

Program EVALPLOT  
(Version 2018-1)

by

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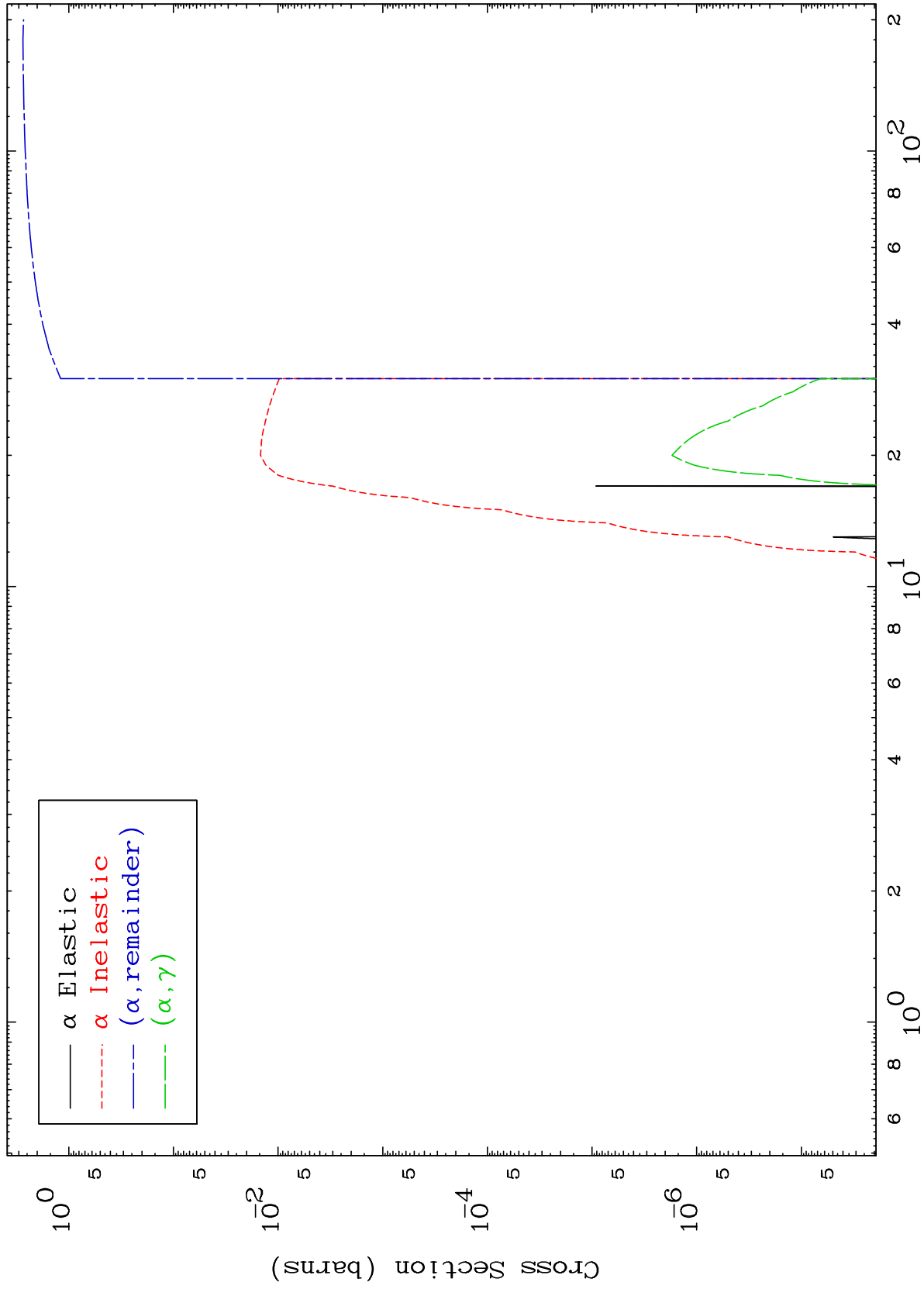
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7734

0 Kelvin  $\alpha$  Major

77-Ir-194



1

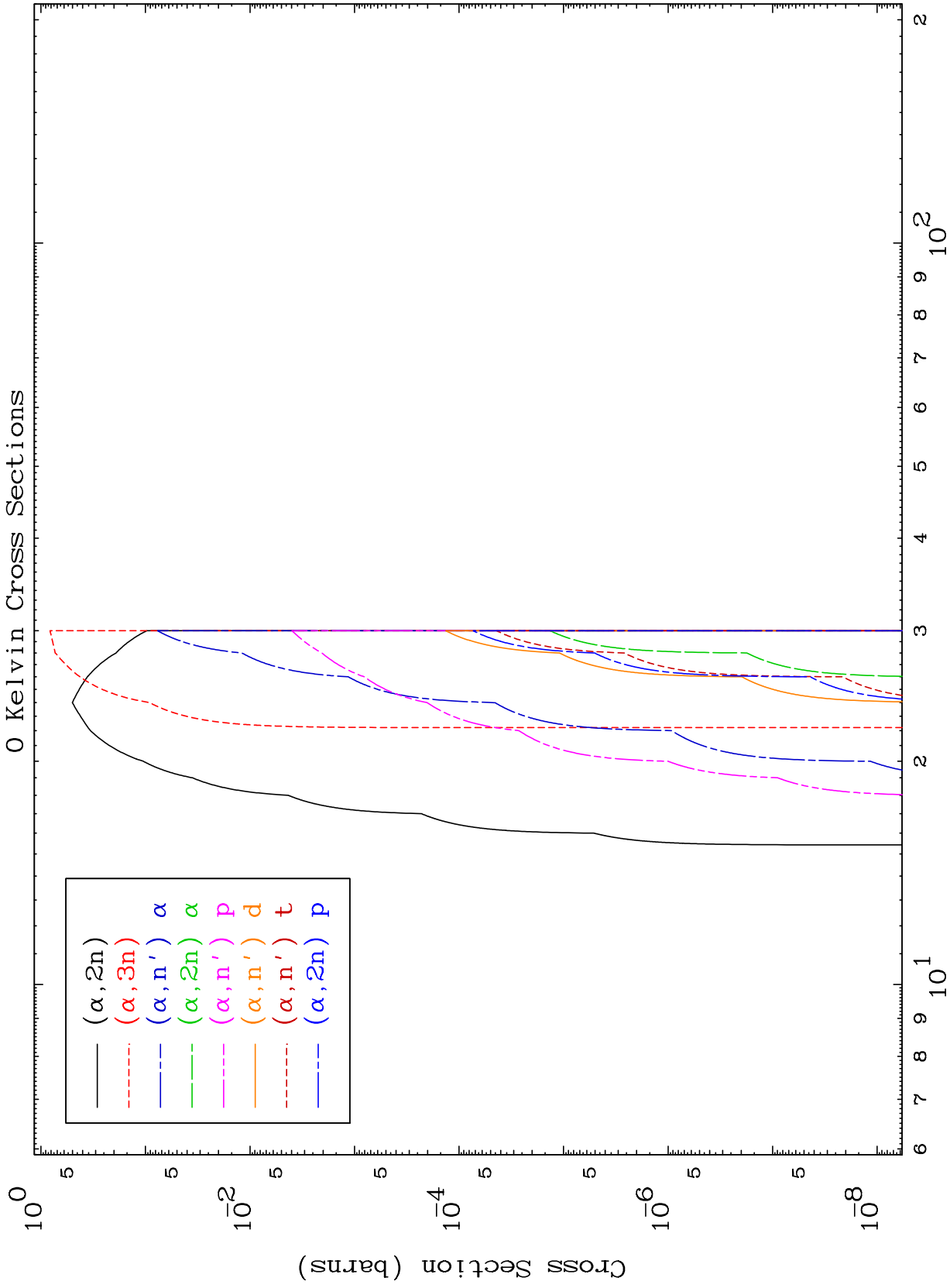
Incident Energy (MeV)

77-Ir-194

MAT 7734

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

77-Ir-194



2

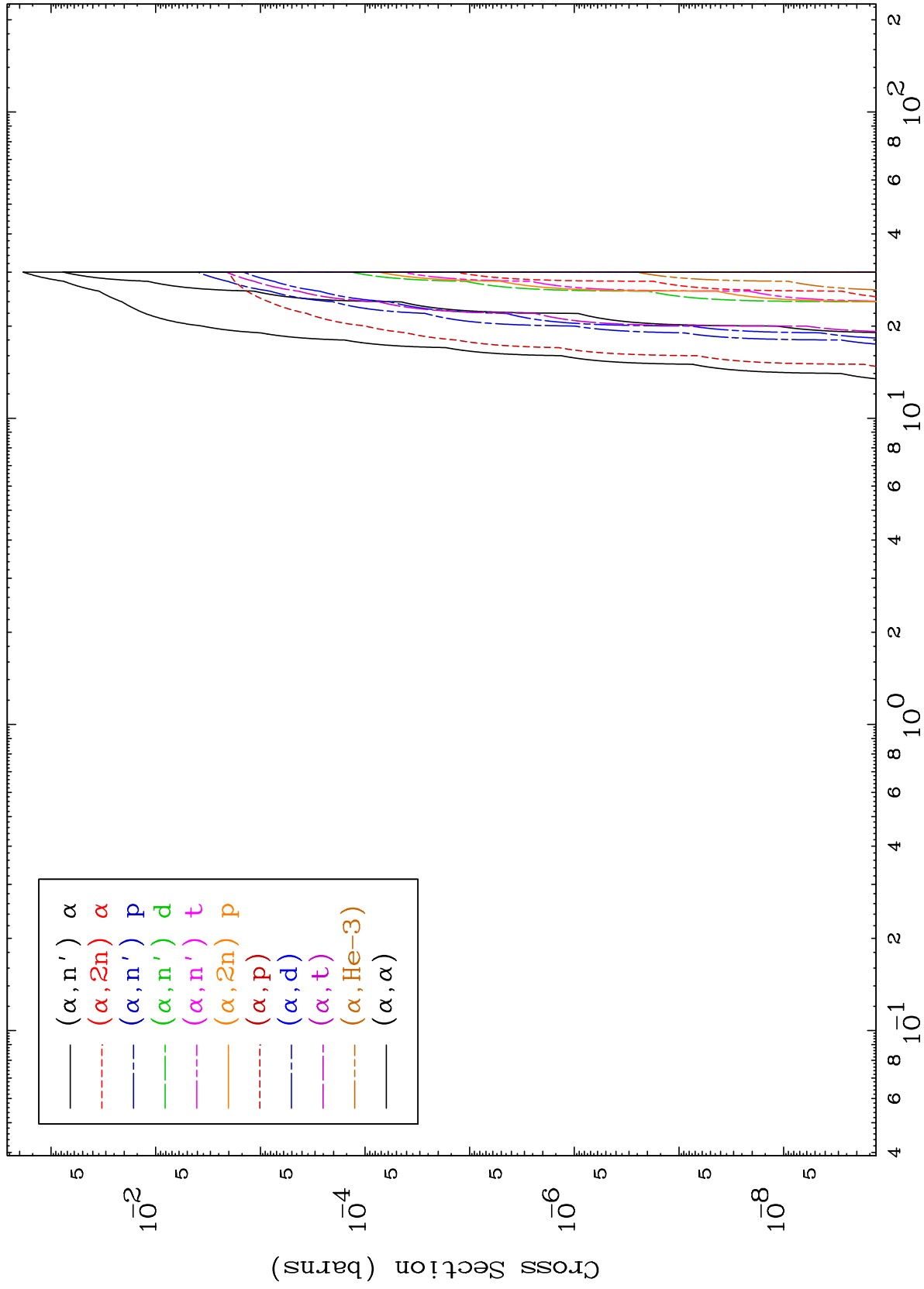
Incident Energy (MeV)

