

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

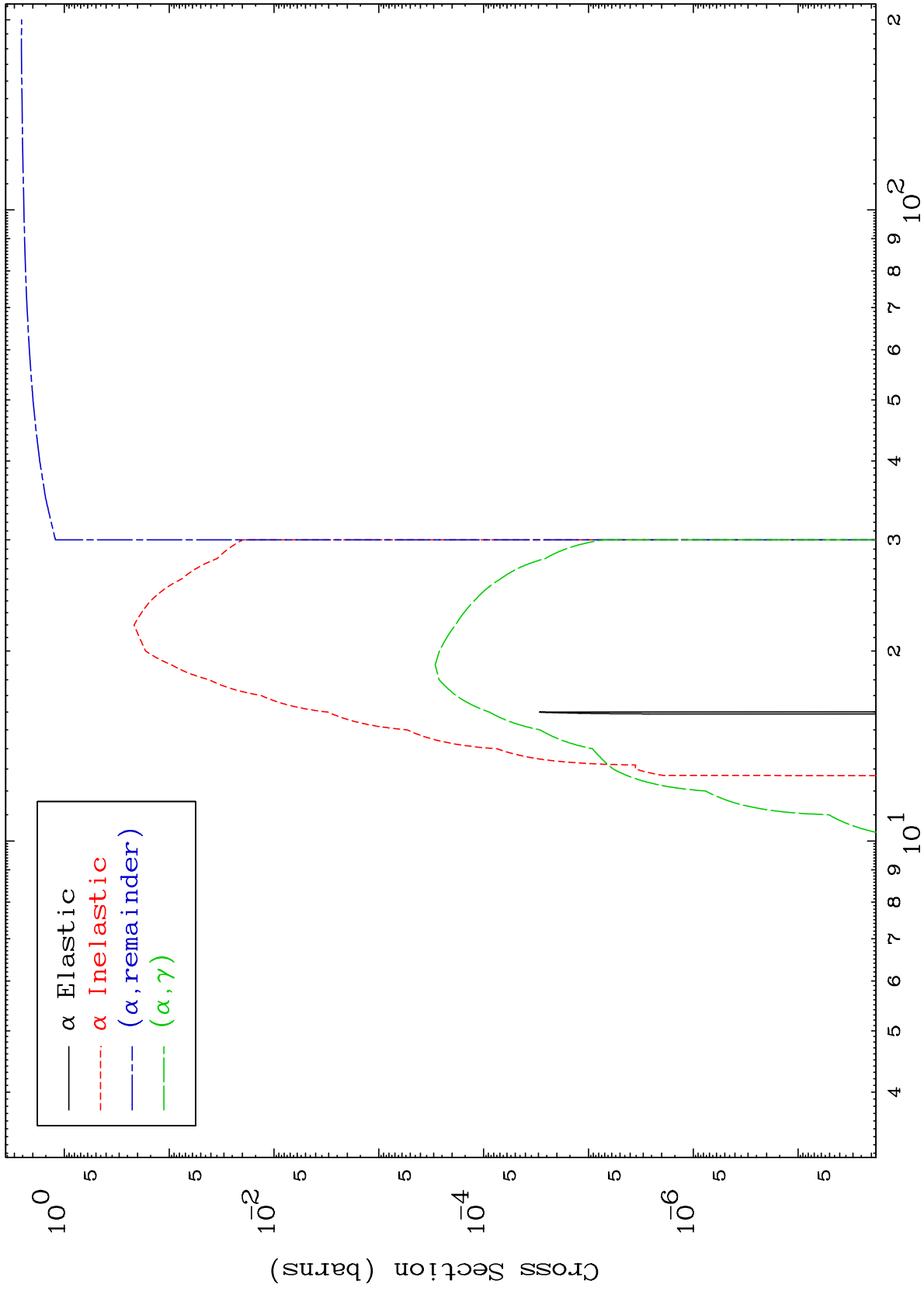
Press Mouse Button to Start

MAT 7107

0 Kelvin

α Major
Cross Sections

71-Lu-169



1

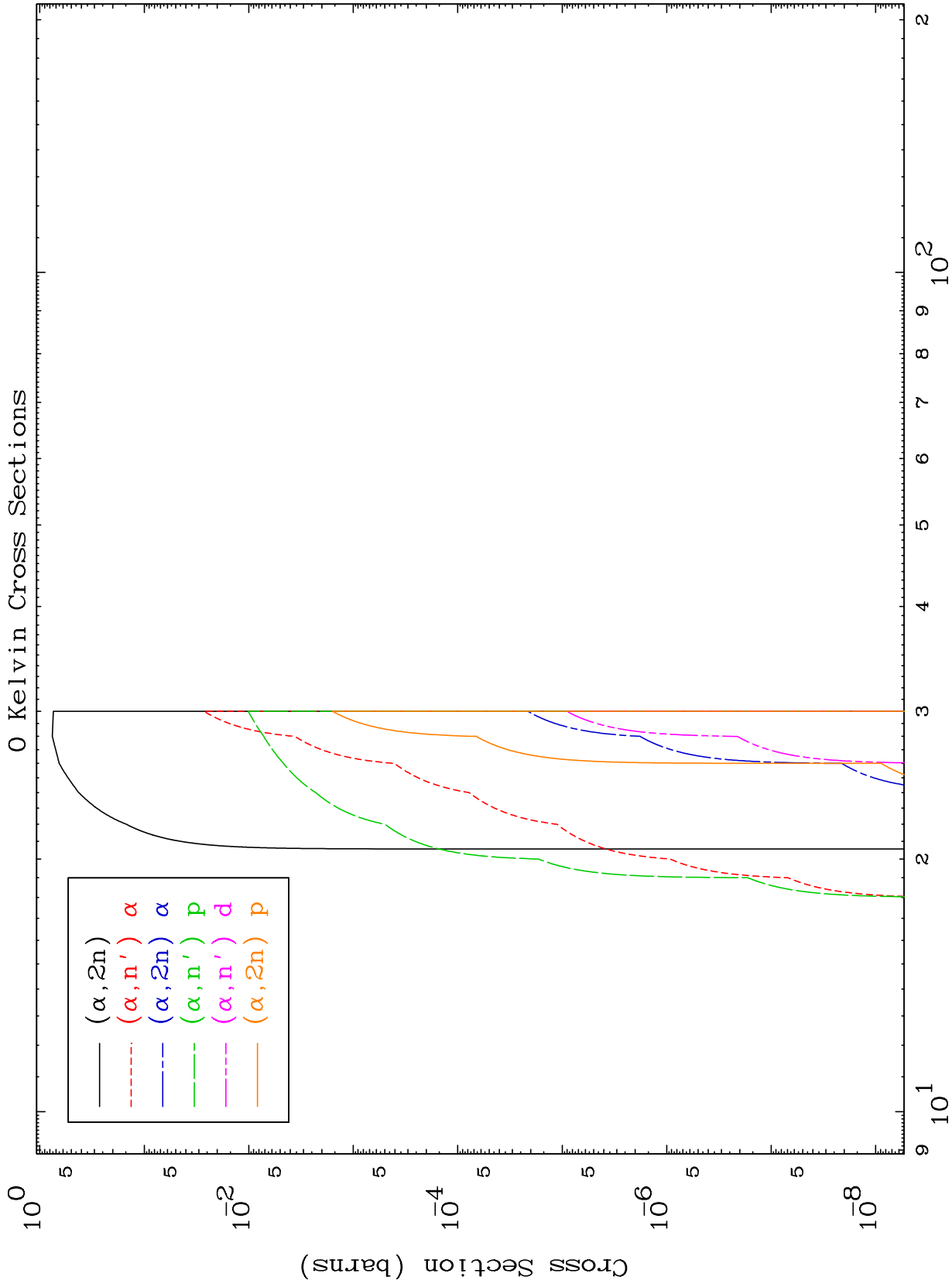
Incident Energy (MeV)

