

Program EVALPLOT  
(Version 2018-1)

by

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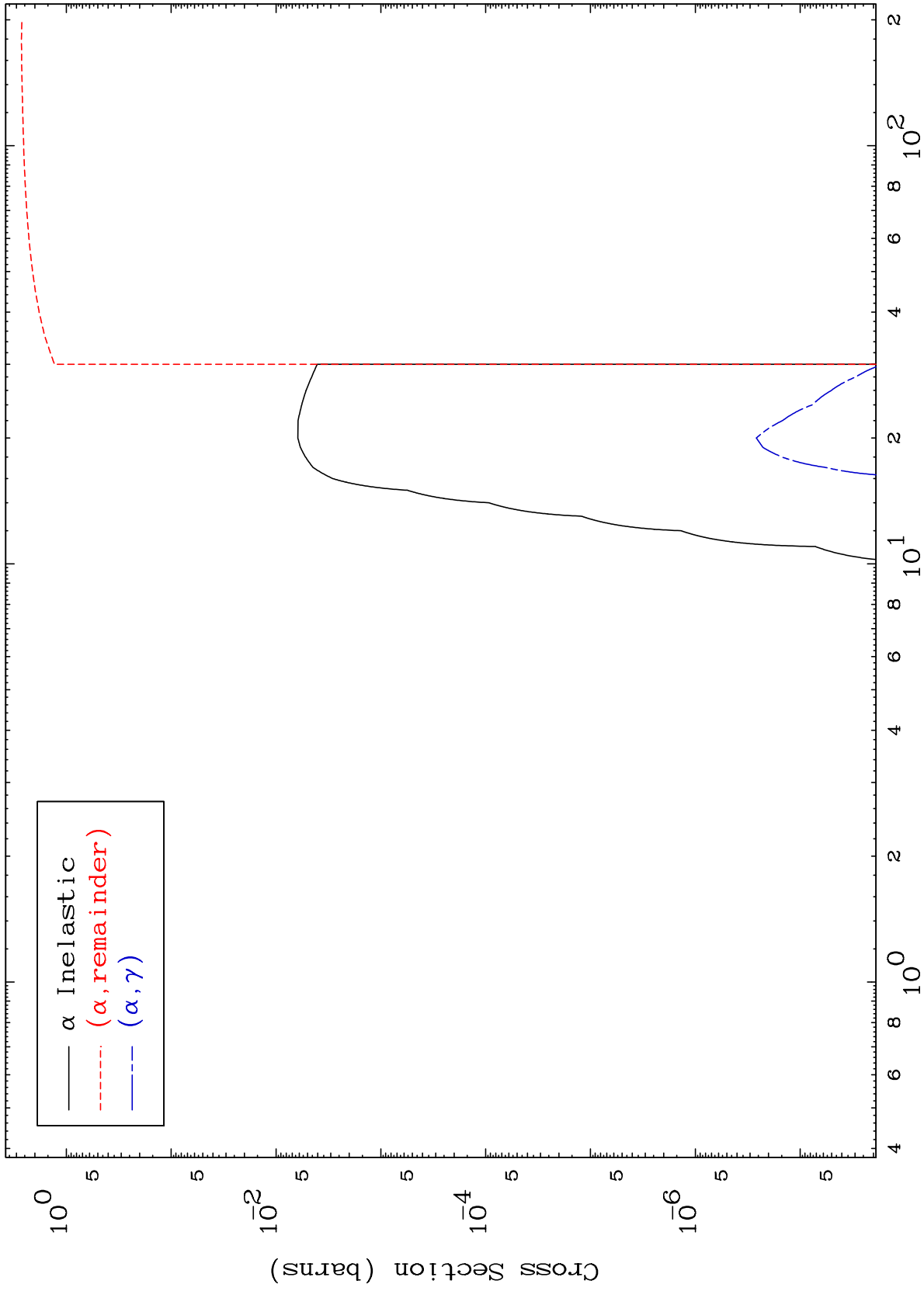
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7255

