

Program EVALPLOT  
(Version 2018-1)

by

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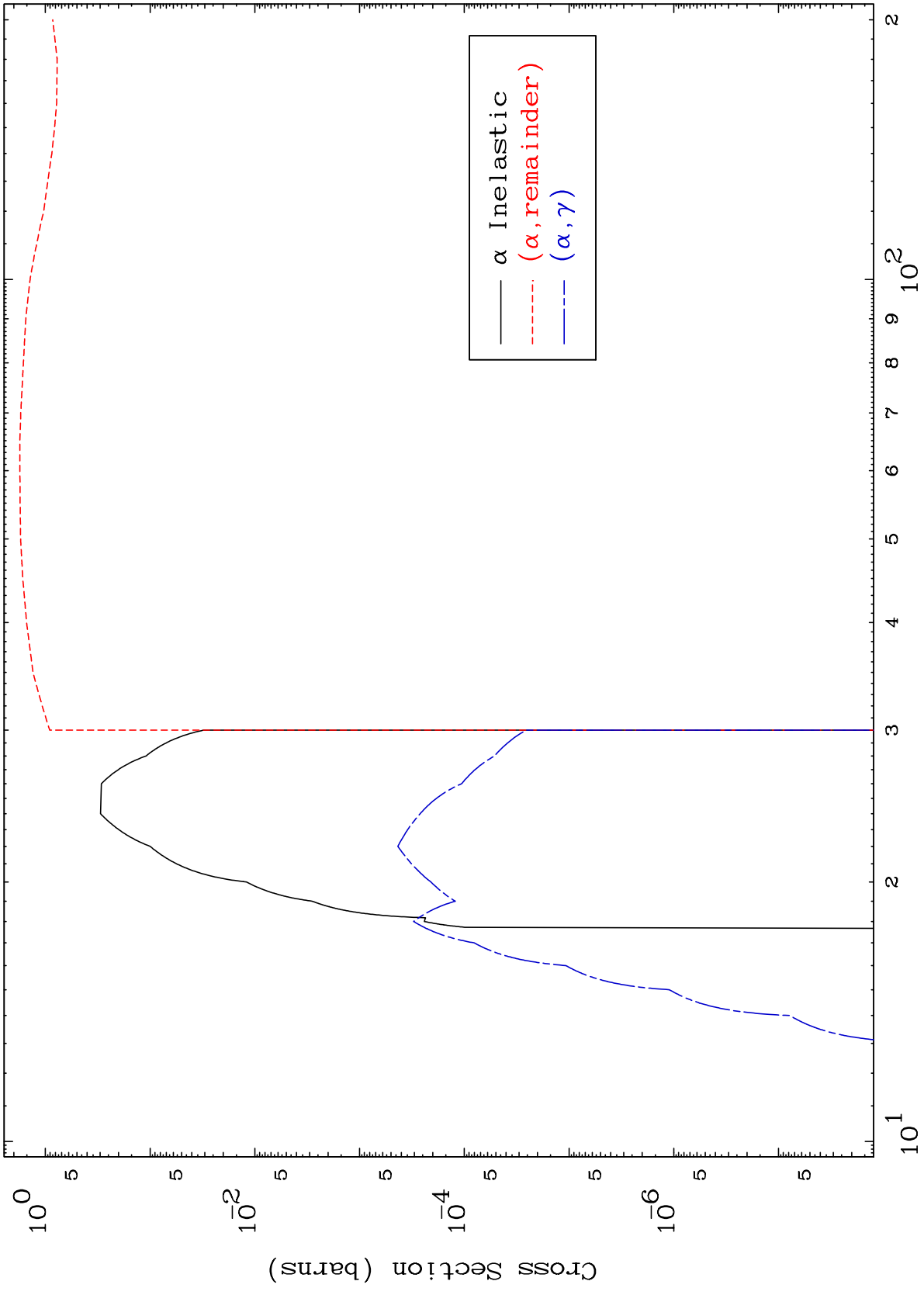
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8728