71-Lu-169

MAT 7107

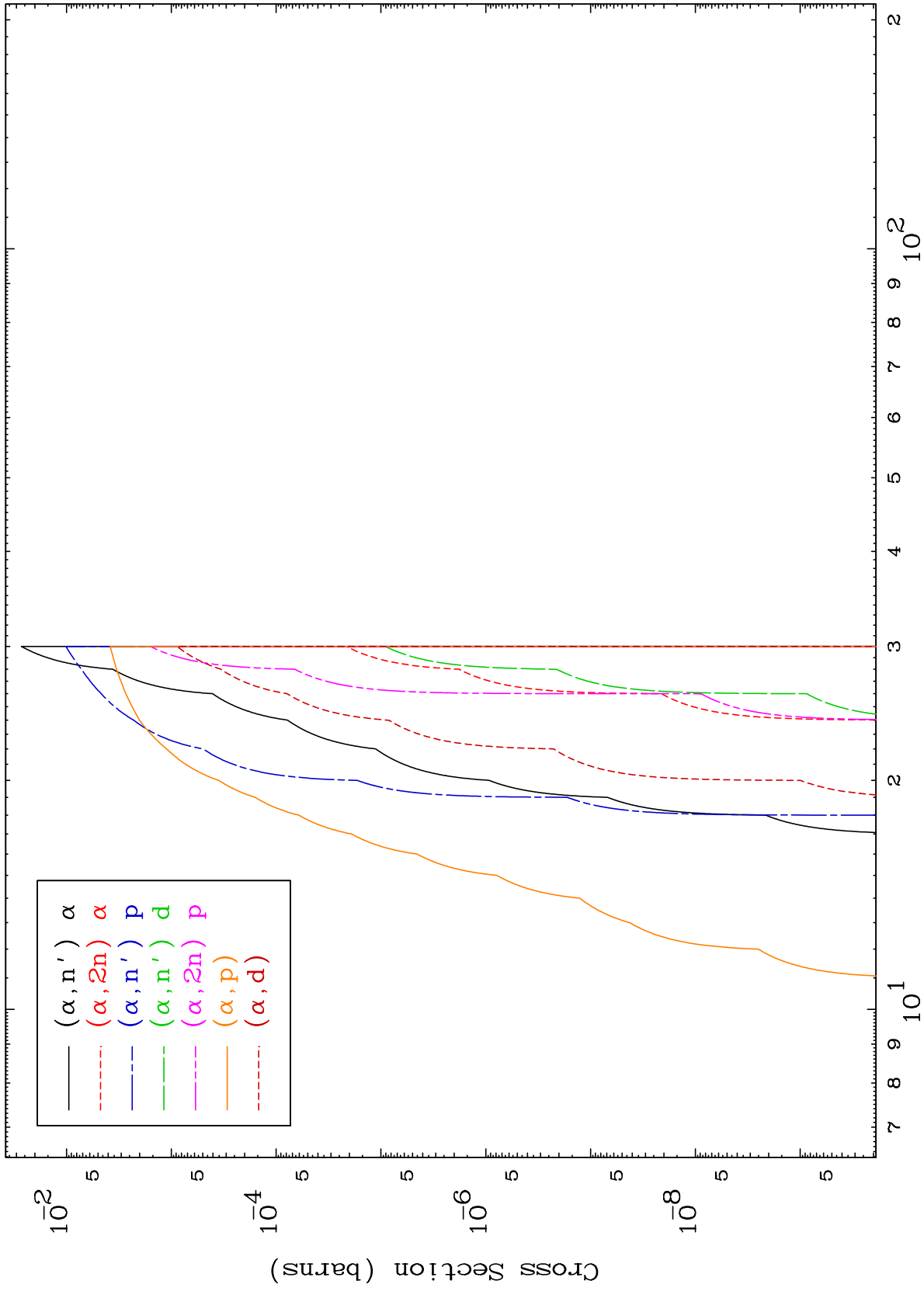
α Neutron Production
0 Kelvin Cross Sections

71-Lu-169



Incident Energy (MeV)