0 Kelvin  $\alpha$  Major  
Cross Sections

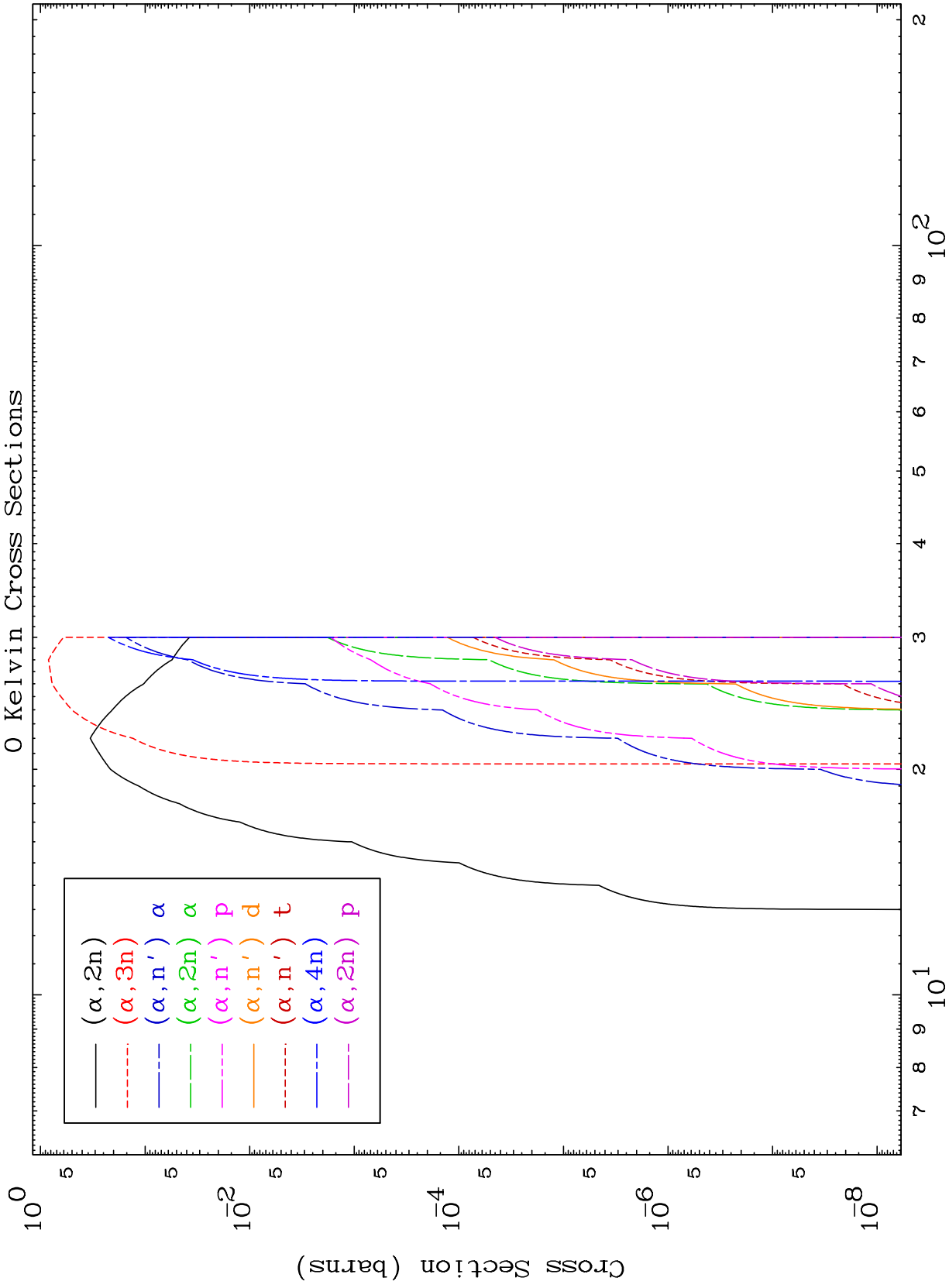
72-Hf-184



MAT 7255

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

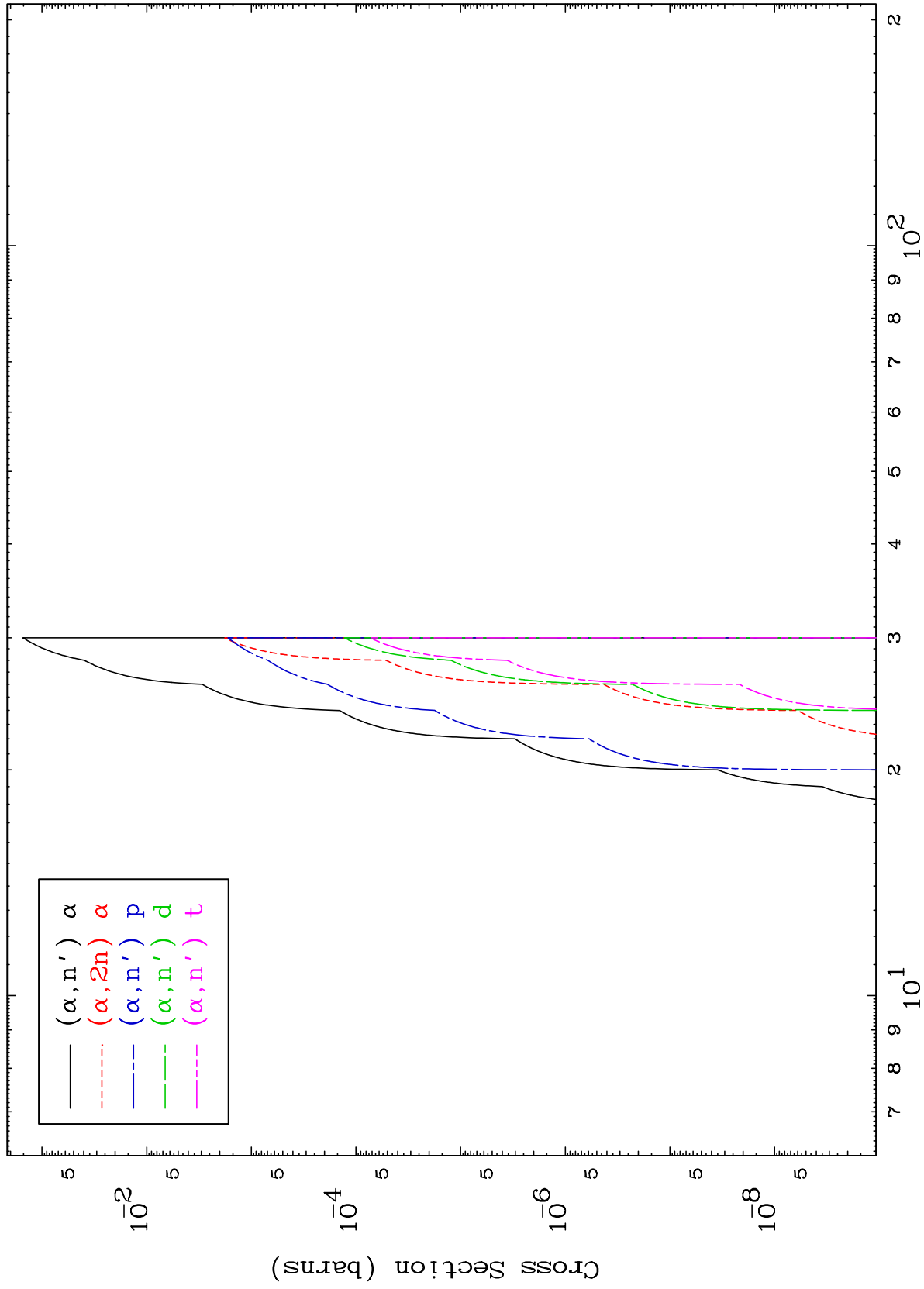
72-Hf-184



2

Incident Energy (MeV)

