

Program EVALPLOT
(Version 2018-1)

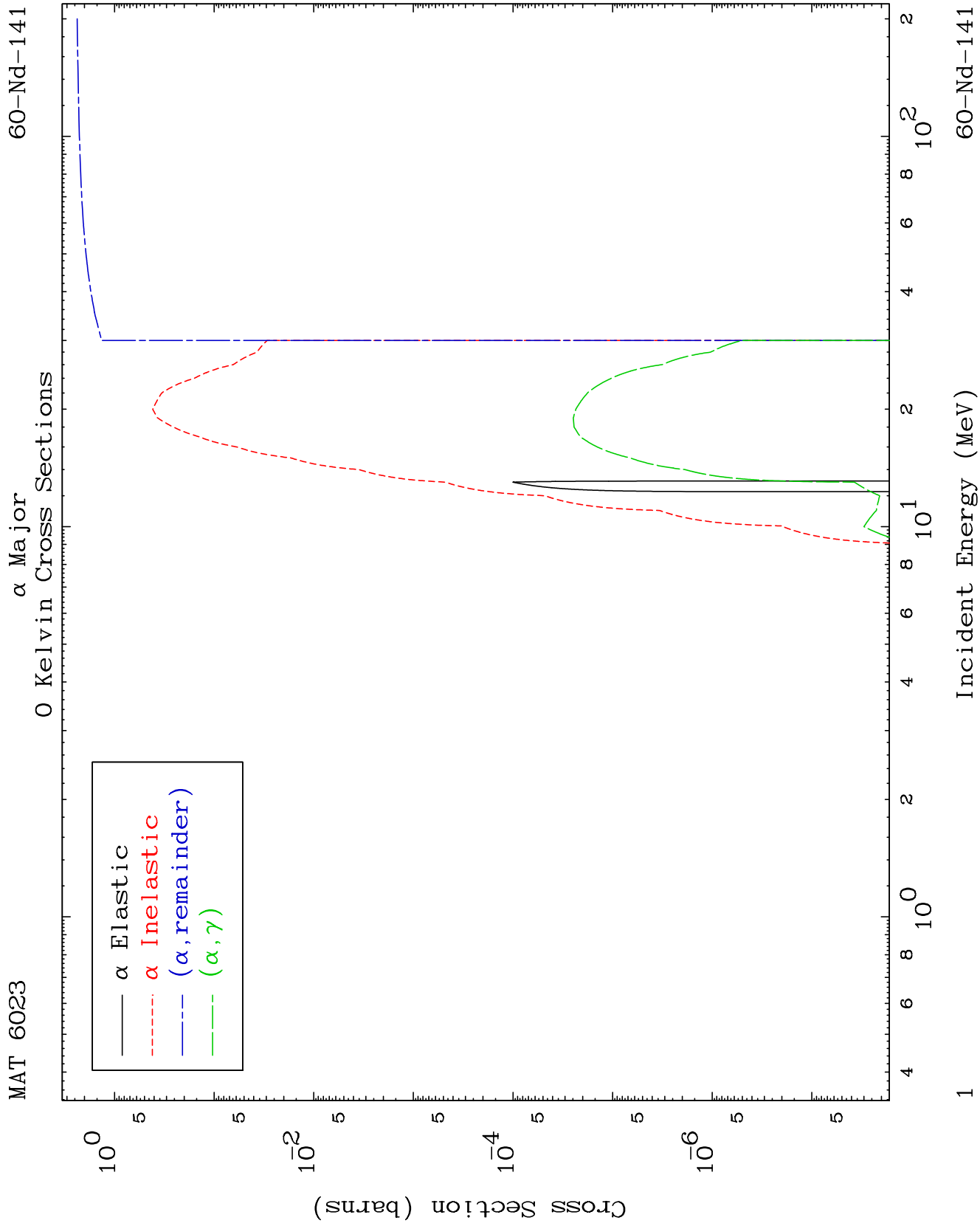
by

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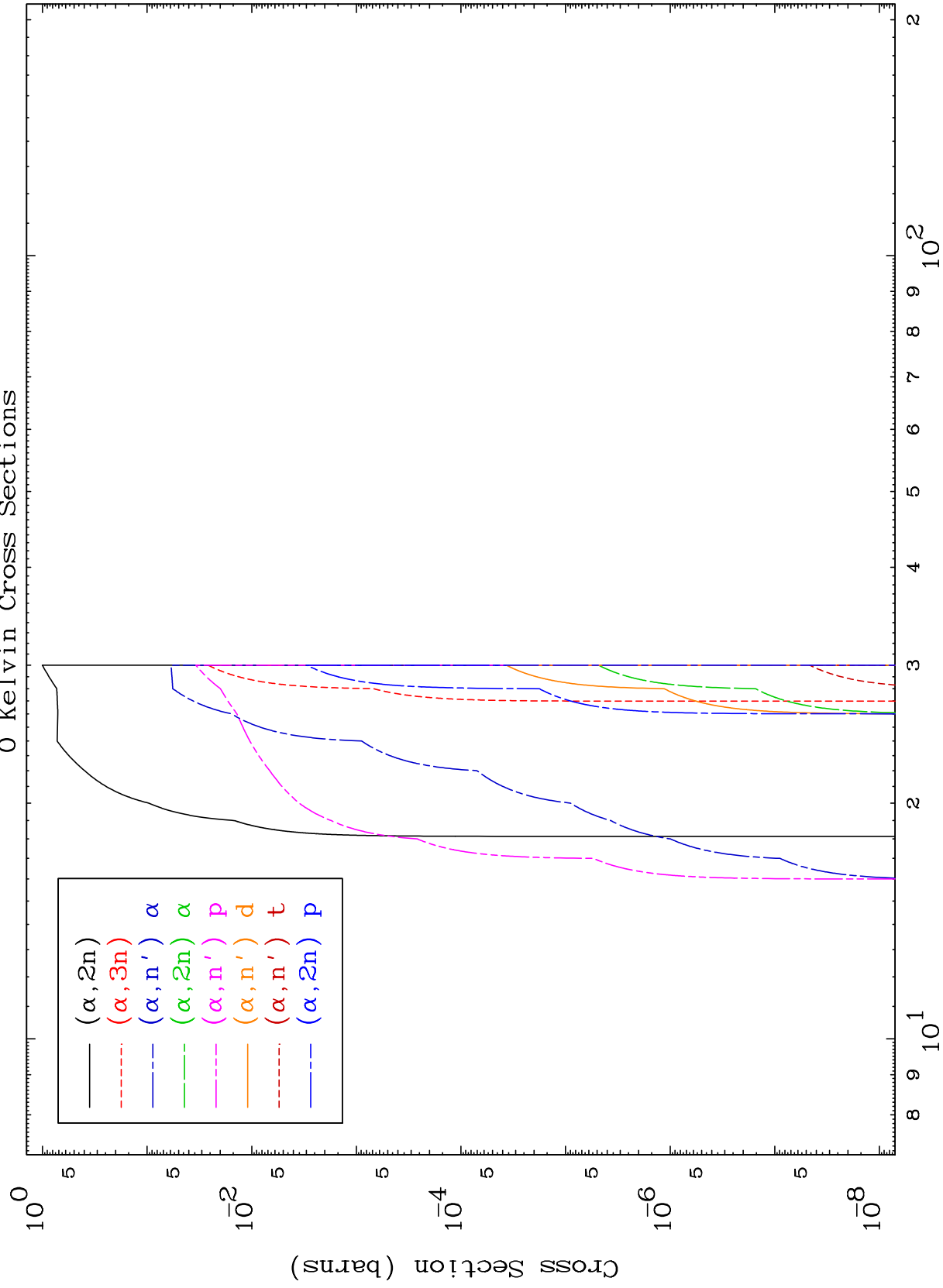