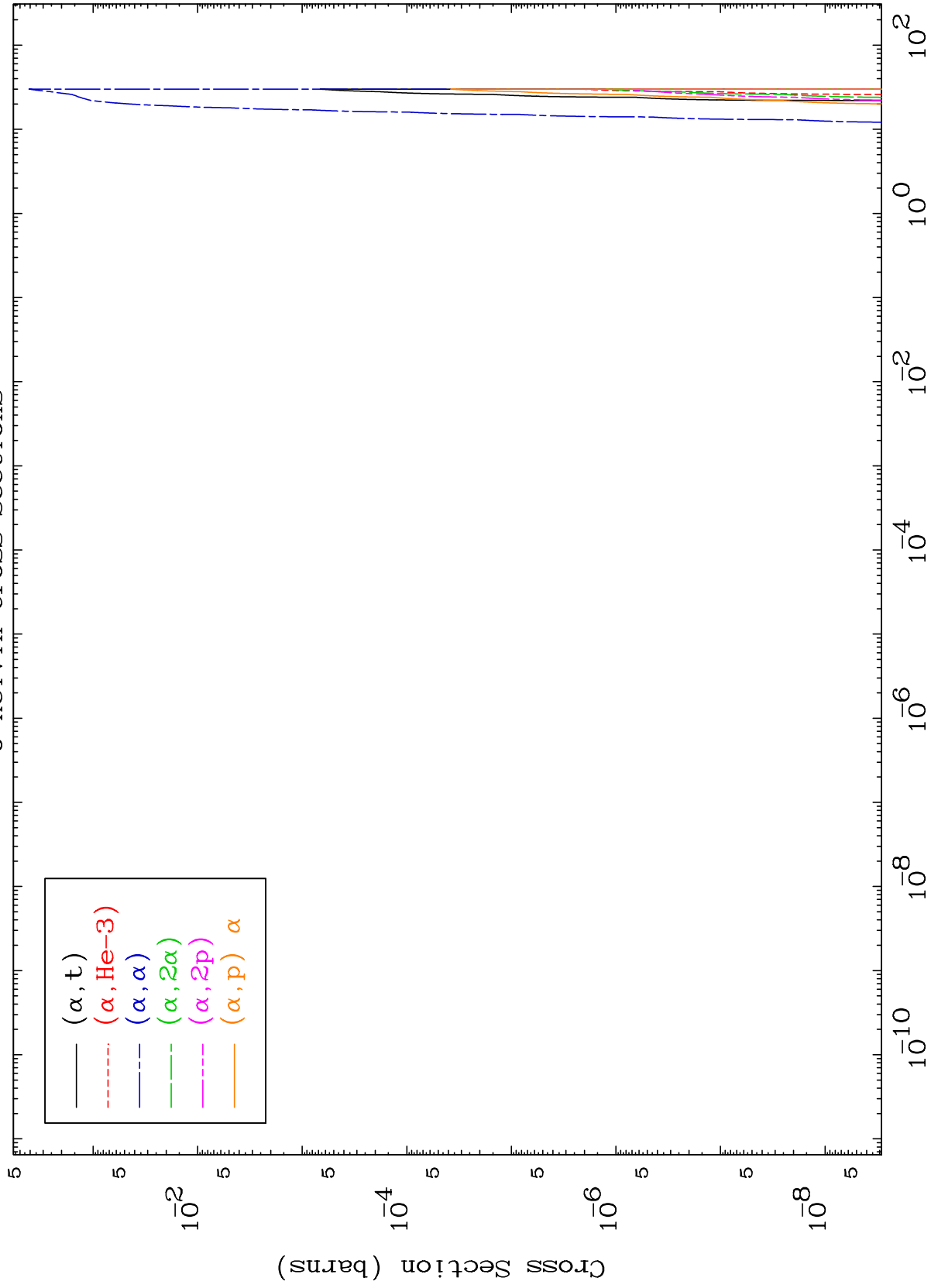
71-Lu-169

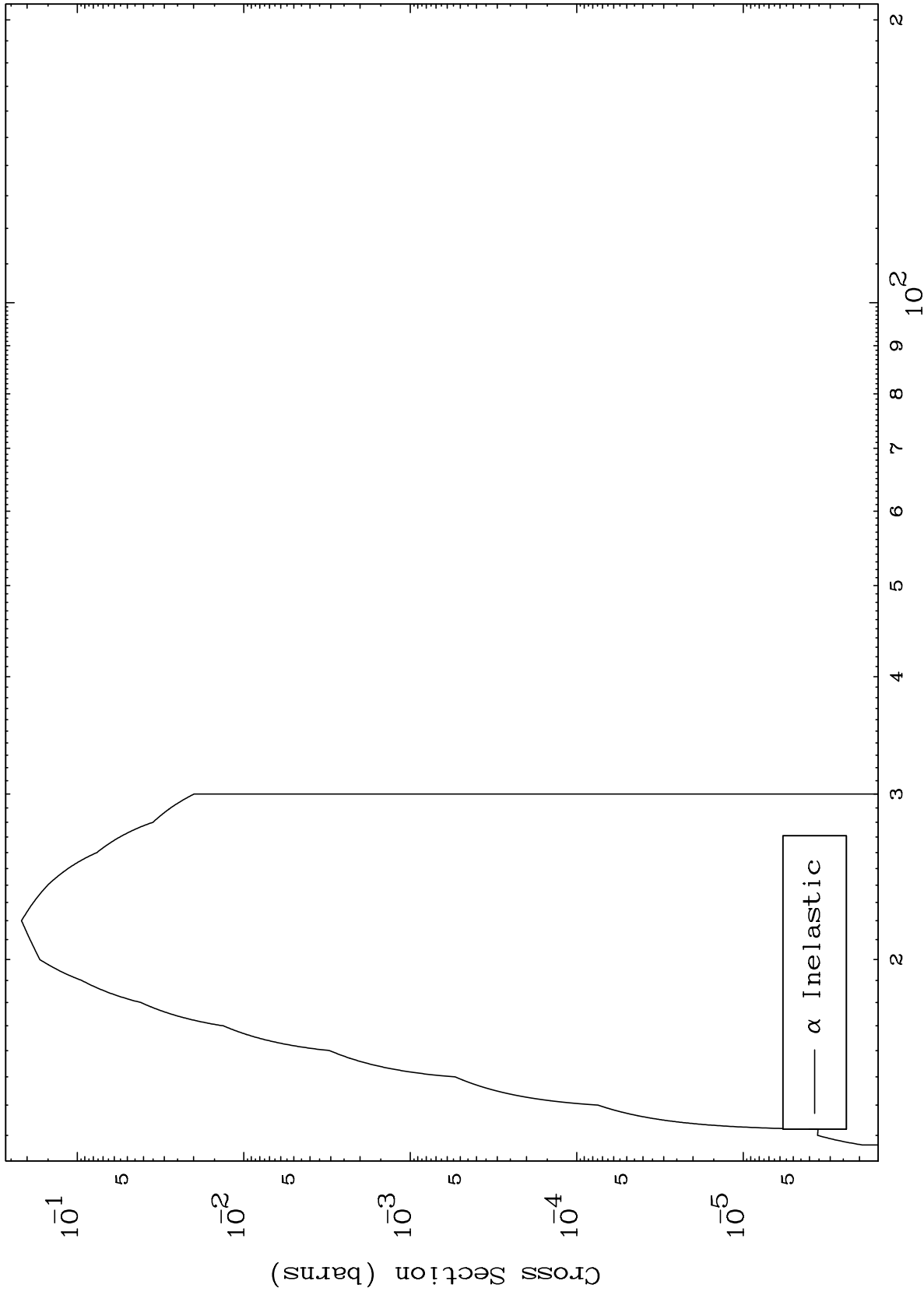


MAT 7107

α Charged Particle
0 Kelvin Cross Sections

71-Lu-169

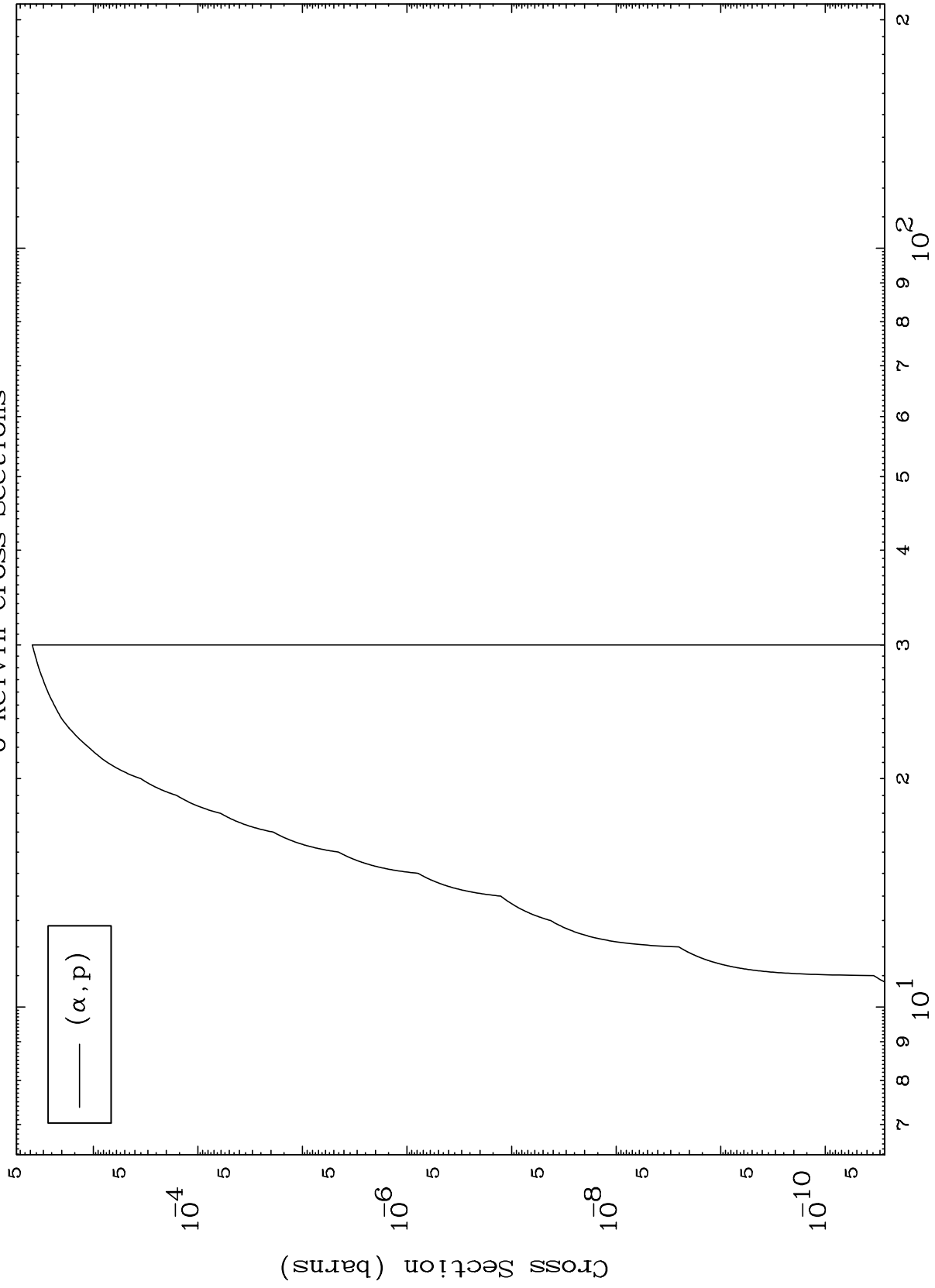




MAT 7107

(α, p) Levels
0 Kelvin Cross Sections

71-Lu-169



6

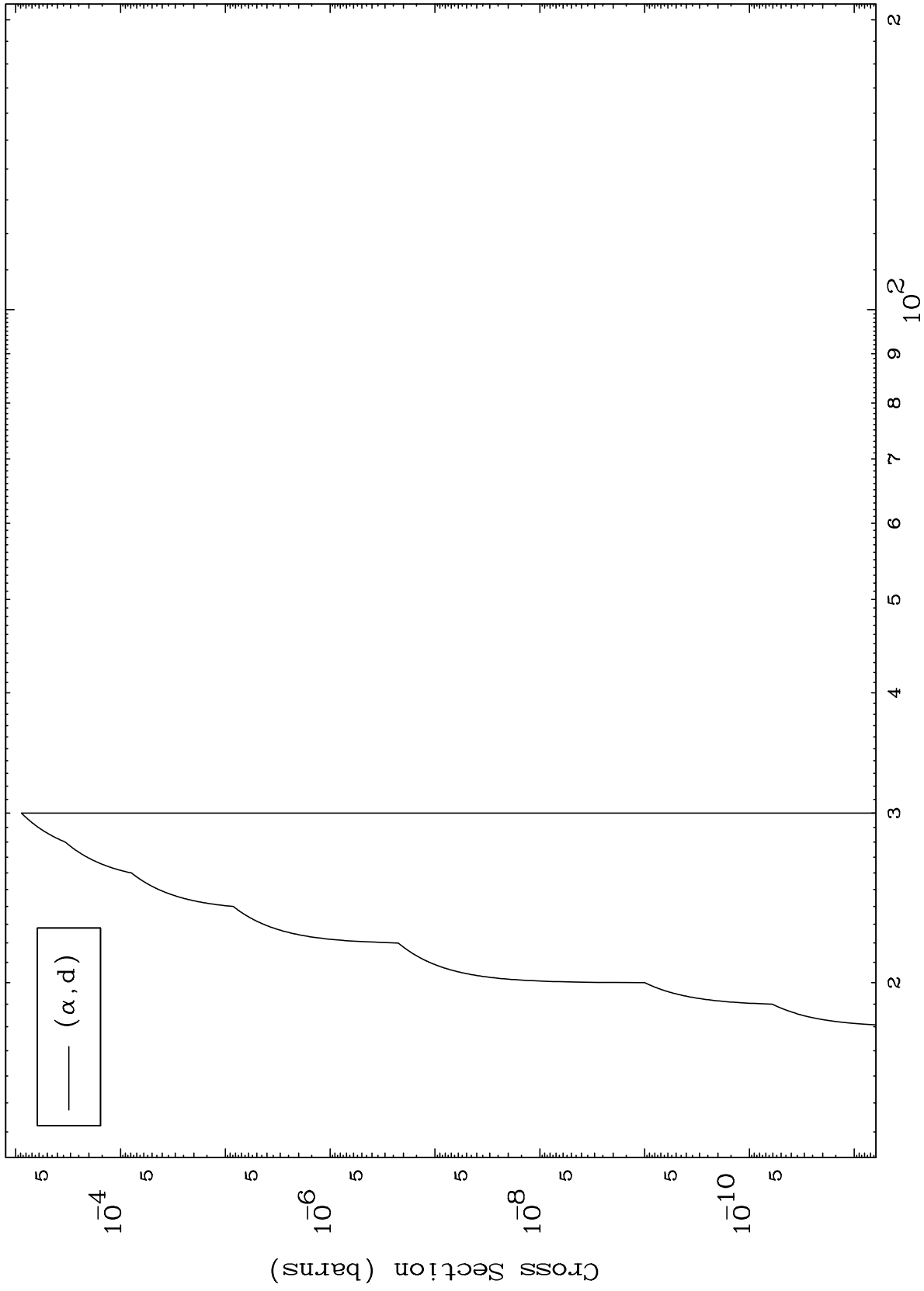
Incident Energy (MeV)