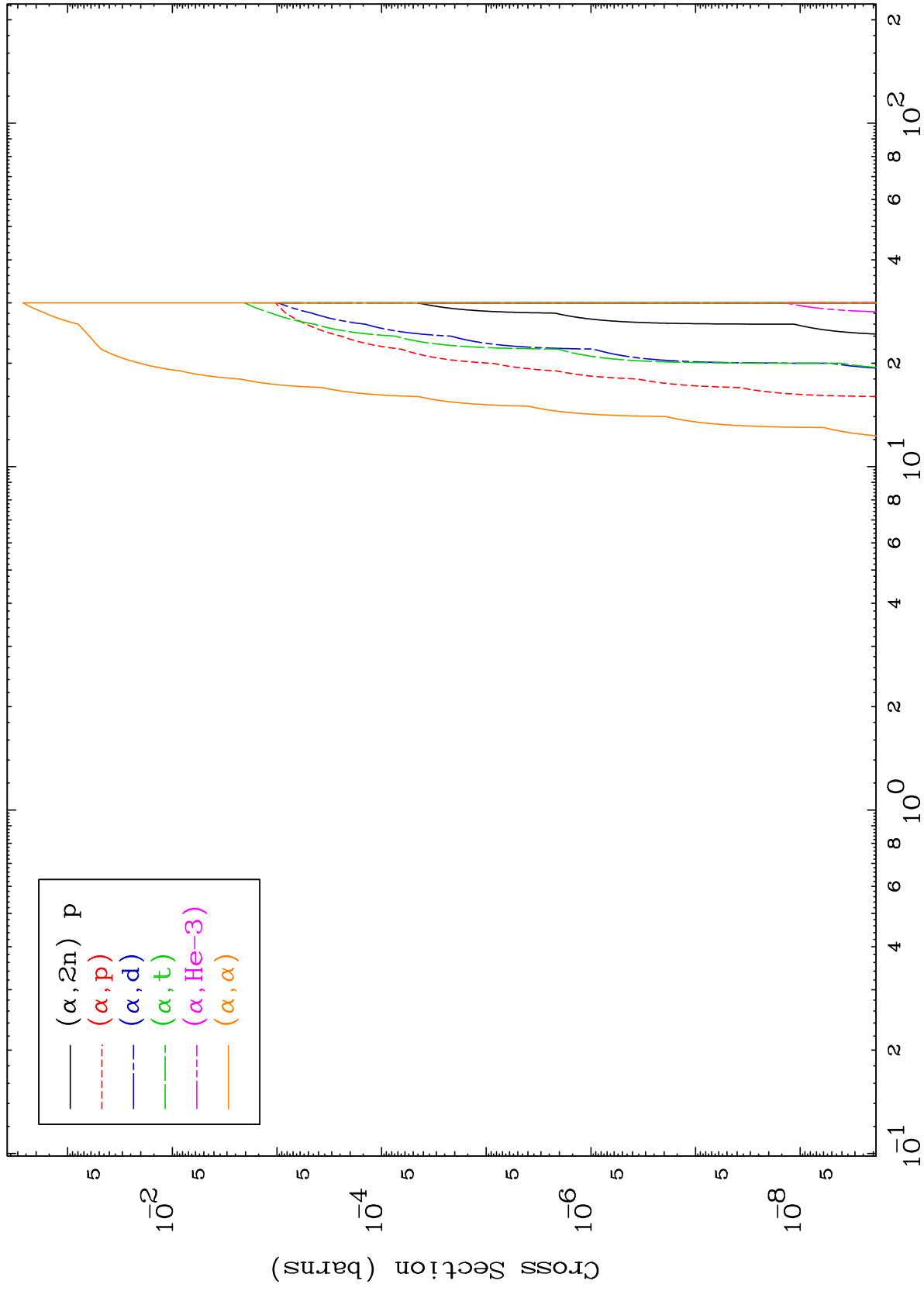
72-Hf-184



MAT 7255

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

72-Hf-184



4

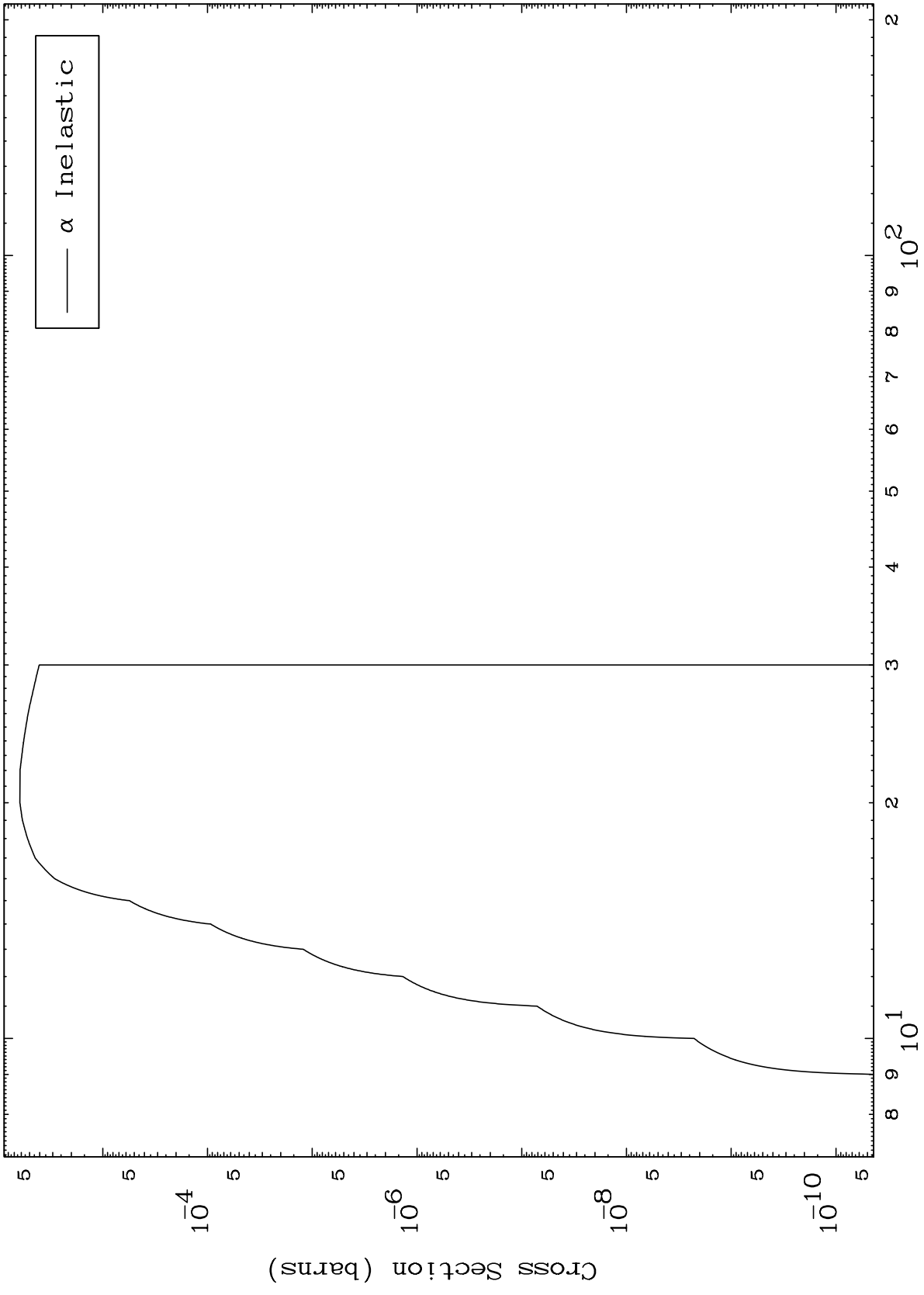
Incident Energy (MeV)

