

Program EVALPLOT
(Version 2018-1)

by

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(Present Contact Information)

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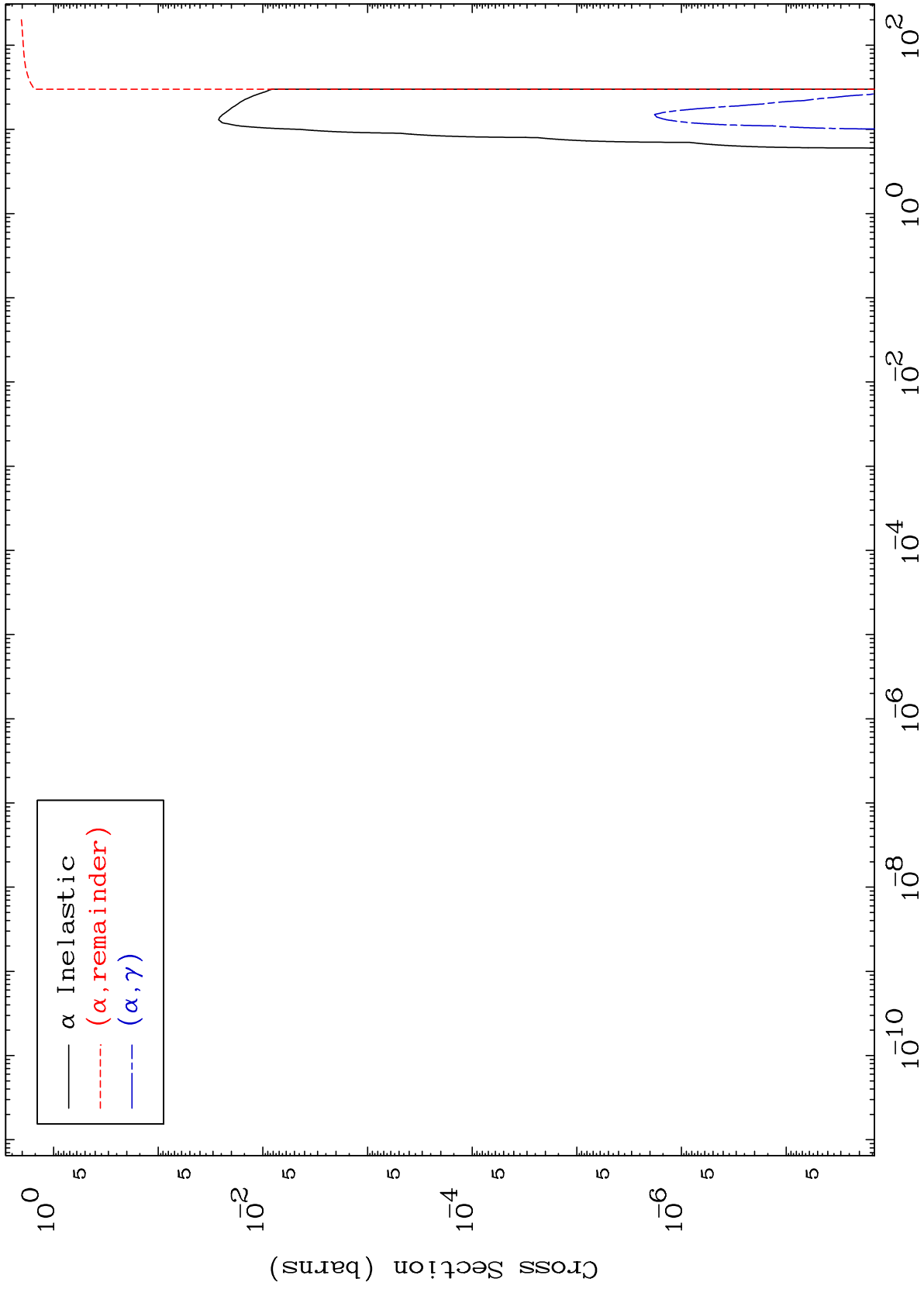
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 4146

0 Kelvin α Major

41-Nb-100



1

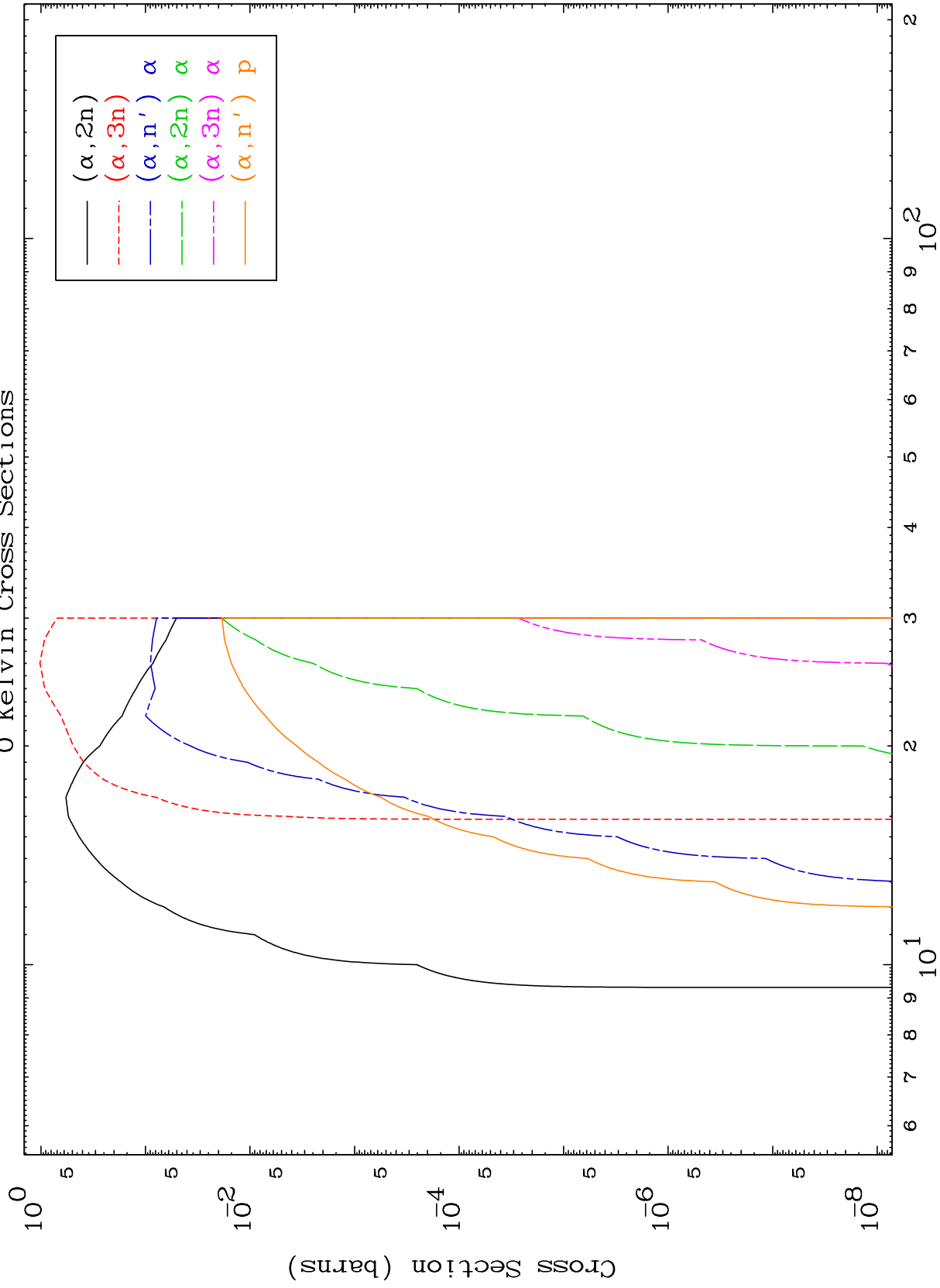
Incident Energy (MeV)