Press Mouse Button to Start



MAT 6023

α Neutron Production
0 Kelvin Cross Sections

60-Nd-141



2

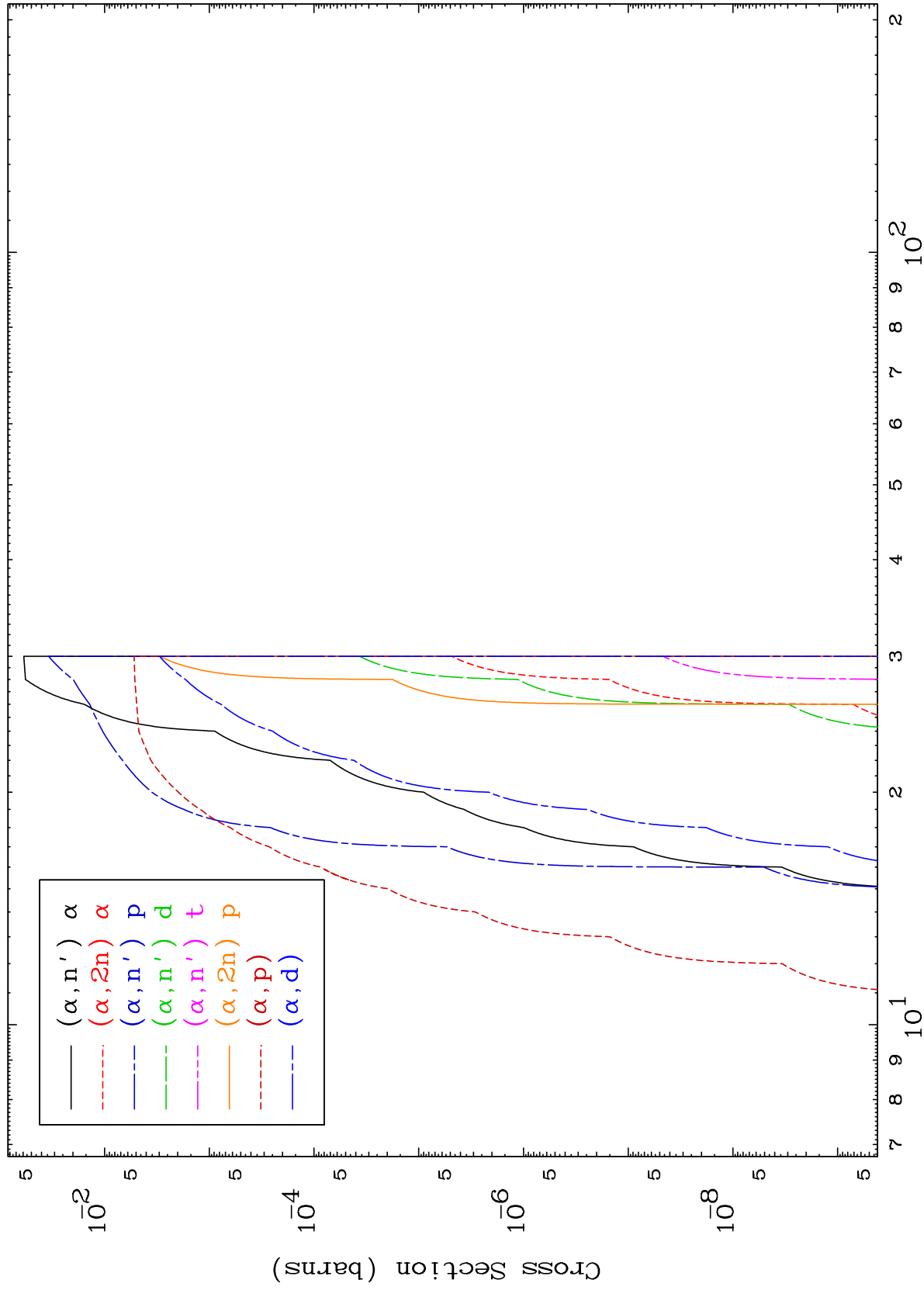
Incident Energy (MeV)