72-Hf-184

MAT 7255

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

72-Hf-184



5

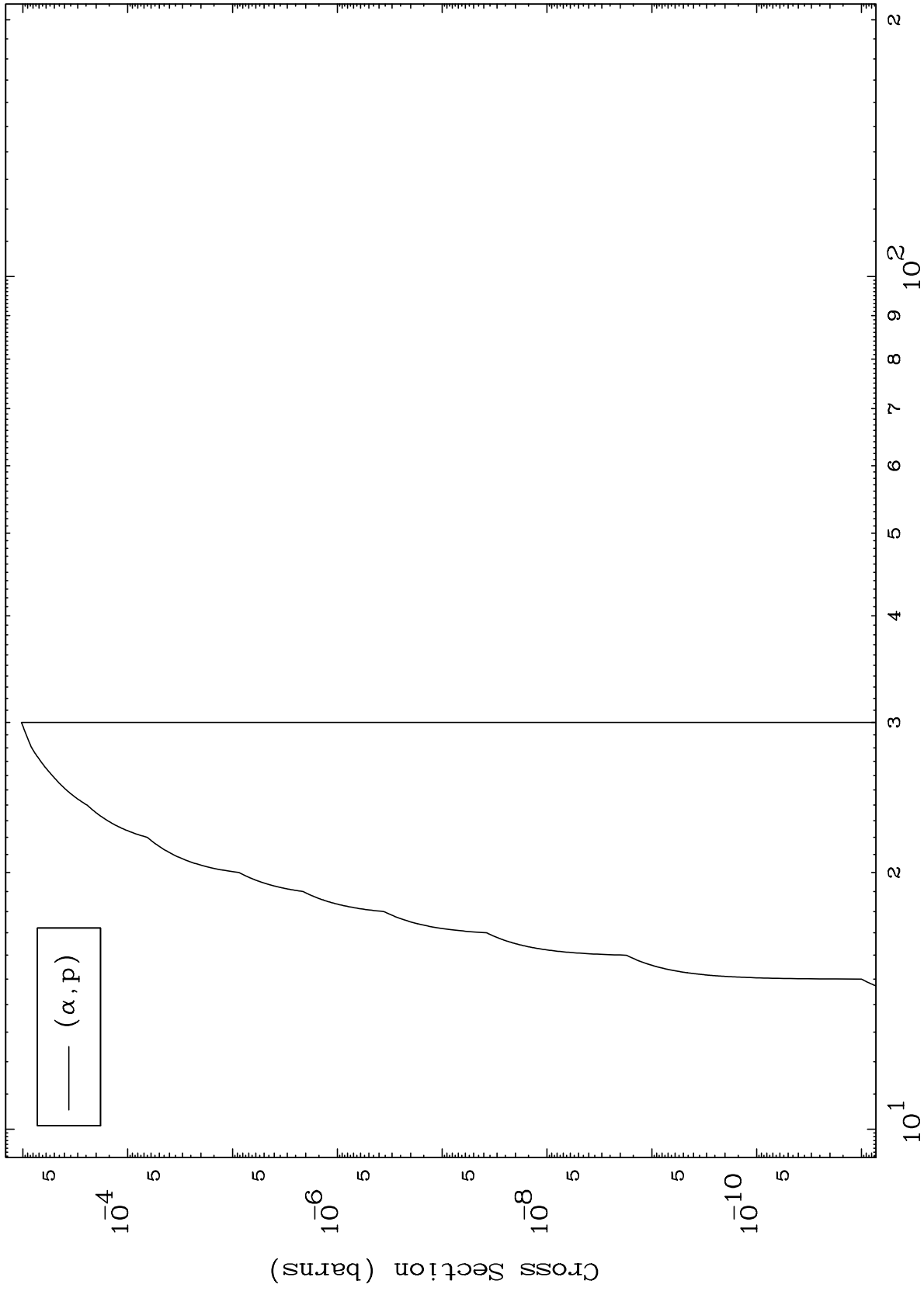
Incident Energy (MeV)

72-Hf-184

MAT 7255

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

72-Hf-184



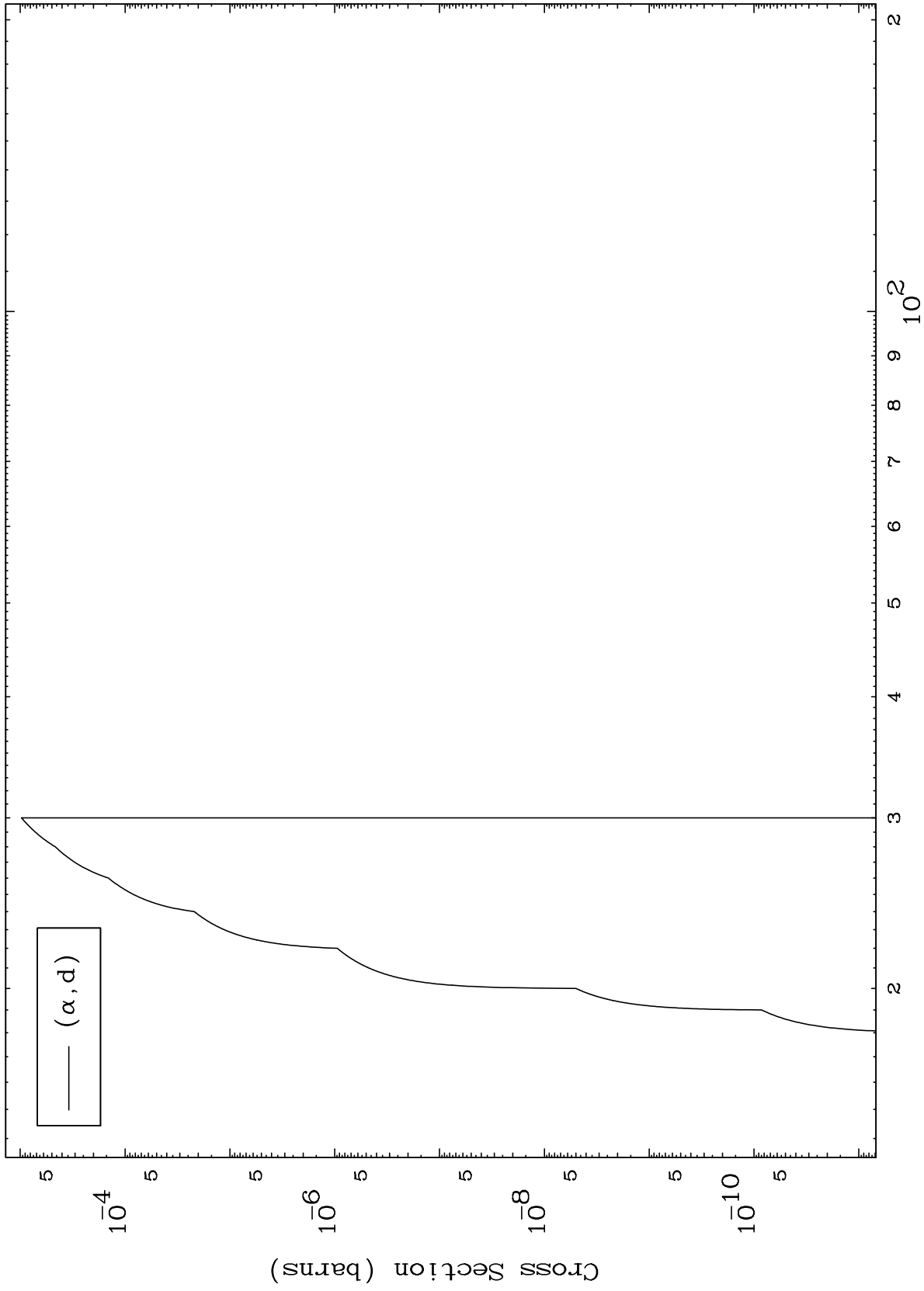
Incident Energy (MeV)

72-Hf-184

MAT 7255

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

72-Hf-184



7

Incident Energy (MeV)