$\alpha$  Major  
0 Kelvin Cross Sections

87-Fr-213



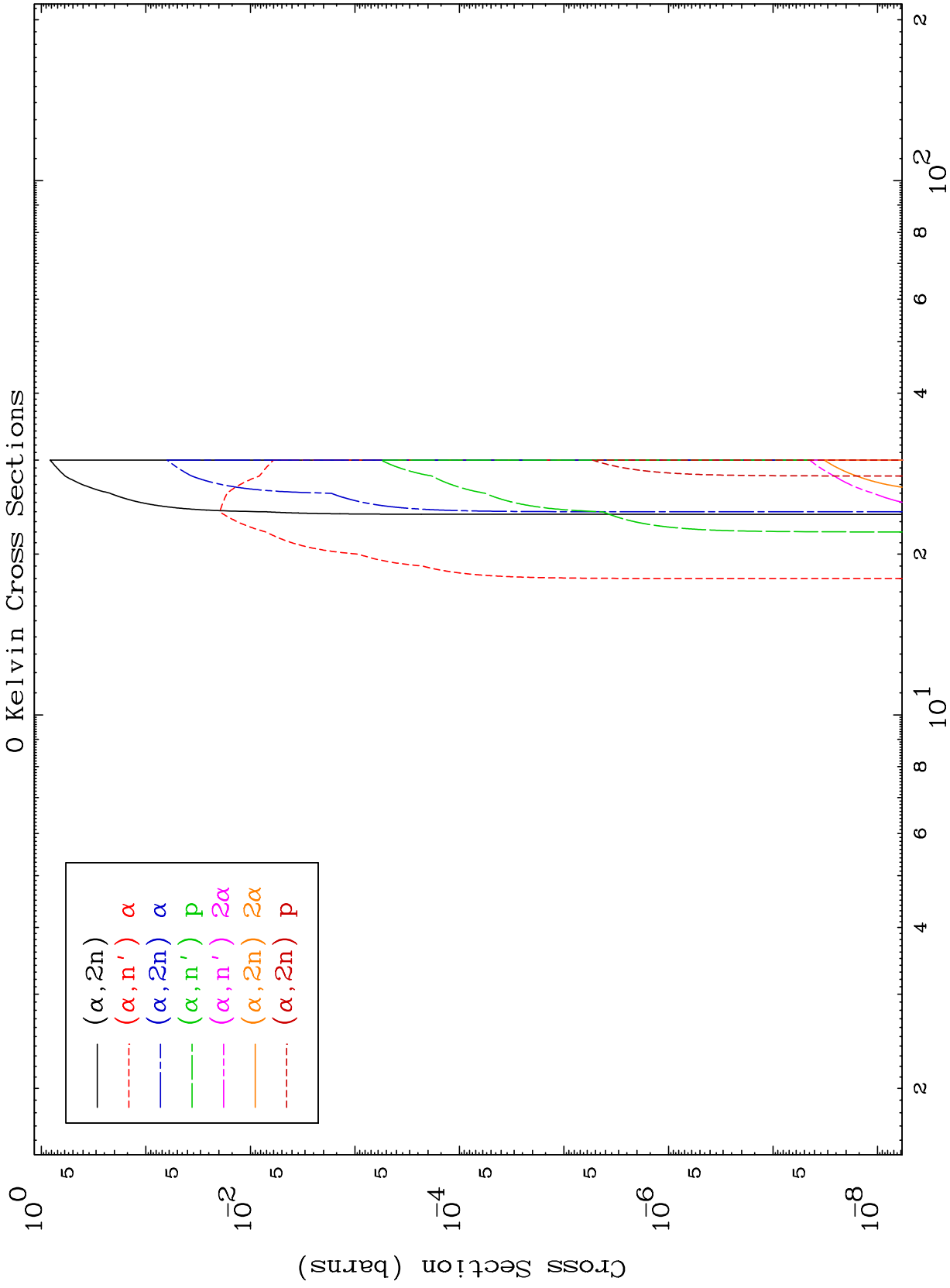
Incident Energy (MeV)