71-Lu-169

MAT 7107

(α, d) Levels
0 Kelvin Cross Sections

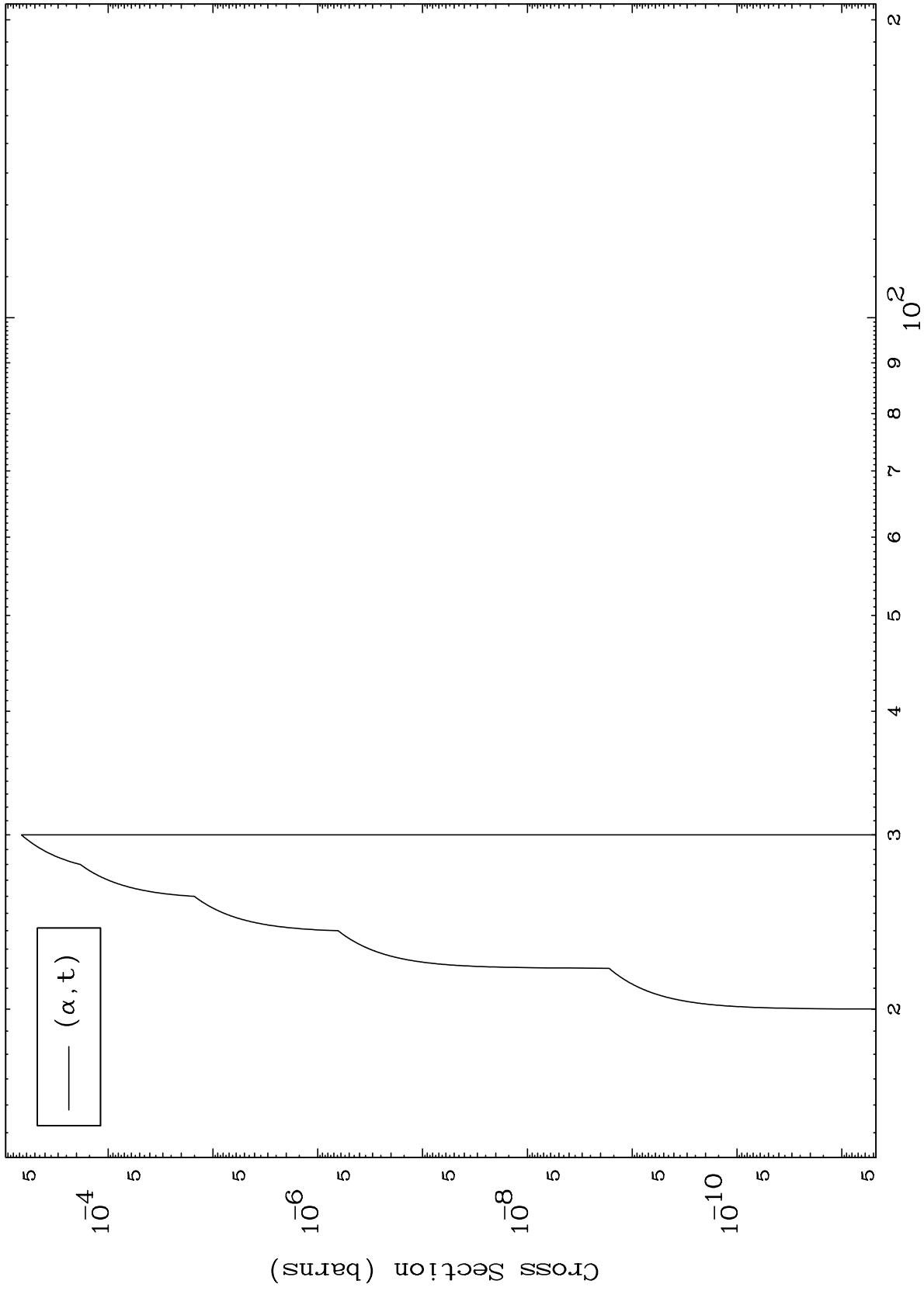
71-Lu-169

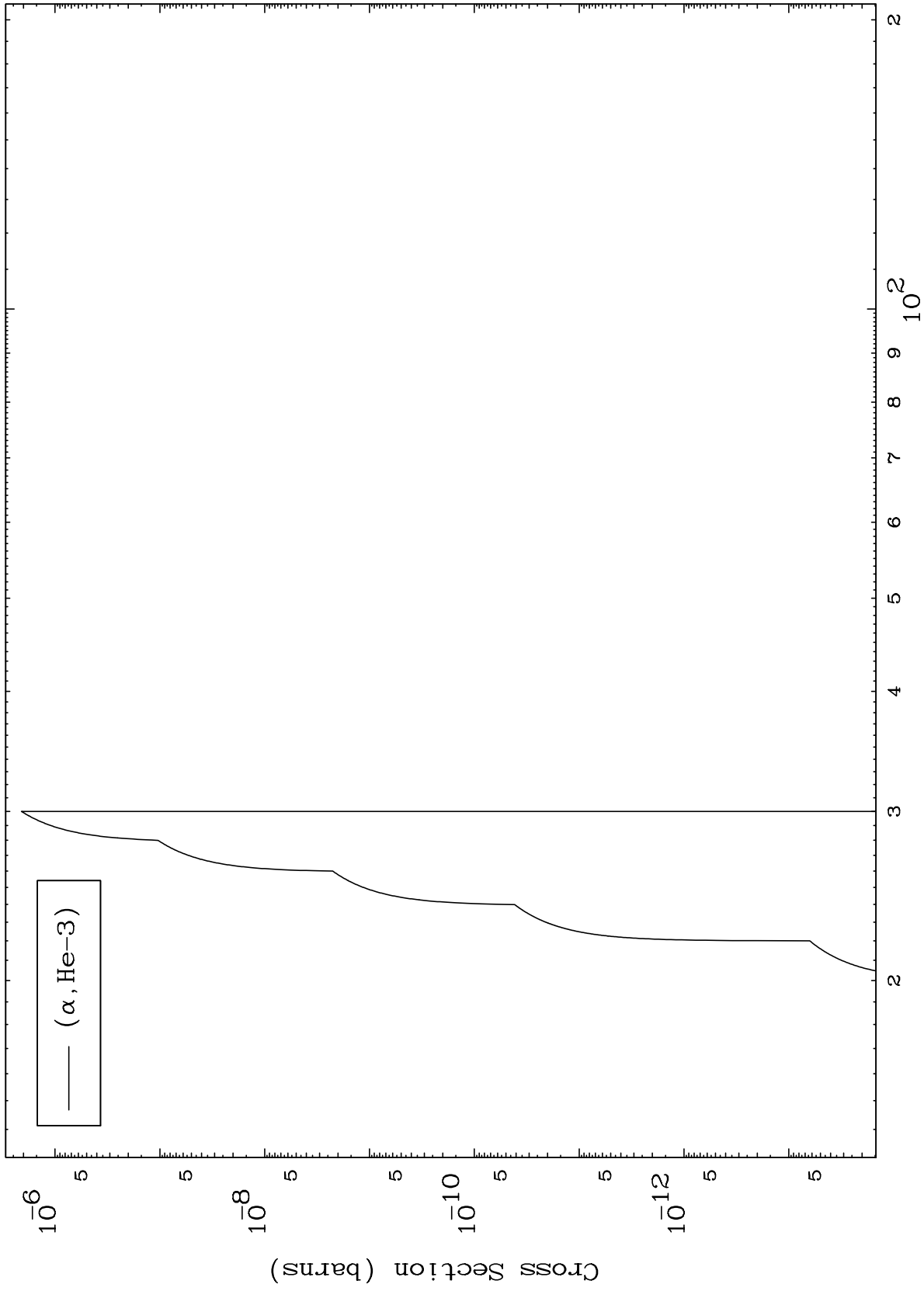


7

Incident Energy (MeV)

71-Lu-169

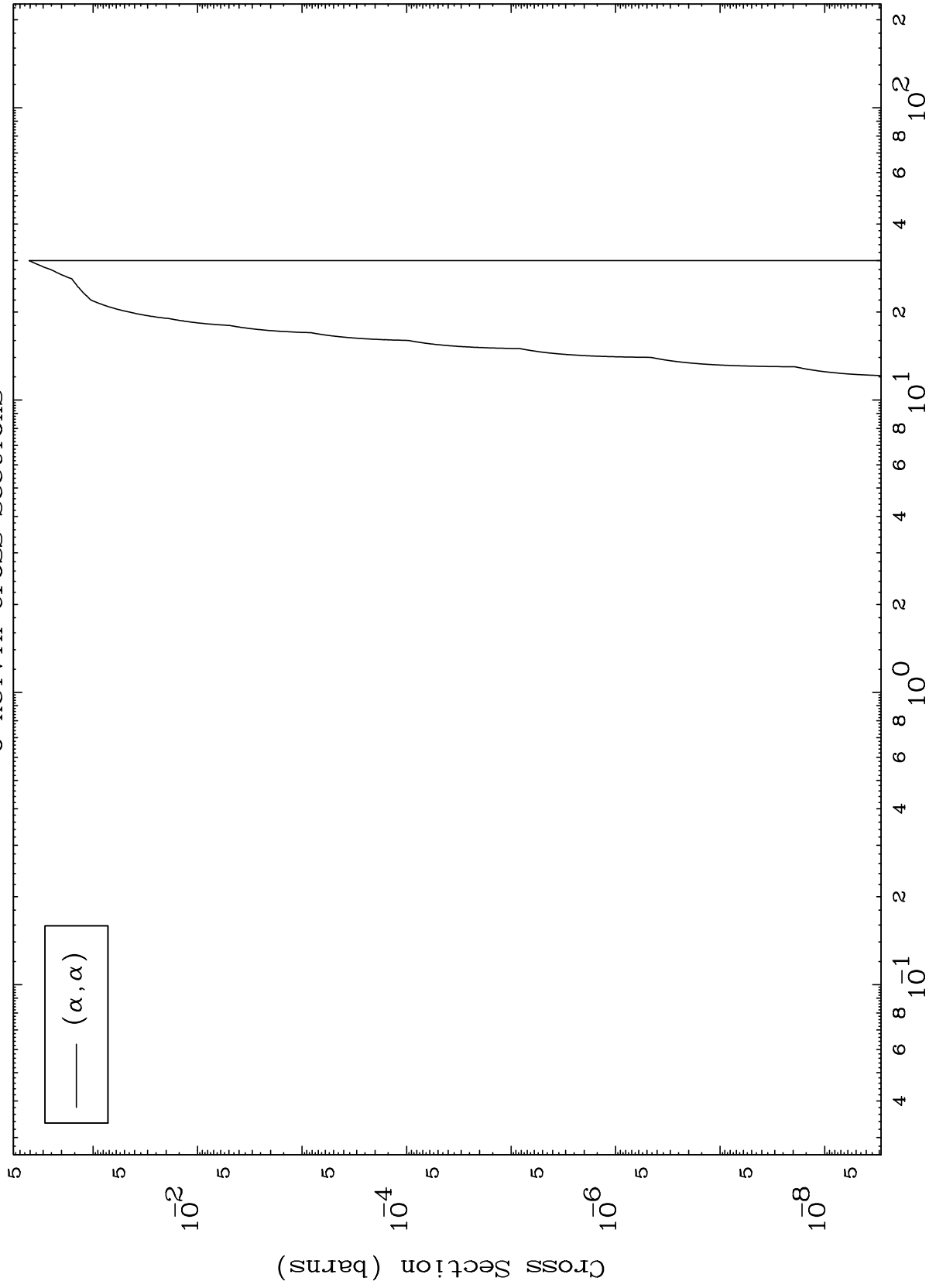




MAT 7107

(α, α) Levels