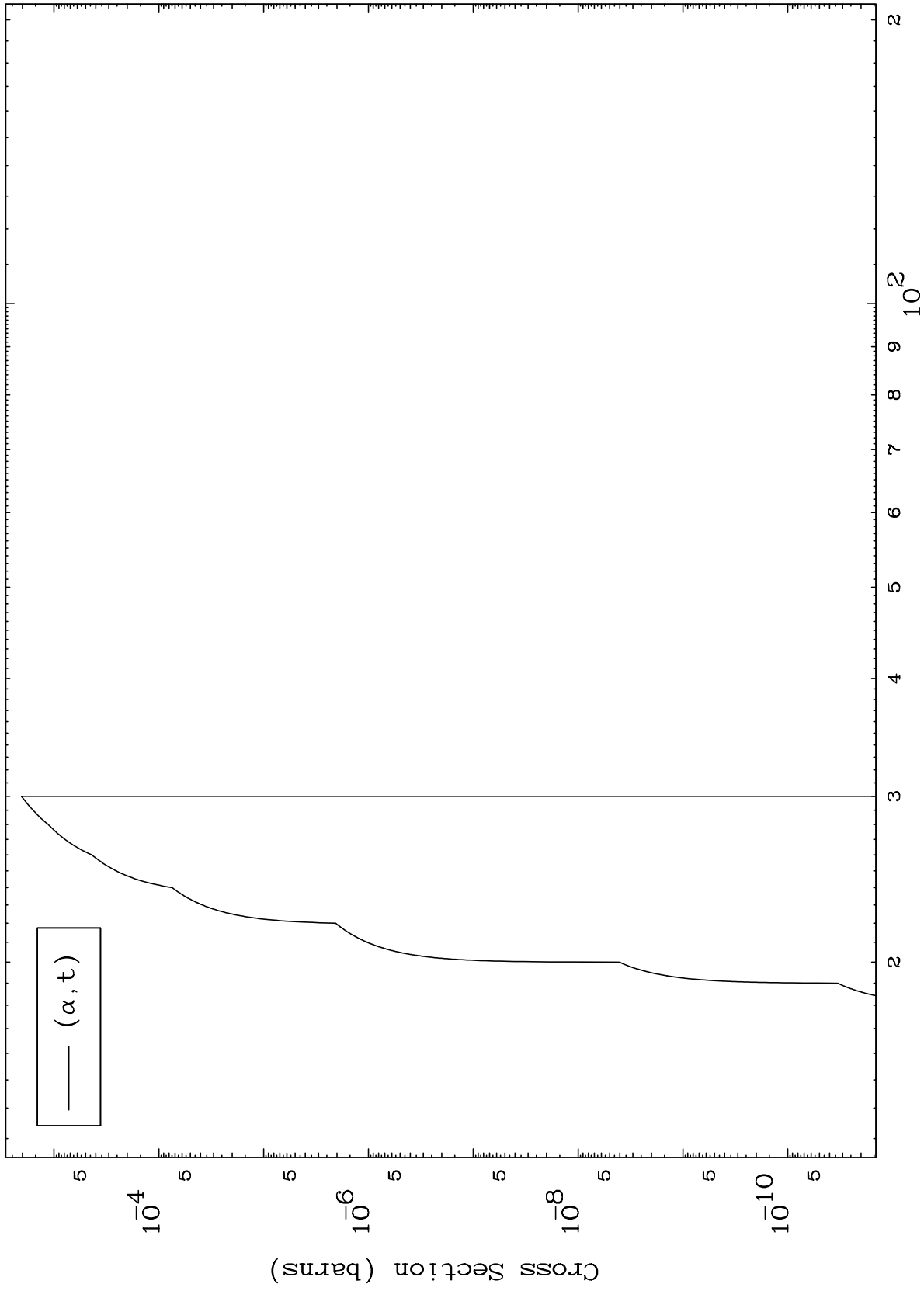
72-Hf-184

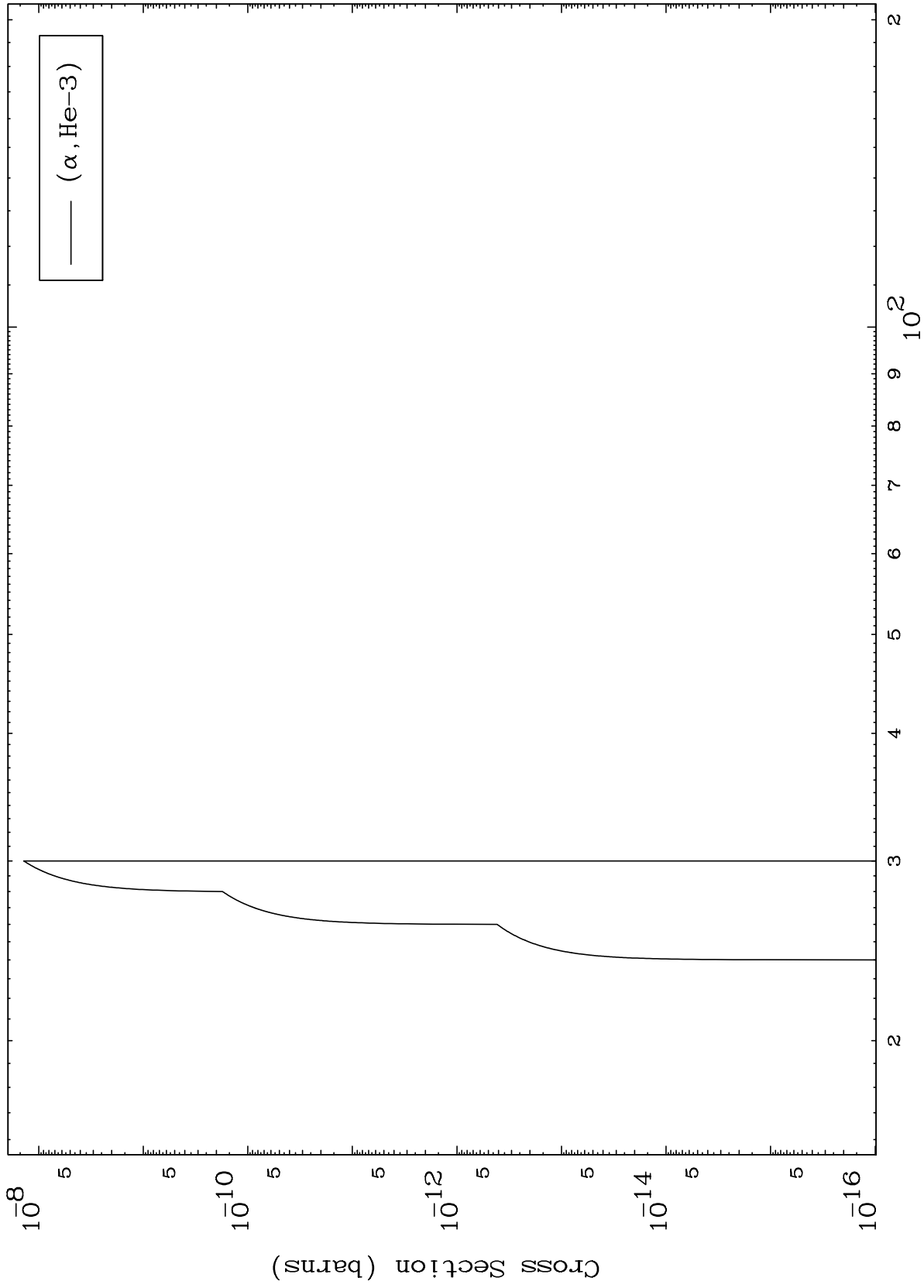


MAT 7255

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