60-Nd-141

MAT 6023

α Charged Particle
0 Kelvin Cross Sections

60-Nd-141



3

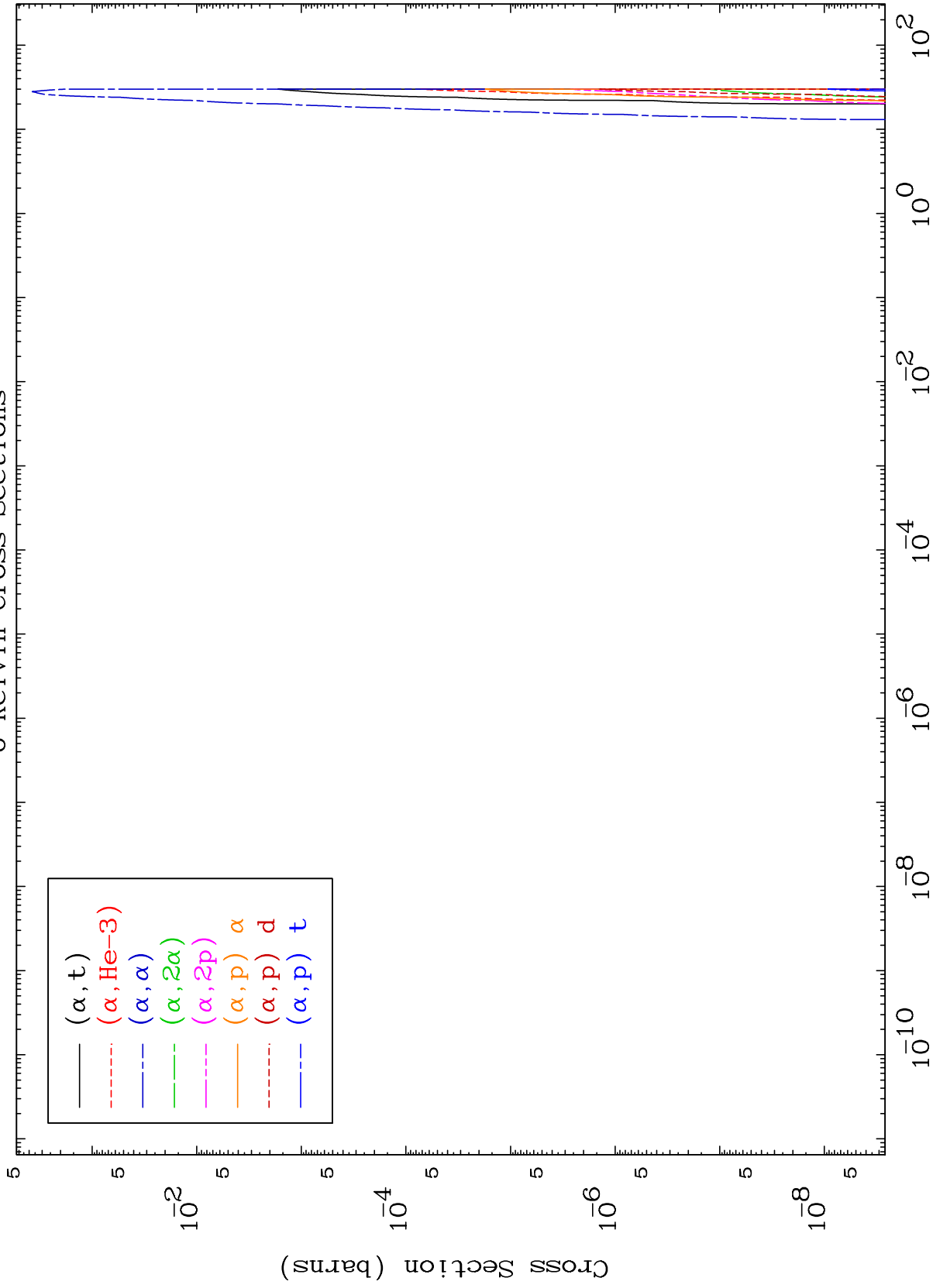
Incident Energy (MeV)