87-Fr-213

MAT 8728

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

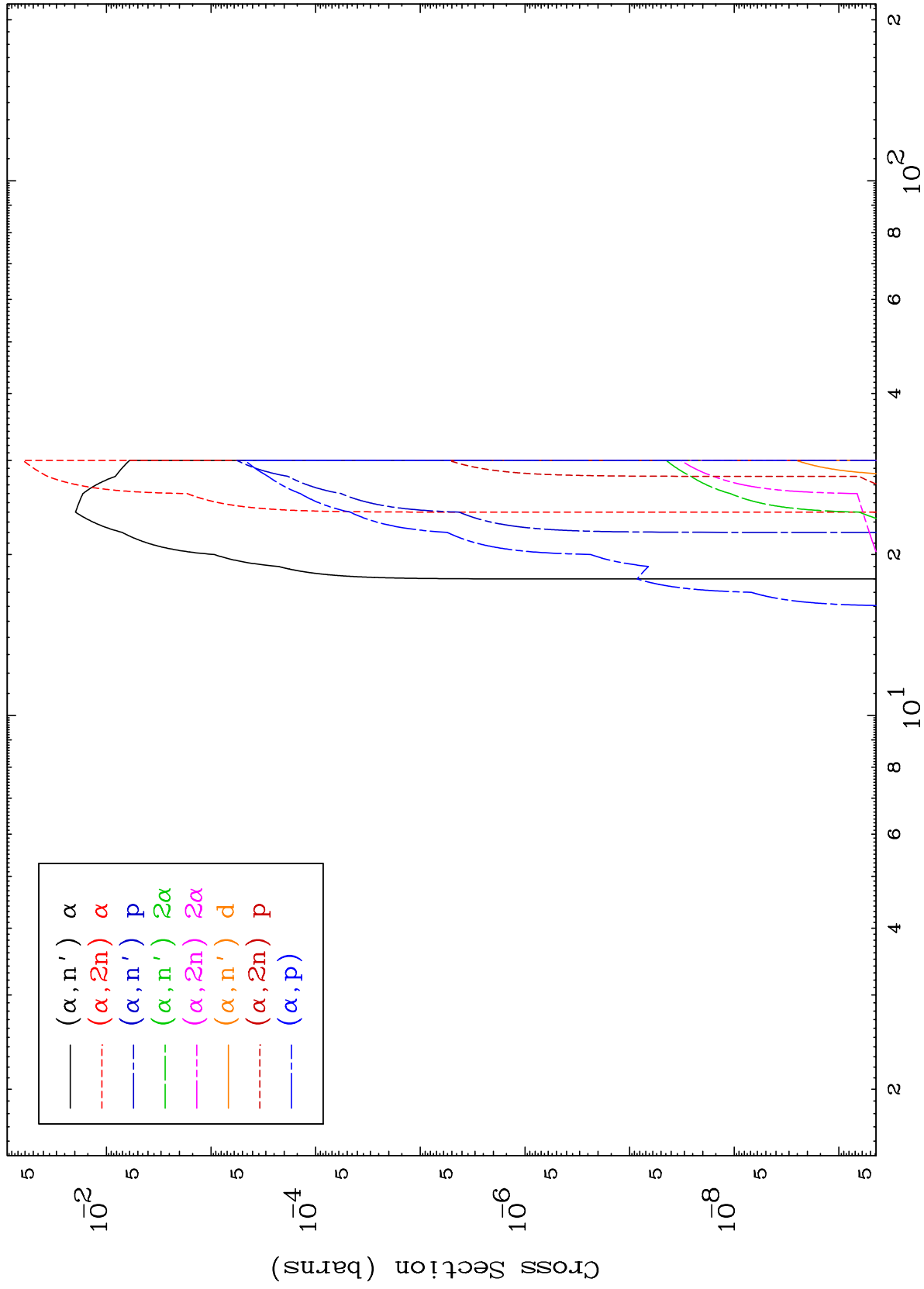
87-Fr-213



2

Incident Energy (MeV)

