

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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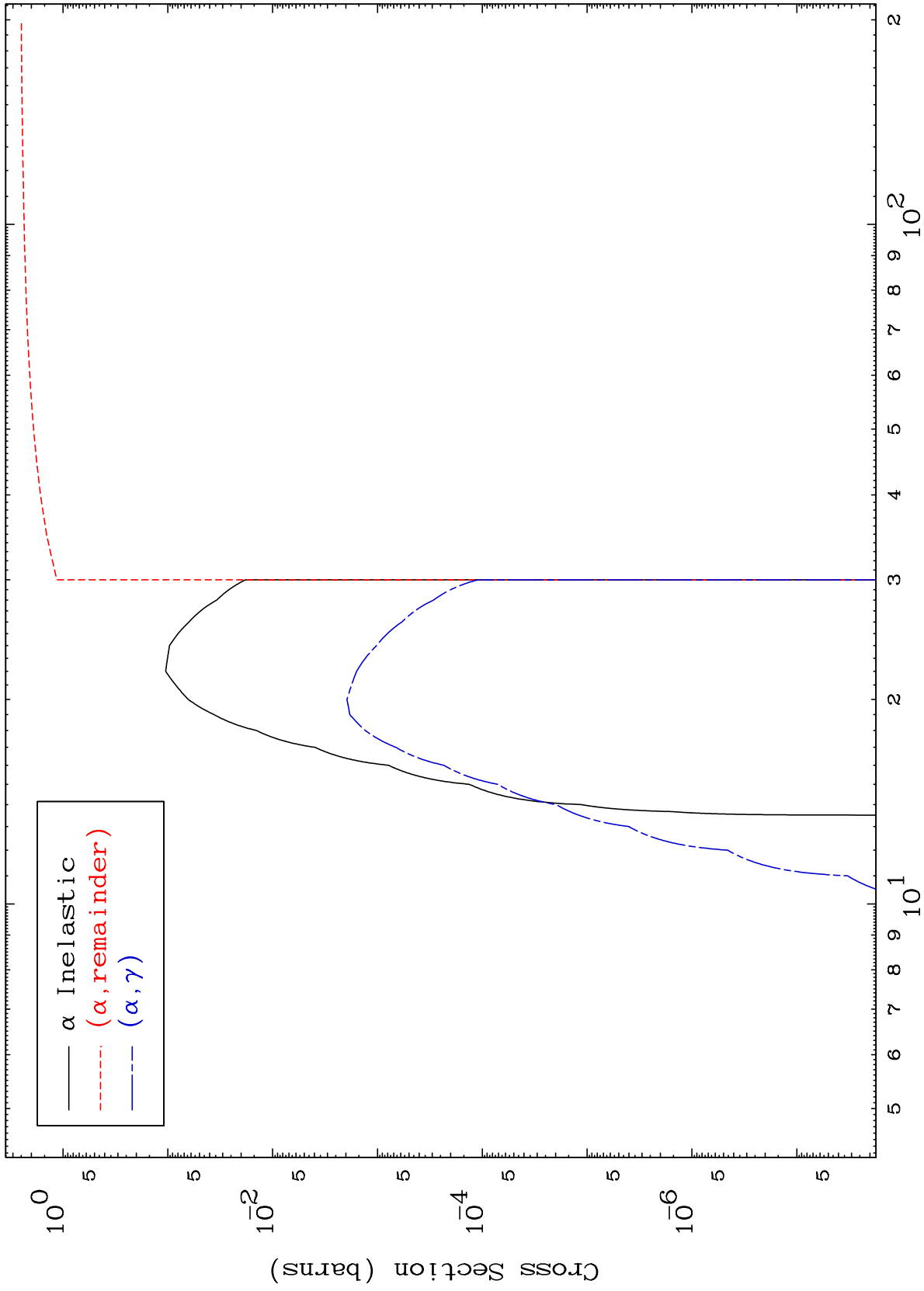
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7081