72-Hf-184

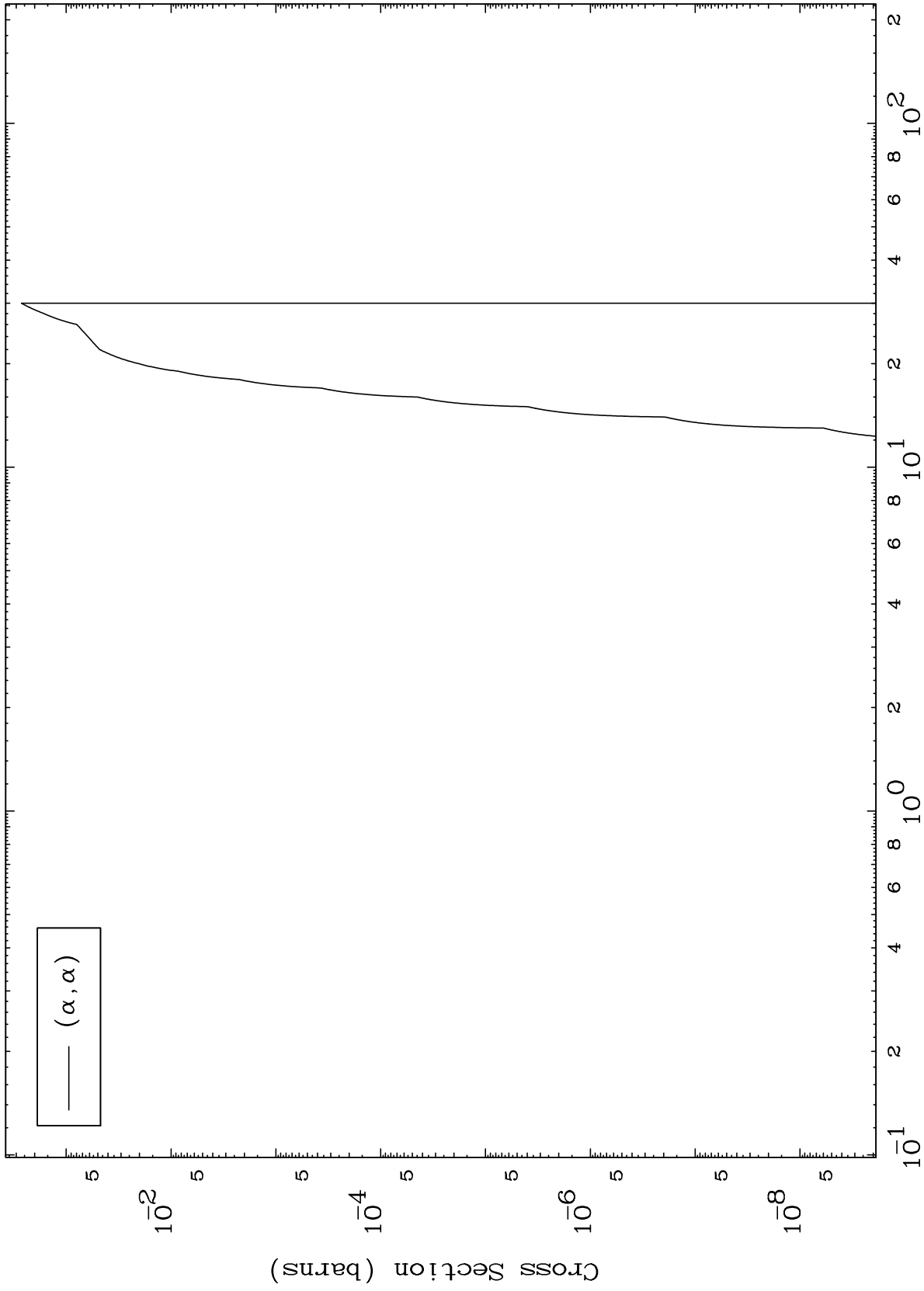




MAT 7255

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

72-Hf-184



Incident Energy (MeV)