41-Nb-100

MAT 4146

α Neutron Production
0 Kelvin Cross Sections

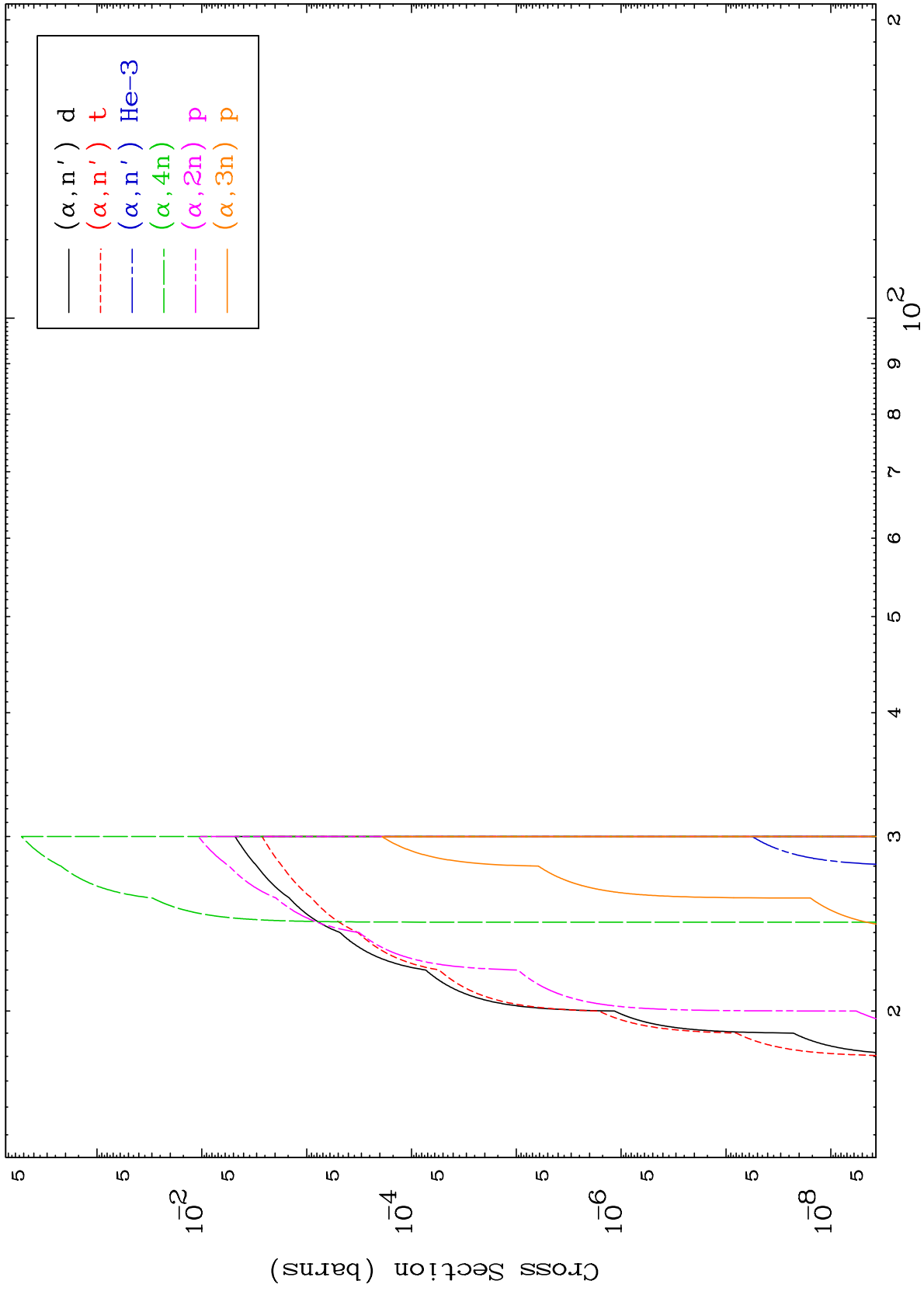
41-Nb-100



2

Incident Energy (MeV)

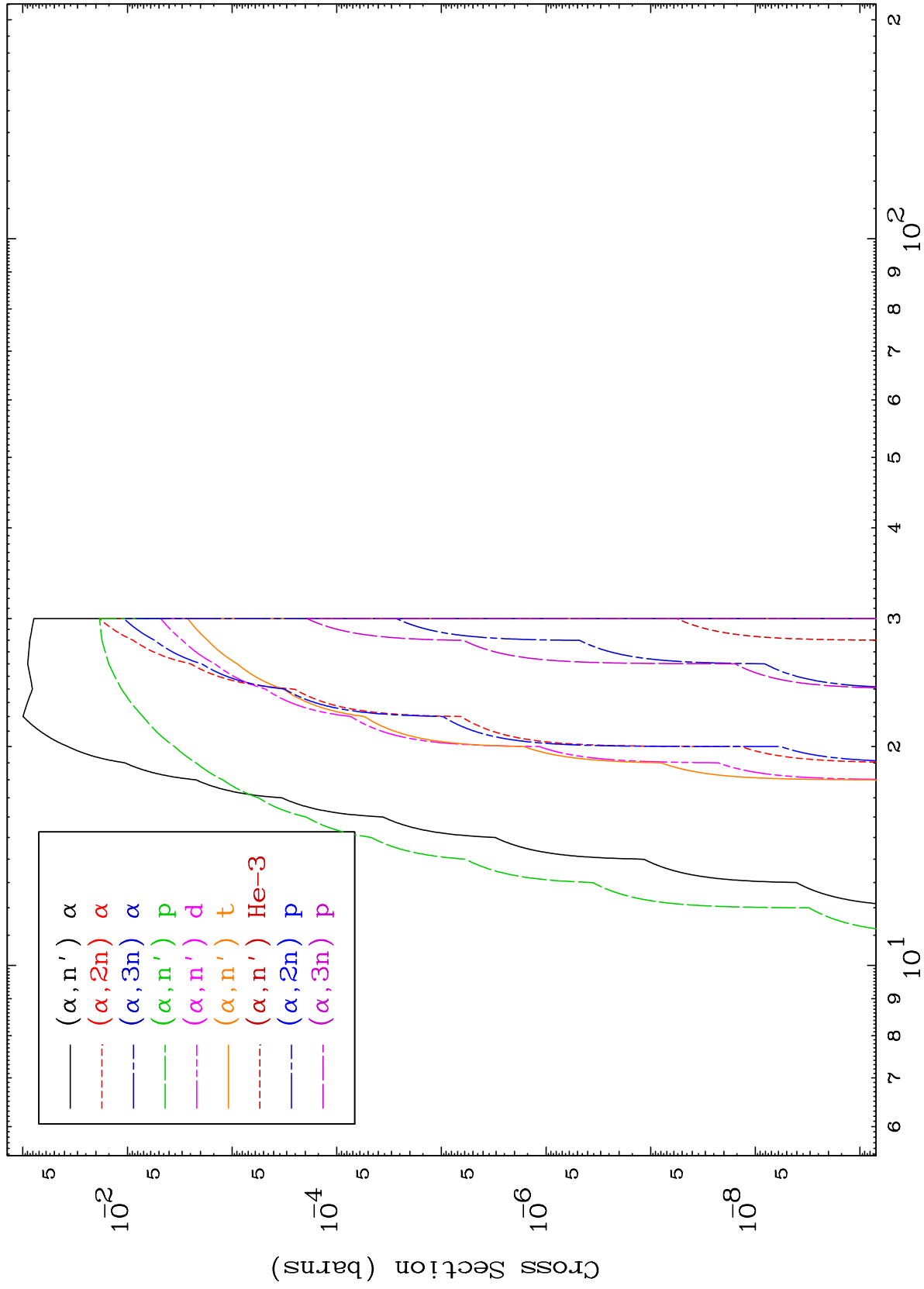
41-Nb-100



MAT 4146

α Charged Particle
0 Kelvin Cross Sections

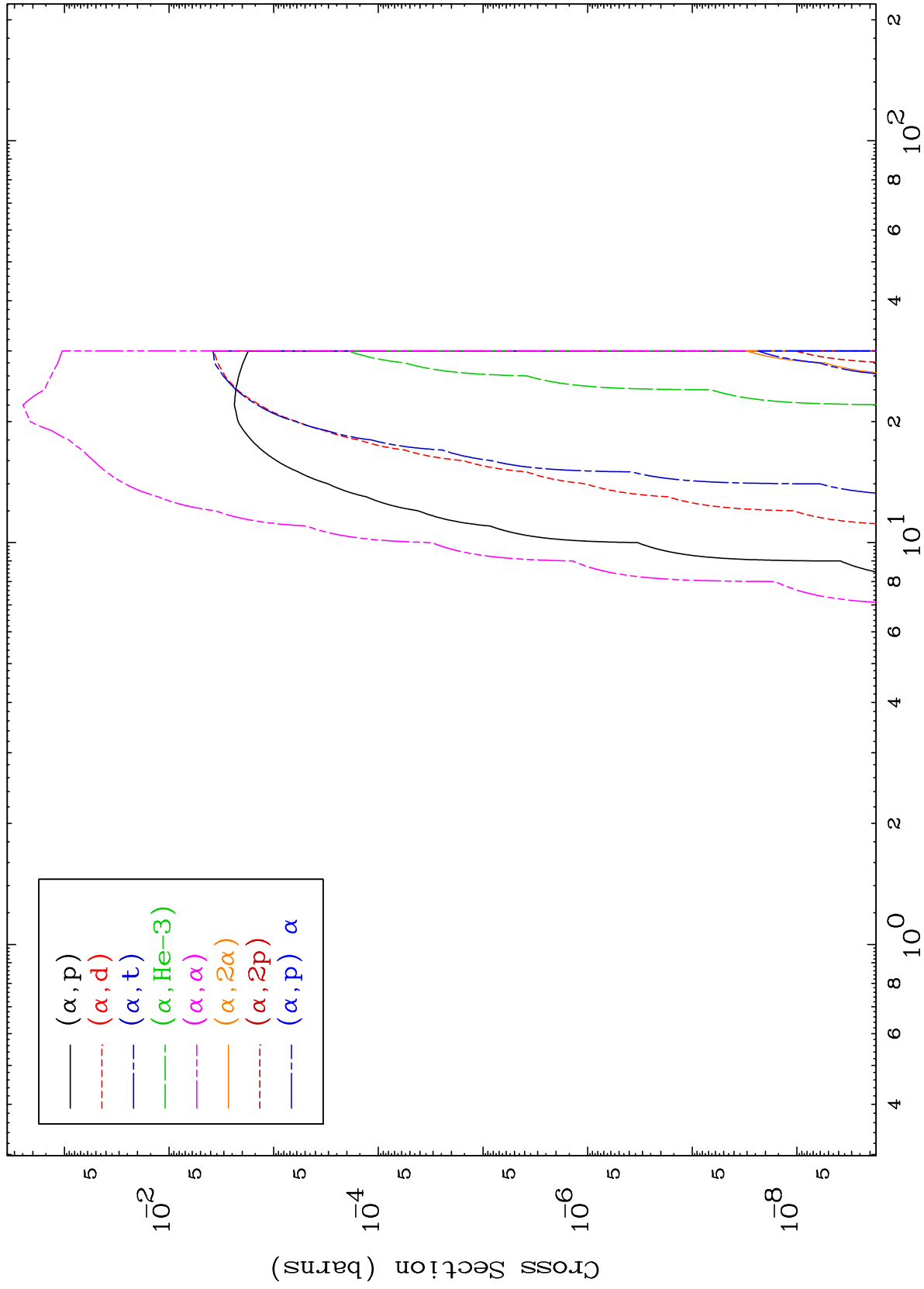
41-Nb-100



4

Incident Energy (MeV)