60-Nd-141

MAT 6023

α Charged Particle
0 Kelvin Cross Sections

60-Nd-141

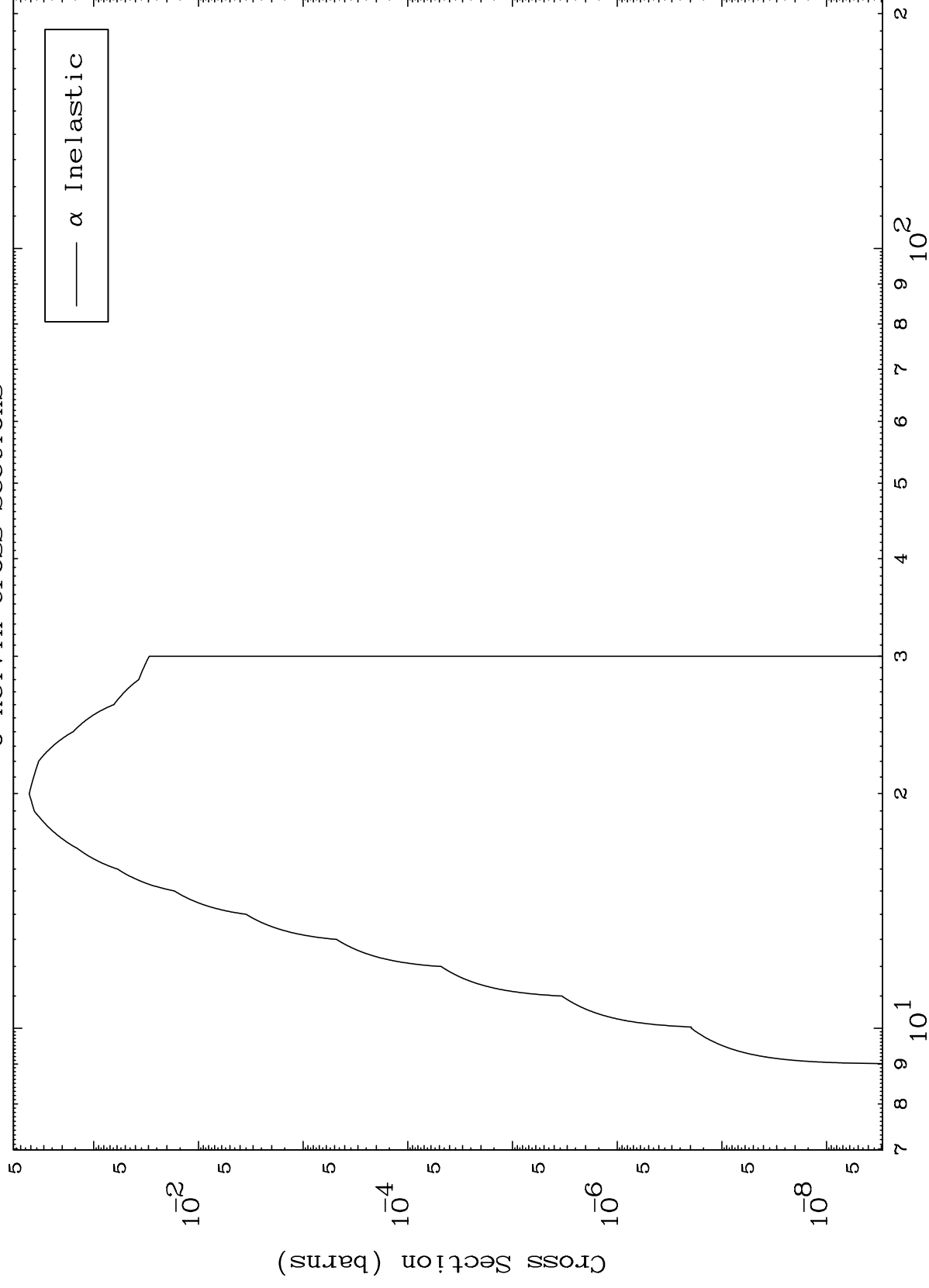


MAT 6023

 (α, n') Level

60-Nd-141

0 Kelvin Cross Sections



5