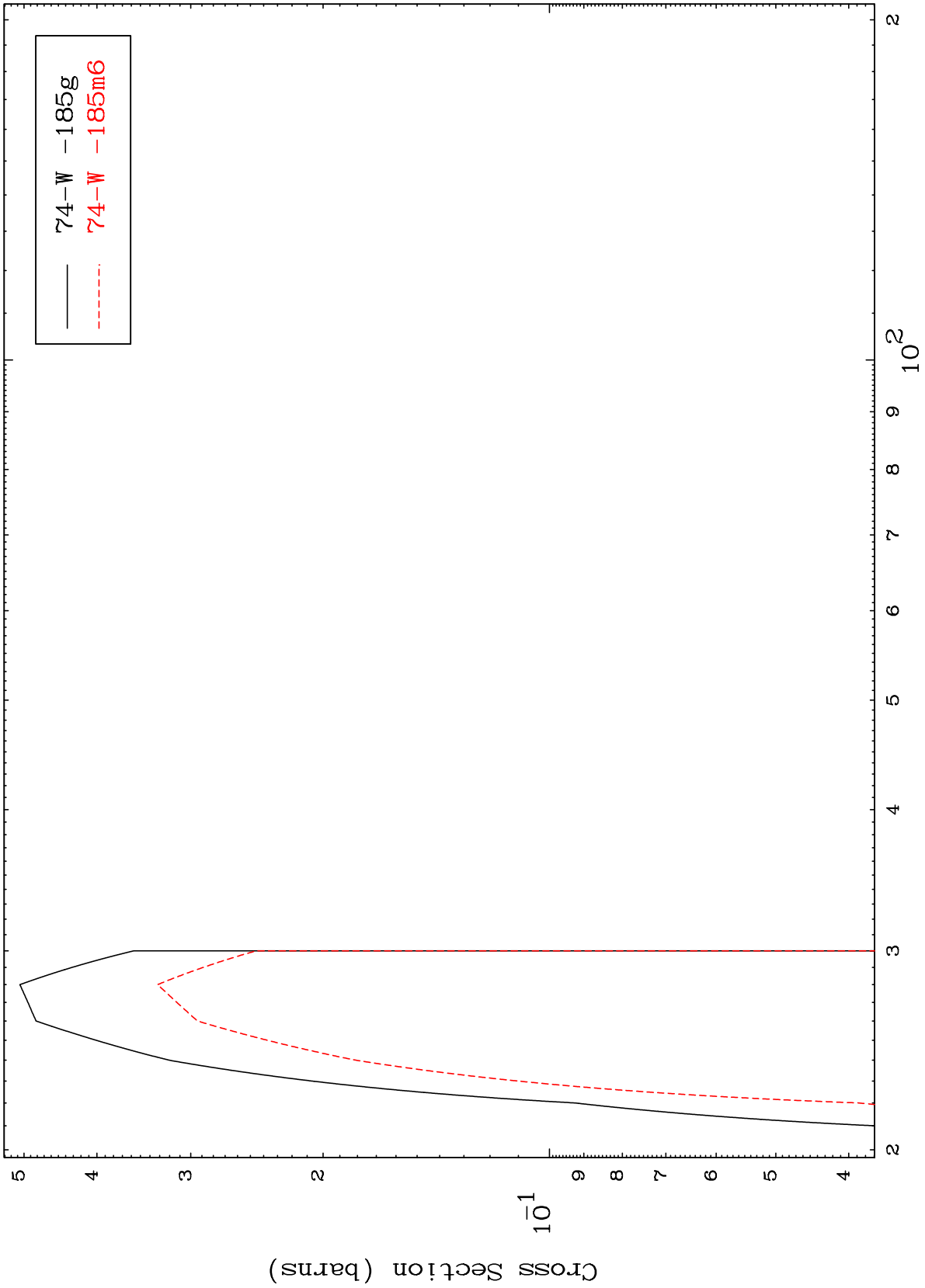
72-Hf-184

MAT 7255

( $\alpha, 3n$ )

72-Hf-184

Radionuclide Production Cross Section



74-W -185g  
74-W -185m6

11

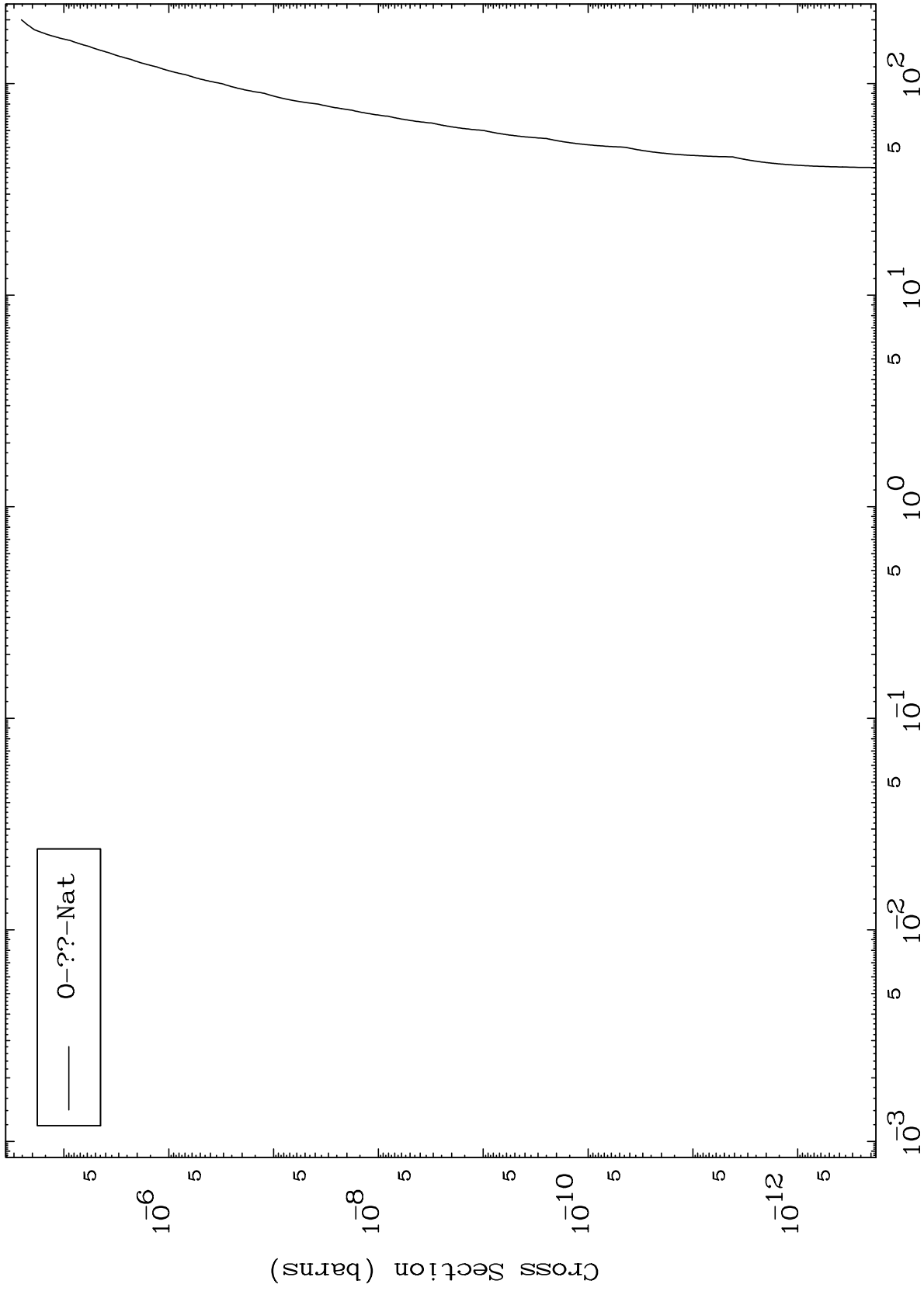
Incident Energy (MeV)

72-Hf-184

MAT 7255

$\alpha$  Fission  
Radionuclide Production Cross Section

72-Hf-184



12

Incident Energy (MeV)