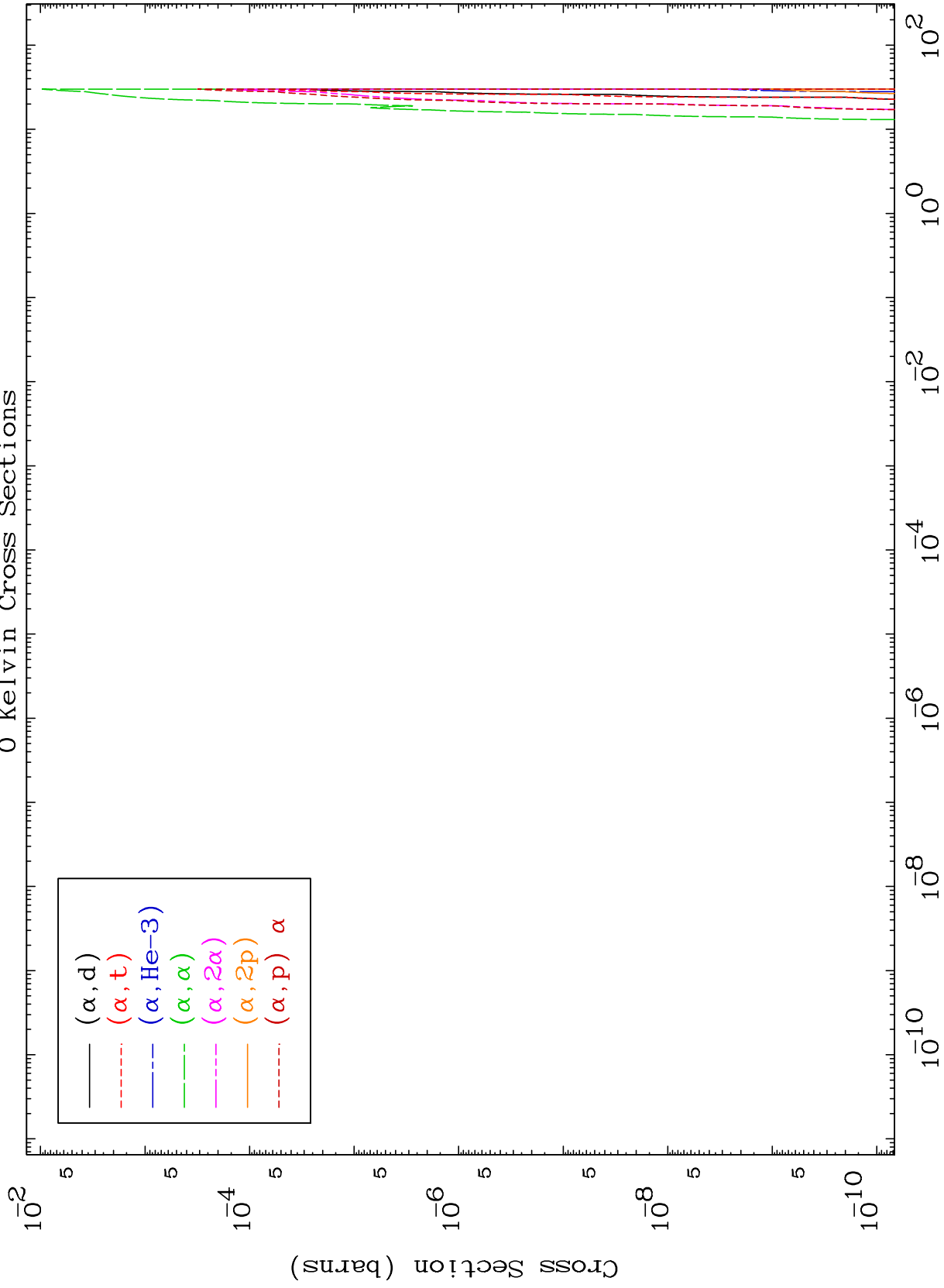
87-Fr-213



MAT 8728

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

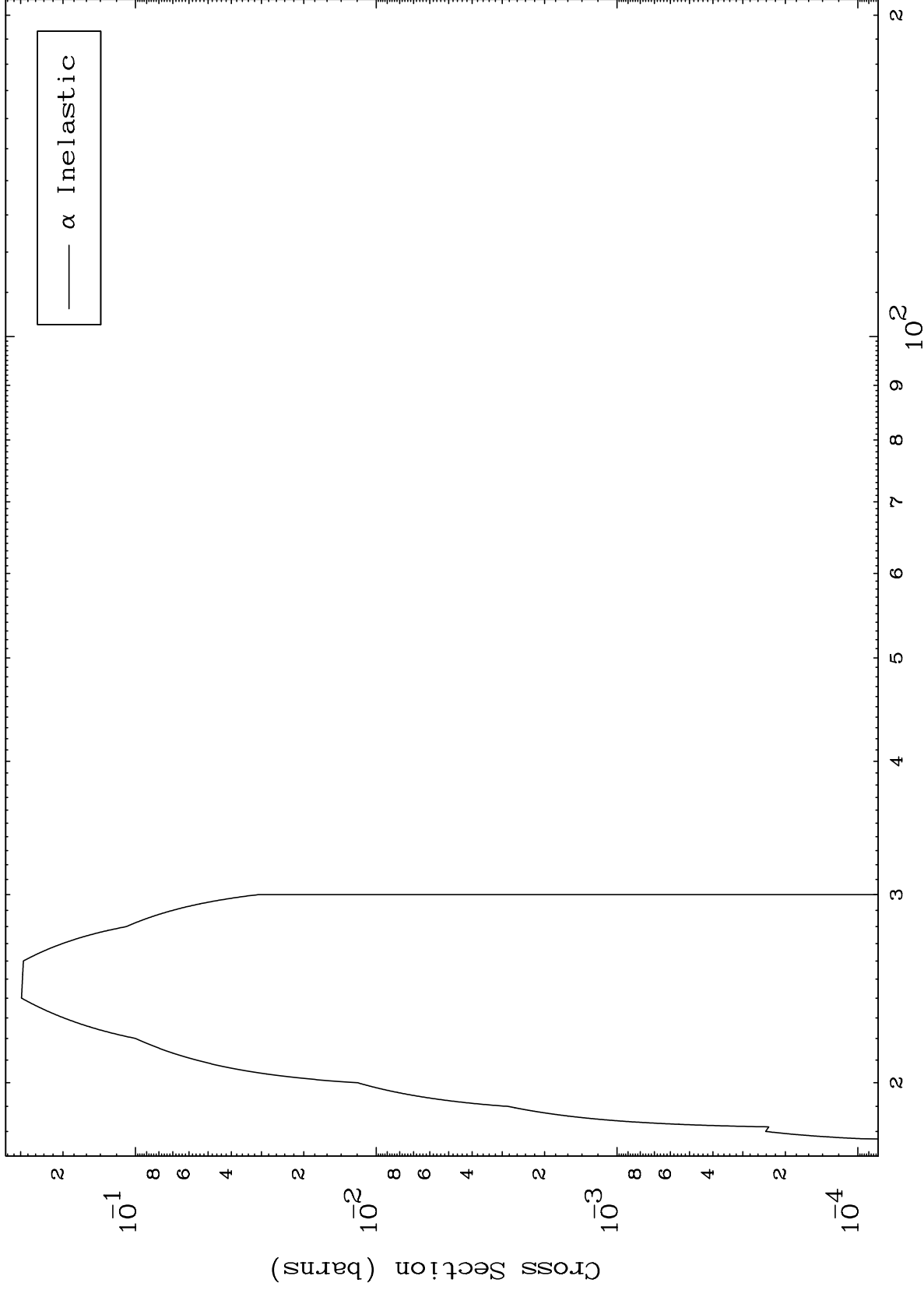
87-Fr-213



MAT 8728

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

87-Fr-213



5

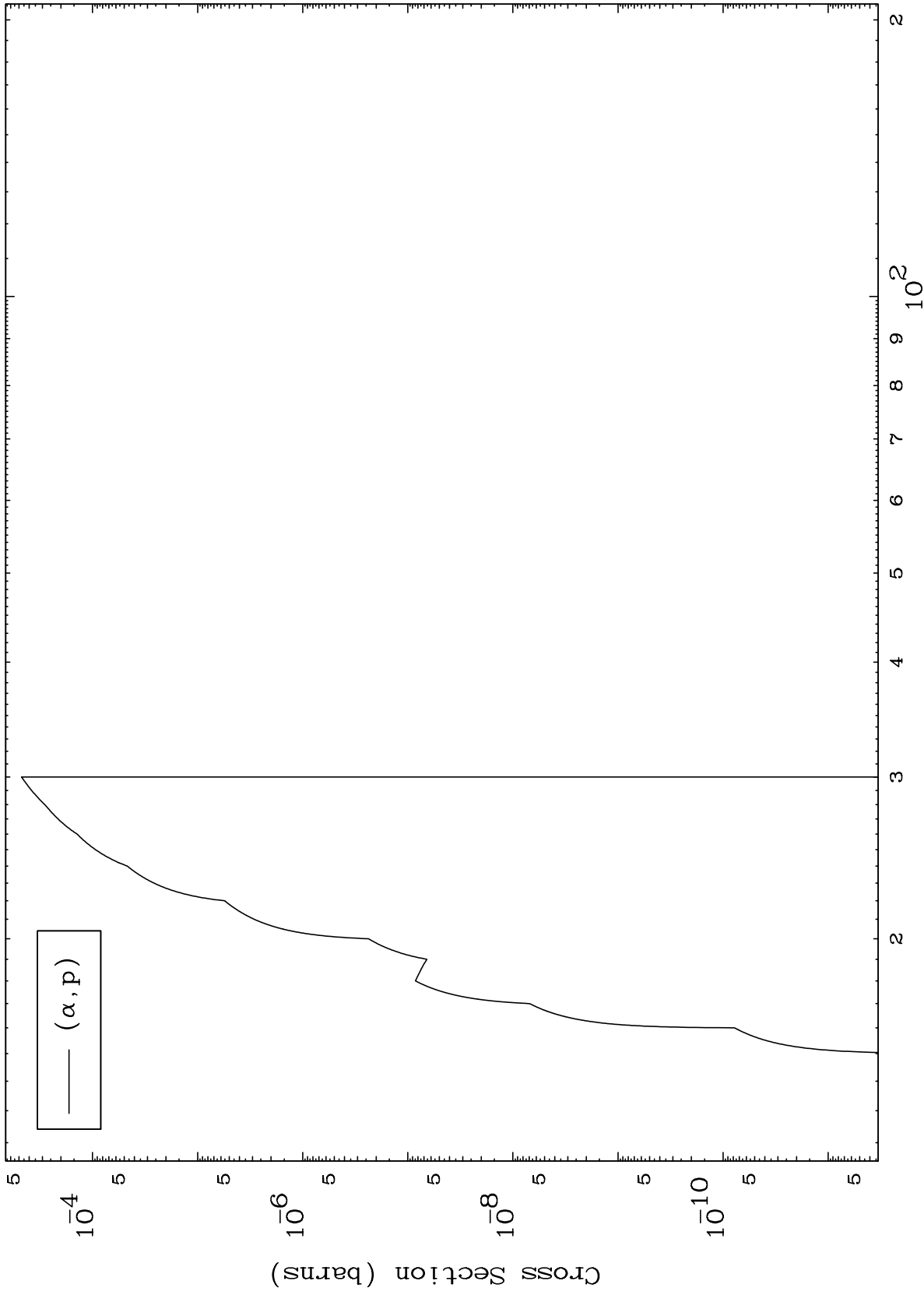
Incident Energy (MeV)

