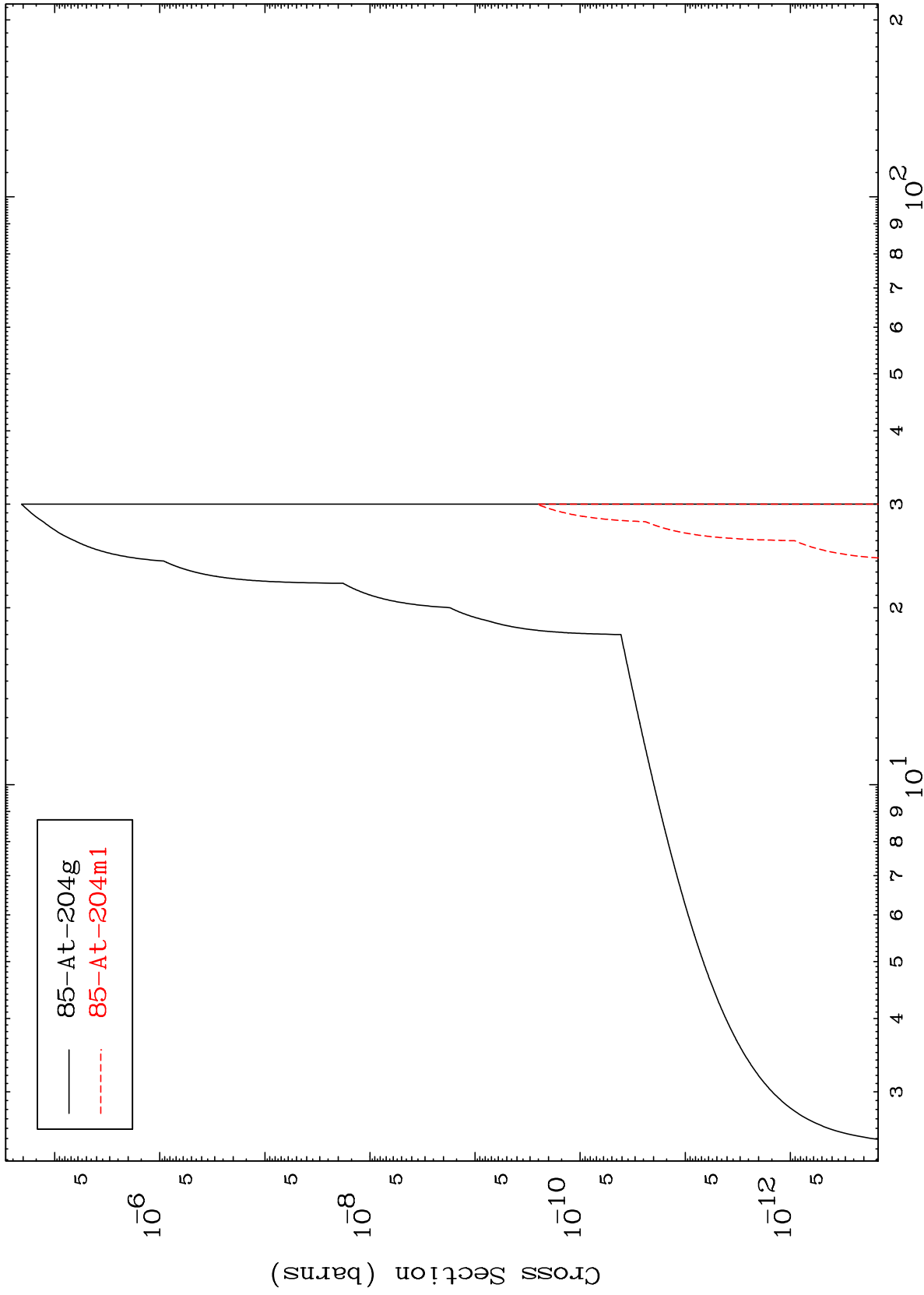
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MAT 8716

( $\alpha, n'$ )  $2\alpha$

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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