77-Ir-194

MAT 7734

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

77-Ir-194



3

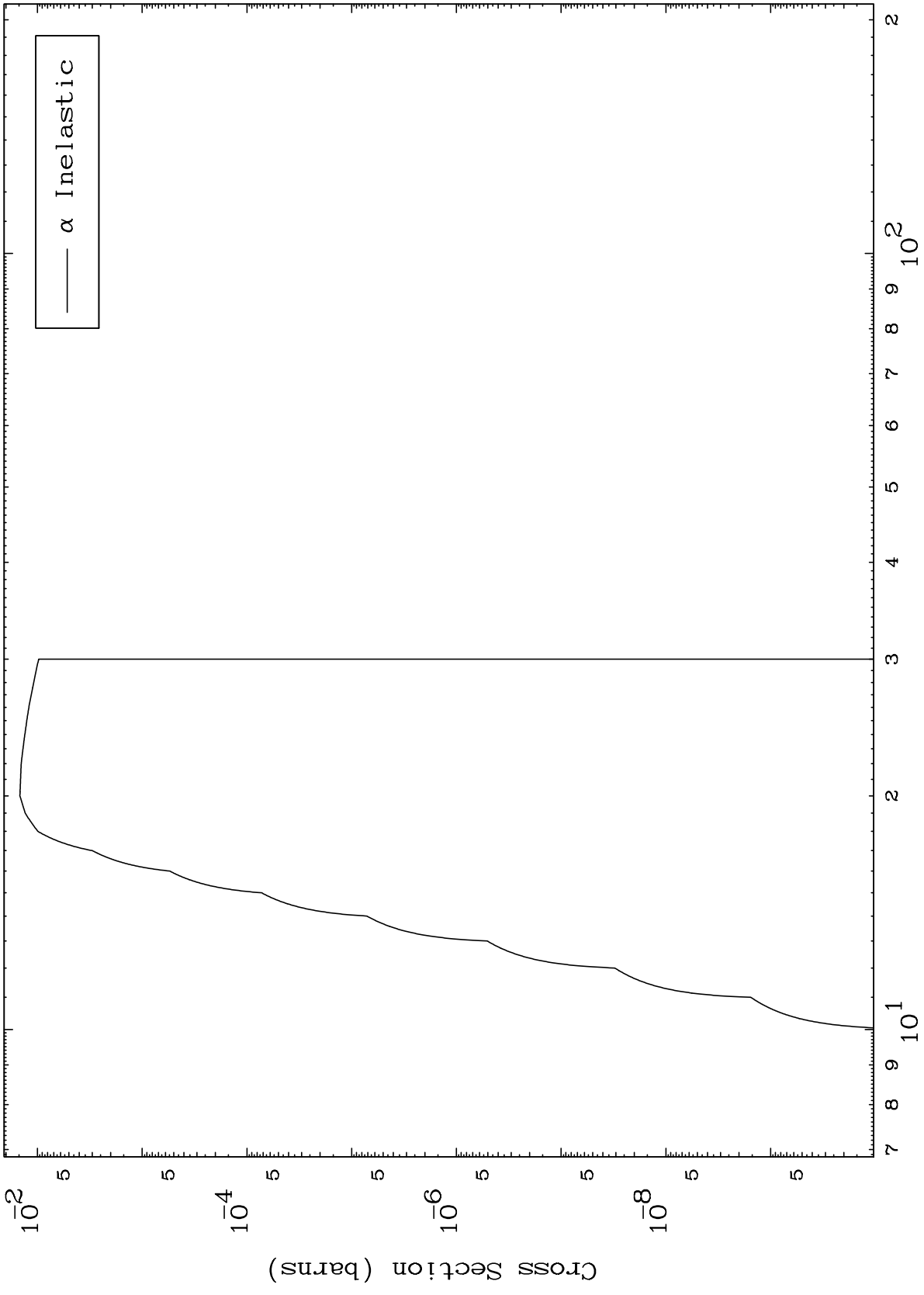
Incident Energy (MeV)

77-Ir-194

MAT 7734

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

77-Ir-194



4

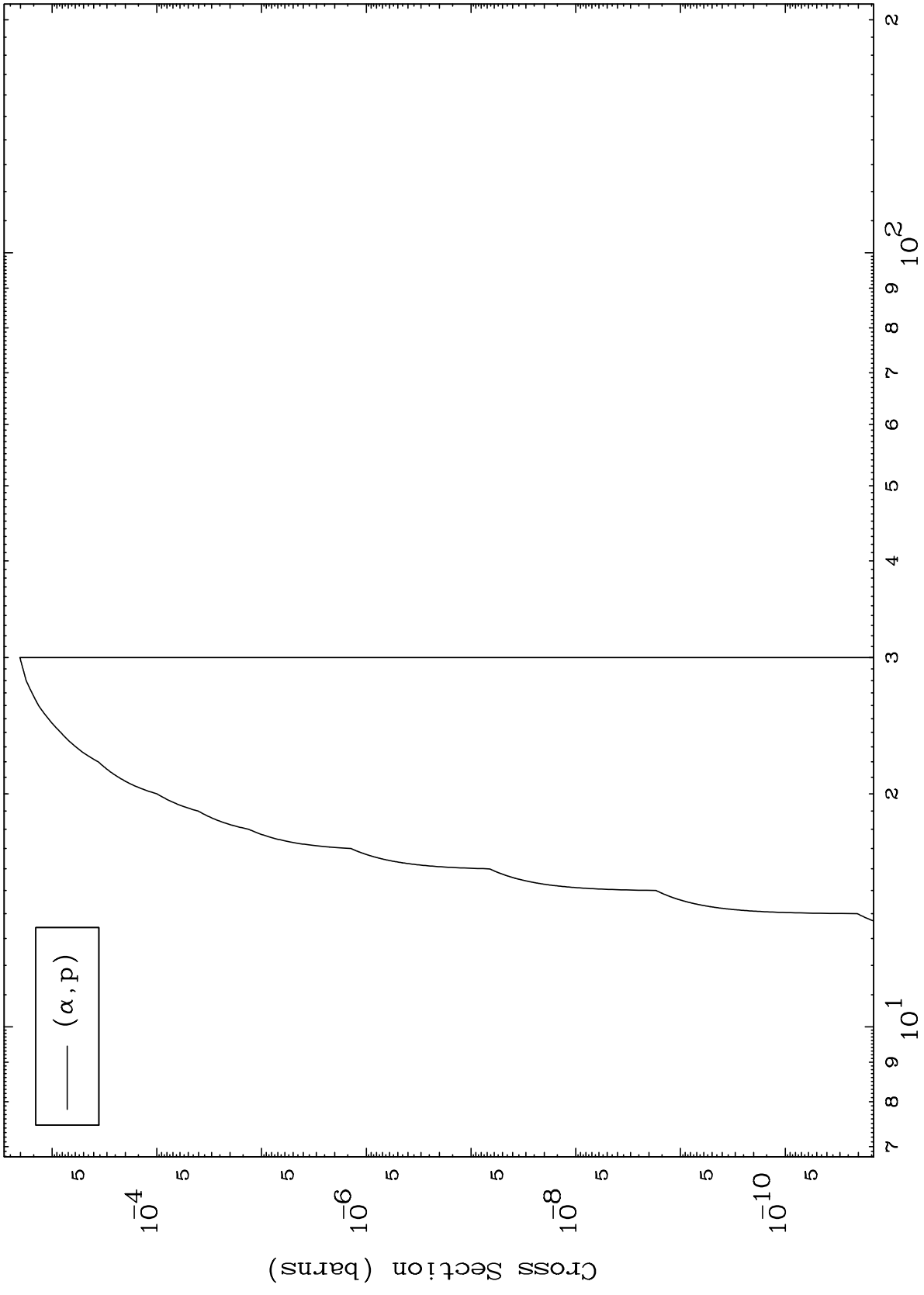
Incident Energy (MeV)

77-Ir-194

MAT 7734

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

77-Ir-194



5

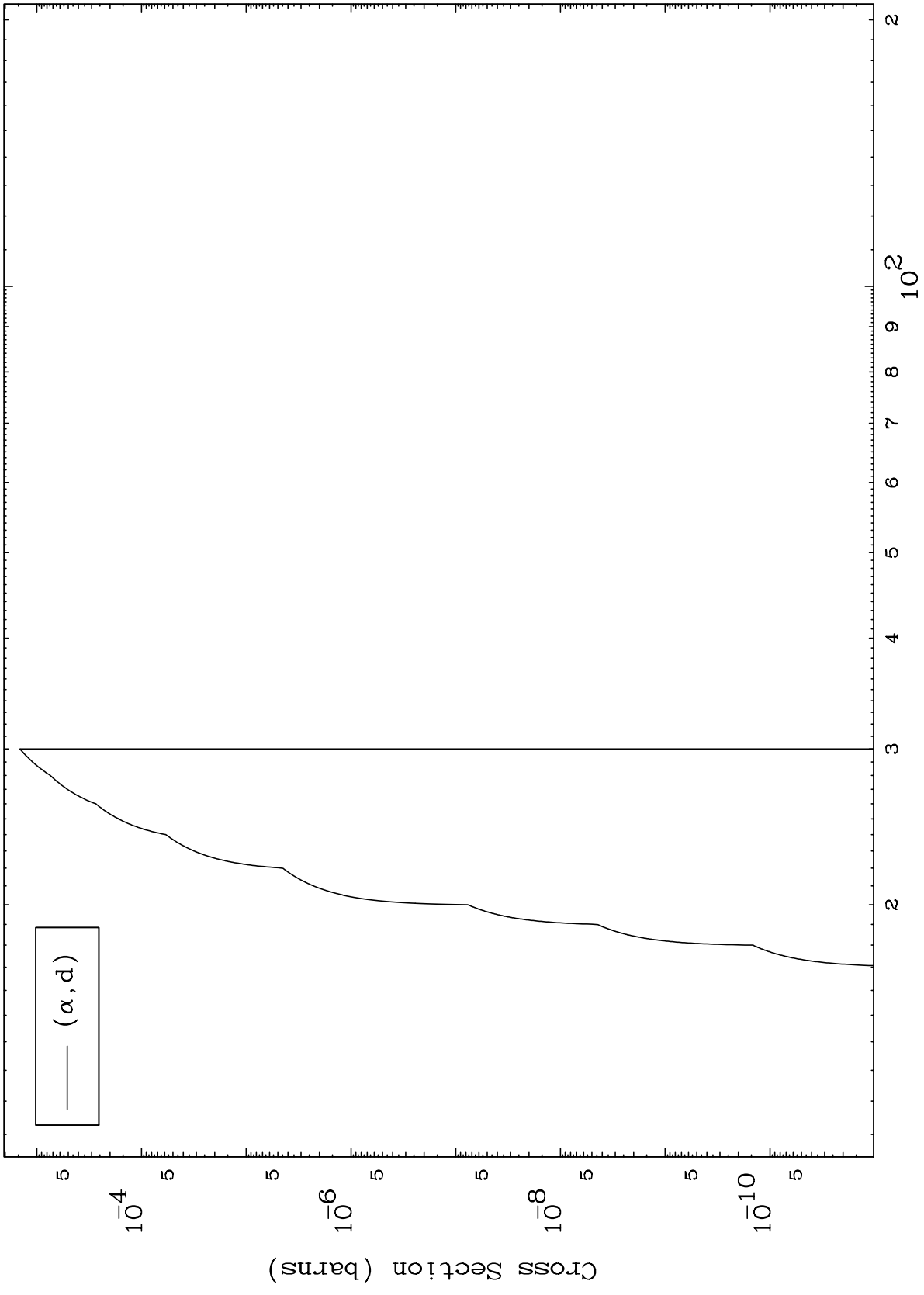
Incident Energy (MeV)