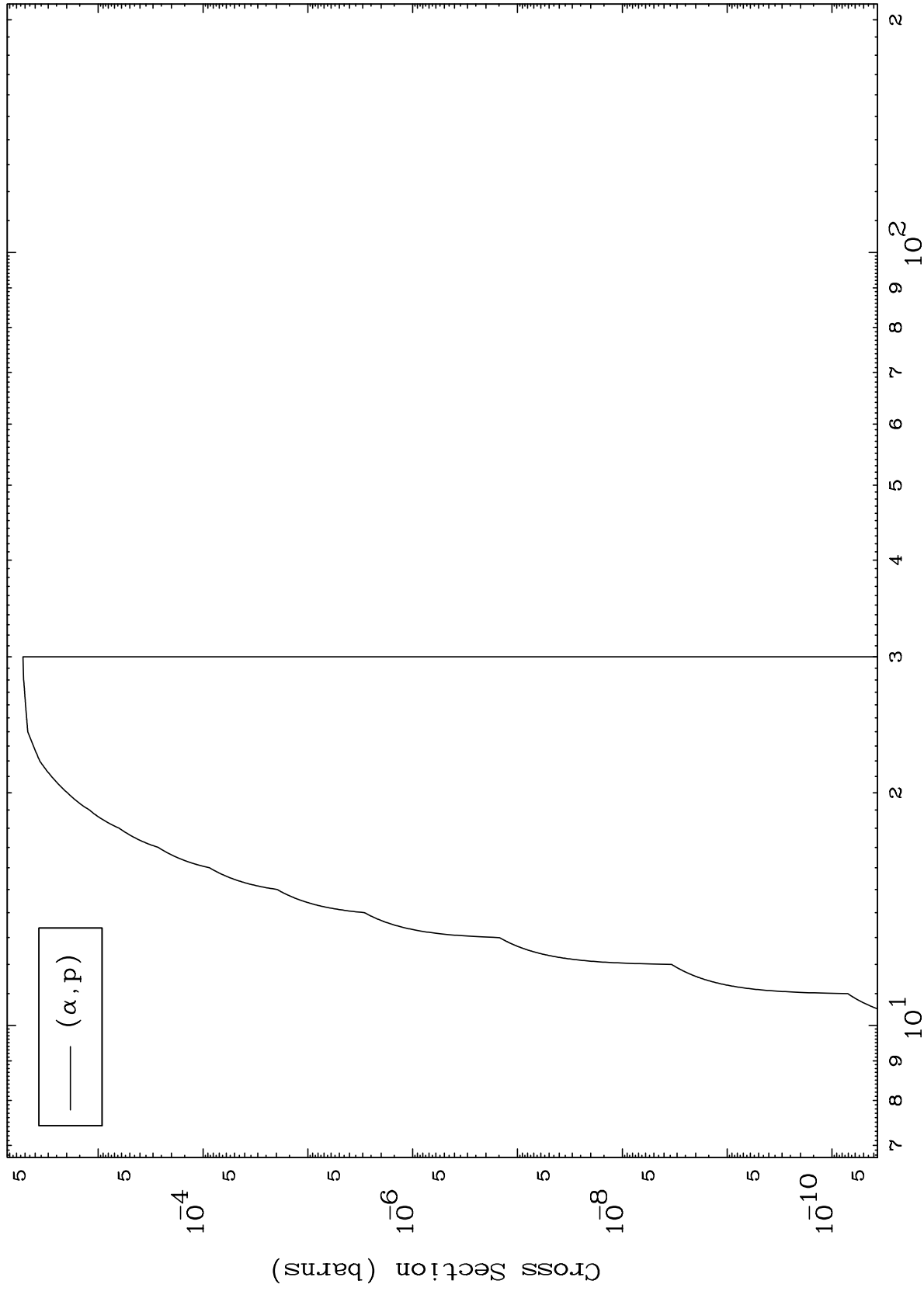
Incident Energy (MeV)

 $^{60}\text{Nd}-141$

MAT 6023

60-Nd-141

(α ,p) Levels
0 Kelvin Cross Sections



6

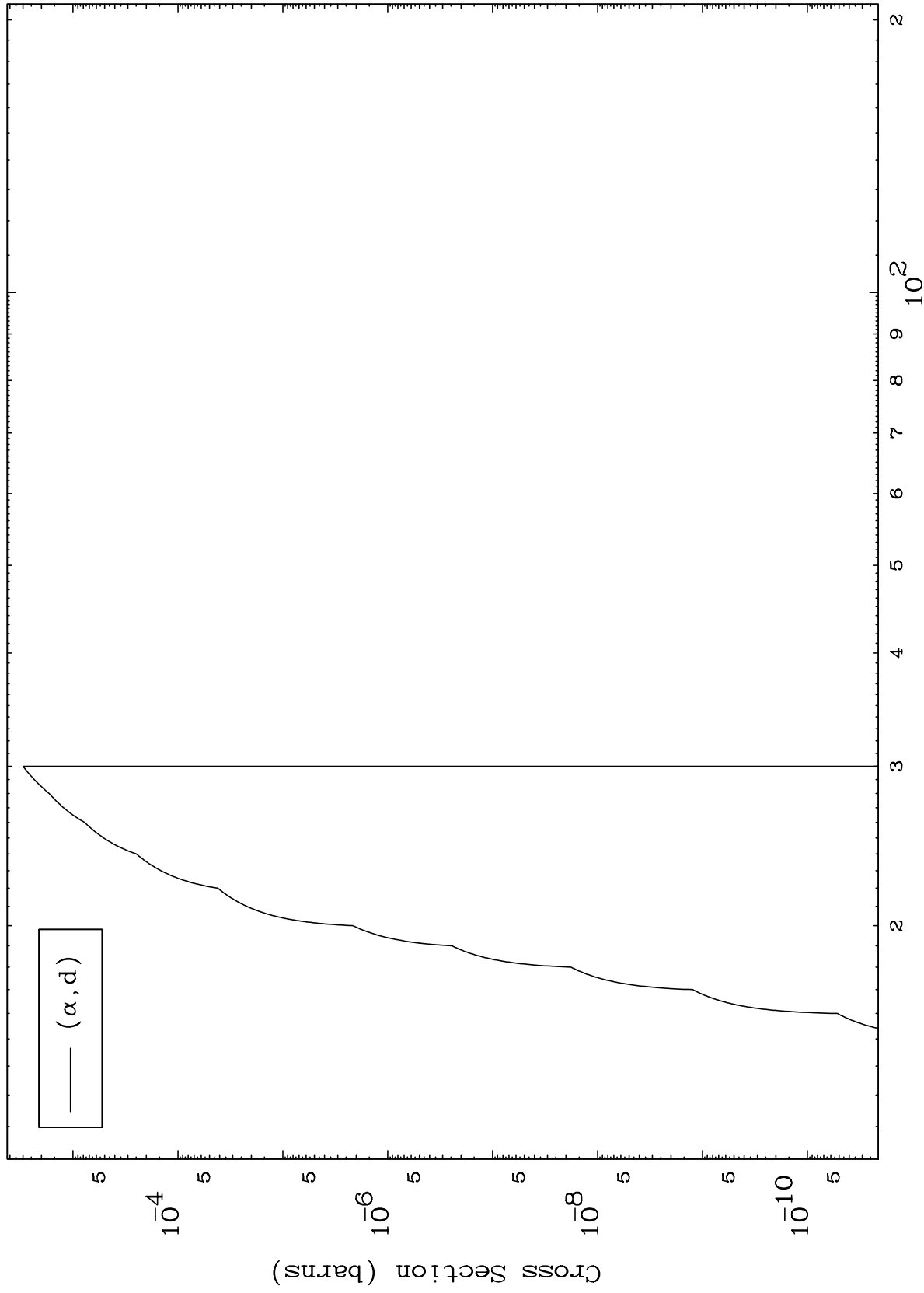
Incident Energy (MeV)