0 Kelvin  $\alpha$  Major  
Cross Sections

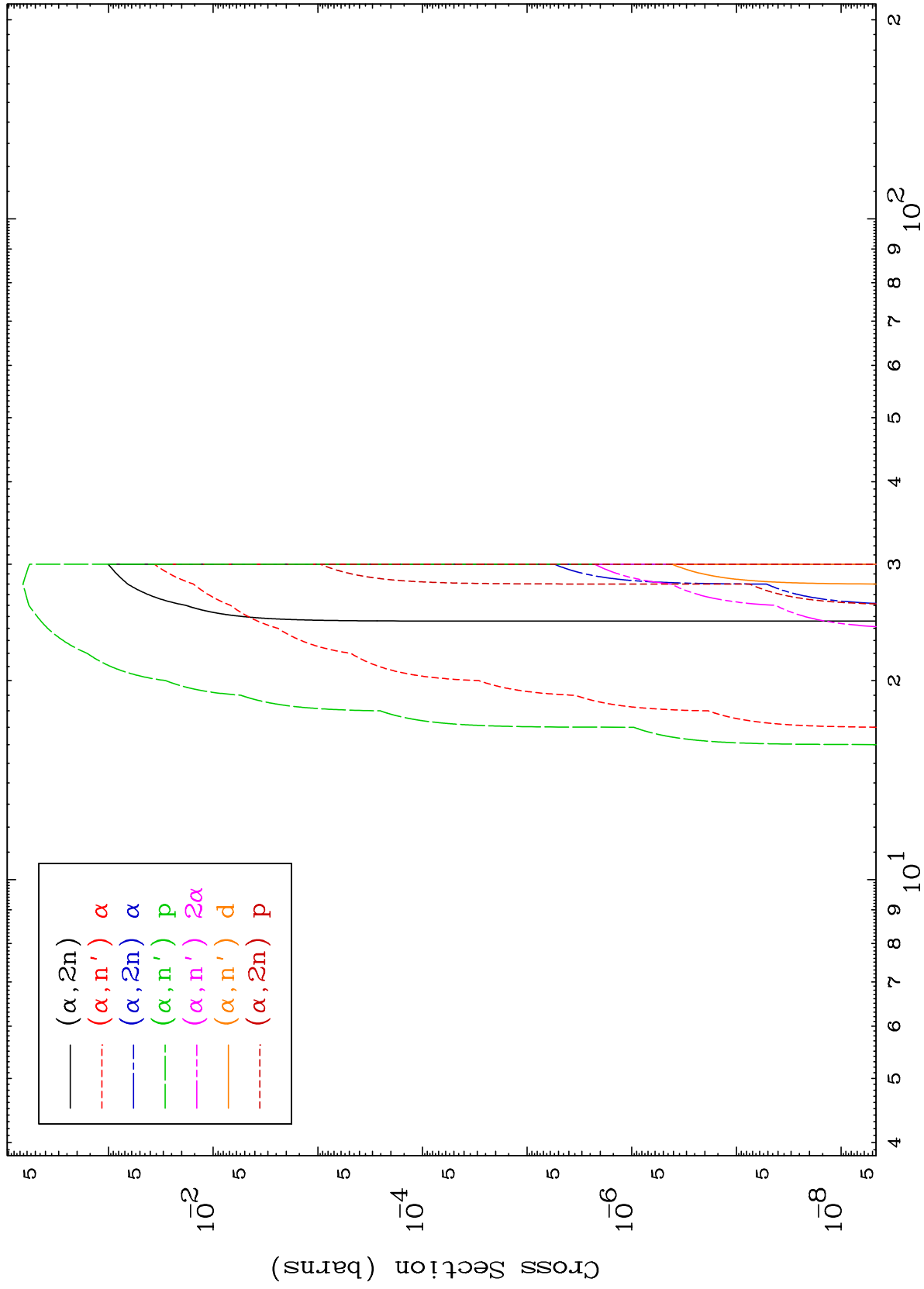
$^{71}\text{Lu-160}$



MAT 7081

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

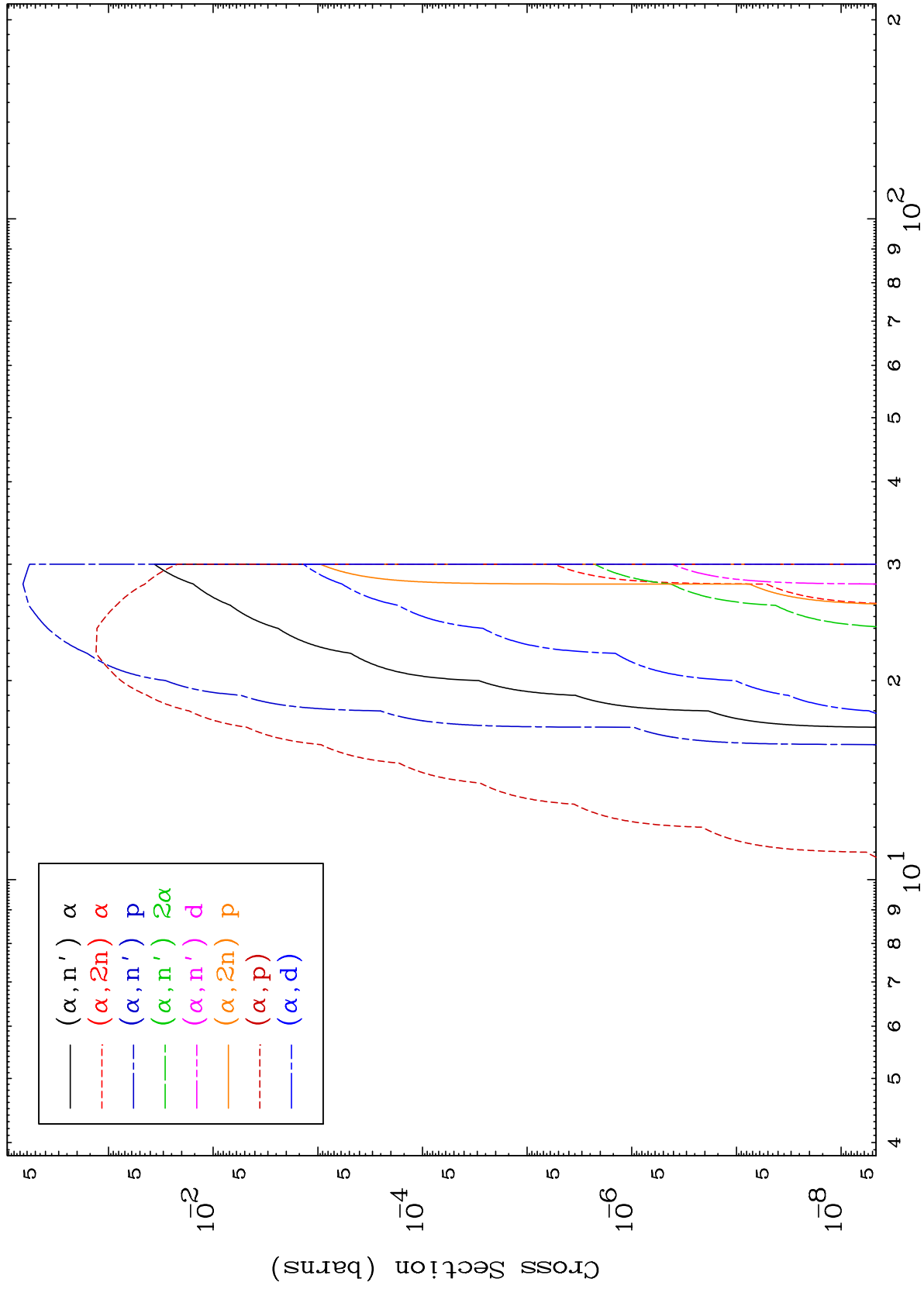
71-Lu-160



2

Incident Energy (MeV)