77-Ir-194

MAT 7734

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

77-Ir-194



6

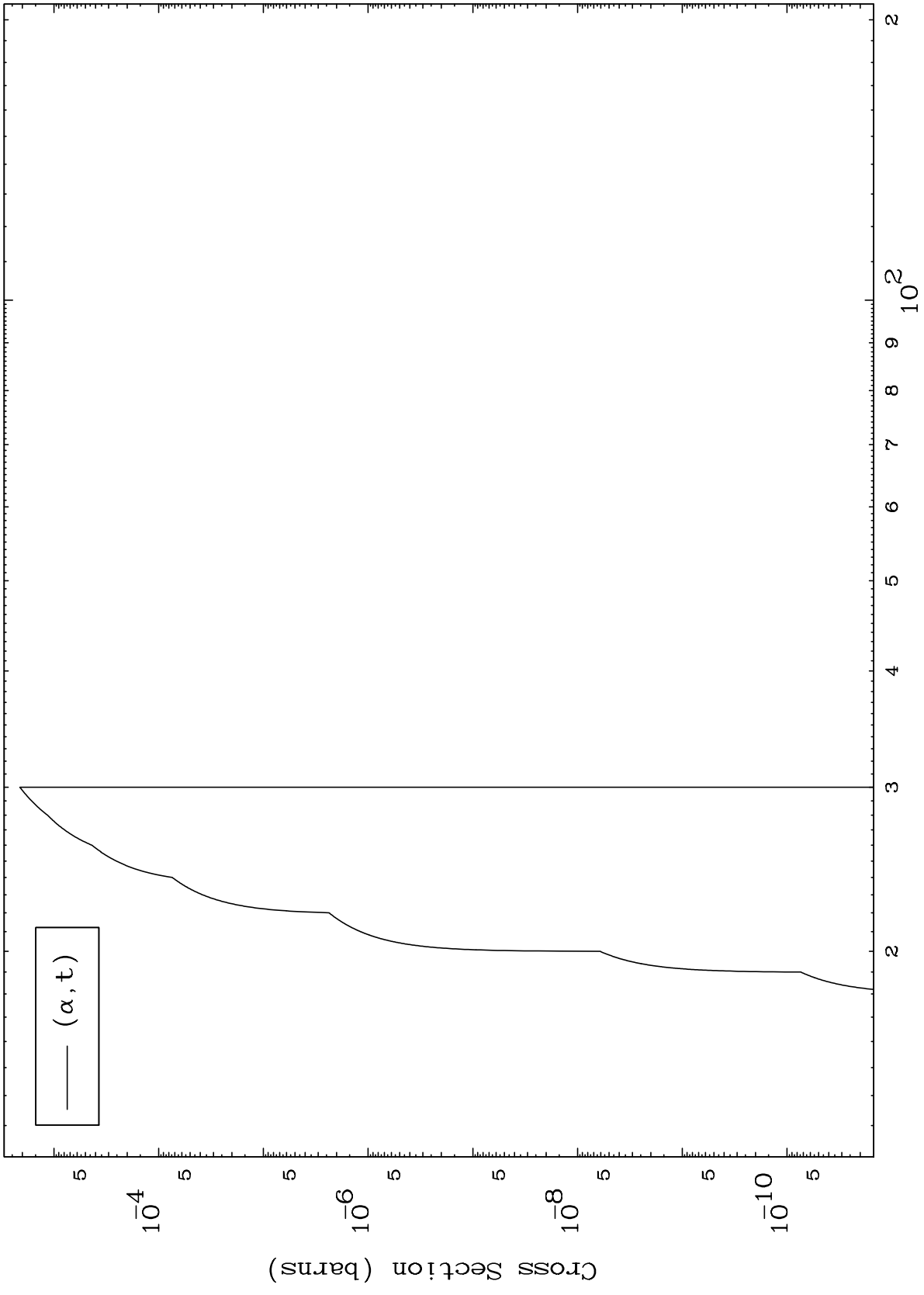
Incident Energy (MeV)