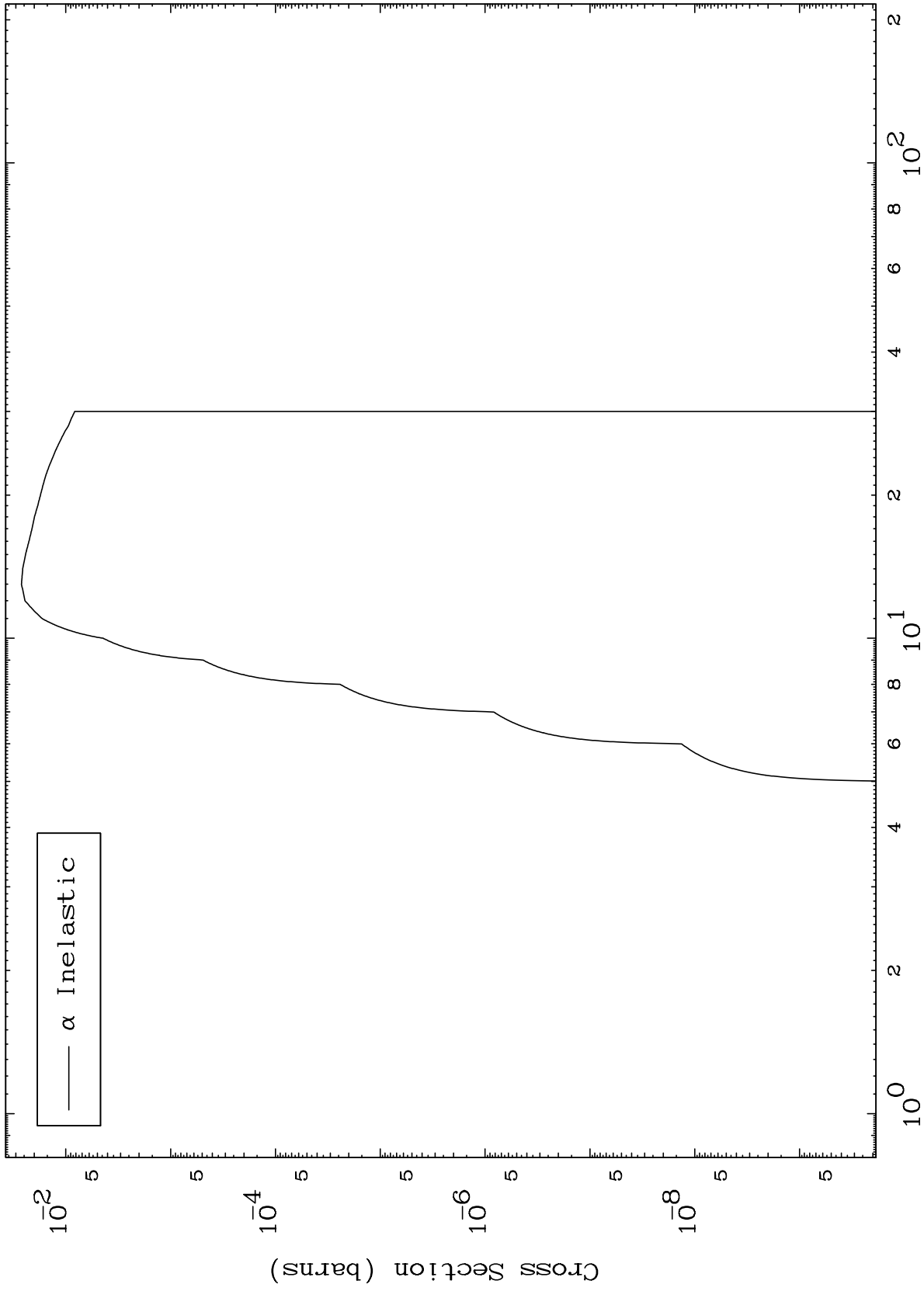
41-Nb-100



MAT 4146

(α, n') Level
0 Kelvin Cross Sections

41-Nb-100



6

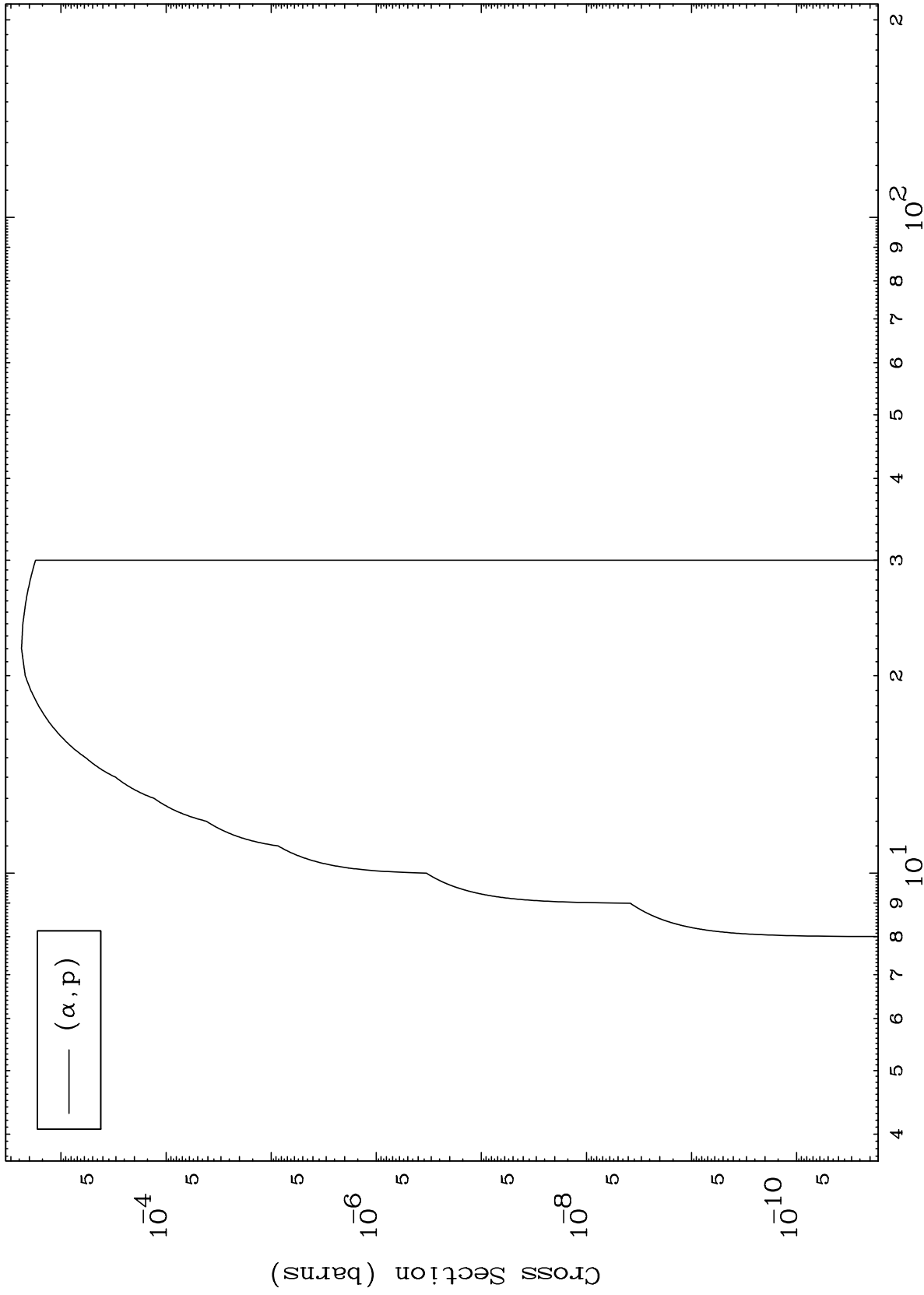
Incident Energy (MeV)