87-Fr-213

MAT 8728

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

87-Fr-213



6

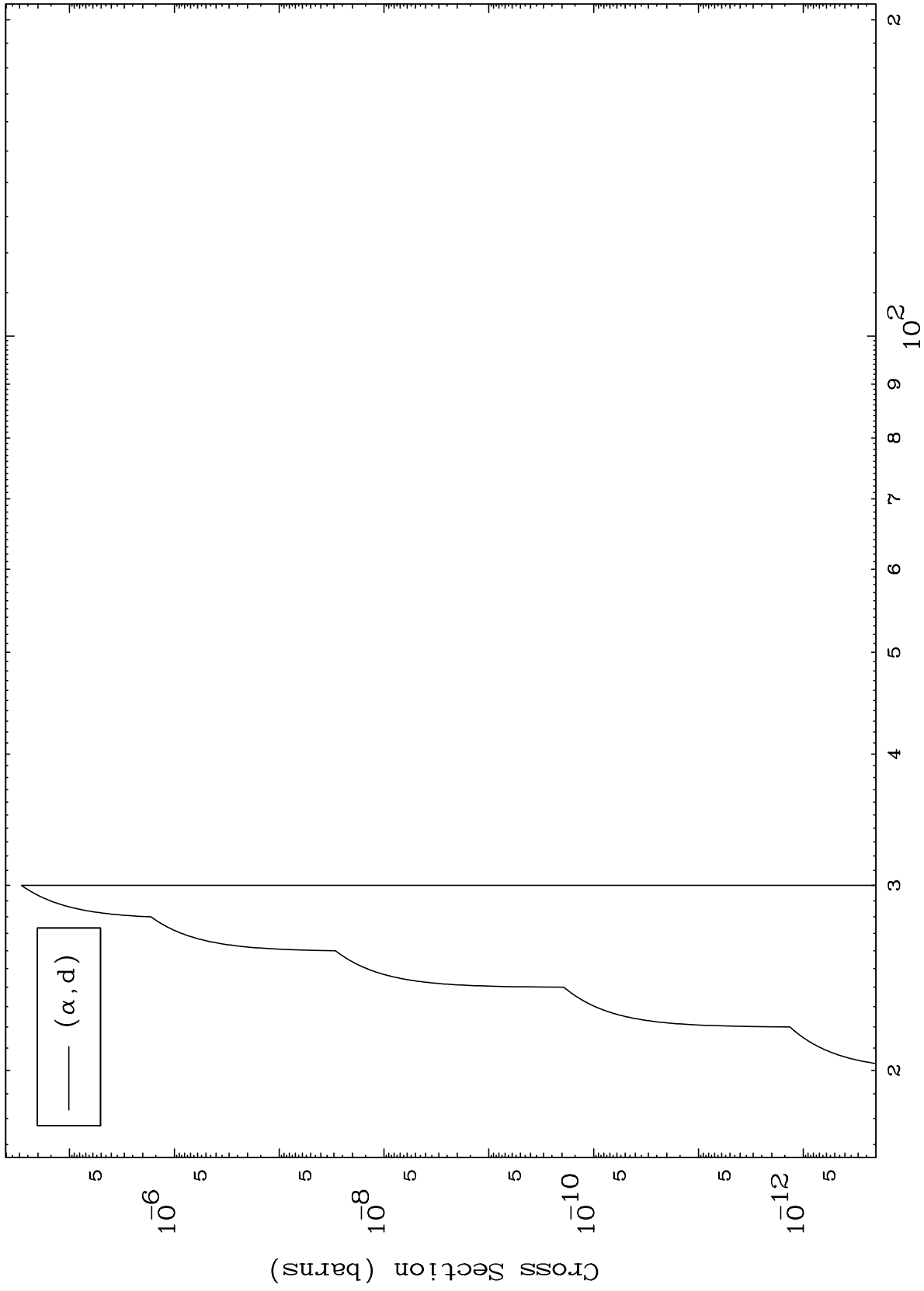
Incident Energy (MeV)