77-Ir-194

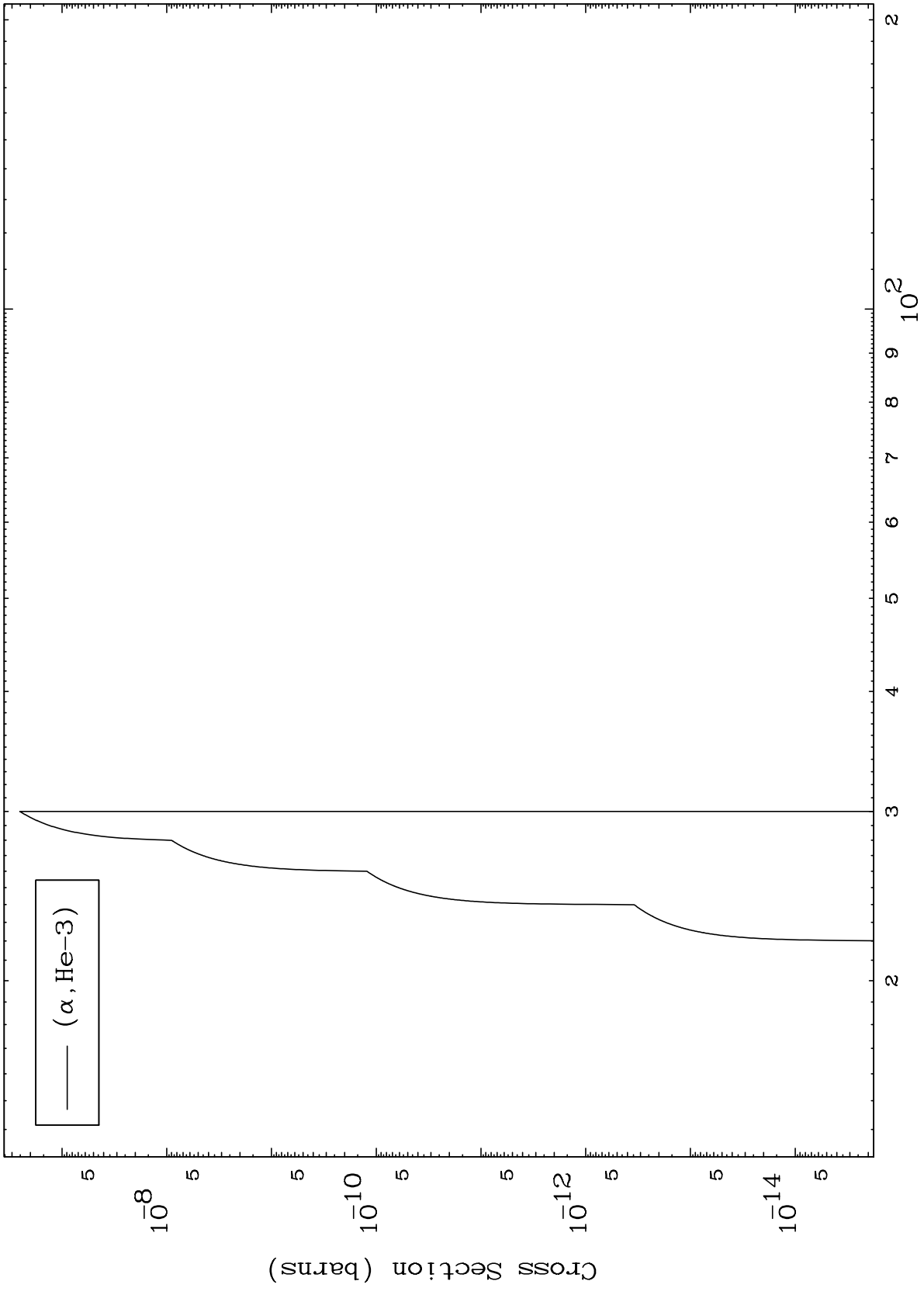
MAT 7734

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

77-Ir-194



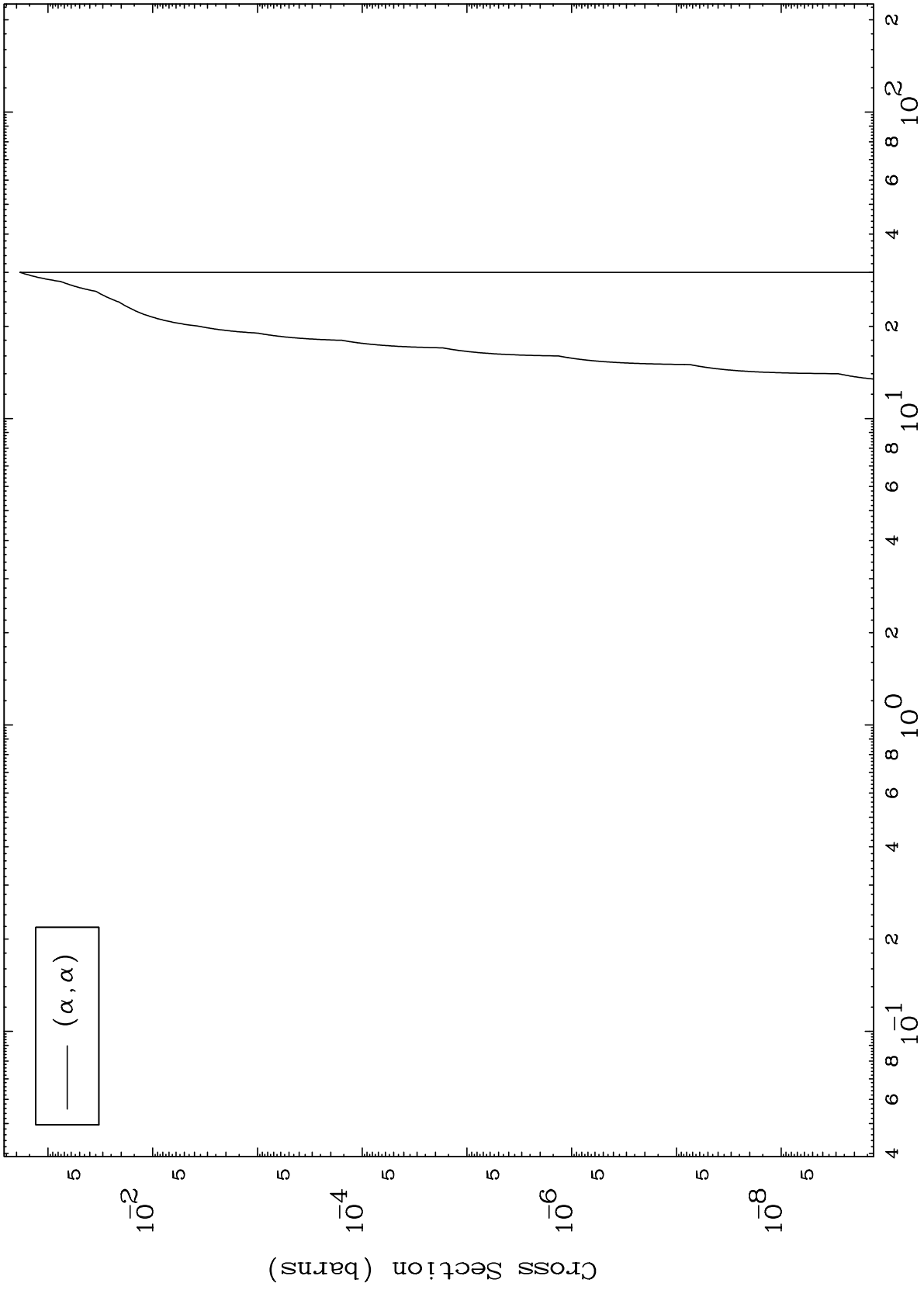




MAT 7734

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

77-Ir-194



9

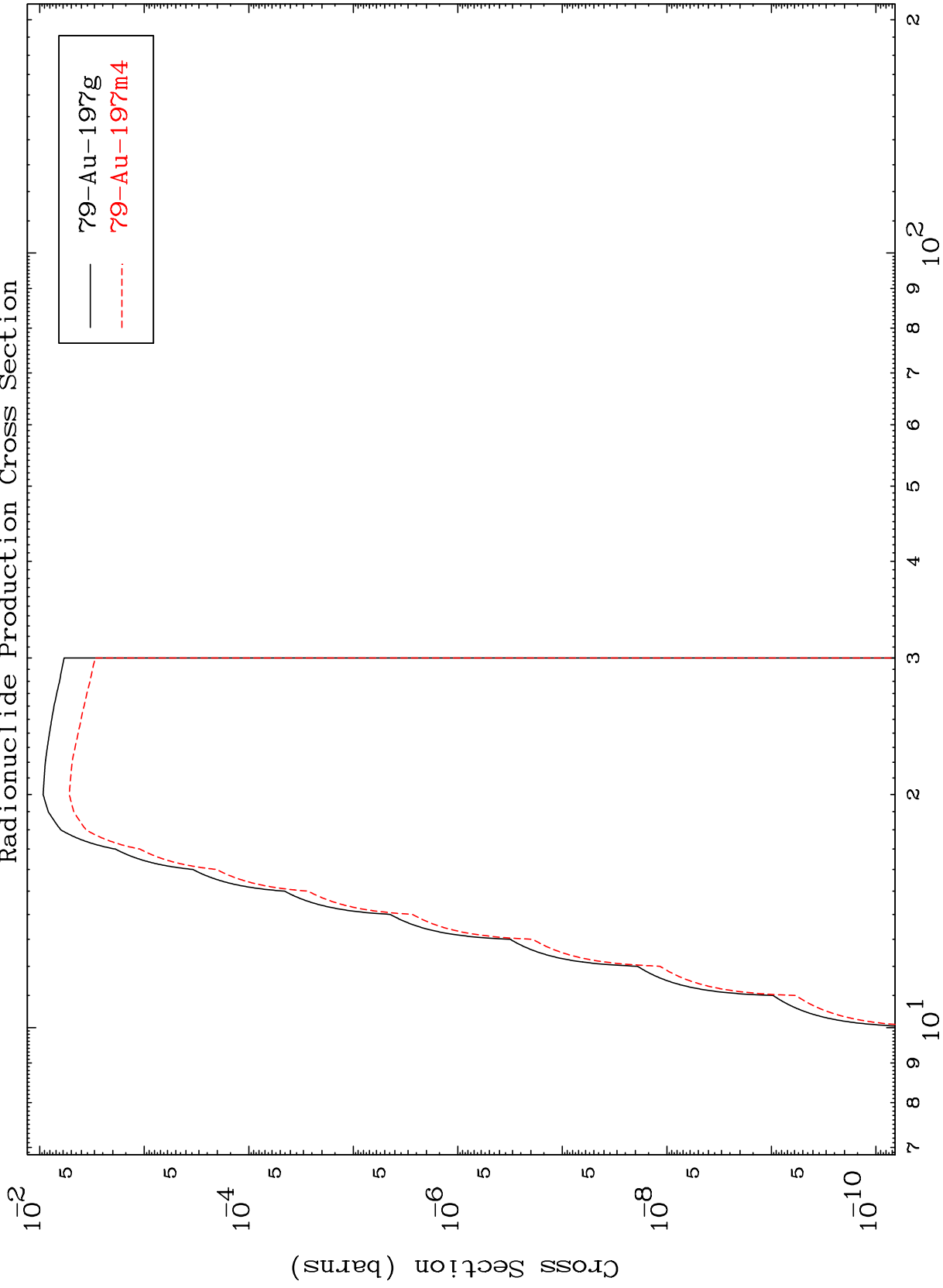
Incident Energy (MeV)

77-Ir-194

MAT 7734

$\alpha$  Inelastic  
Radionuclide Production Cross Section

77-Ir-194



10

Incident Energy (MeV)

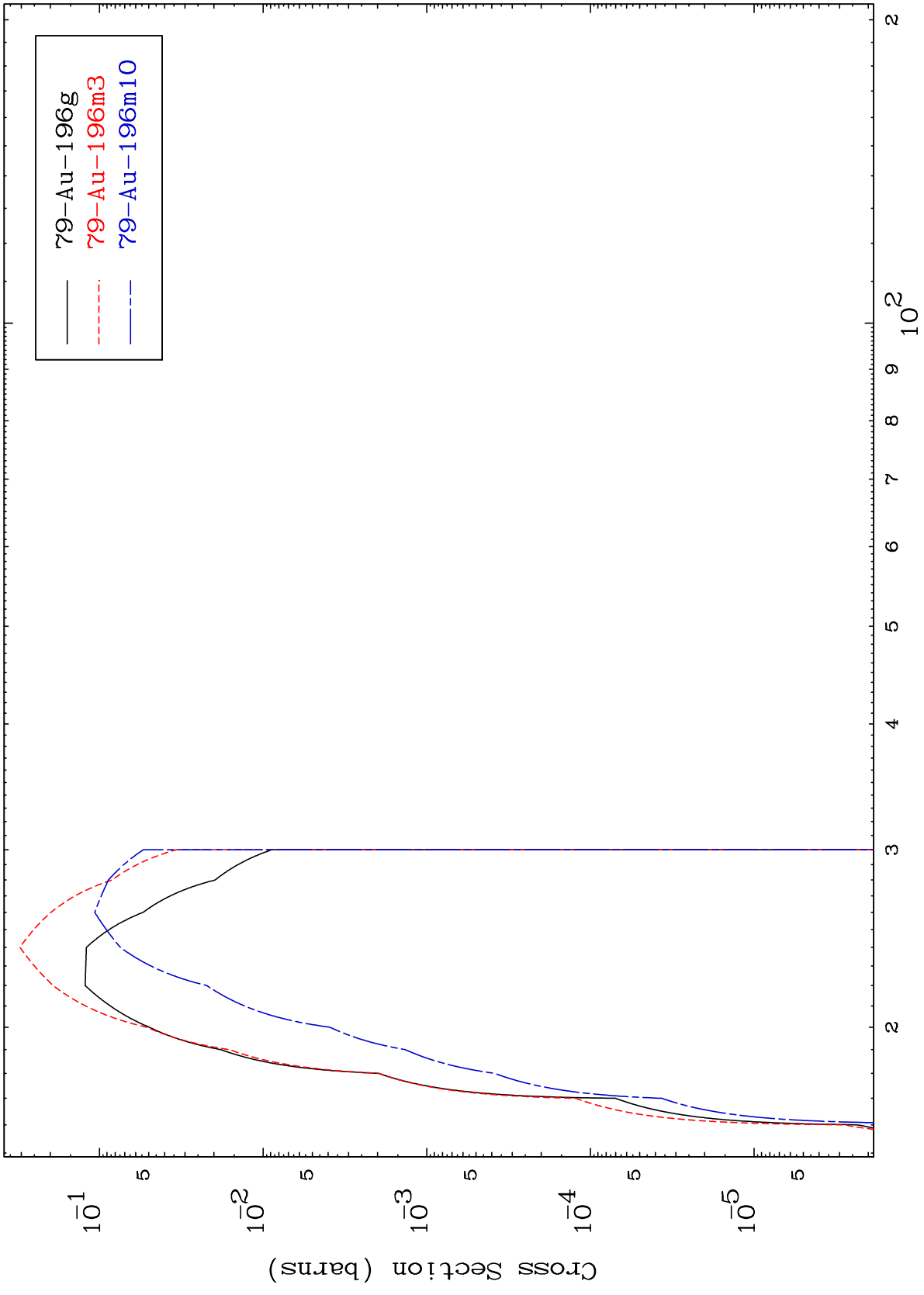
77-Ir-194

MAT 7734

( $\alpha, 2n$ )