41-Nb-100

MAT 4146

(α, p) Levels
0 Kelvin Cross Sections

41-Nb-100



7

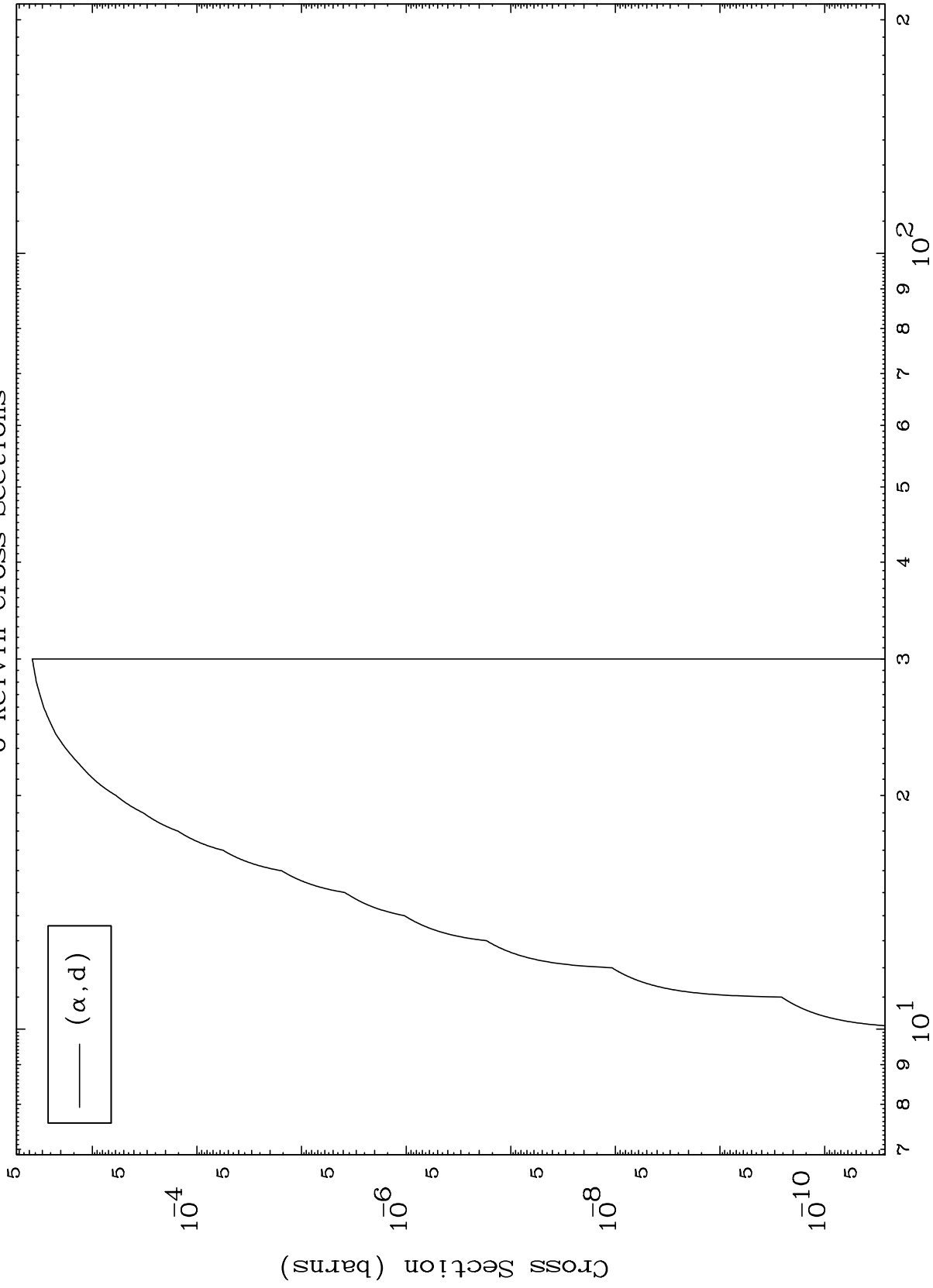
Incident Energy (MeV)

41-Nb-100

MAT 4146

(α, d) Levels
0 Kelvin Cross Sections

41-Nb-100



8

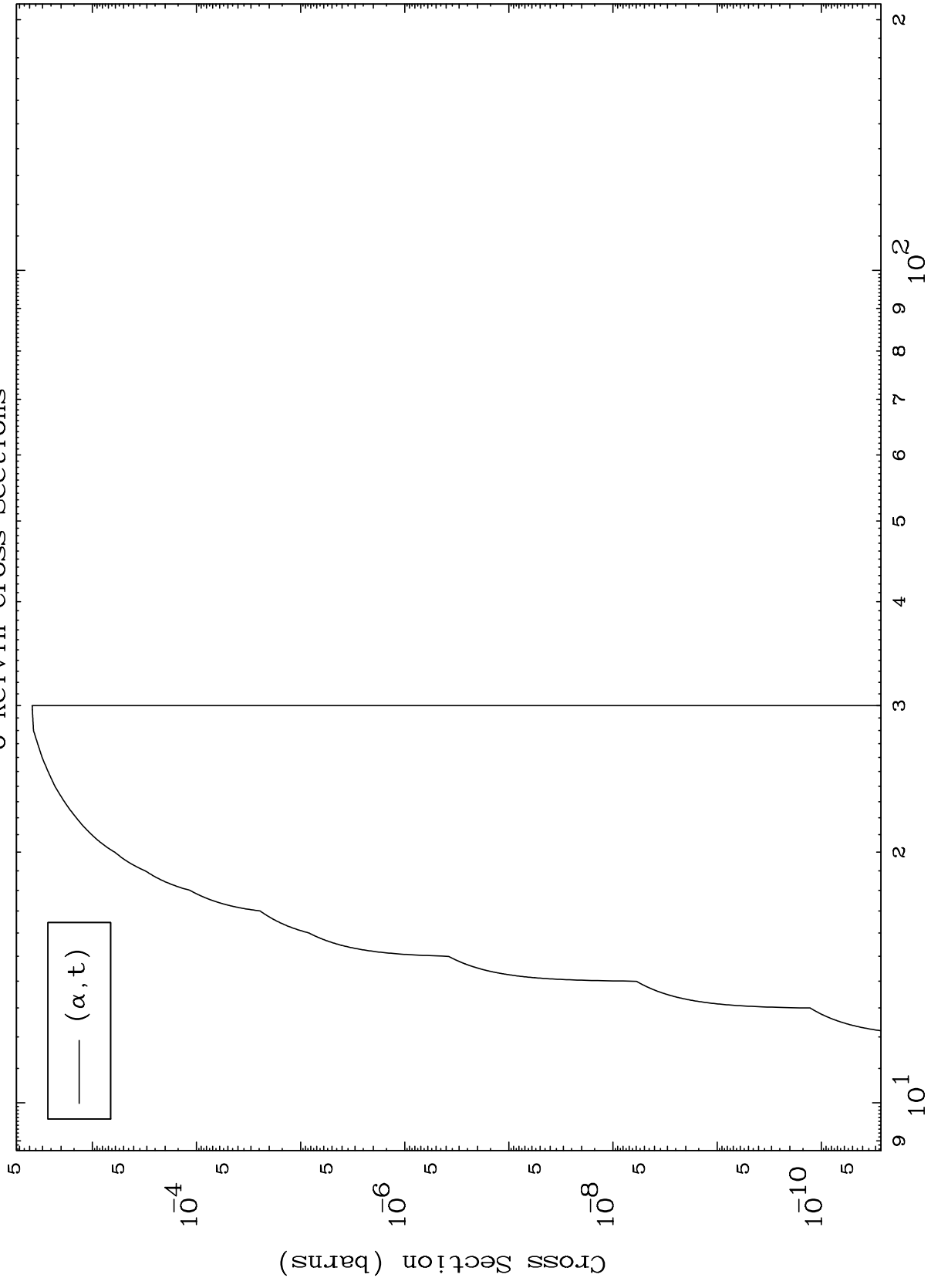
Incident Energy (MeV)

41-Nb-100

MAT 4146

(α, t) Levels
0 Kelvin Cross Sections

41-Nb-100



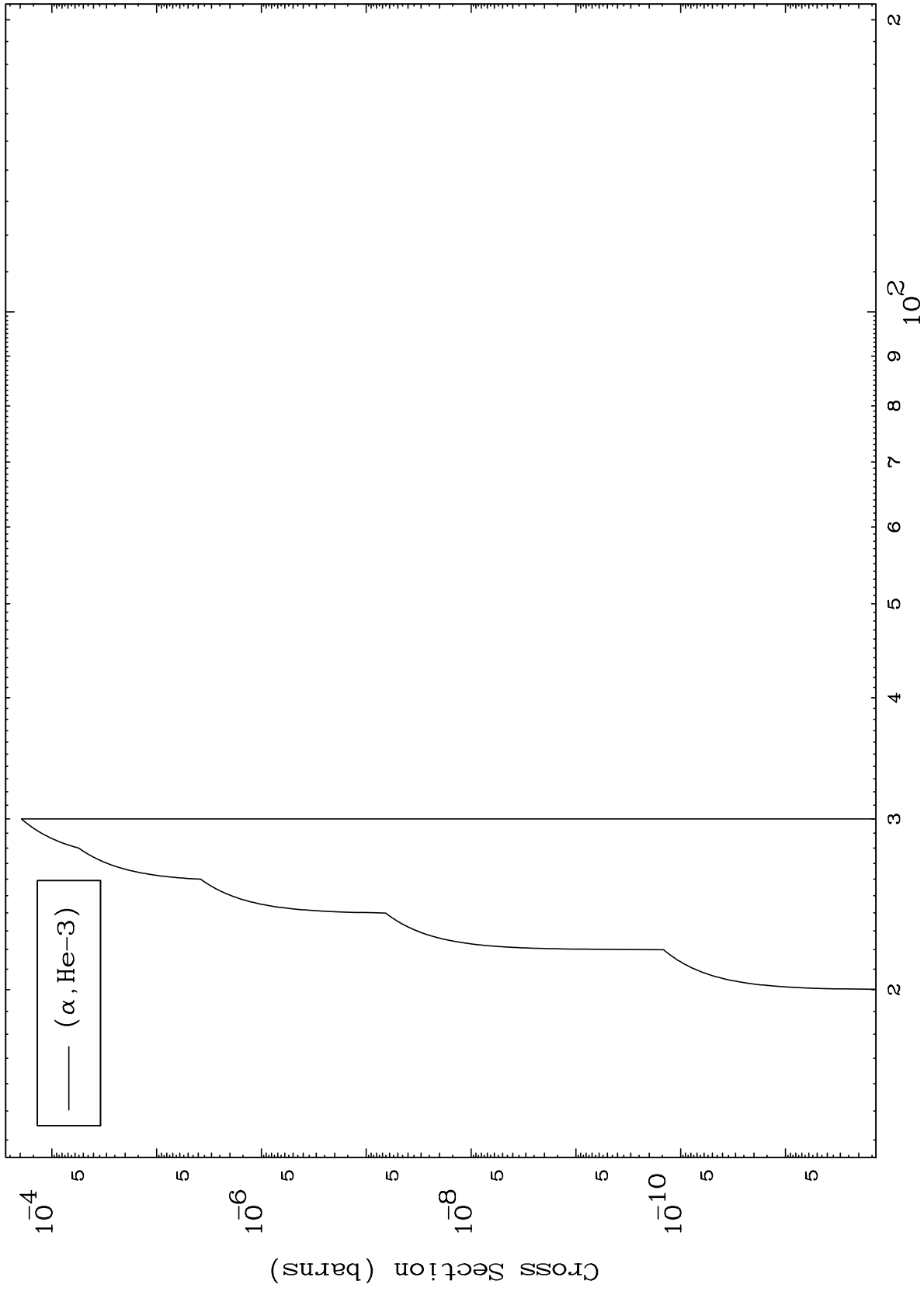
Incident Energy (MeV)