87-Fr-213

MAT 8728

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

87-Fr-213



7

Incident Energy (MeV)

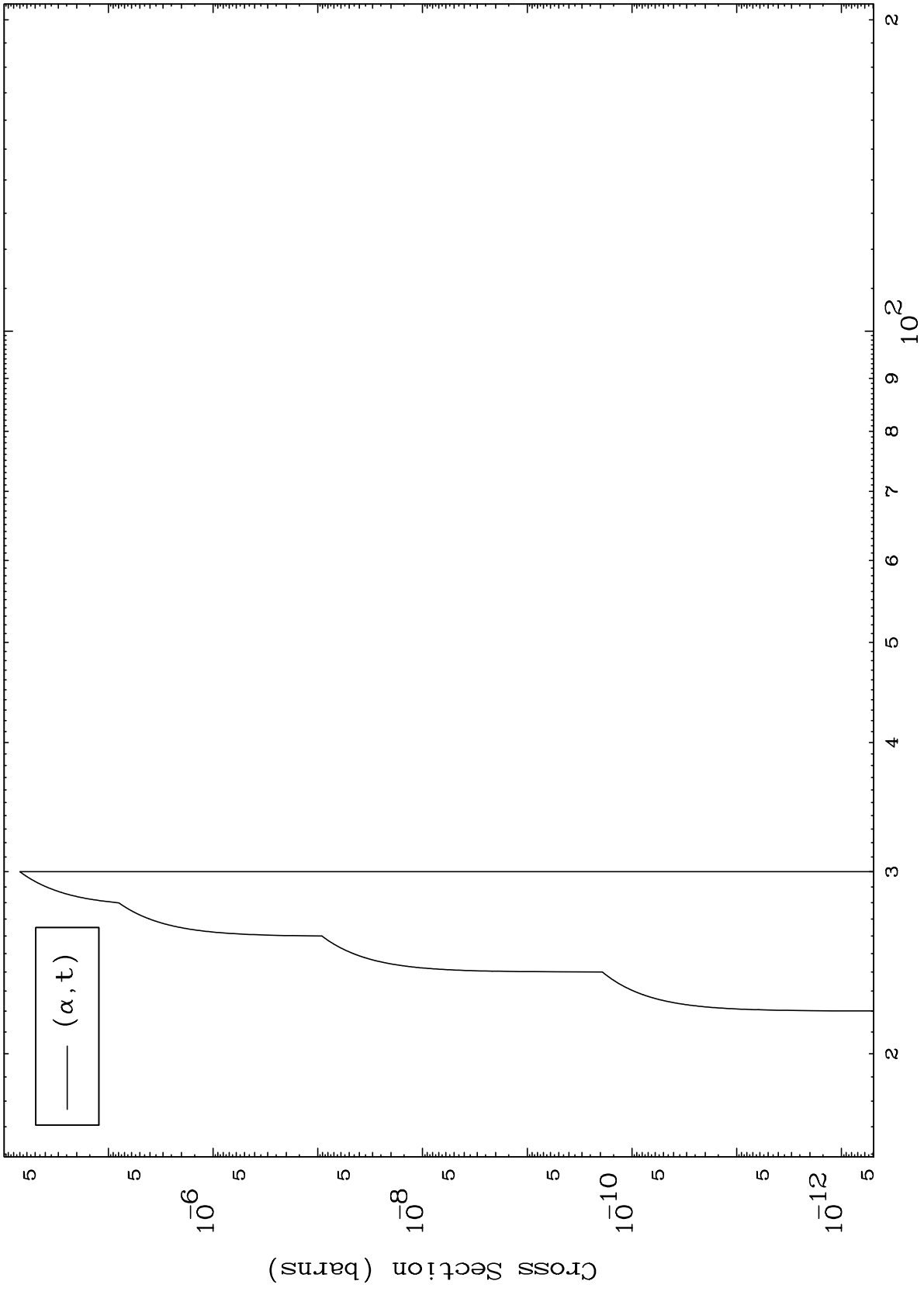
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MAT 8728