77-Ir-194

Radionuclide Production Cross Section



11

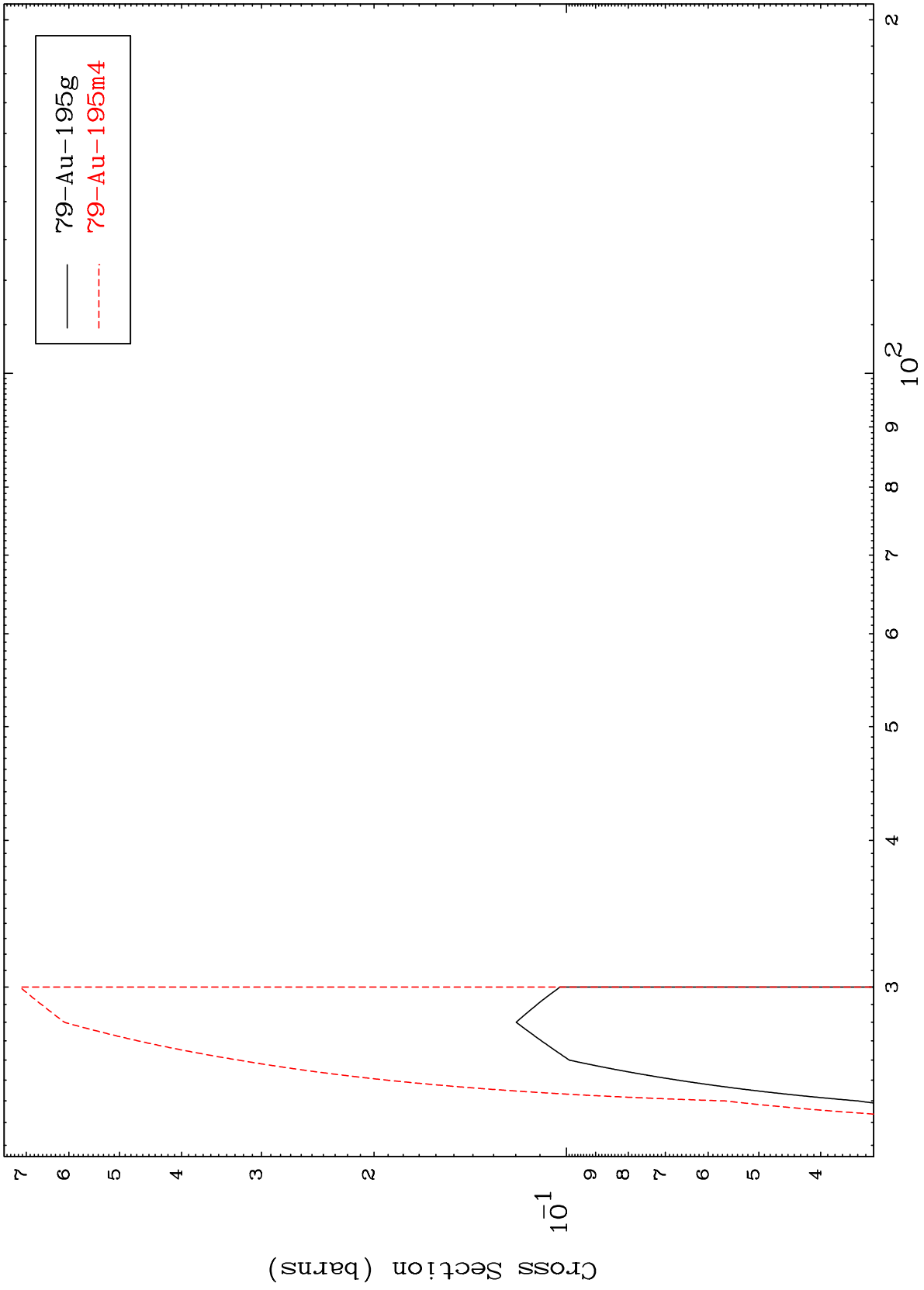
Incident Energy (MeV)

77-Ir-194

MAT 7734

77-Ir-194

( $\alpha, 3n$ )  
Radionuclide Production Cross Section



12

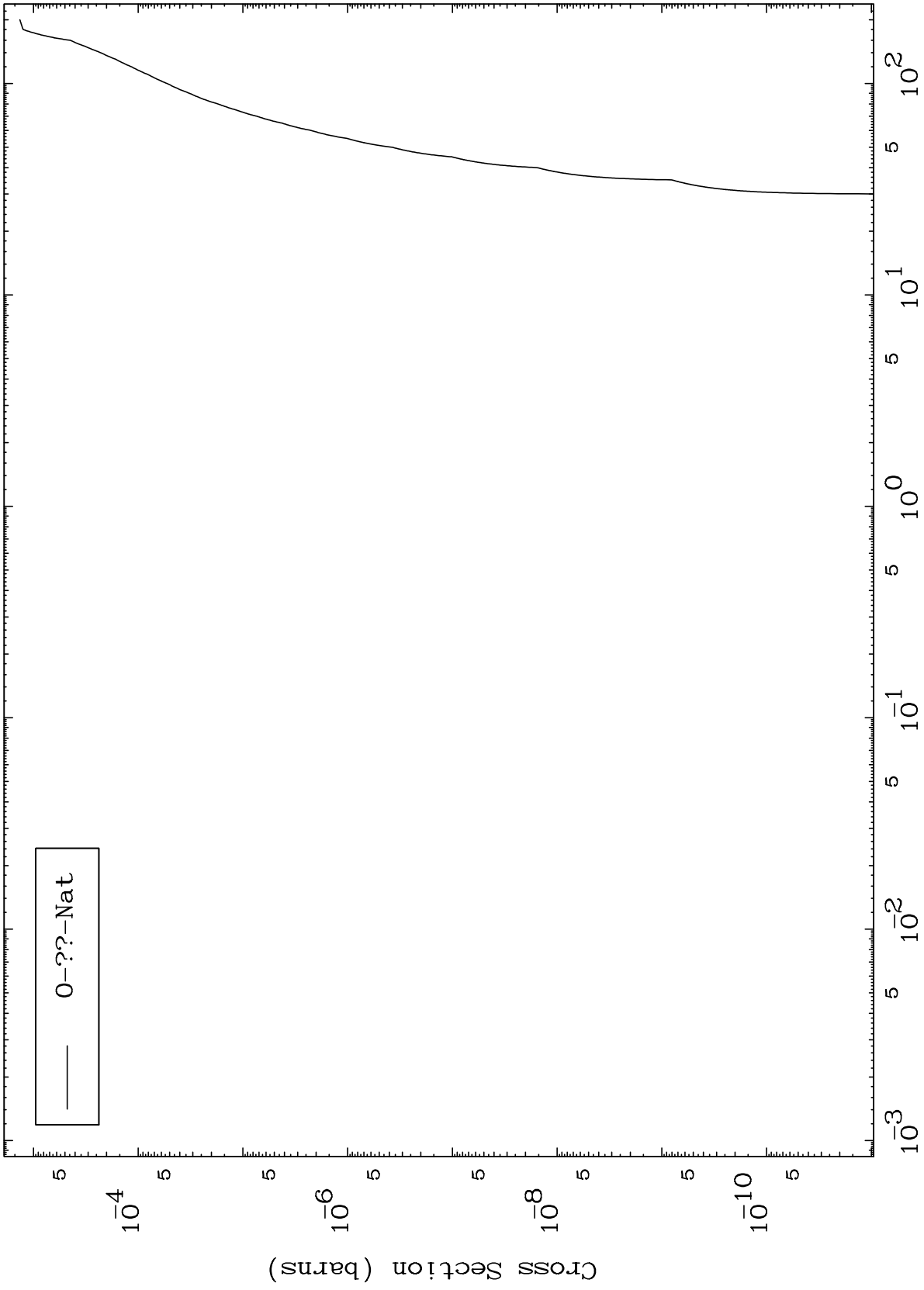
Incident Energy (MeV)

77-Ir-194

MAT 7734

$\alpha$  Fission  
Radionuclide Production Cross Section

77-Ir-194



13

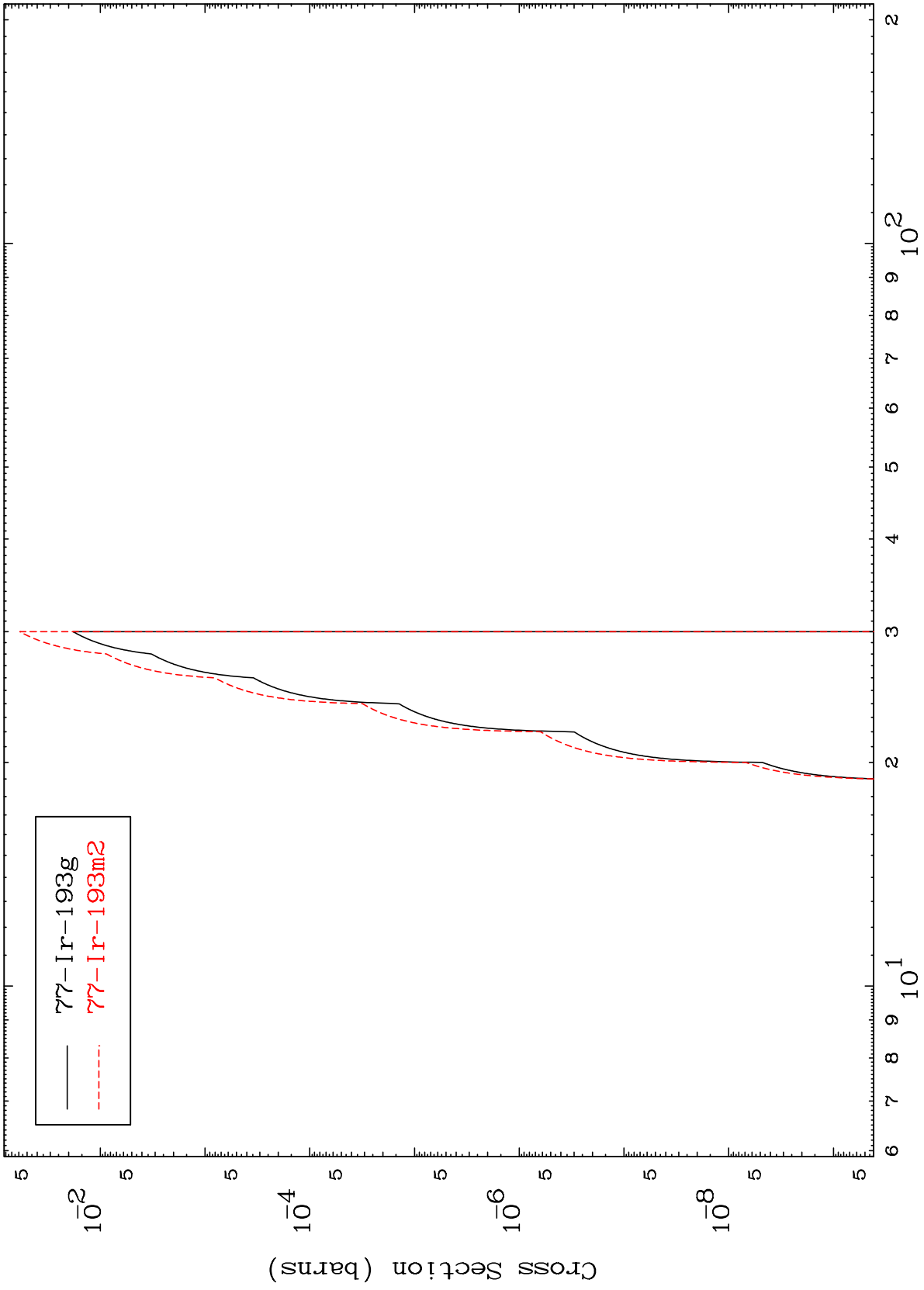
Incident Energy (MeV)