0 Kelvin Cross Sections

71-Lu-169



10

Incident Energy (MeV)

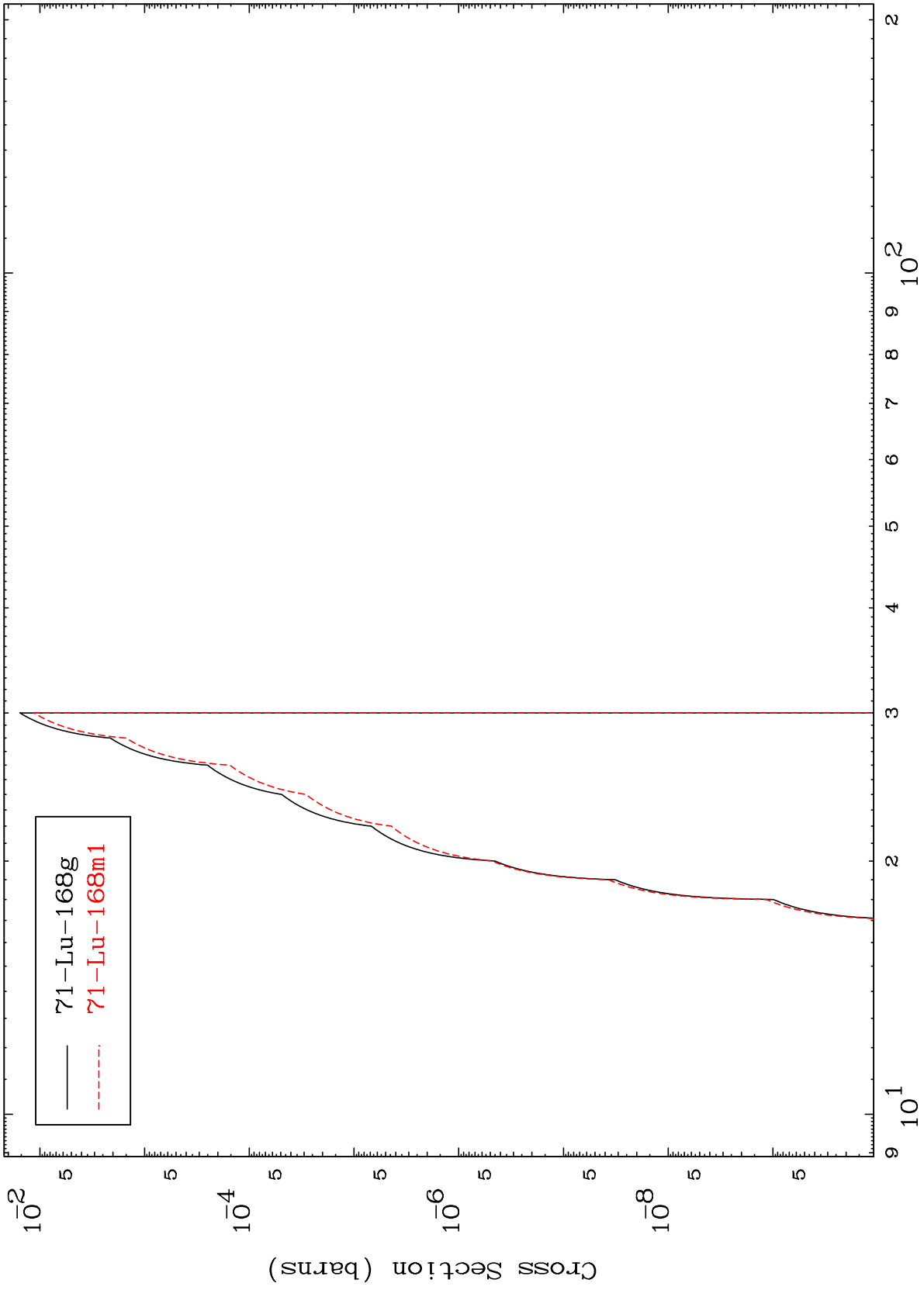
71-Lu-169

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(α, n') α

$^{71}\text{Lu-169}$

Radionuclide Production Cross Section



11

Incident Energy (MeV)