60-Nd-141

MAT 6023

(α ,d) Levels
0 Kelvin Cross Sections

60-Nd-141



7

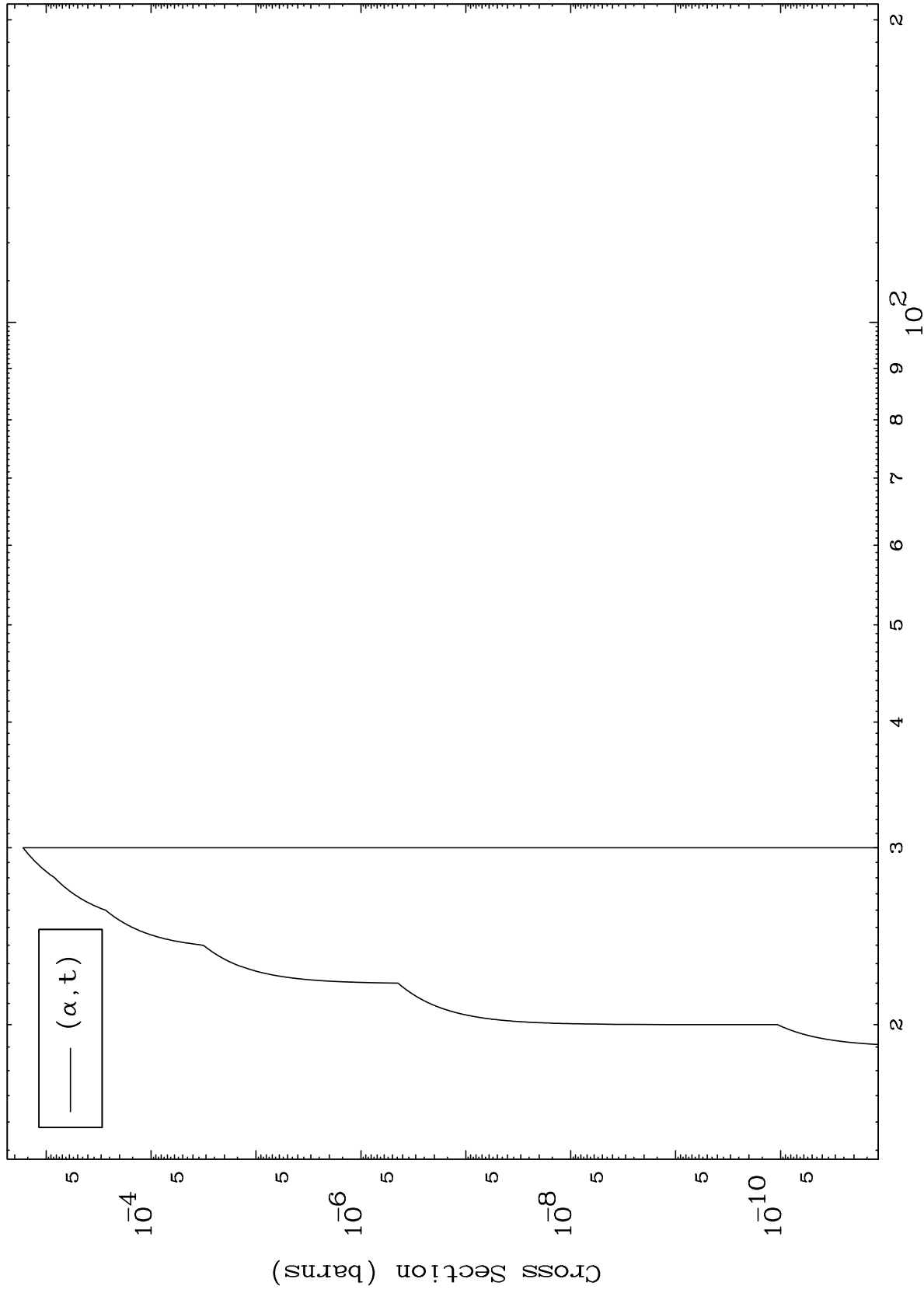
Incident Energy (MeV)