77-Ir-194

MAT 7734

77-Ir-194

$(\alpha, n')$   $\alpha$   
Radionuclide Production Cross Section



14

Incident Energy (MeV)

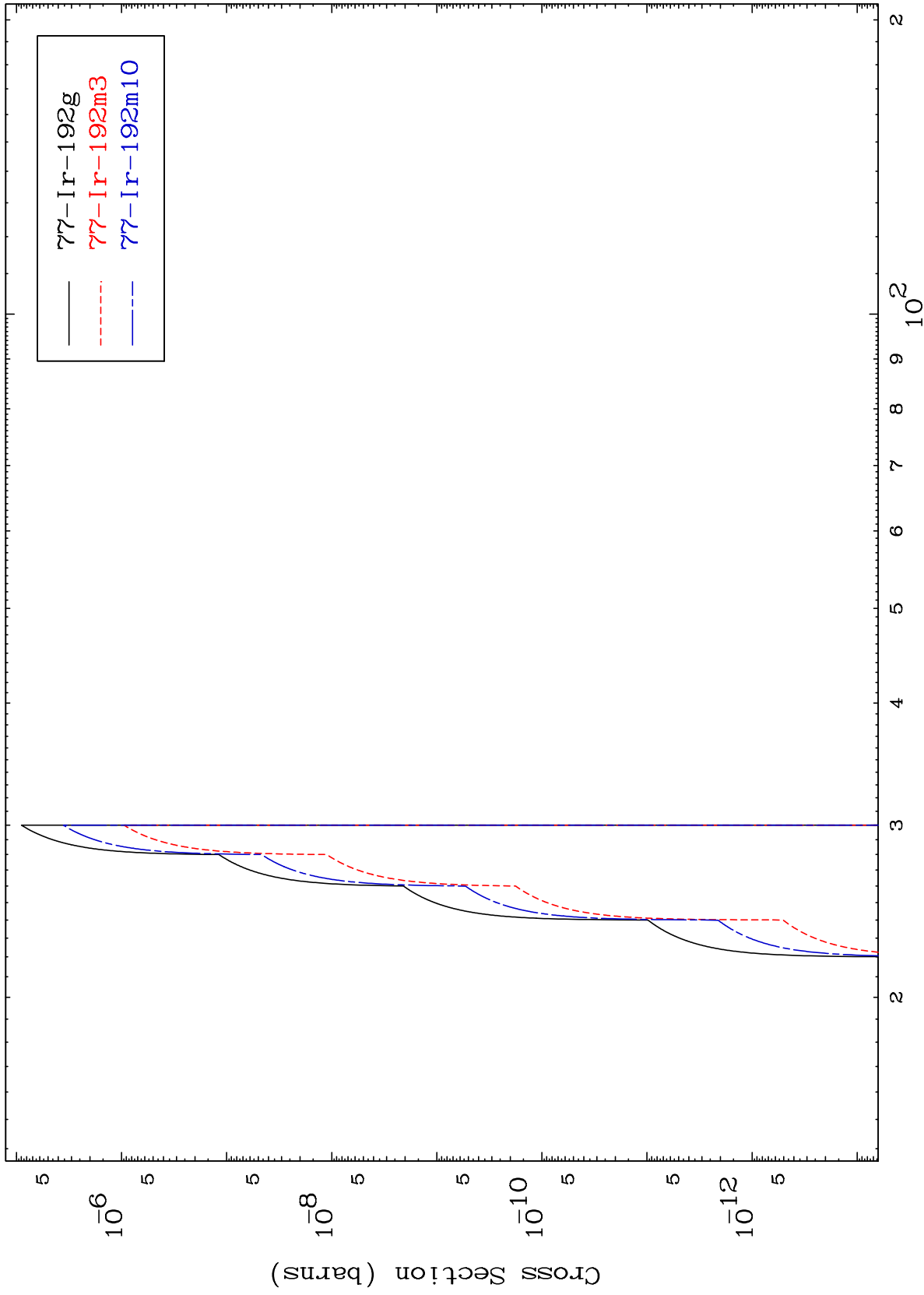
77-Ir-194

MAT 7734

$(\alpha, 2n) \alpha$

$^{77}\text{Ir-194}$

Radionuclide Production Cross Section



15

Incident Energy (MeV)

$^{77}\text{Ir-194}$

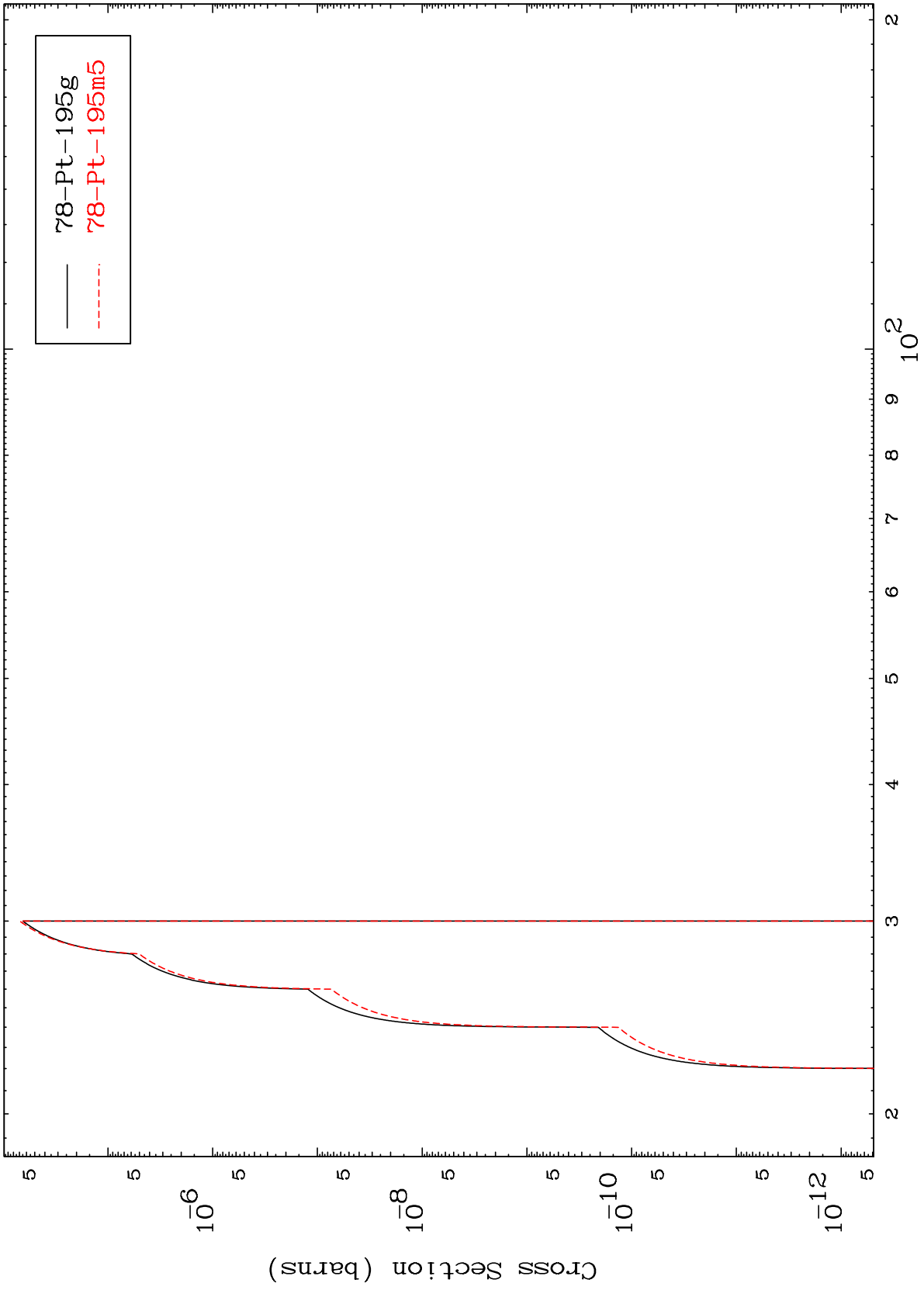


MAT 7734

( $\alpha, n'$ ) d

77-Ir-194

Radionuclide Production Cross Section



16

Incident Energy (MeV)