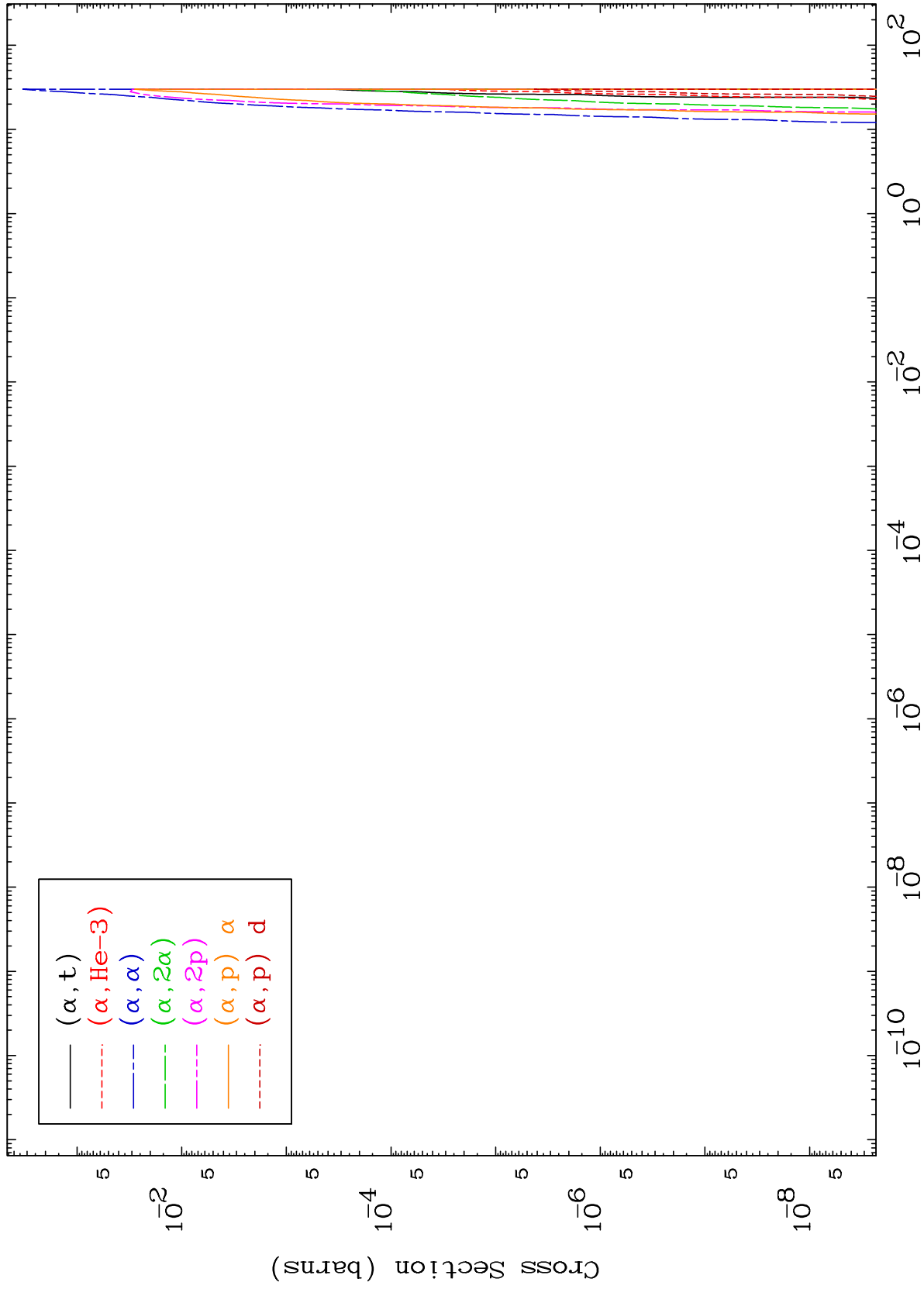
71-Lu-160



MAT 7081

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

71-Lu-160

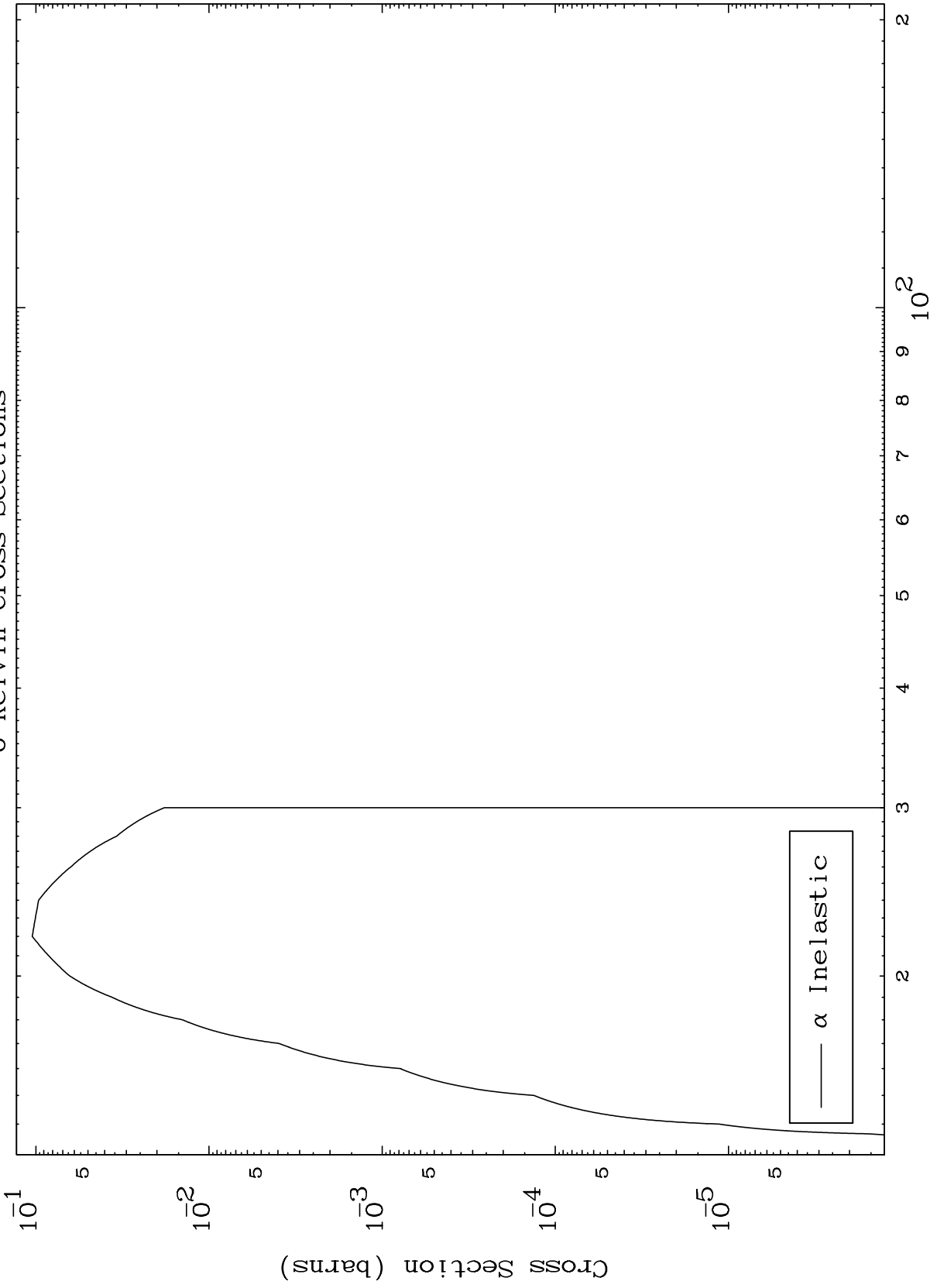


71-Lu-160

MAT 7081

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