60-Nd-141

MAT 6023

60-Nd-141

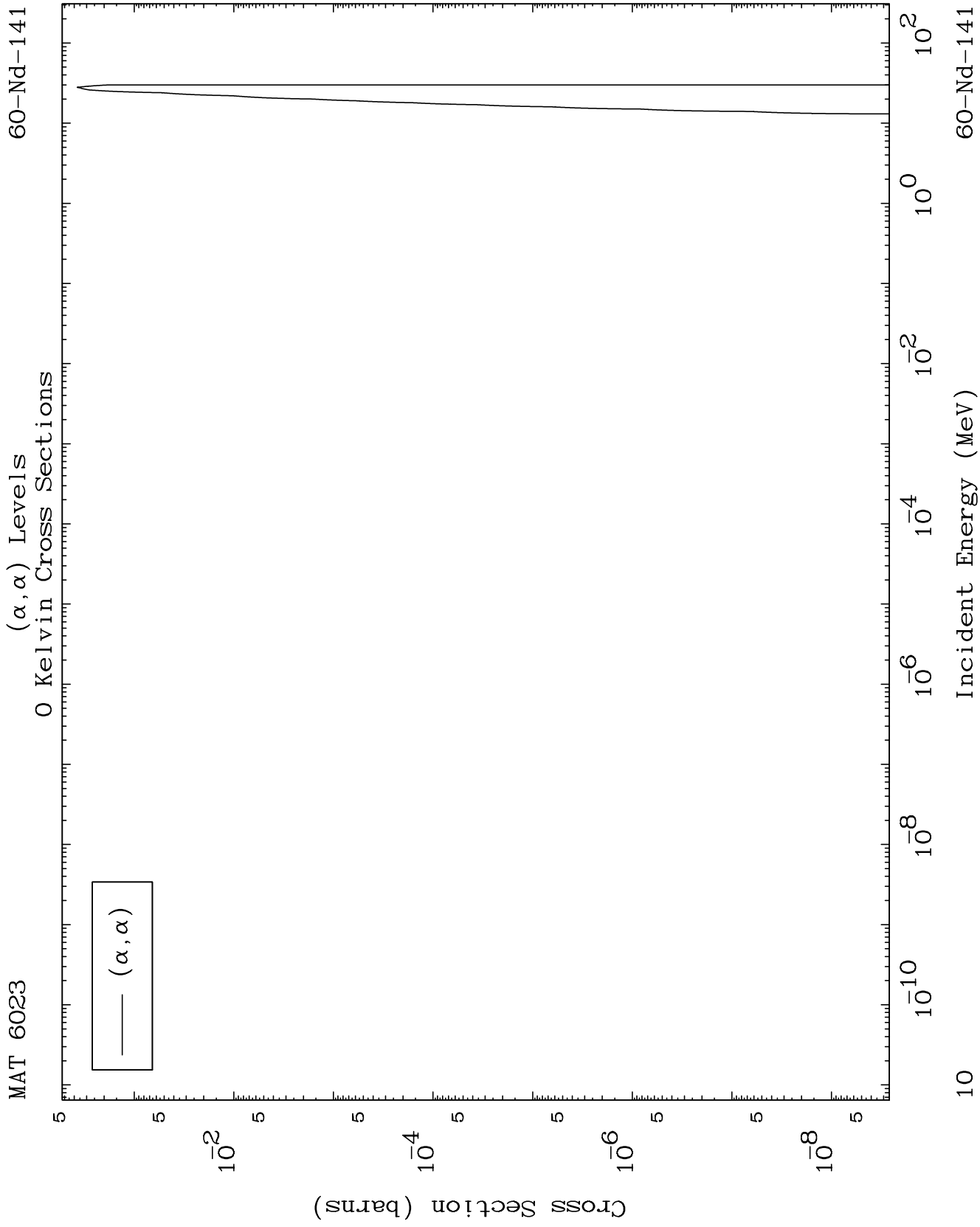
(α, t) Levels
0 Kelvin Cross Sections