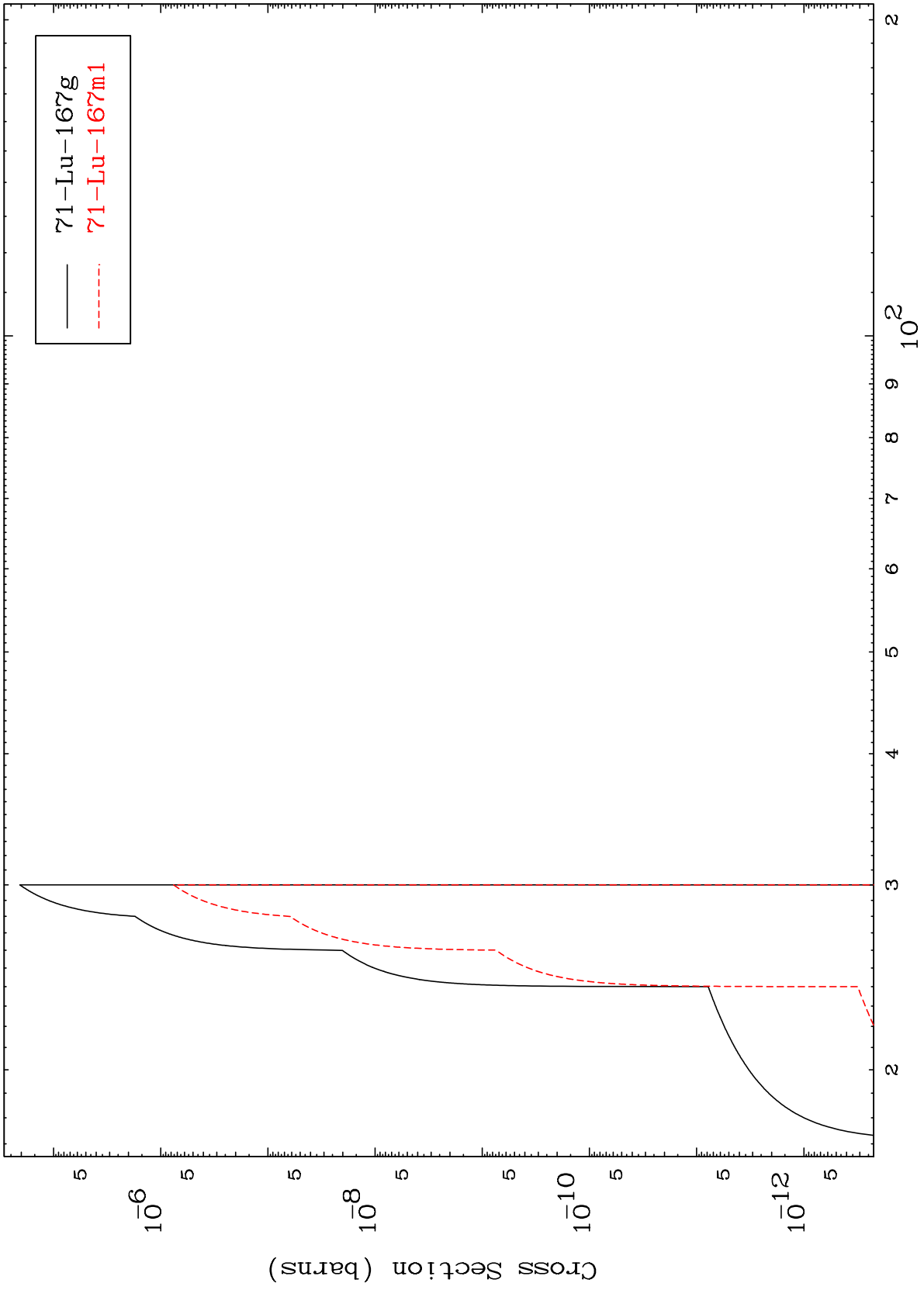
$^{71}\text{Lu-169}$

MAT 7107

$(\alpha, 2n) \alpha$

$^{71}\text{Lu-169}$

Radionuclide Production Cross Section



12

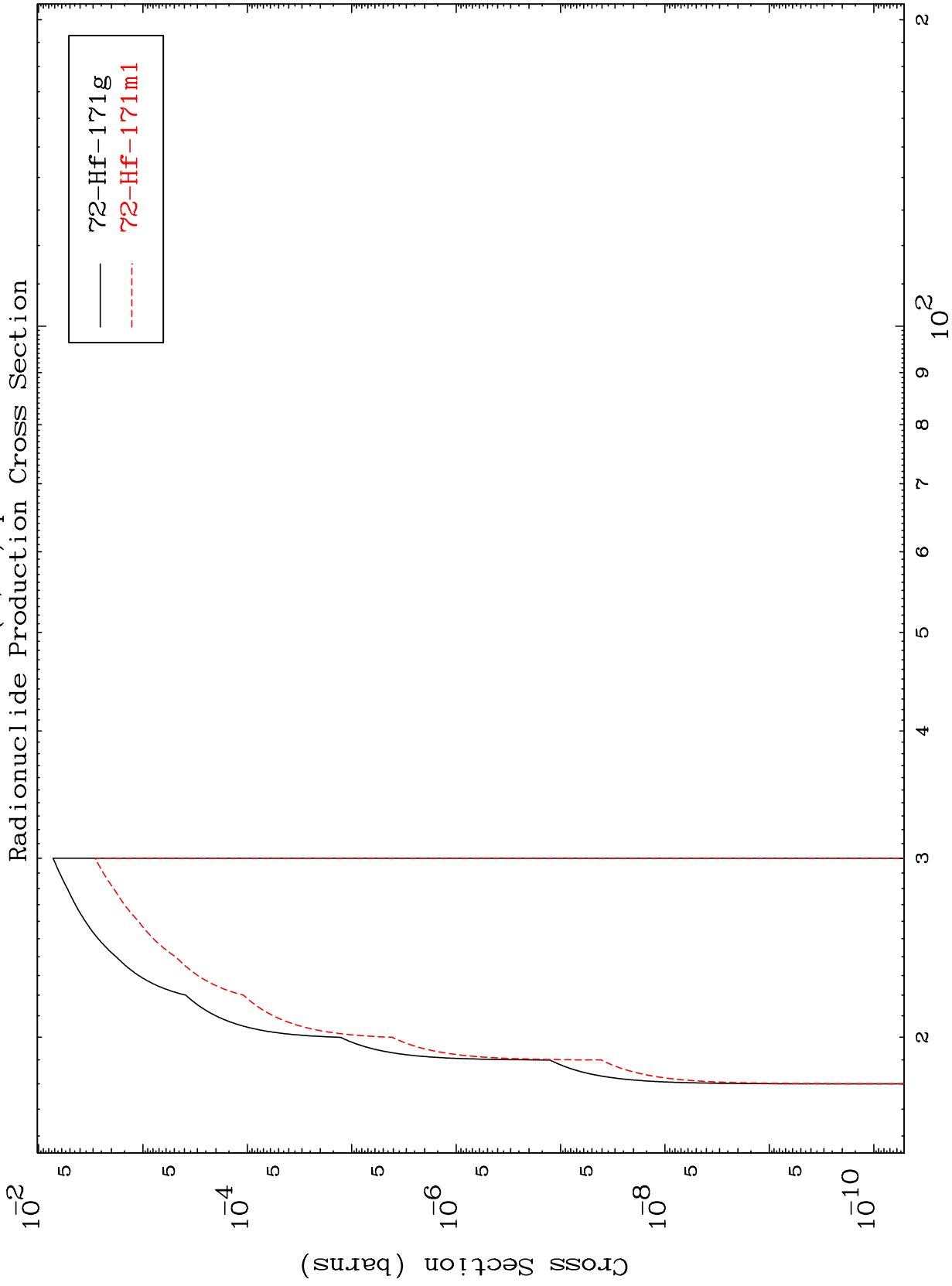
Incident Energy (MeV)