41-Nb-100

MAT 4146

(α ,He3) Levels
0 Kelvin Cross Sections

41-Nb-100



10

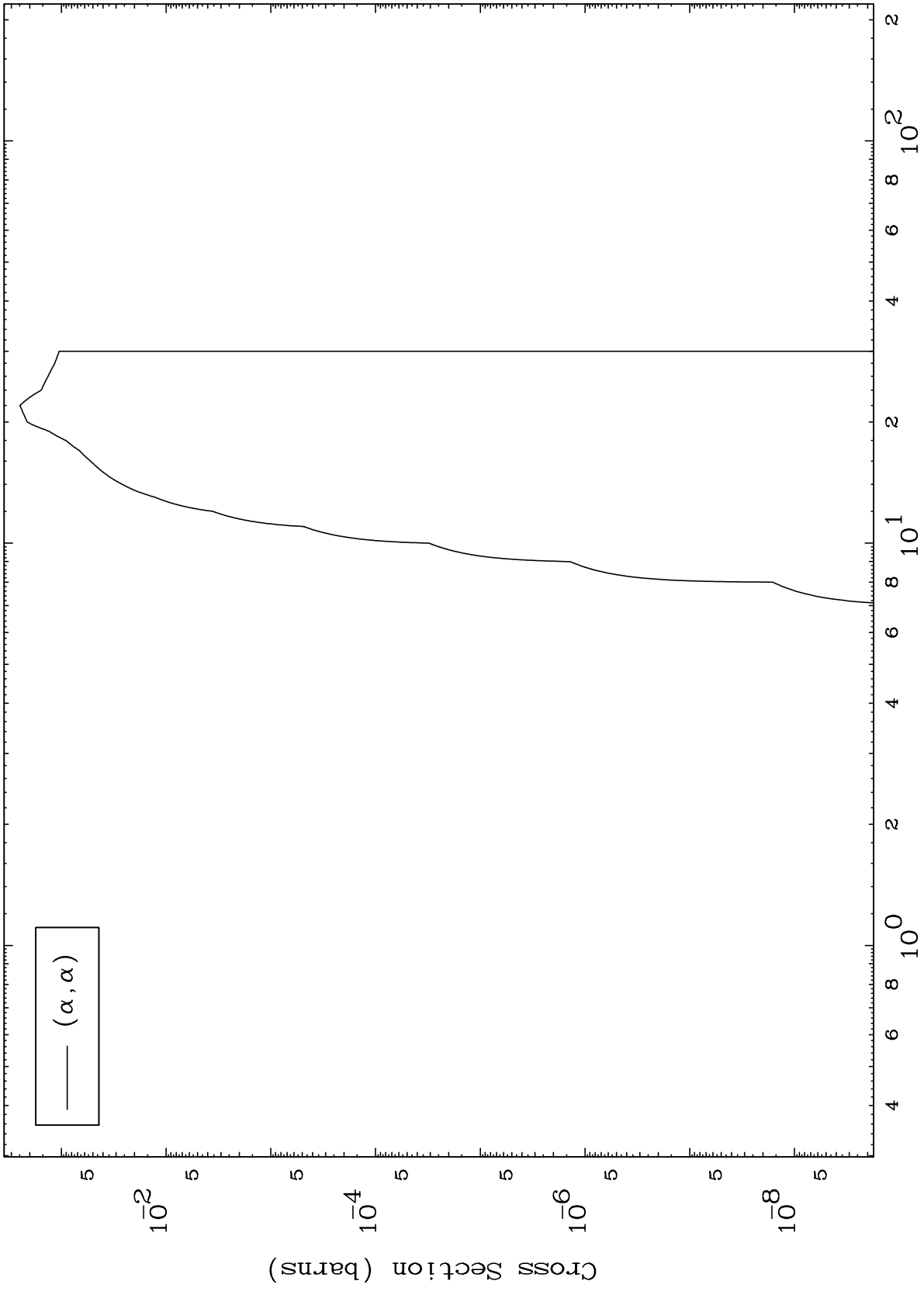
Incident Energy (MeV)

41-Nb-100

MAT 4146

41-Nb-100

(α, α) Levels
0 Kelvin Cross Sections



(α, α)

41-Nb-100

Incident Energy (MeV)

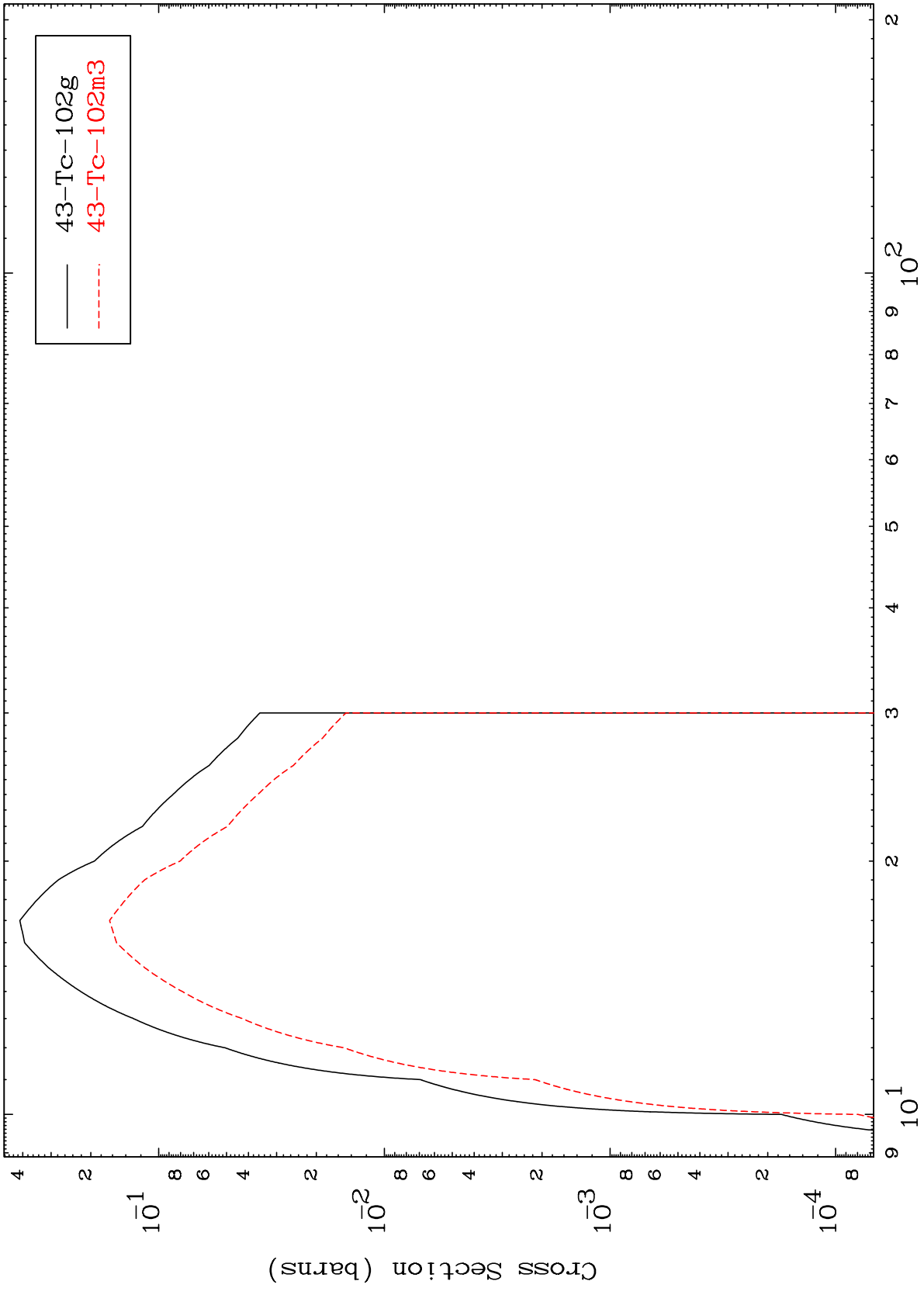
11

MAT 4146

($\alpha, 2n$)