71-Lu-160



5

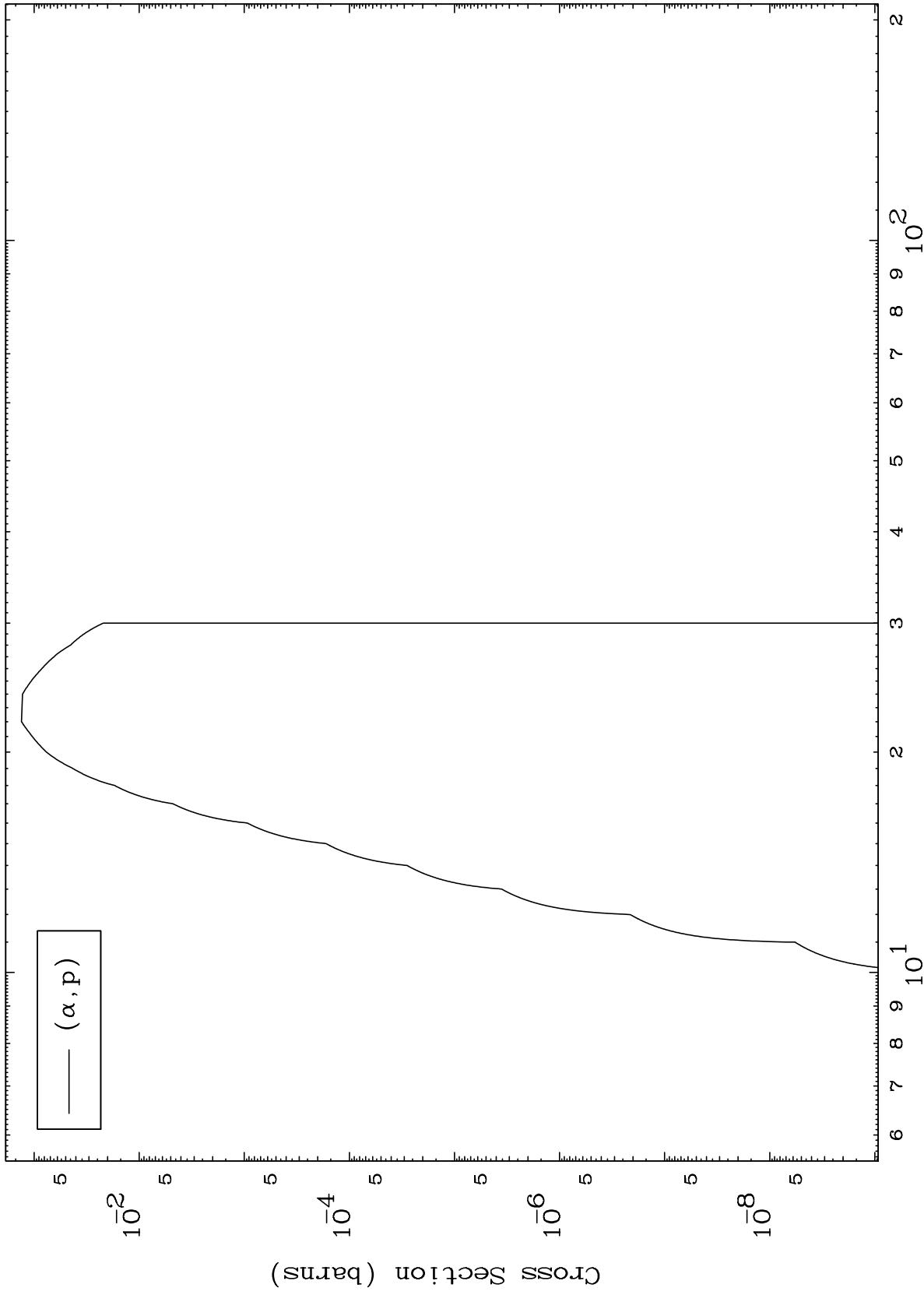
Incident Energy (MeV)

71-Lu-160

MAT 7081

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

71-Lu-160



6

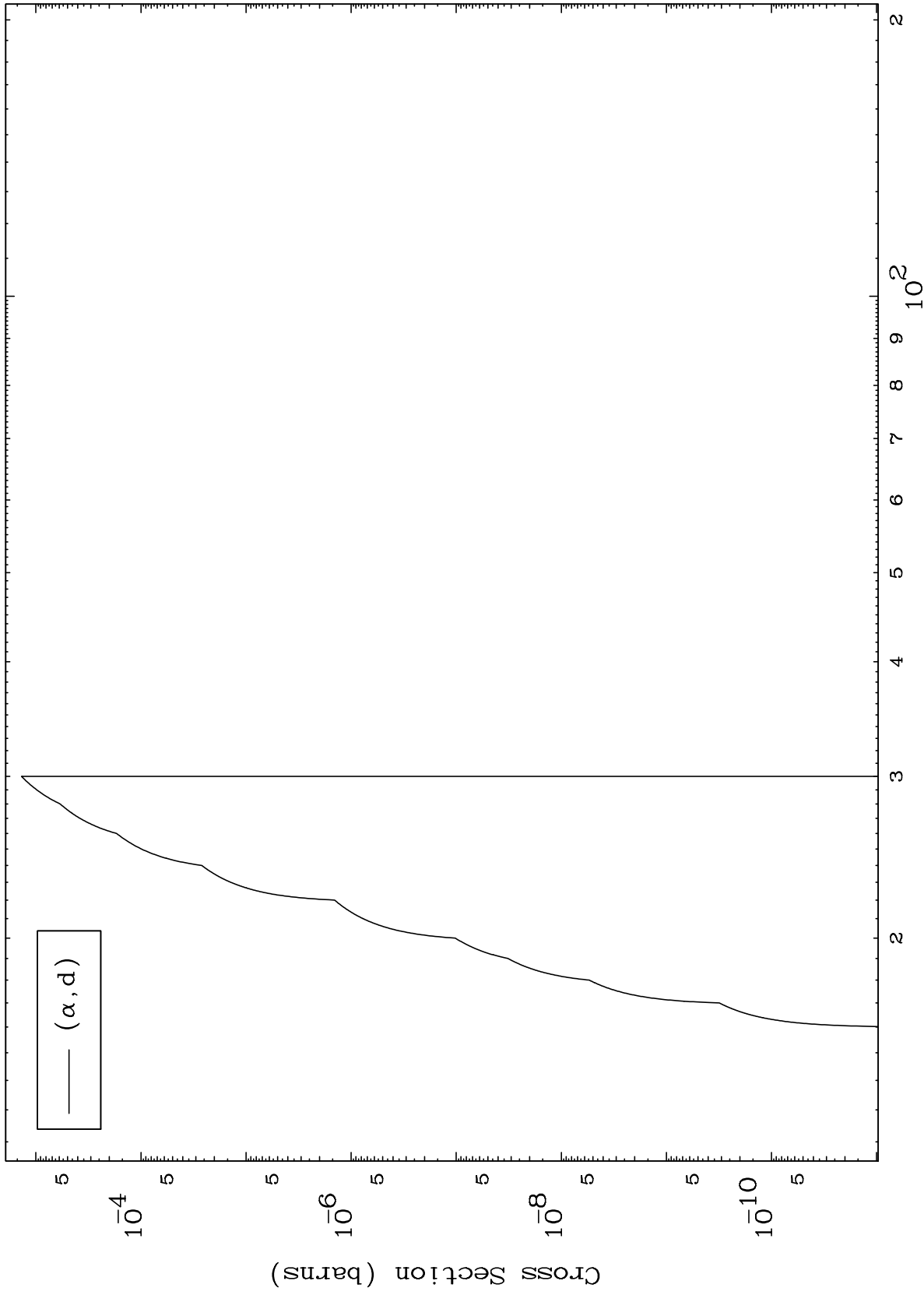
Incident Energy (MeV)