$^{71}\text{Lu-169}$

MAT 7107

(α, n') p

71-Lu-169



13

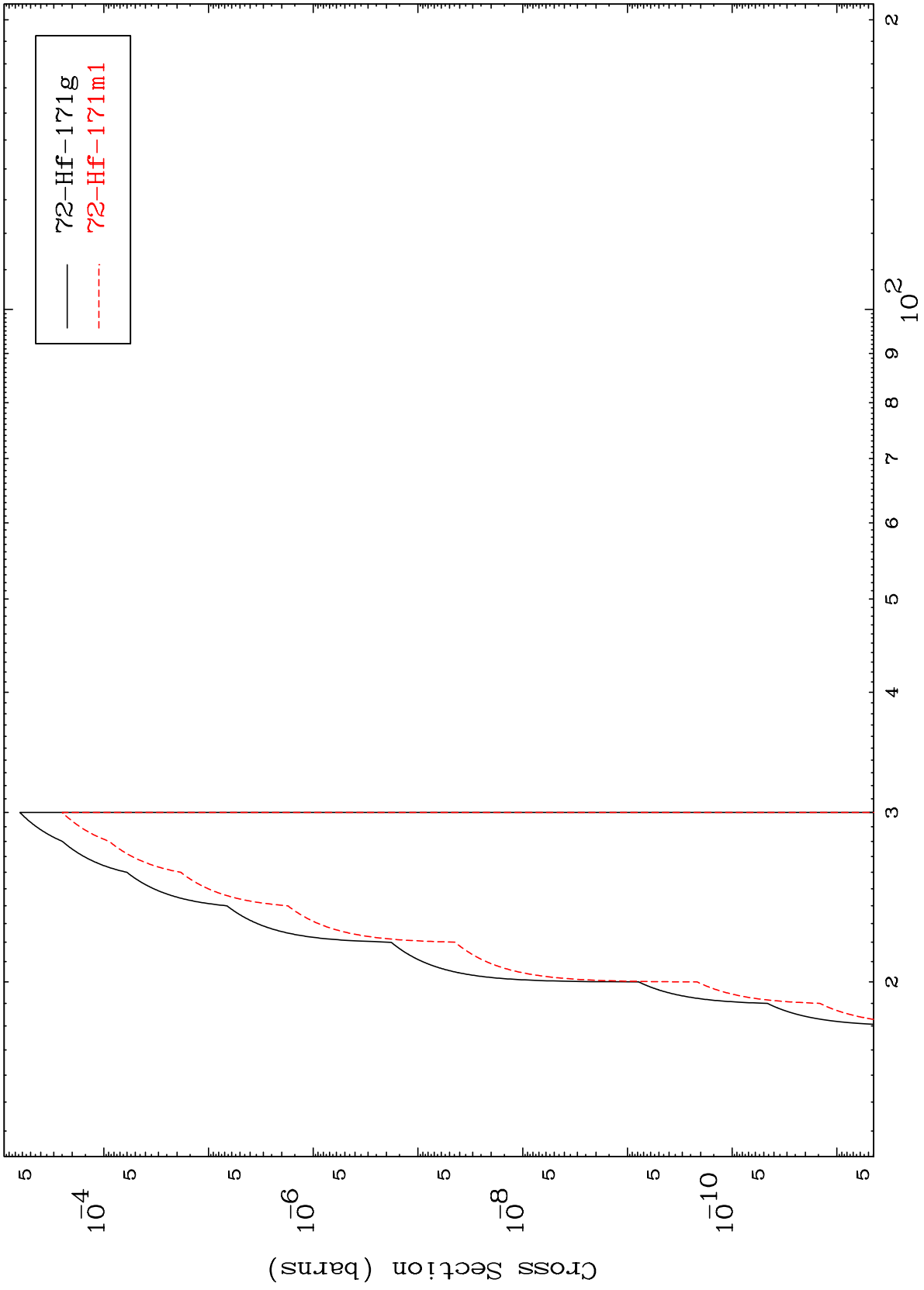
71-Lu-169

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(α, d)

71-Lu-169

Radionuclide Production Cross Section



14

Incident Energy (MeV)

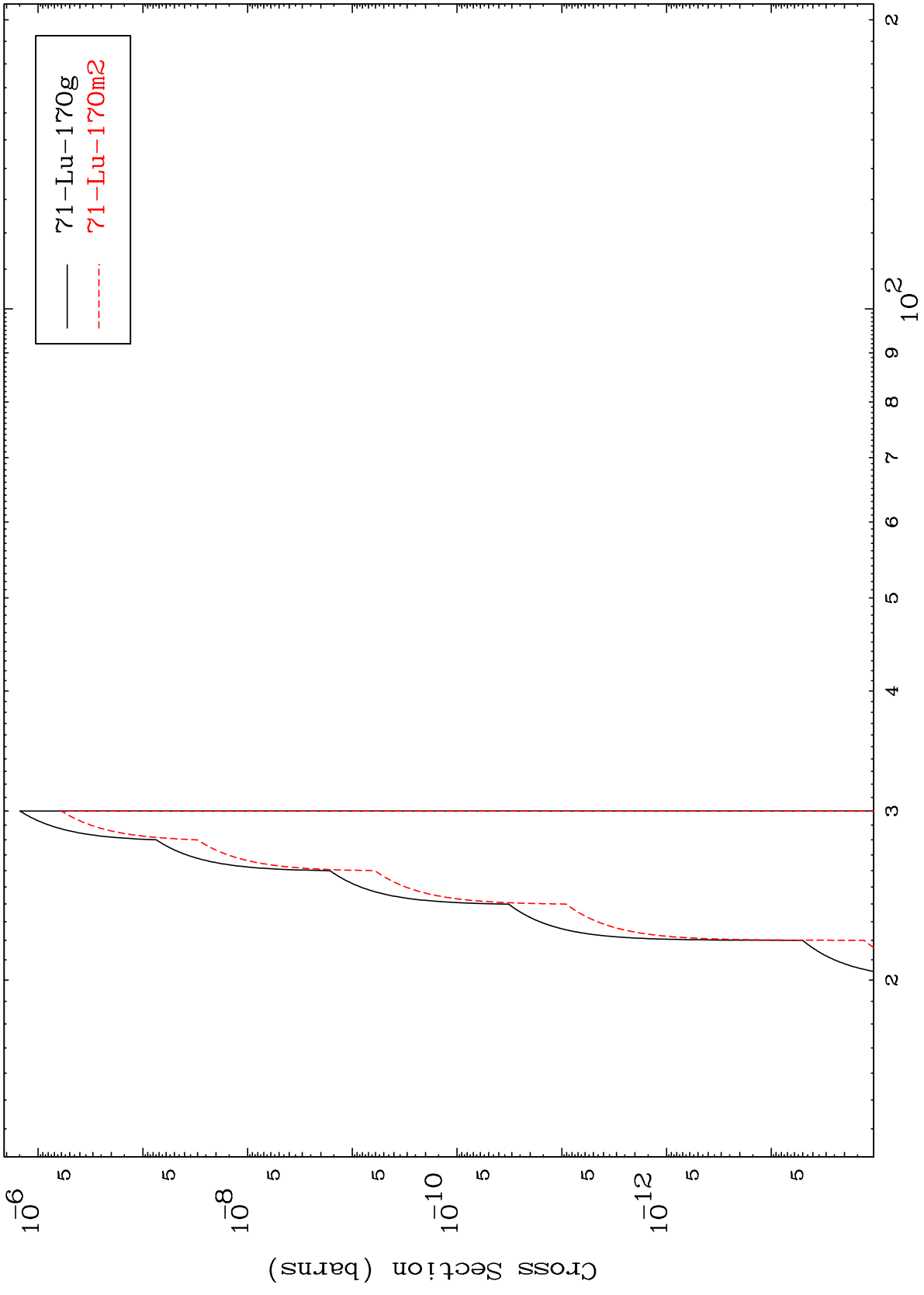
71-Lu-169

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($\alpha, \text{He-3}$)