41-Nb-100

Radionuclide Production Cross Section



— 43-Tc-102g
- - - 43-Tc-102m3

12

Incident Energy (MeV)

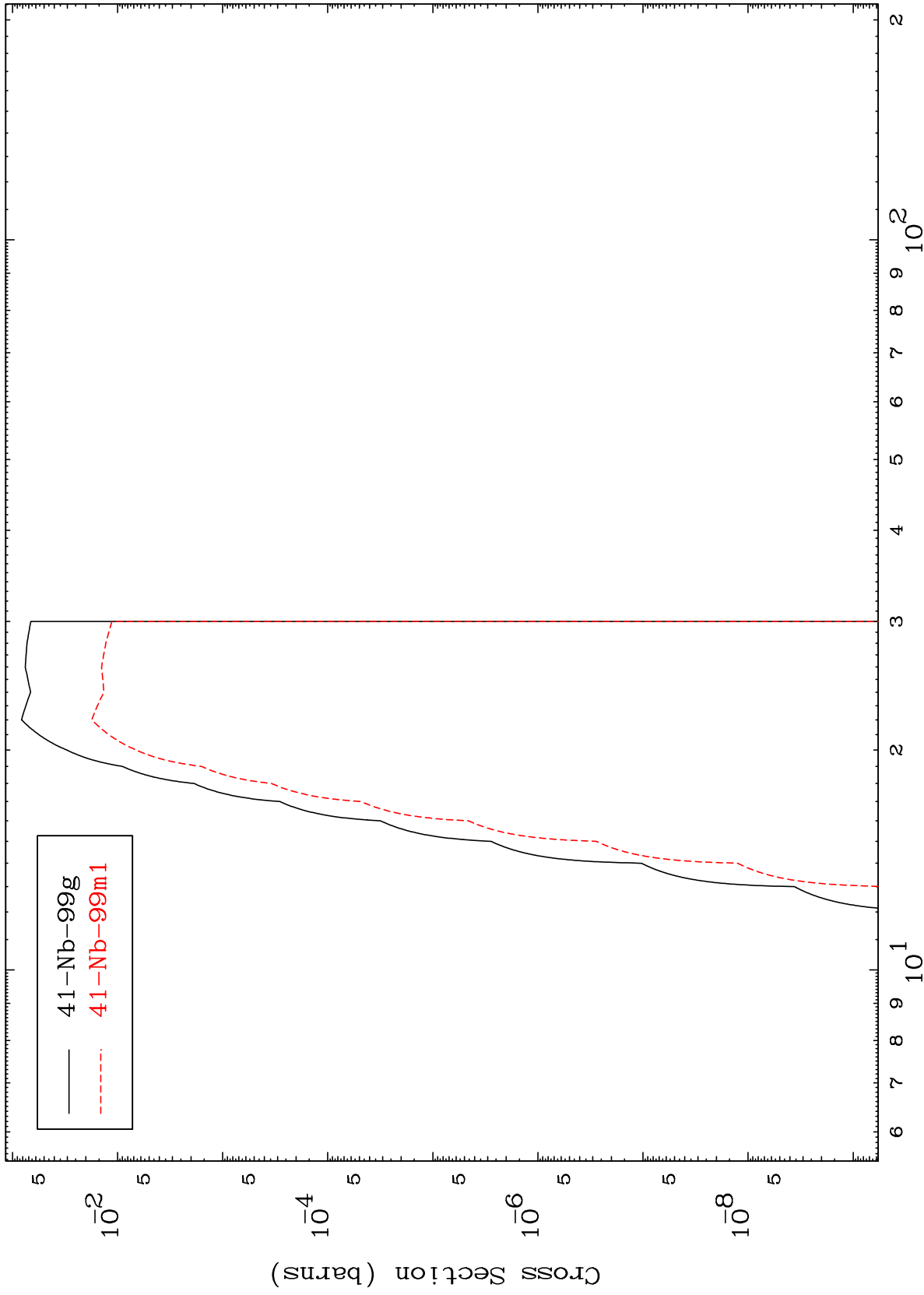
41-Nb-100

MAT 4146

(α, n') α

41-Nb-100

Radionuclide Production Cross Section



13

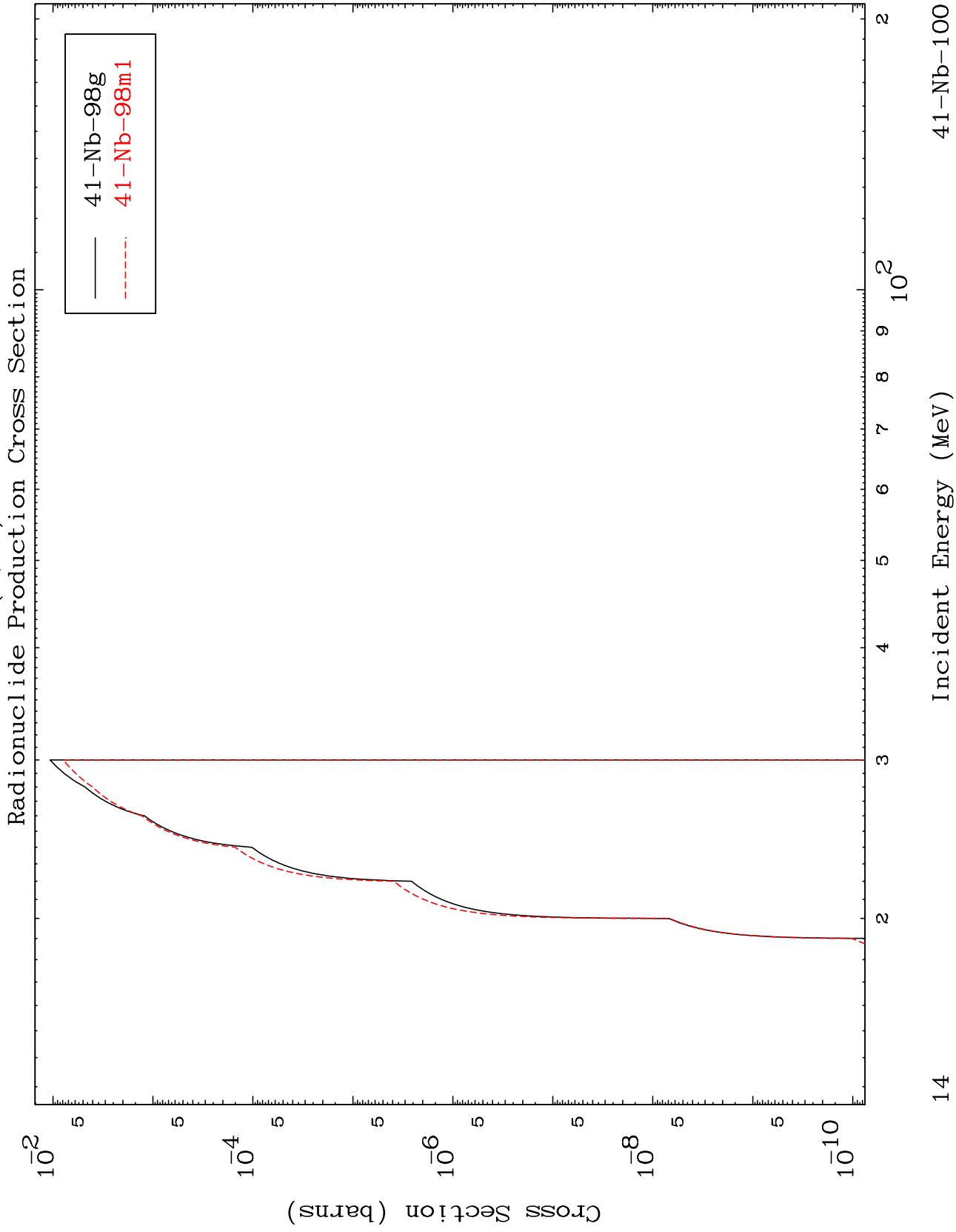
Incident Energy (MeV)