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

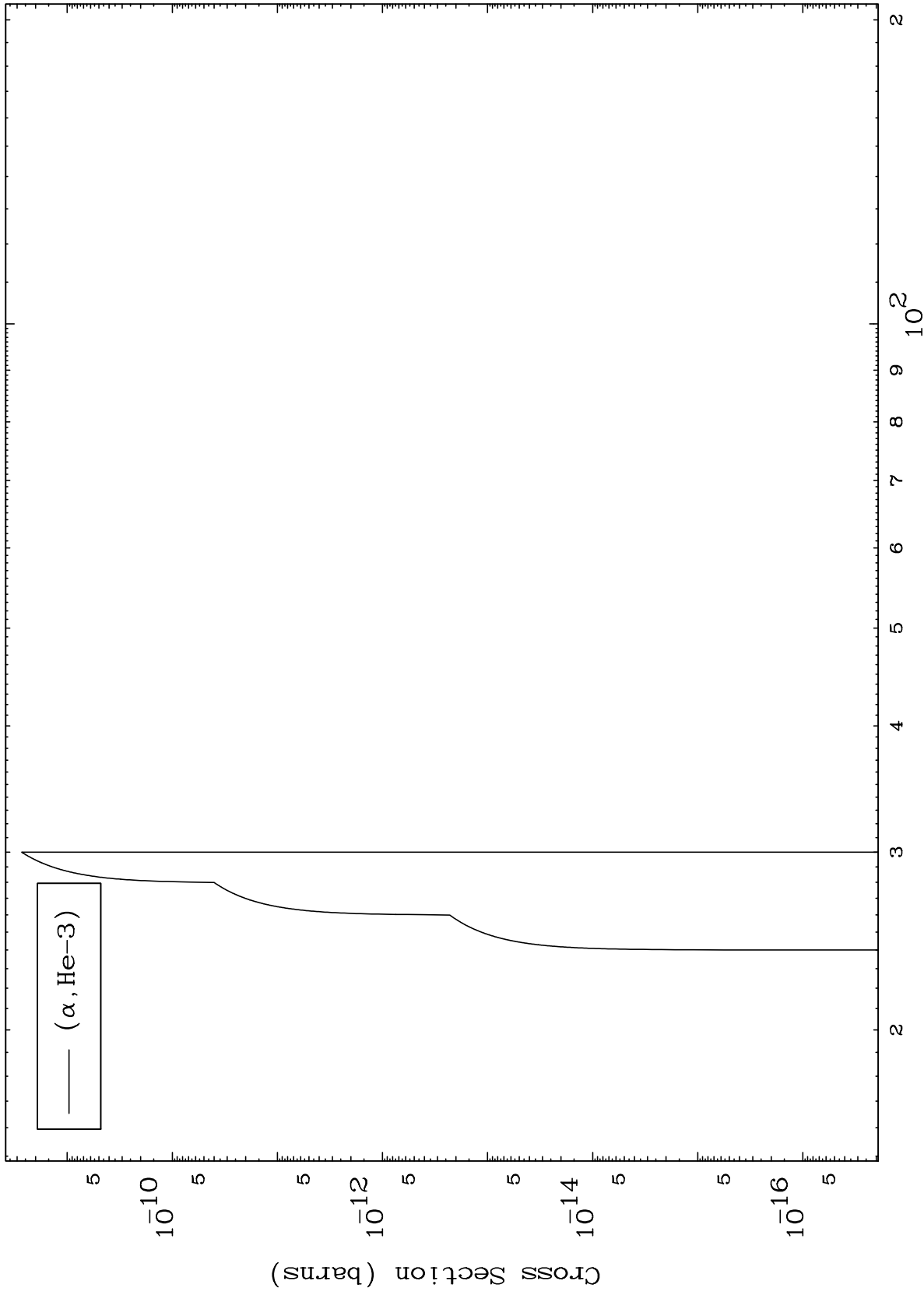
87-Fr-213



8

Incident Energy (MeV)