$^{71}\text{Lu-169}$

Radionuclide Production Cross Section



15

Incident Energy (MeV)

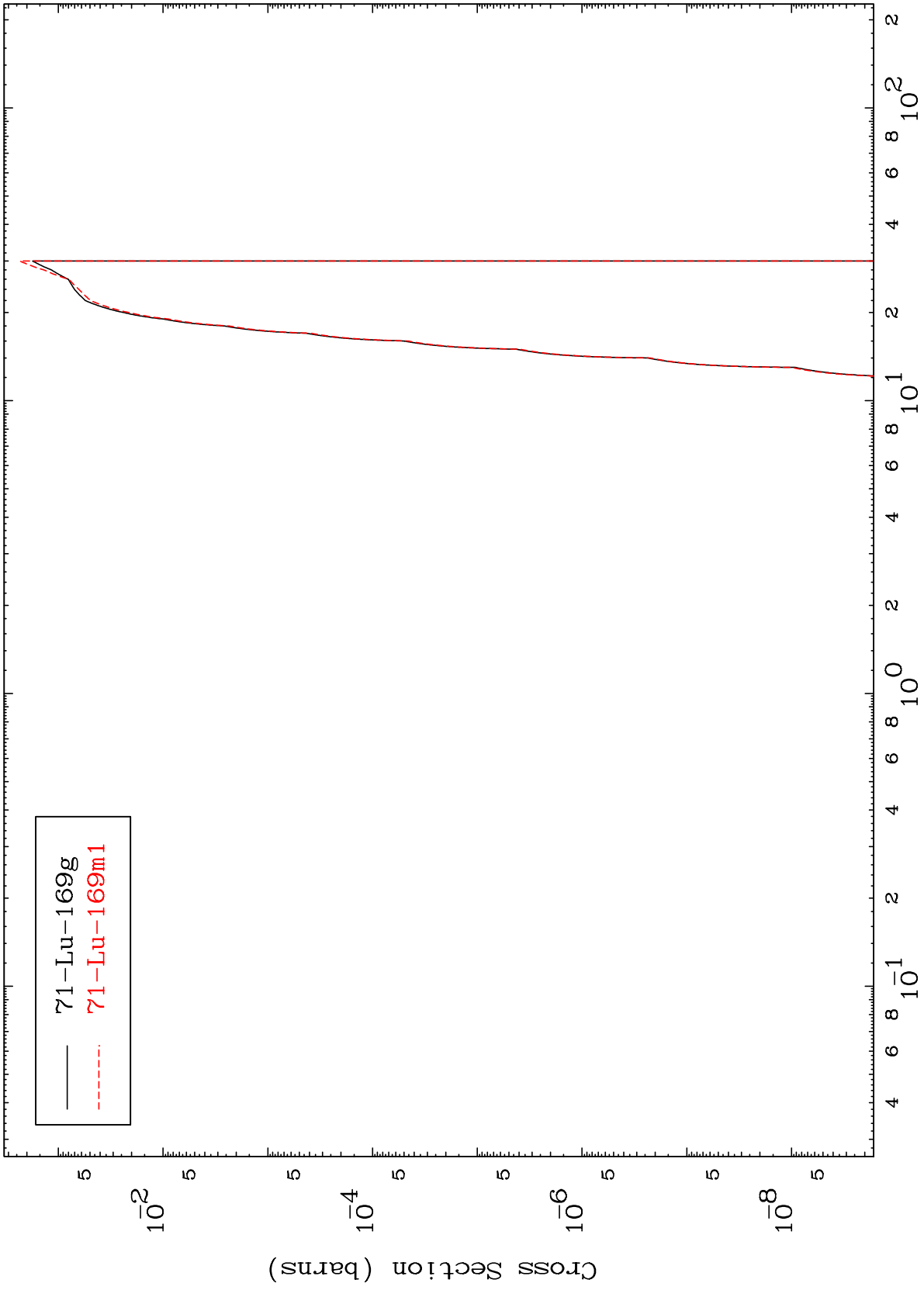
$^{71}\text{Lu-169}$

MAT 7107

(α, α)

⁷¹Lu-169

Radionuclide Production Cross Section



16

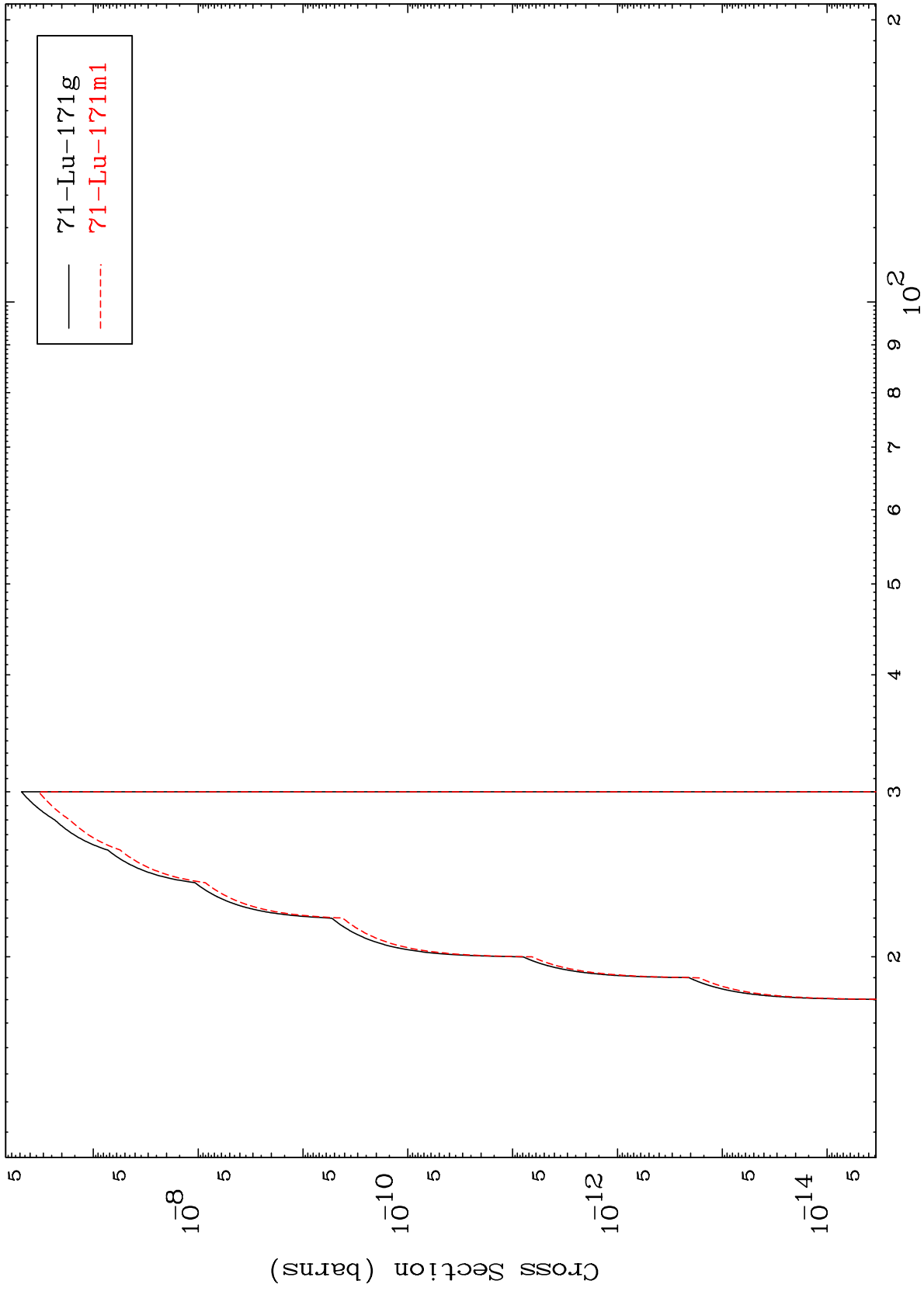
Incident Energy (MeV)

⁷¹Lu-169

MAT 7107

71-Lu-169

($\alpha, 2p$)
Radionuclide Production Cross Section



17

71-Lu-169