41-Nb-100

MAT 4146

$(\alpha, 2n) \alpha$

41-Nb-100



14

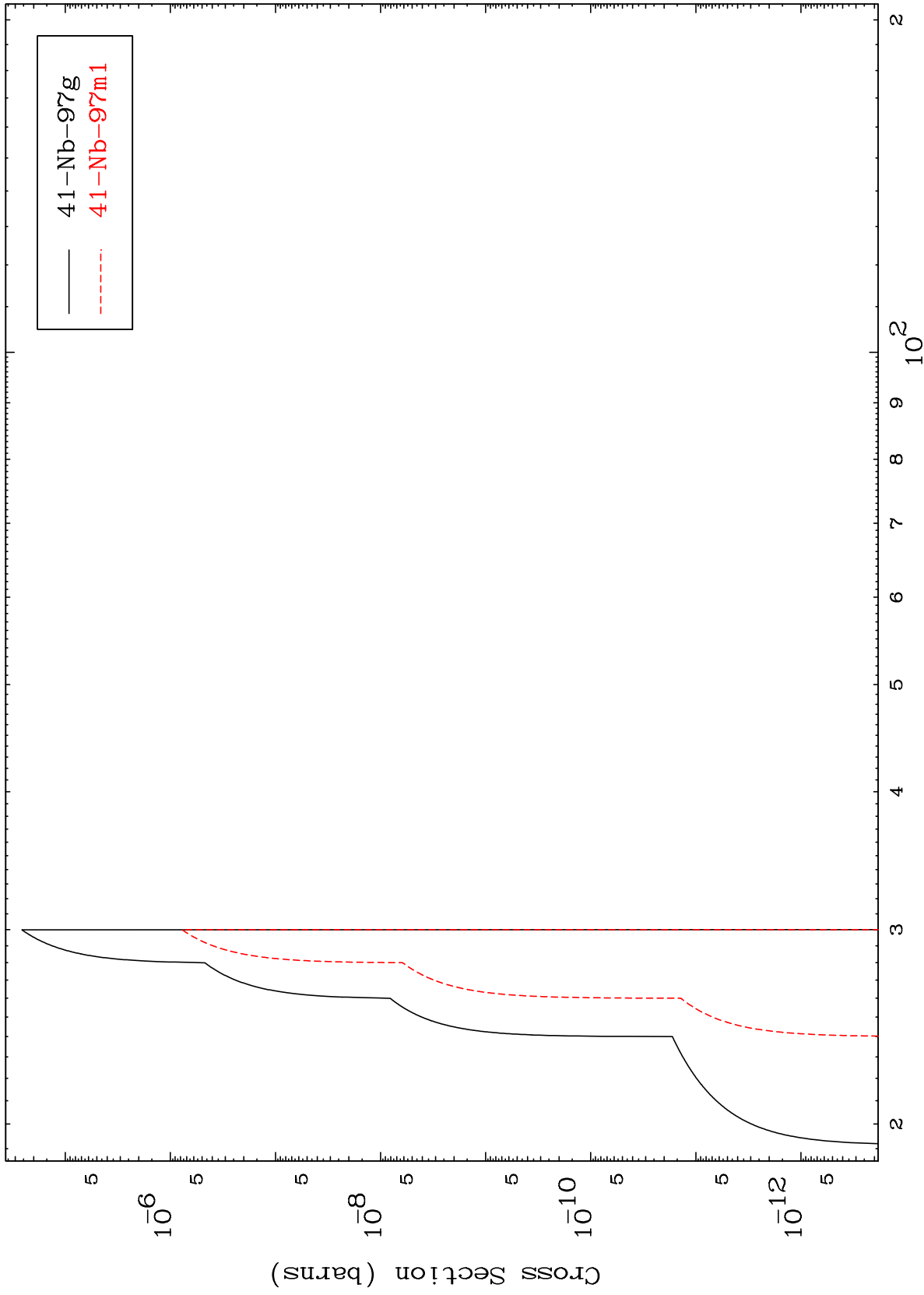
41-Nb-100

MAT 4146

$(\alpha, 3n) \alpha$

41-Nb-100

Radionuclide Production Cross Section



15

Incident Energy (MeV)

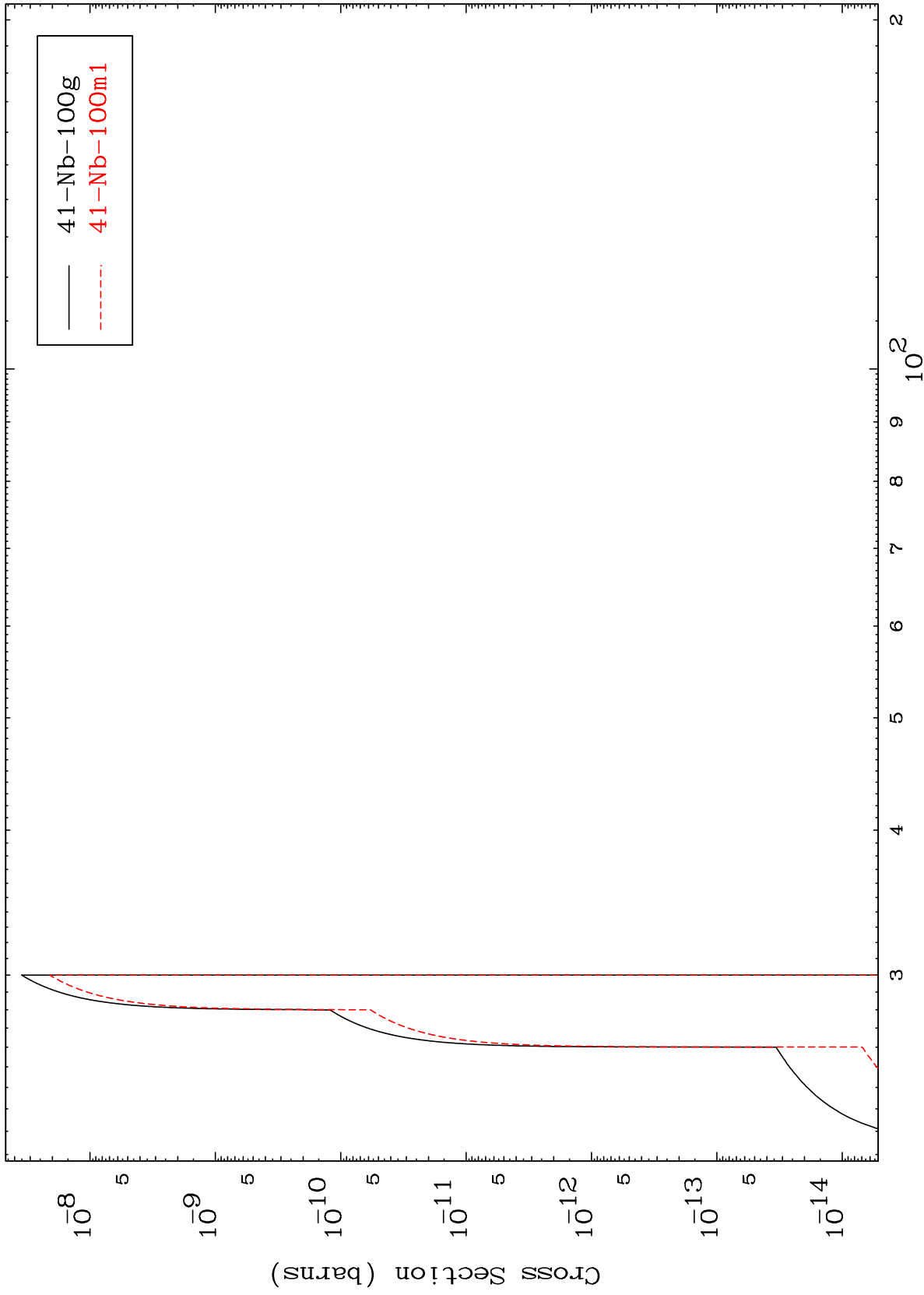
41-Nb-100

MAT 4146

(α, n') He-3

41-Nb-100

Radionuclide Production Cross Section



16

Incident Energy (MeV)

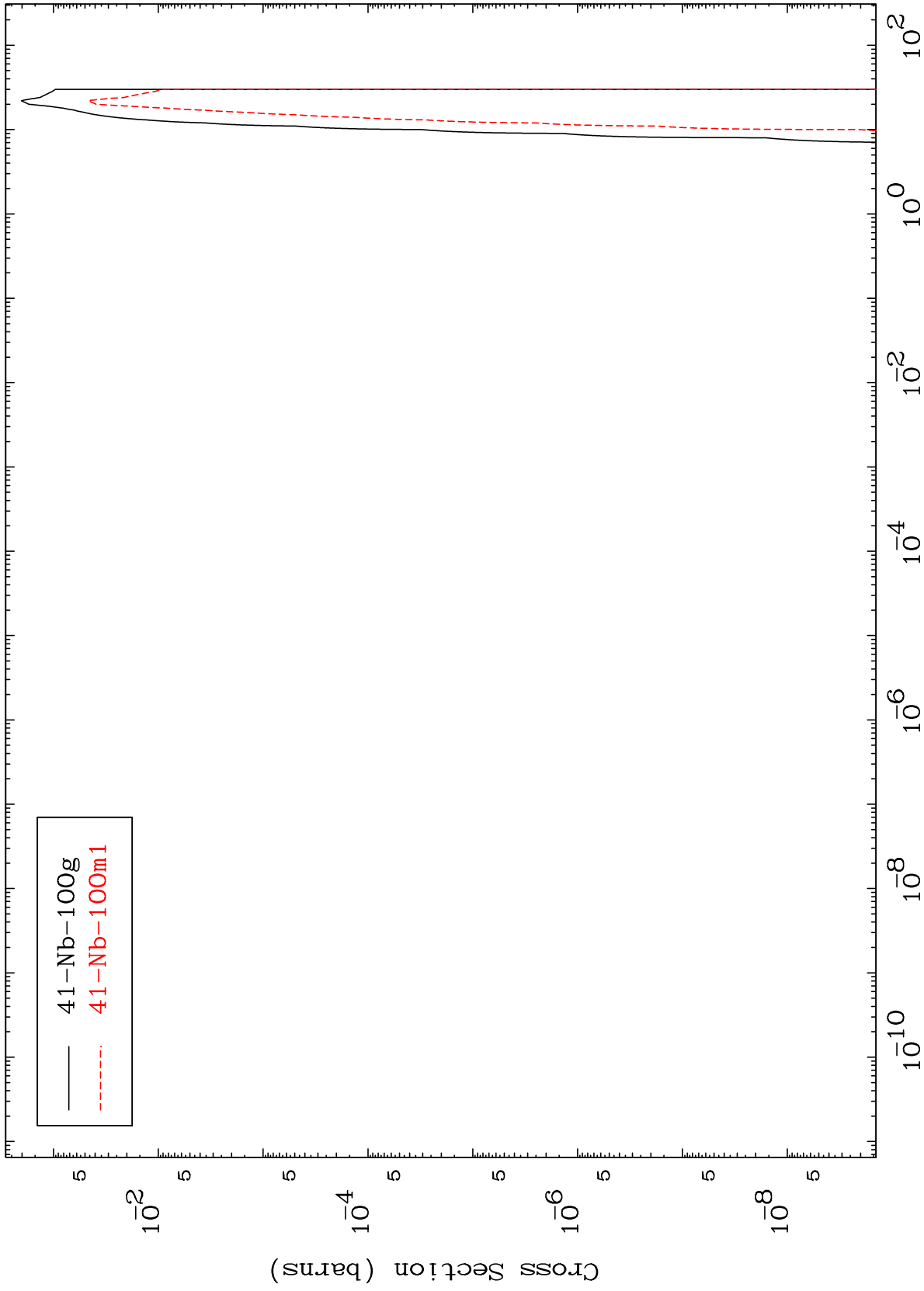
41-Nb-100

MAT 4146

(α, α)