87-Fr-213

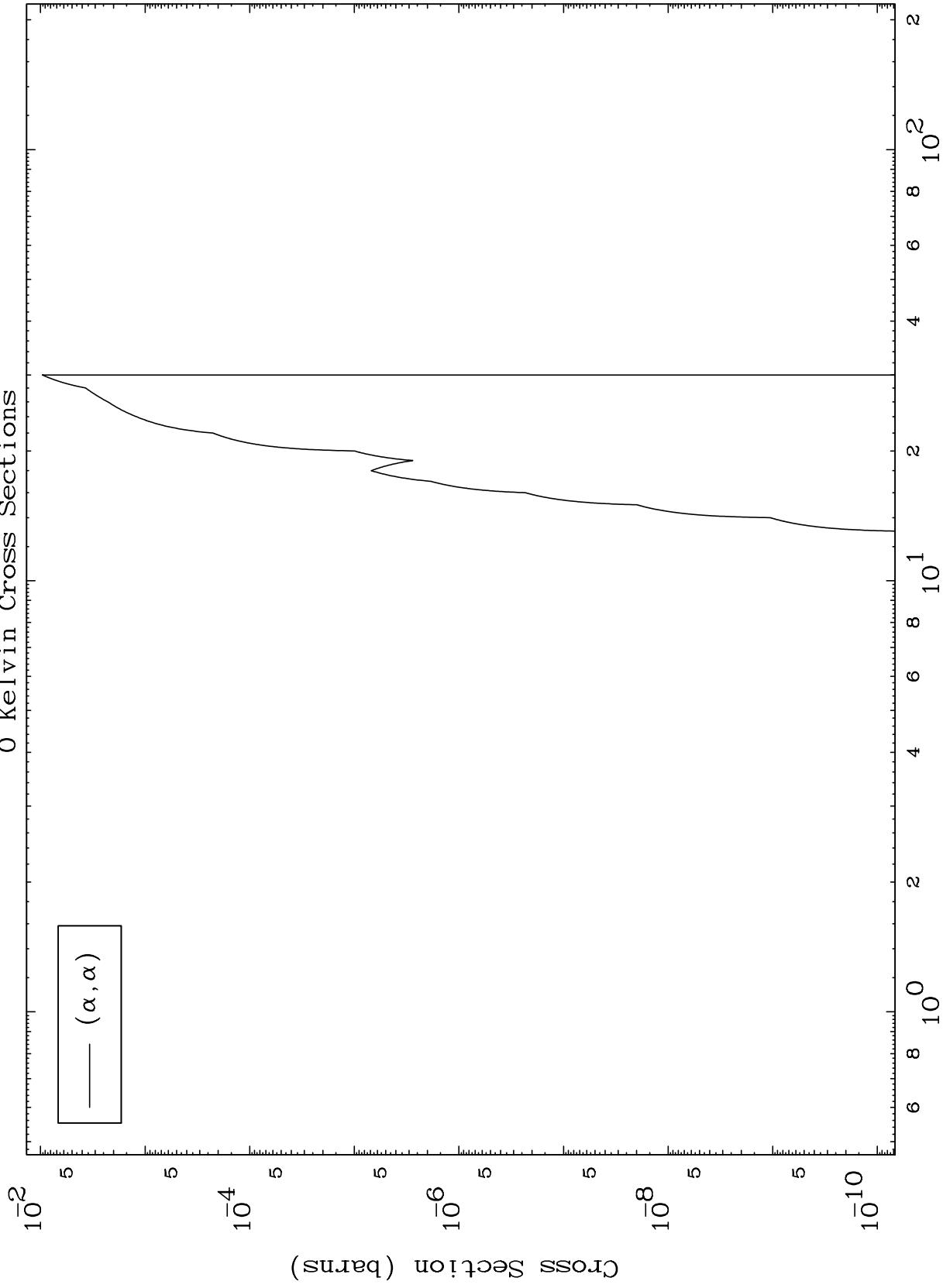


MAT 8728

( $\alpha, \alpha$ ) Levels

87-Fr-213

0 Kelvin Cross Sections



10

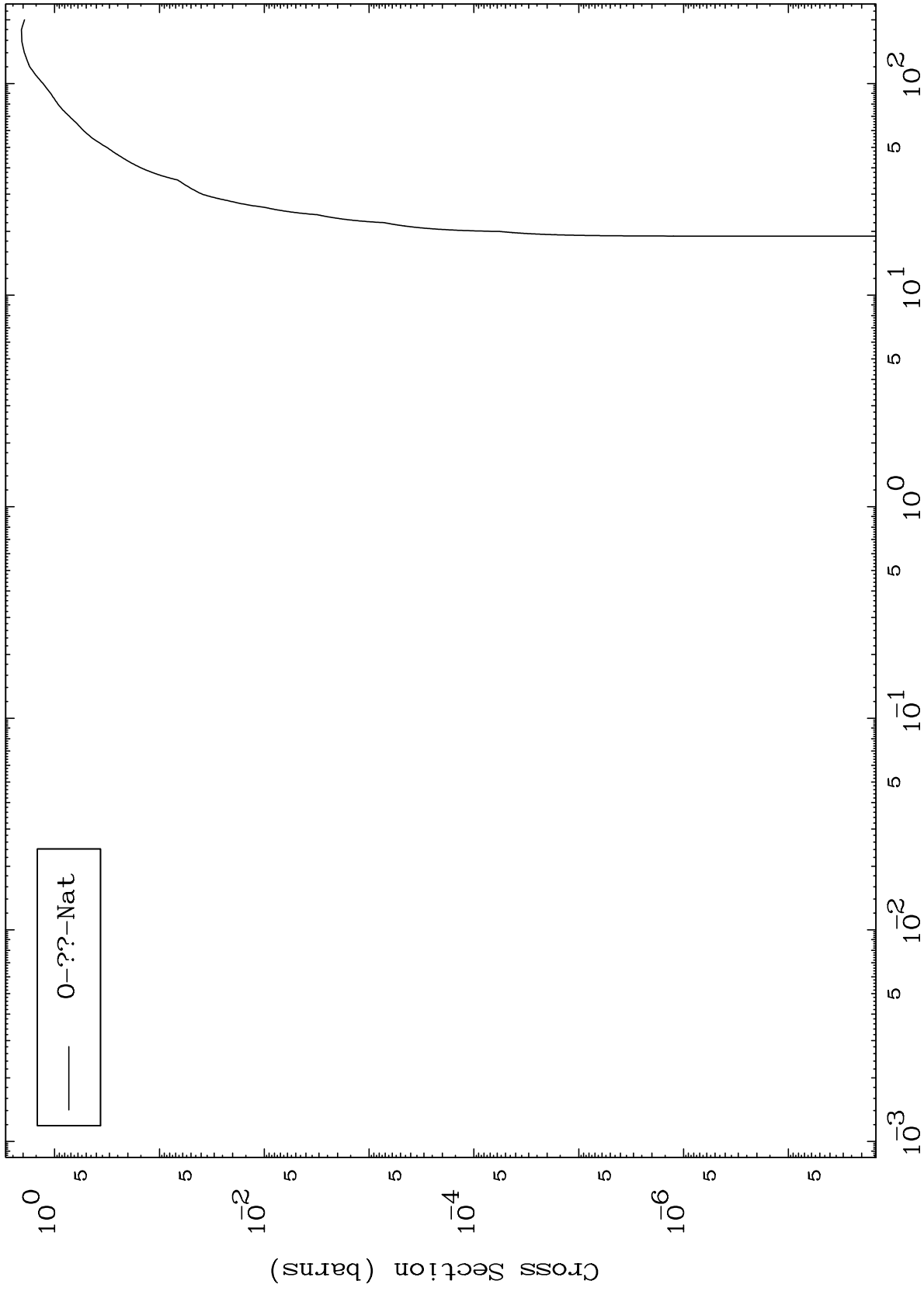
Incident Energy (MeV)

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MAT 8728

$\alpha$  Fission  
Radionuclide Production Cross Section

87-Fr-213



11

Incident Energy (MeV)

87-Fr-213