71-Lu-160

MAT 7081

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

71-Lu-160



7

Incident Energy (MeV)

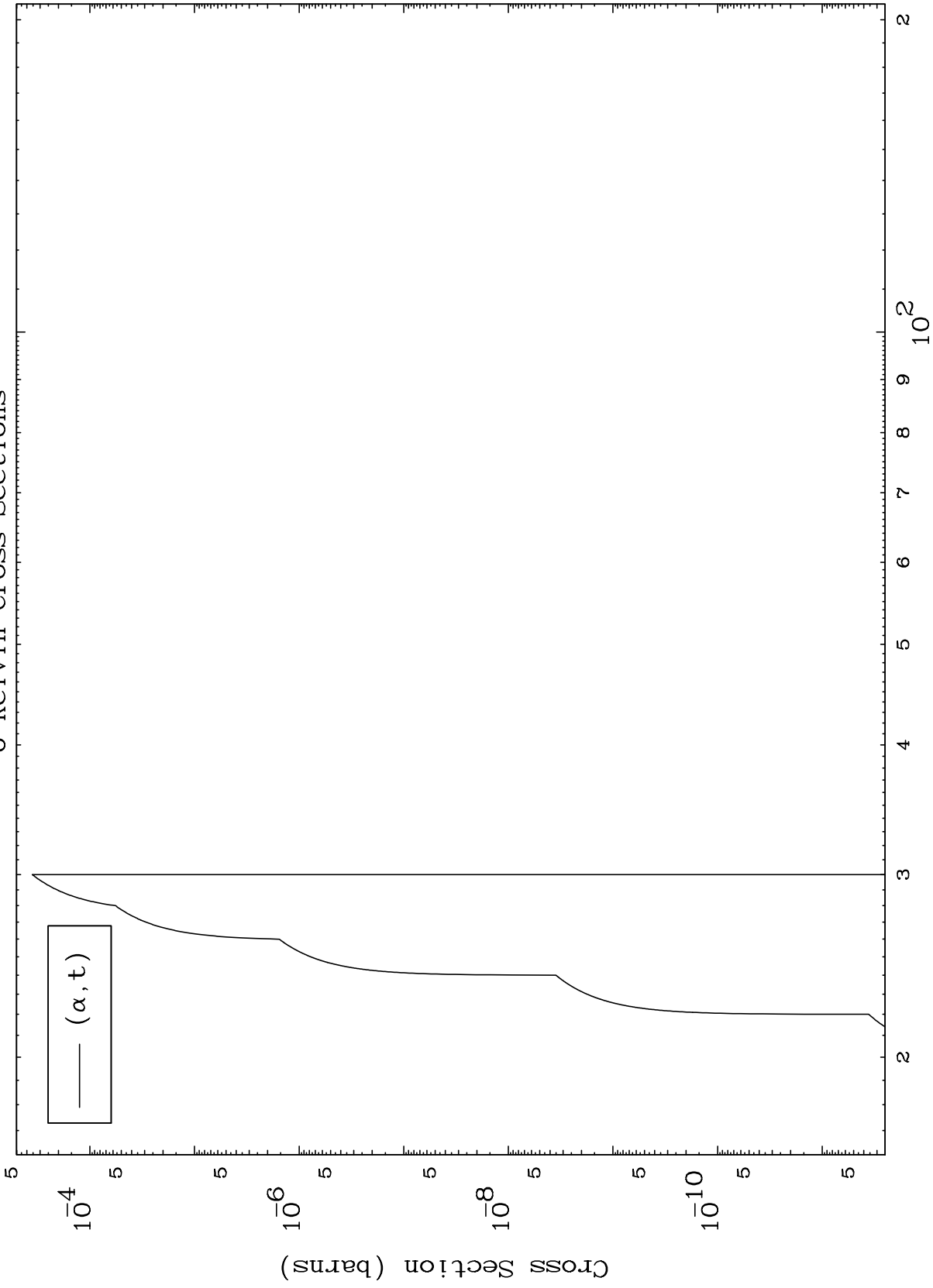
71-Lu-160



MAT 7081

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