41-Nb-100

Radionuclide Production Cross Section



17

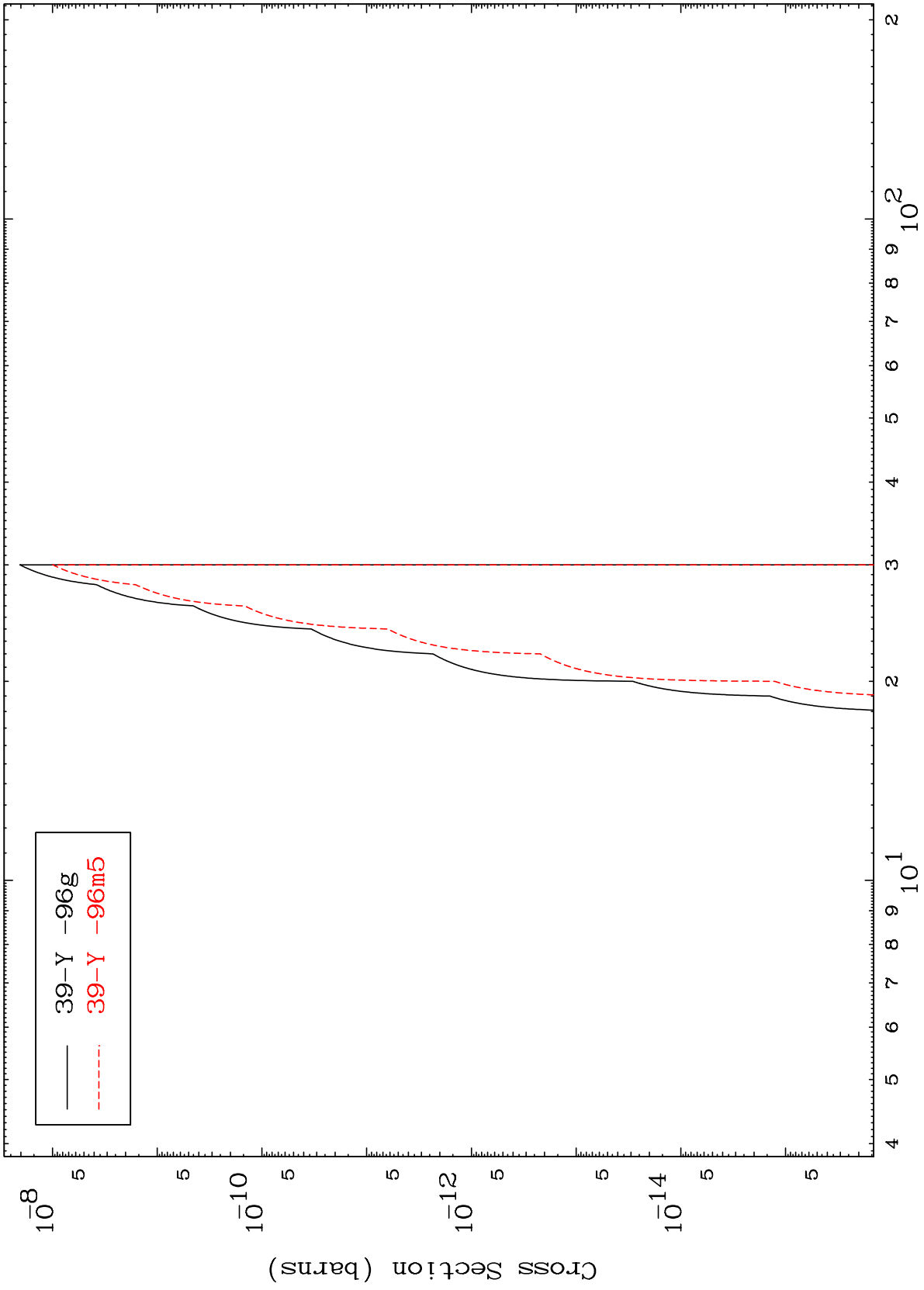
Incident Energy (MeV)

41-Nb-100

MAT 4146

41-Nb-100

Radionuclide Production Cross Section
($\alpha, 2\alpha$)



18

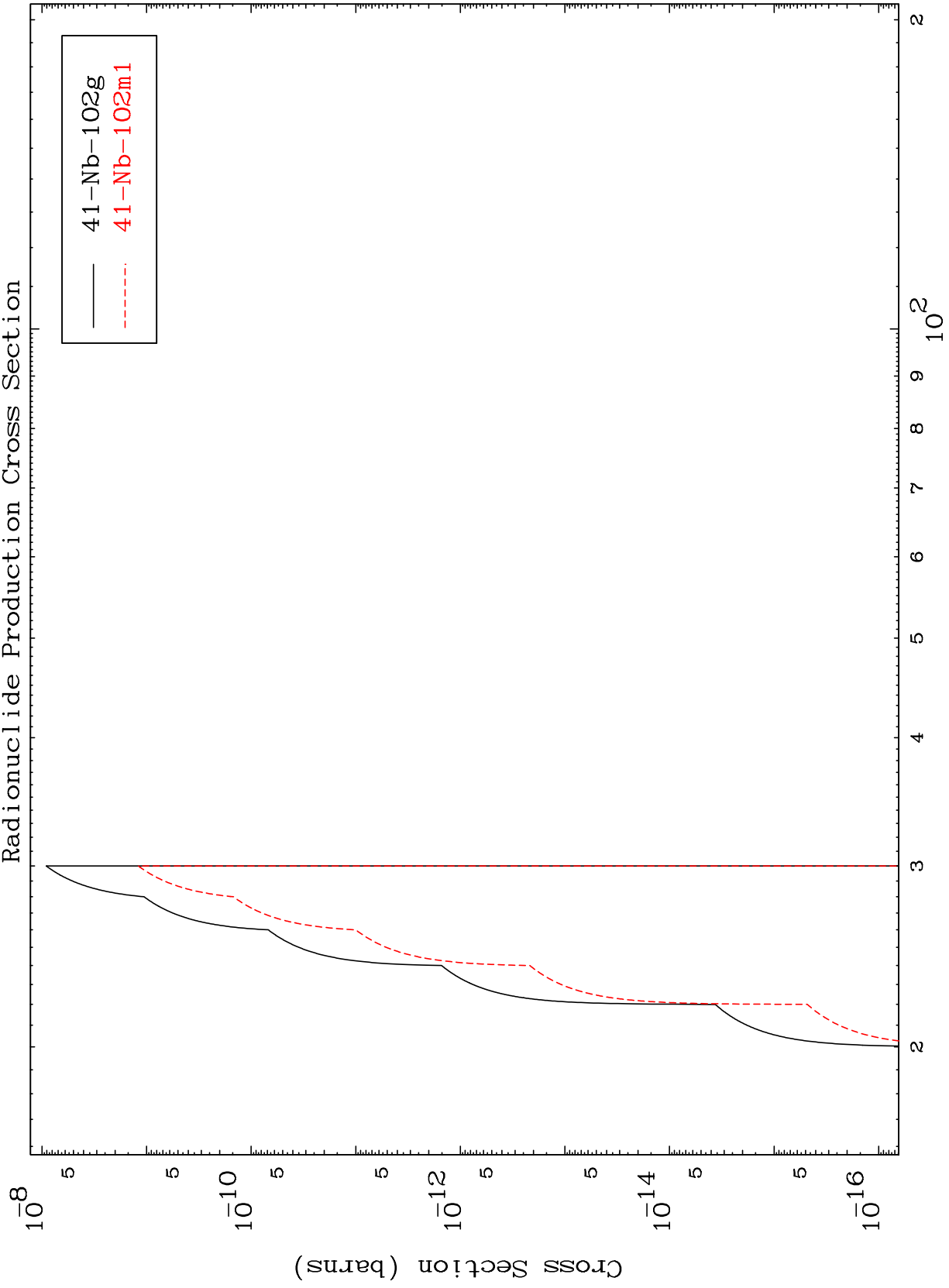
Incident Energy (MeV)

41-Nb-100

MAT 4146

41-Nb-100

($\alpha, 2p$)
Radionuclide Production Cross Section



19

41-Nb-100