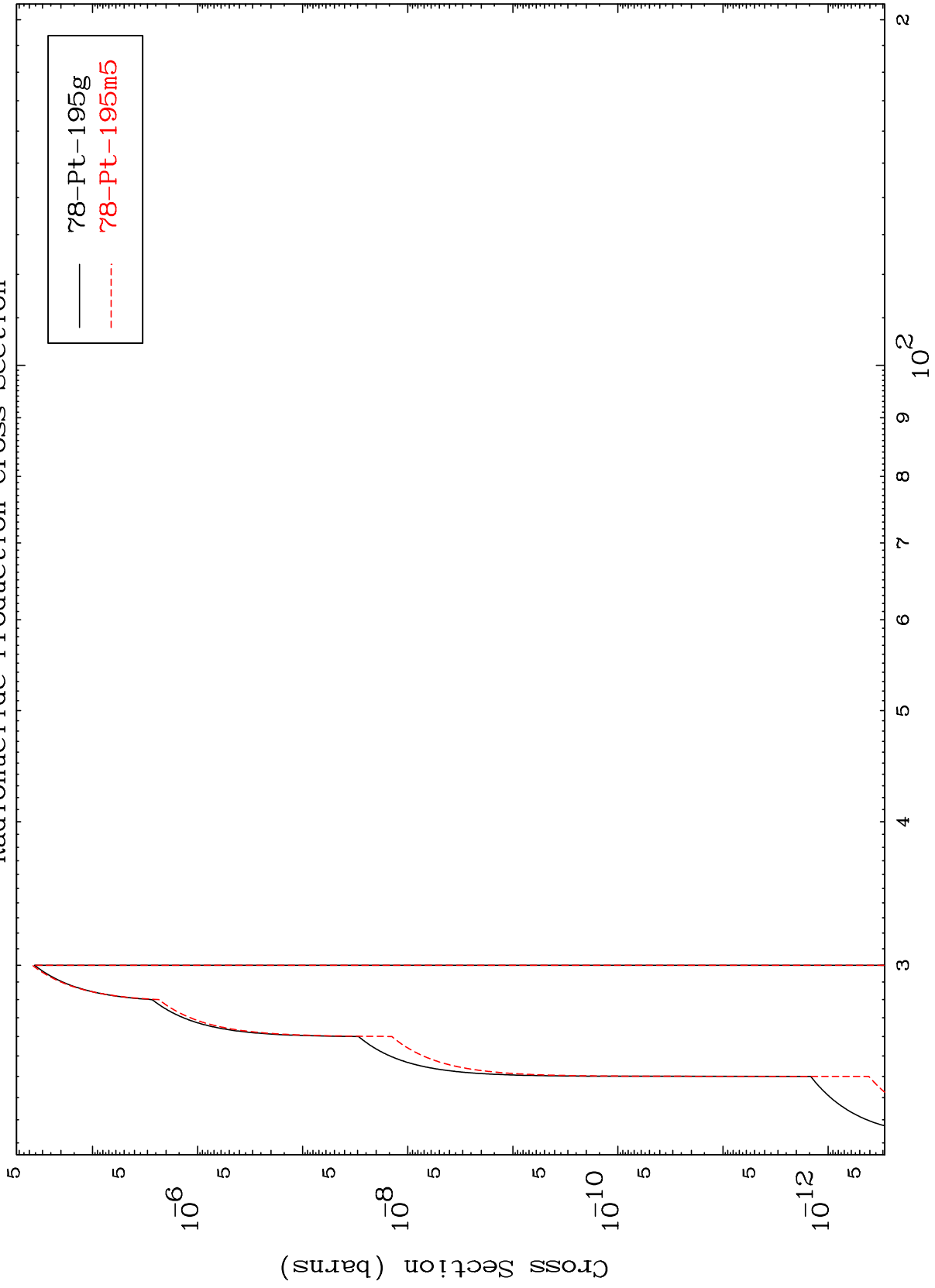
77-Ir-194

MAT 7734

$(\alpha, 2n)$  p

77-Ir-194

Radionuclide Production Cross Section



17

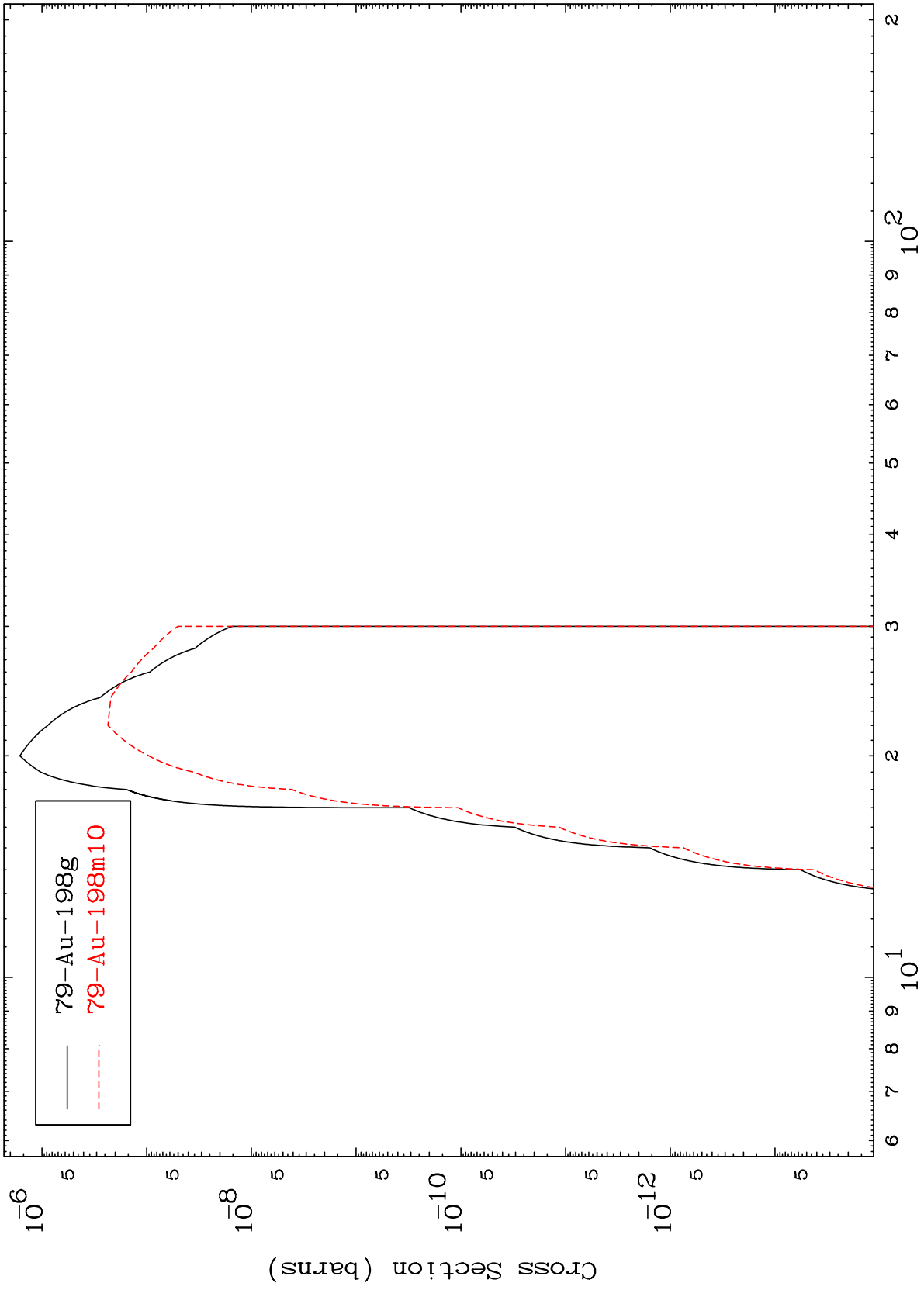
Incident Energy (MeV)

77-Ir-194

MAT 7734

Radionuclide Production Cross Section  
( $\alpha, \gamma$ )

<sup>77</sup>Ir-194

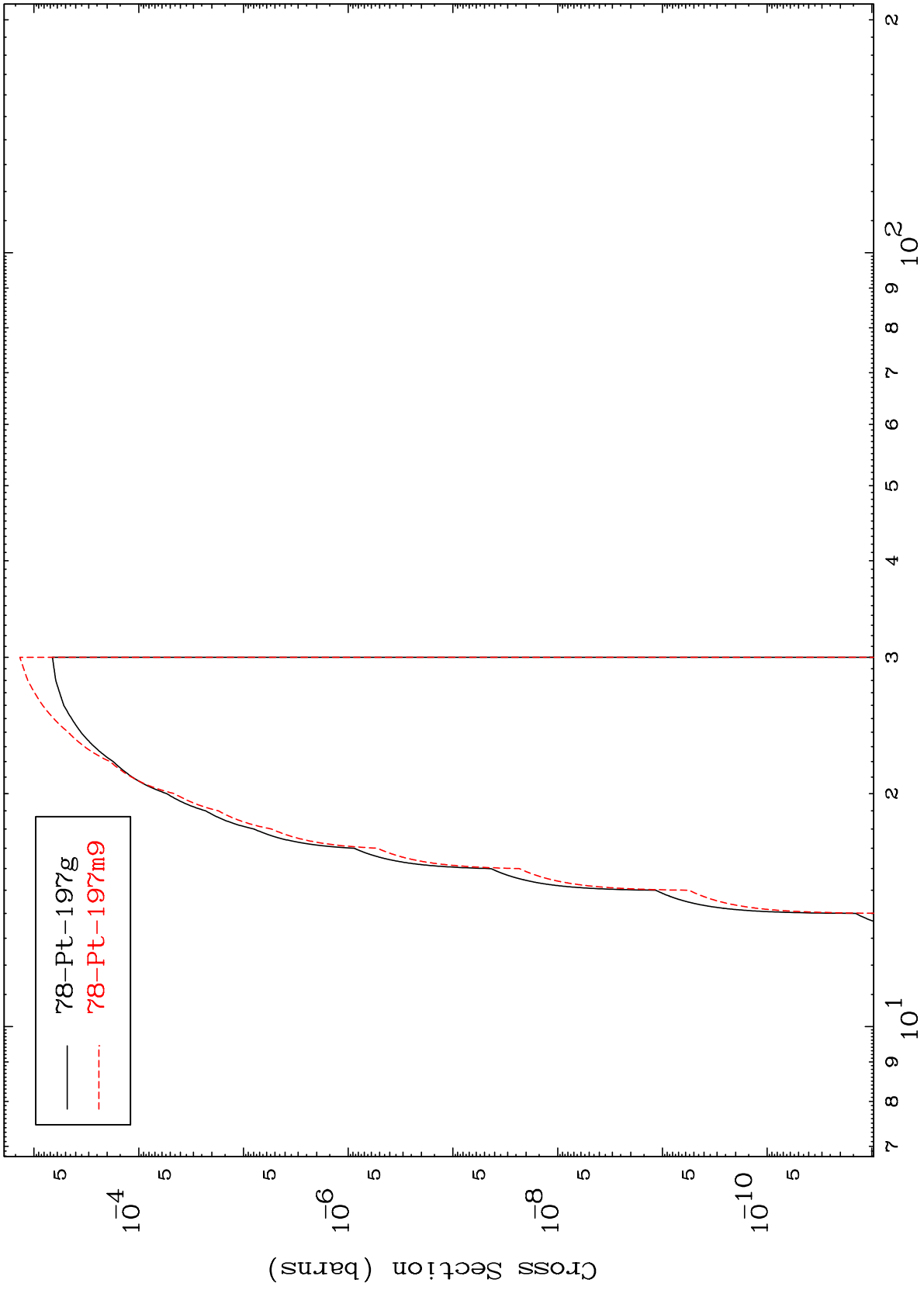


Legend:  
— 79-Au-198g  
- - - 79-Au-198m10

MAT 7734

77-Ir-194

( $\alpha, p$ )  
Radionuclide Production Cross Section



19

Incident Energy (MeV)