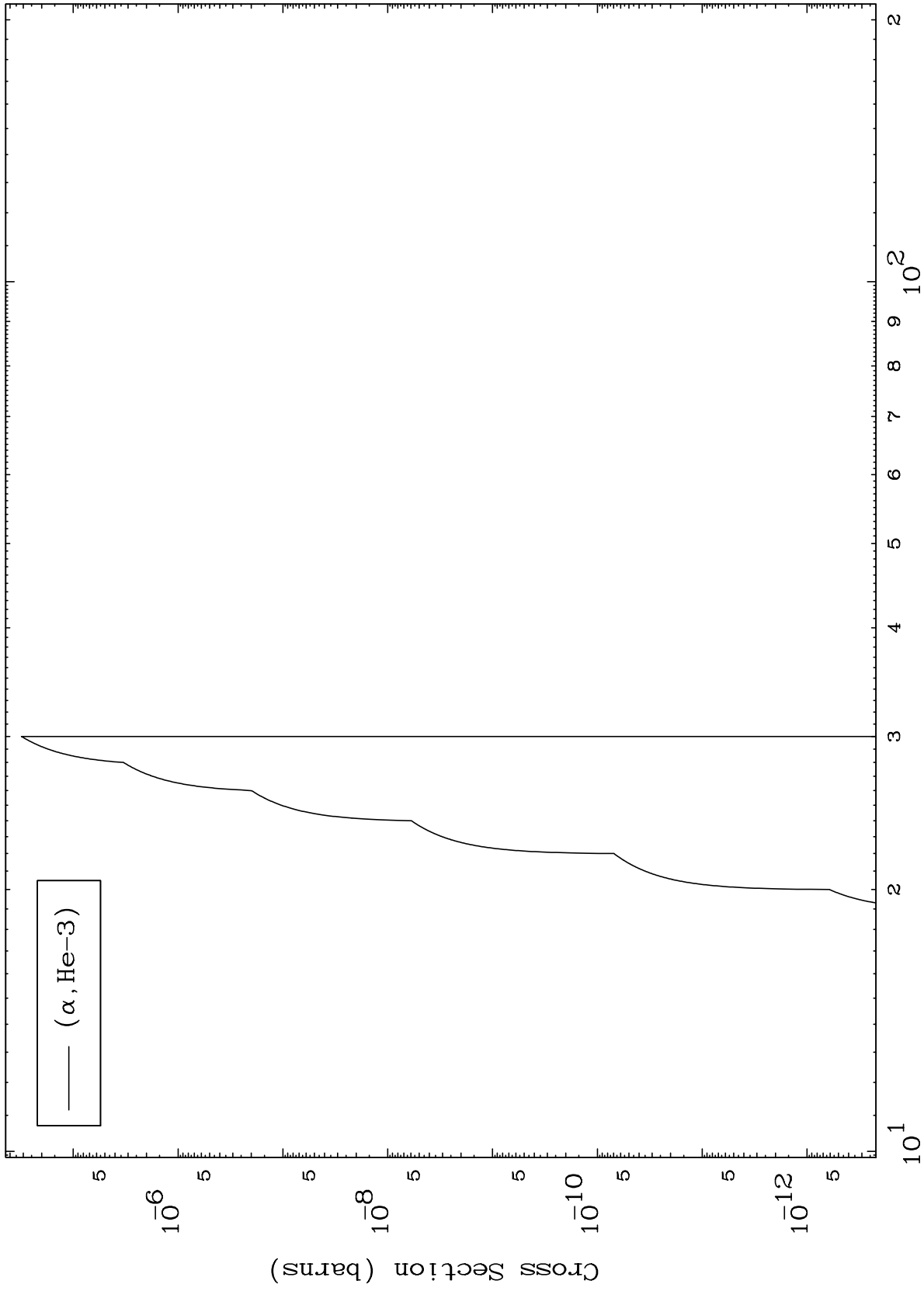
71-Lu-160



MAT 7081

( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections

71-Lu-160



9

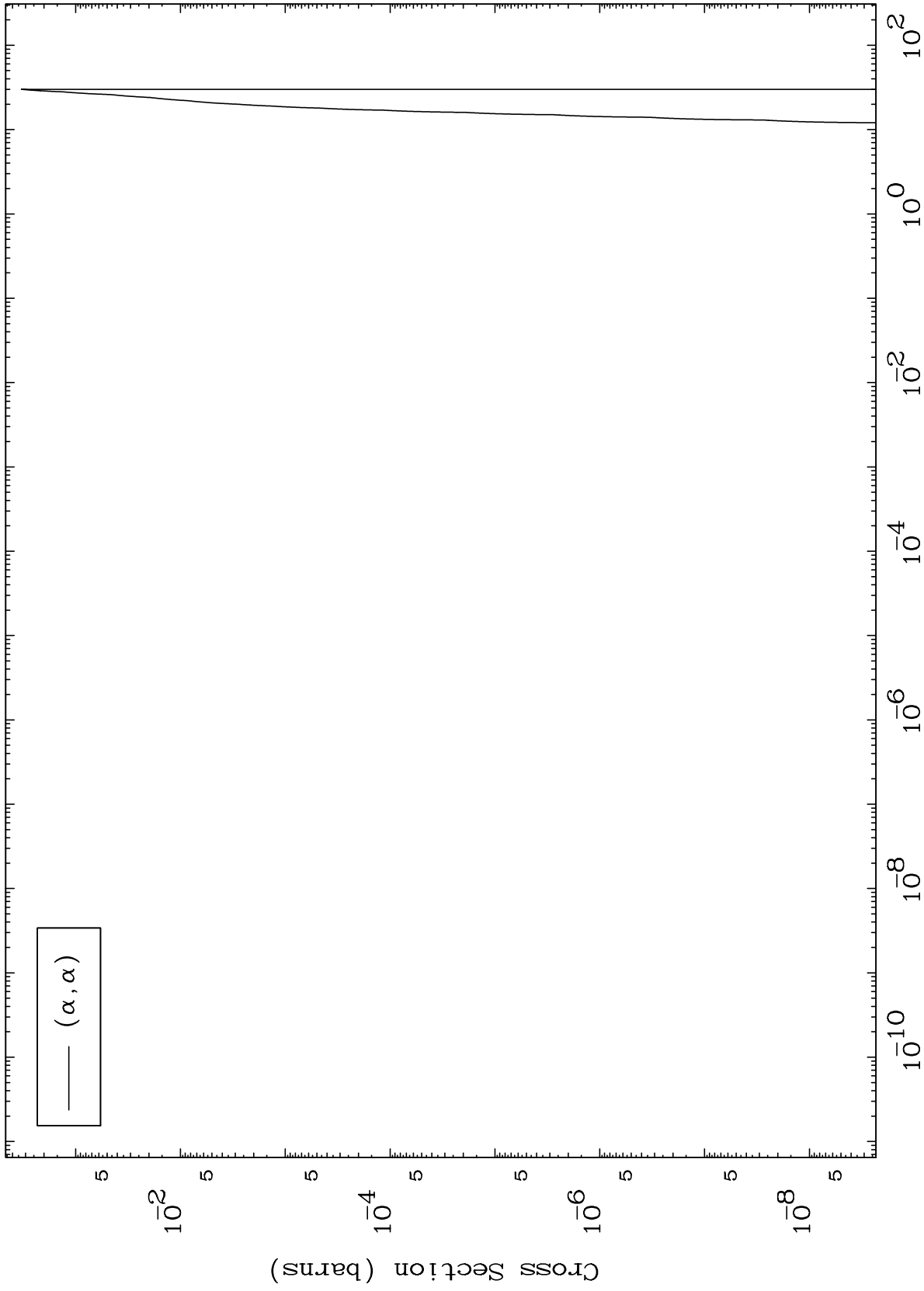
Incident Energy (MeV)