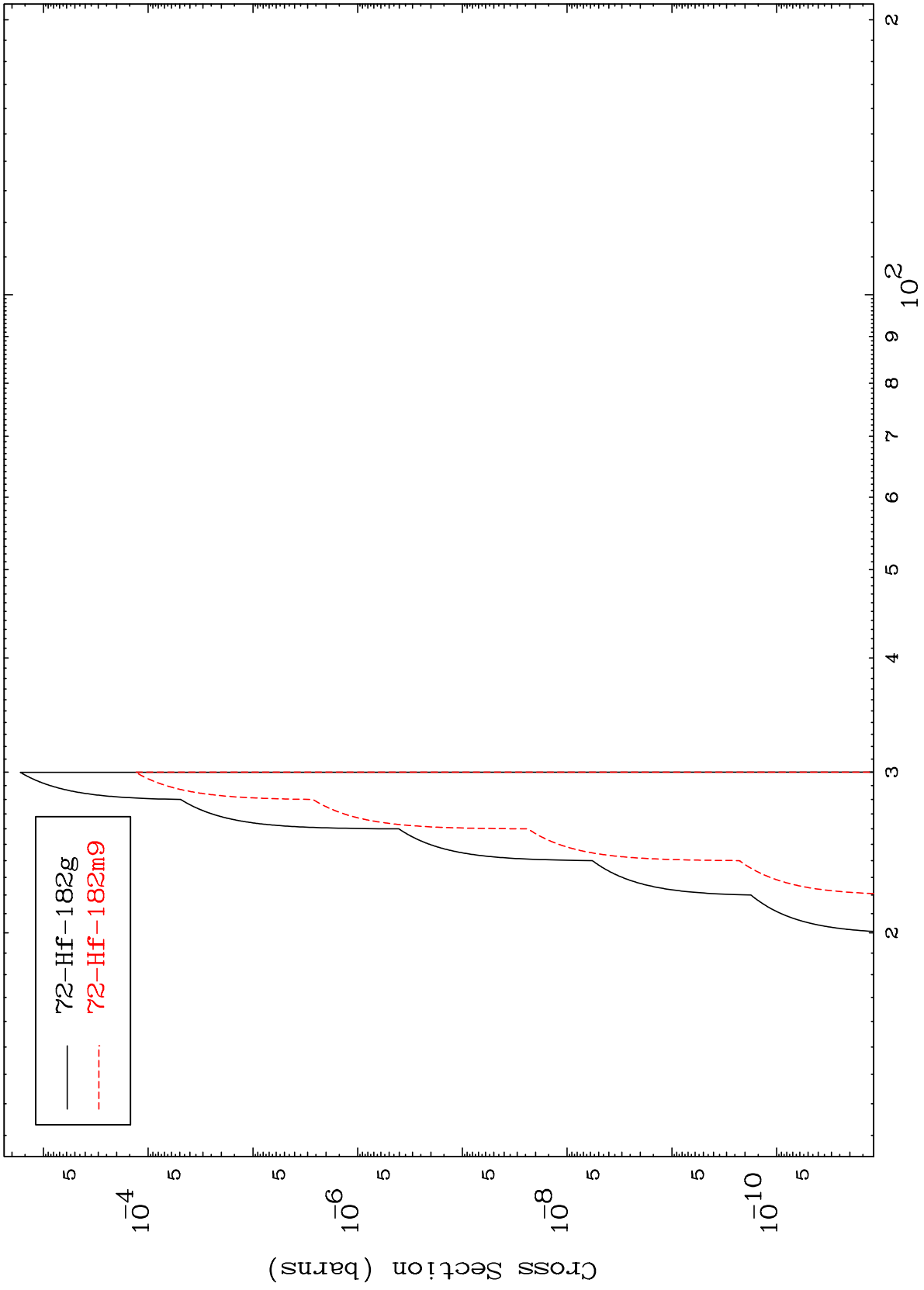
72-Hf-184

MAT 7255

$(\alpha, 2n) \alpha$

$^{72}\text{Hf-184}$

Radionuclide Production Cross Section



13

Incident Energy (MeV)

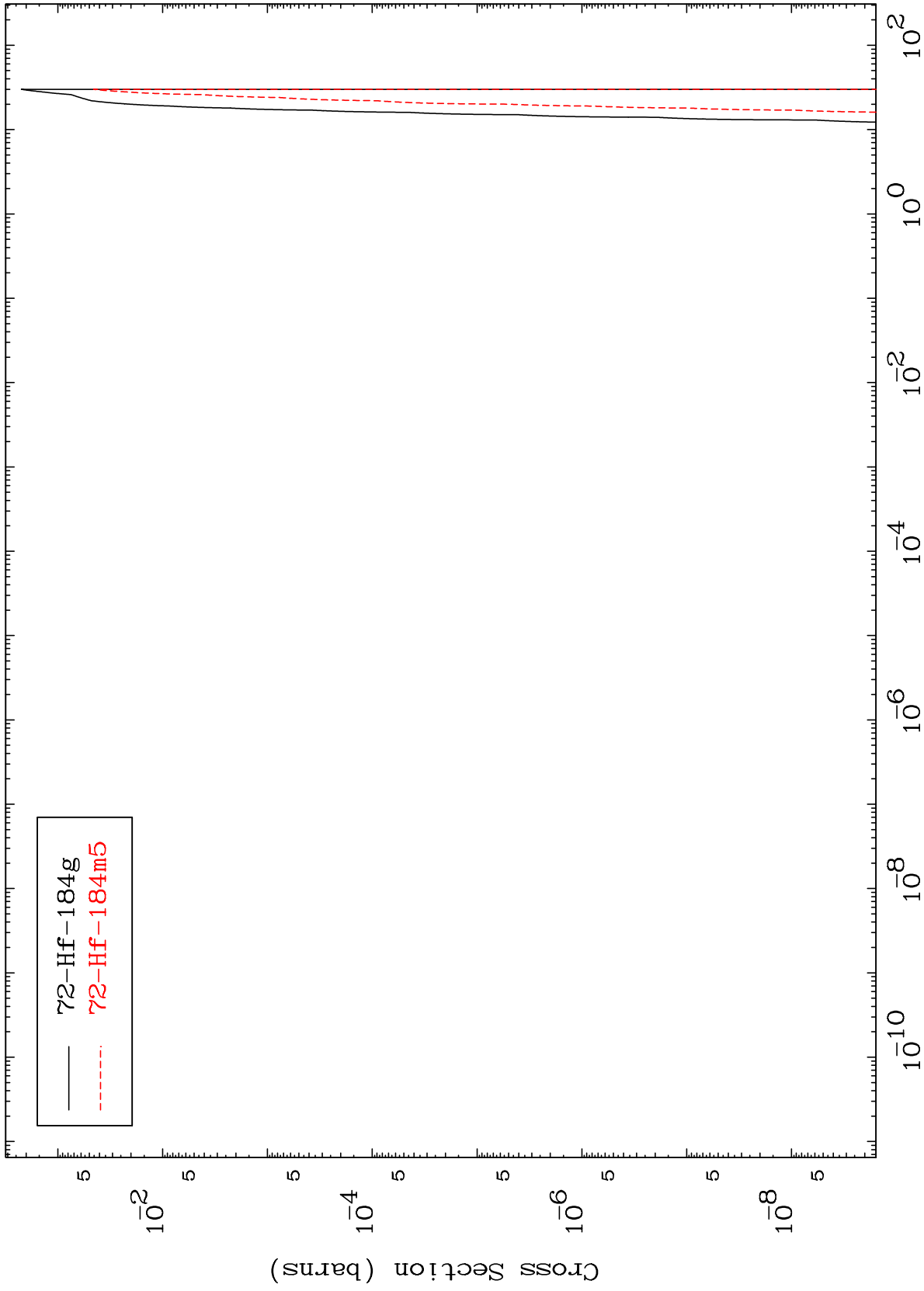
$^{72}\text{Hf-184}$

MAT 7255

( $\alpha, \alpha$ )

<sup>72</sup>Hf-184

Radionuclide Production Cross Section



14

Incident Energy (MeV)

<sup>72</sup>Hf-184