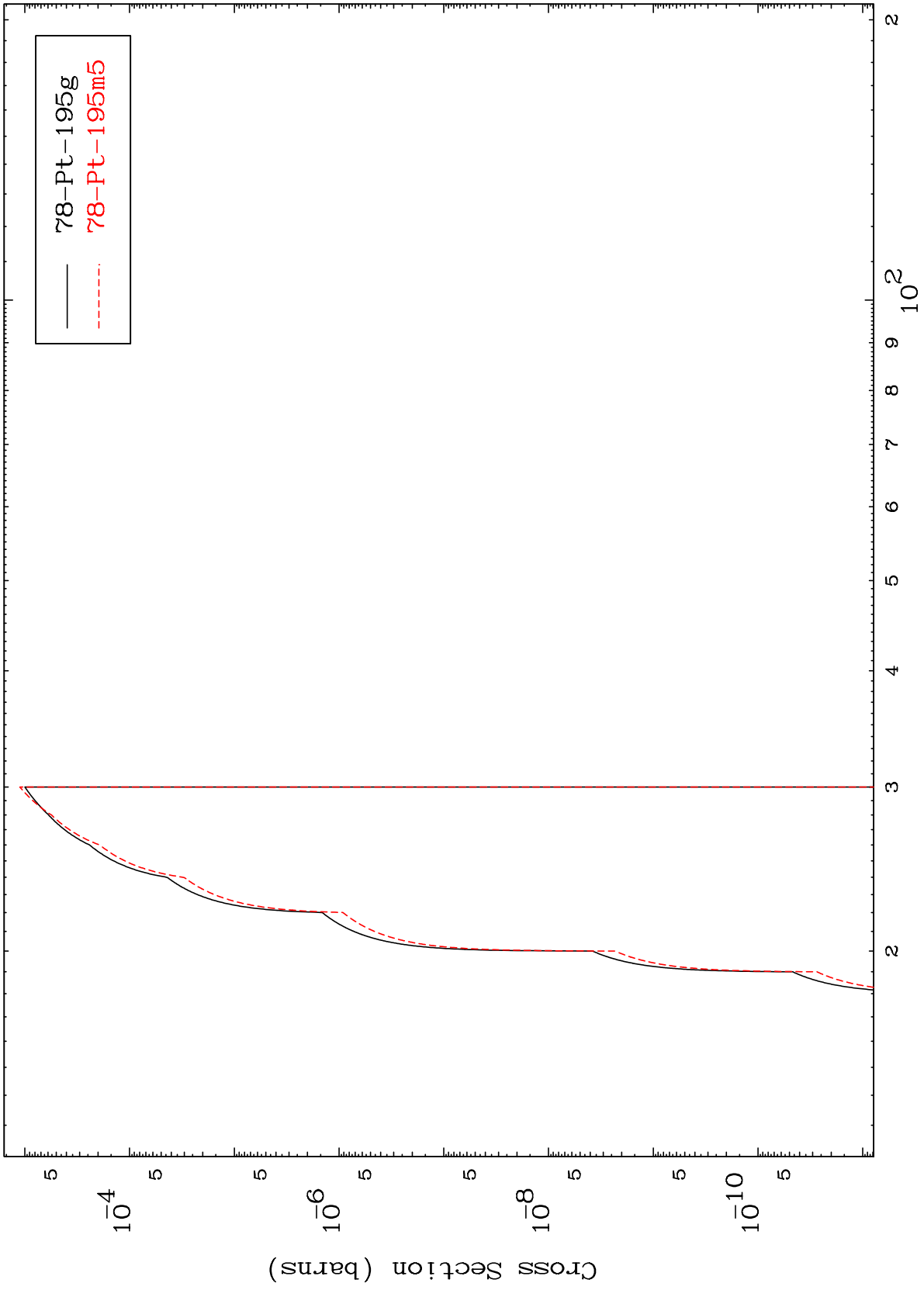
77-Ir-194

MAT 7734

( $\alpha, t$ )

77-Ir-194

Radionuclide Production Cross Section



20

Incident Energy (MeV)

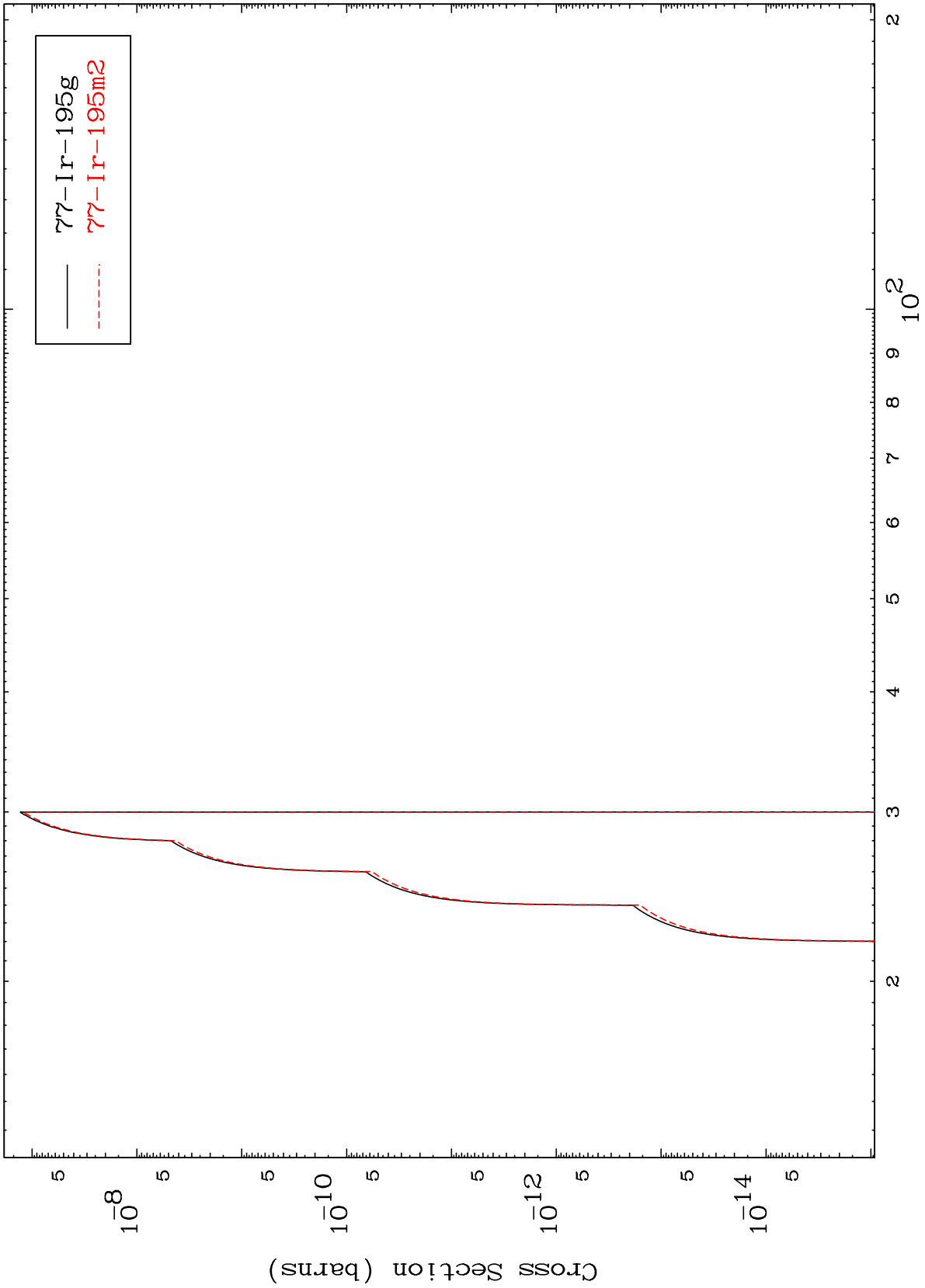
77-Ir-194

MAT 7734

( $\alpha, \text{He-3}$ )

77-Ir-194

Radionuclide Production Cross Section

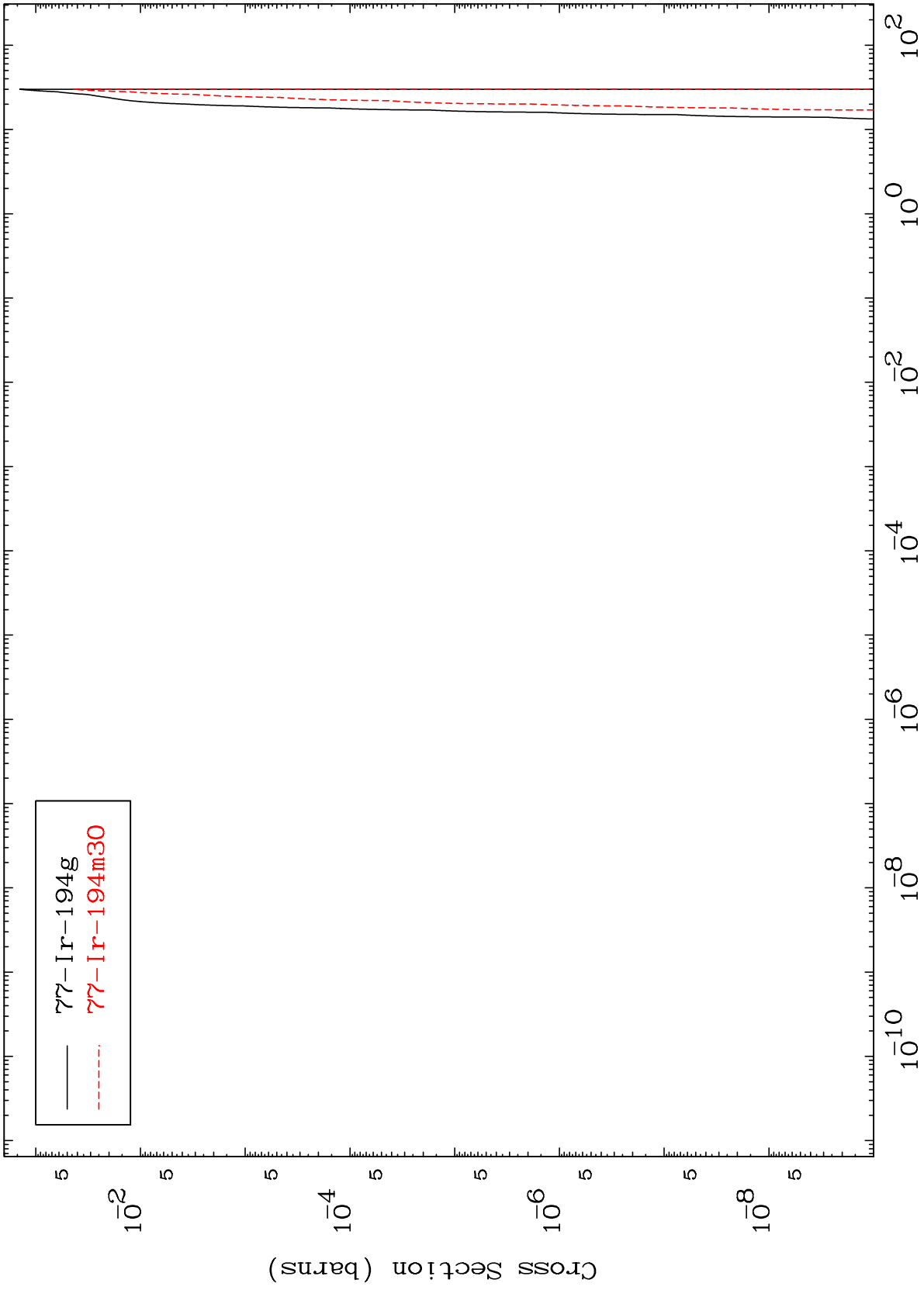


MAT 7734

( $\alpha, \alpha$ )

<sup>77</sup>Ir-194

Radionuclide Production Cross Section



22

Incident Energy (MeV)

<sup>77</sup>Ir-194