71-Lu-160

MAT 7081

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

71-Lu-160



10

Incident Energy (MeV)

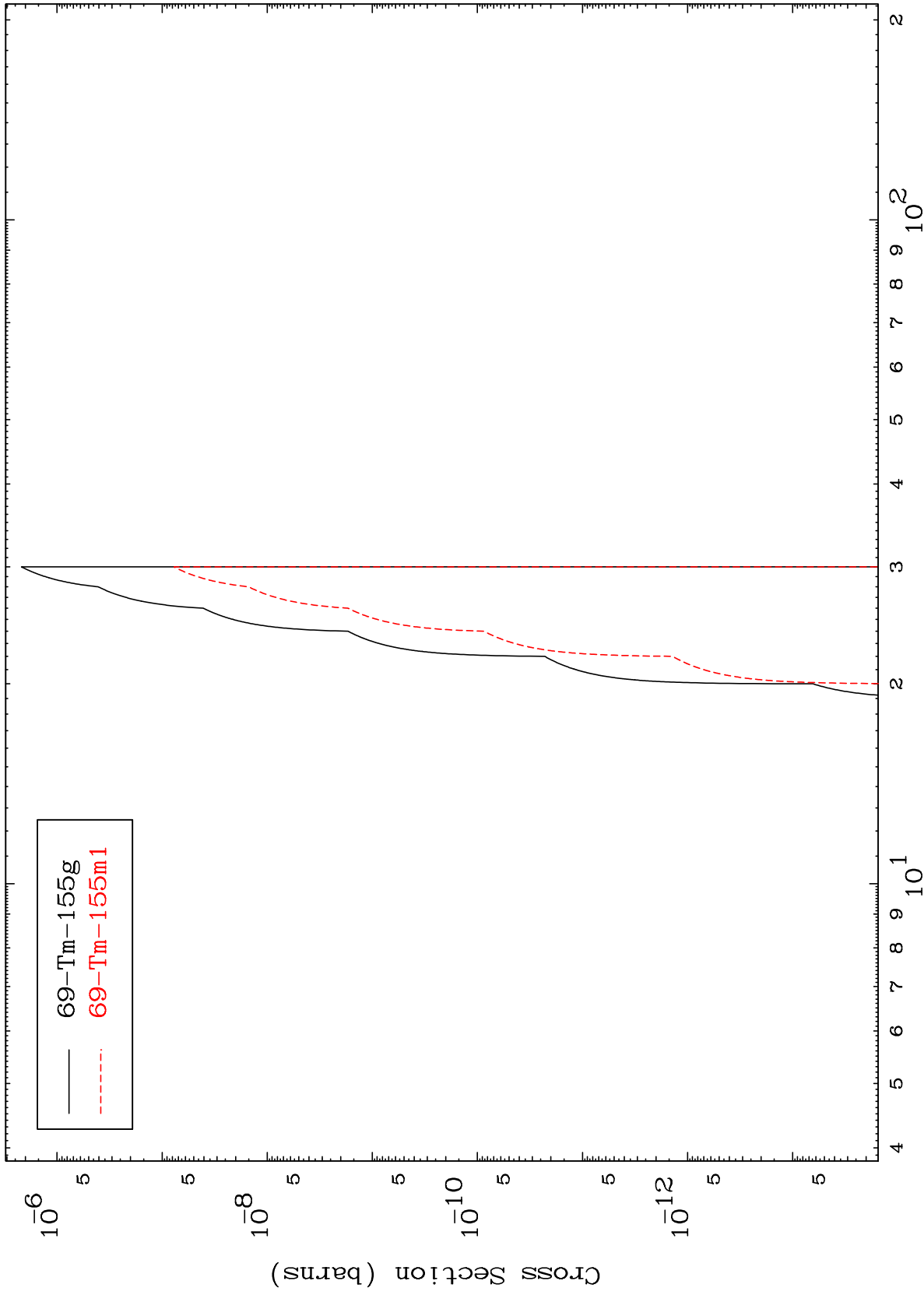
71-Lu-160

MAT 7081

( $\alpha, n'$ )  $2\alpha$

71-Lu-160

Radionuclide Production Cross Section



11

Incident Energy (MeV)

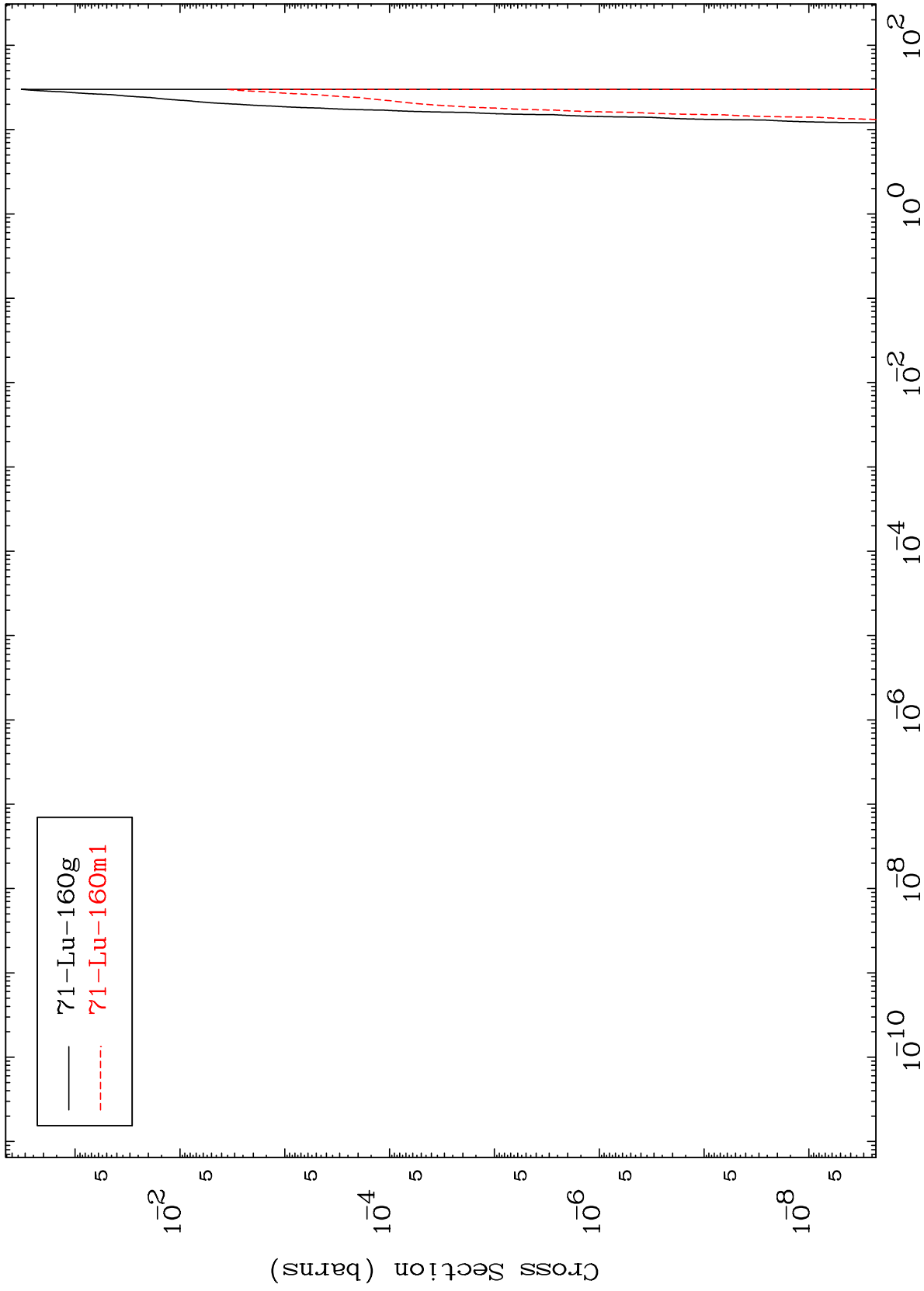
71-Lu-160

MAT 7081

( $\alpha, \alpha$ )

<sup>71</sup>Lu-160

Radionuclide Production Cross Section



12

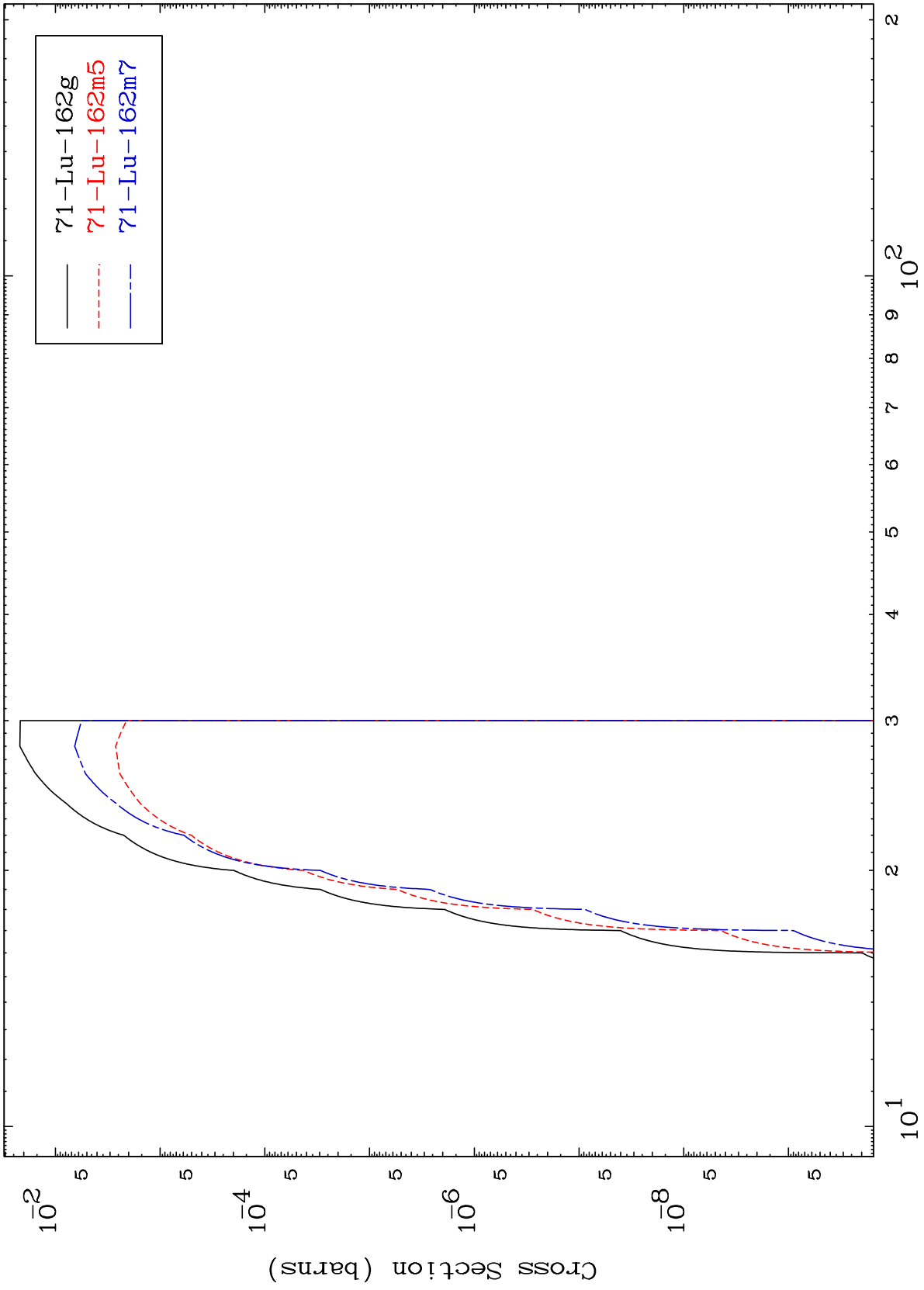
Incident Energy (MeV)

<sup>71</sup>Lu-160

MAT 7081

71-Lu-160

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



71-Lu-160

Incident Energy (MeV)

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