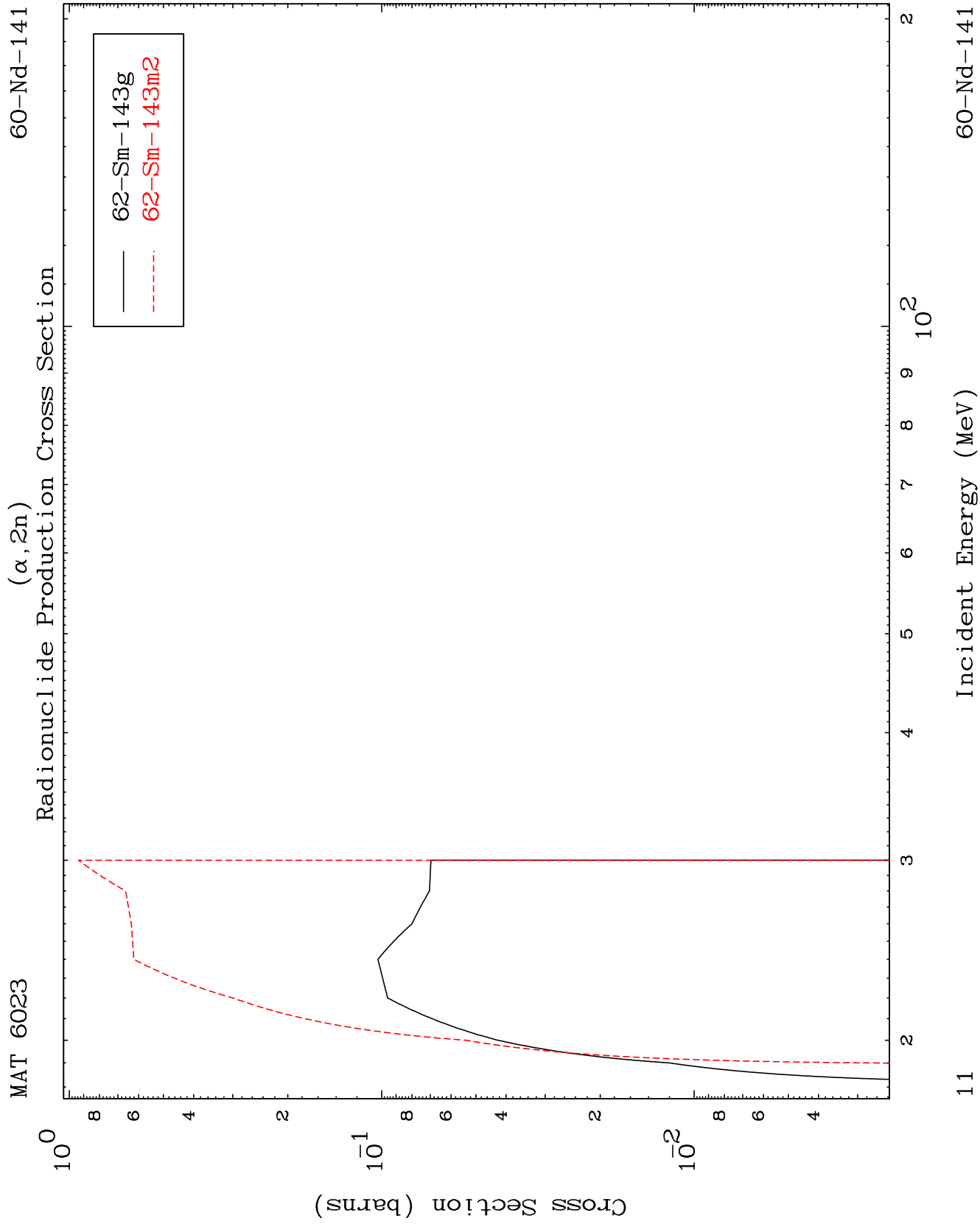


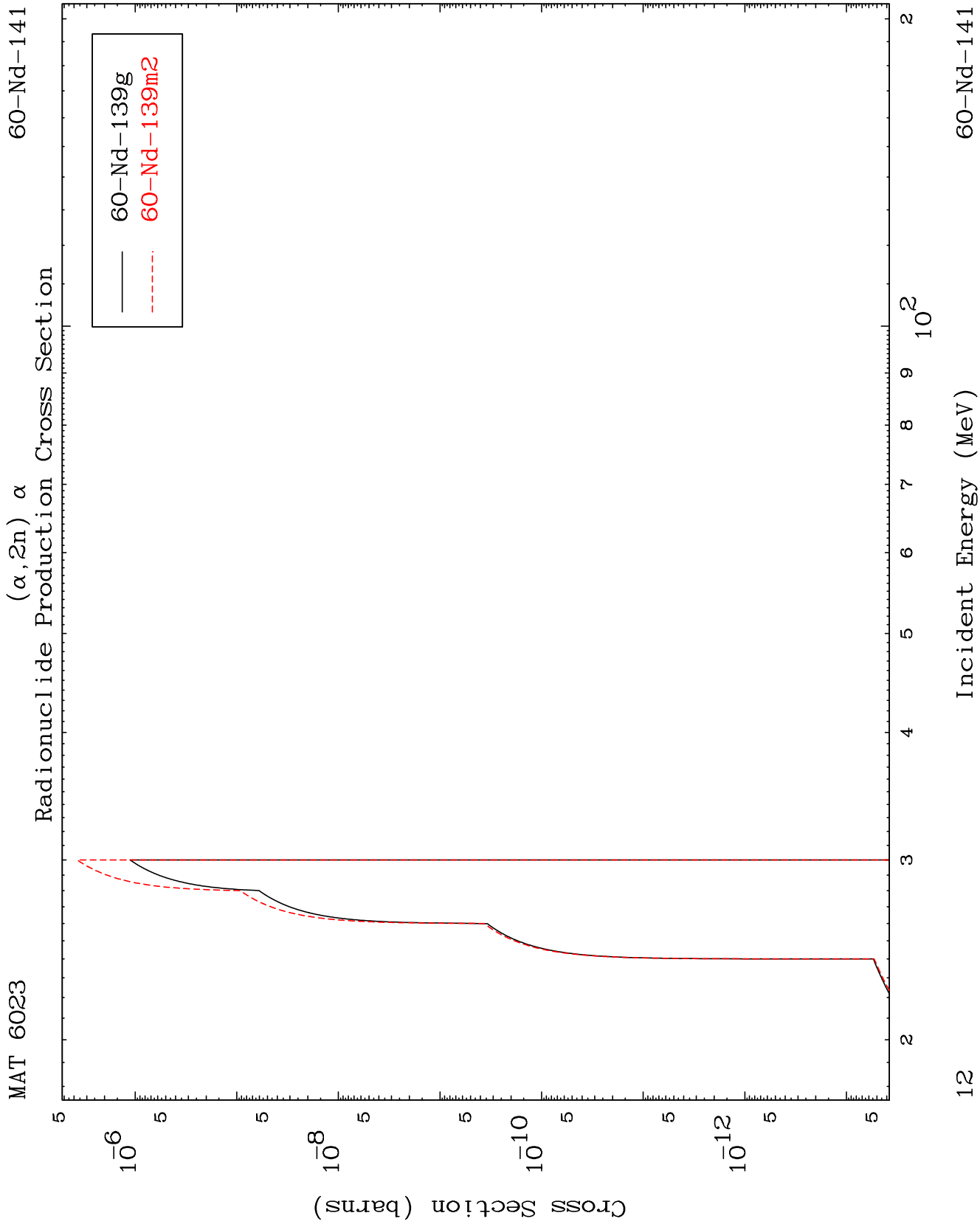
8

Incident Energy (MeV)

60-Nd-141





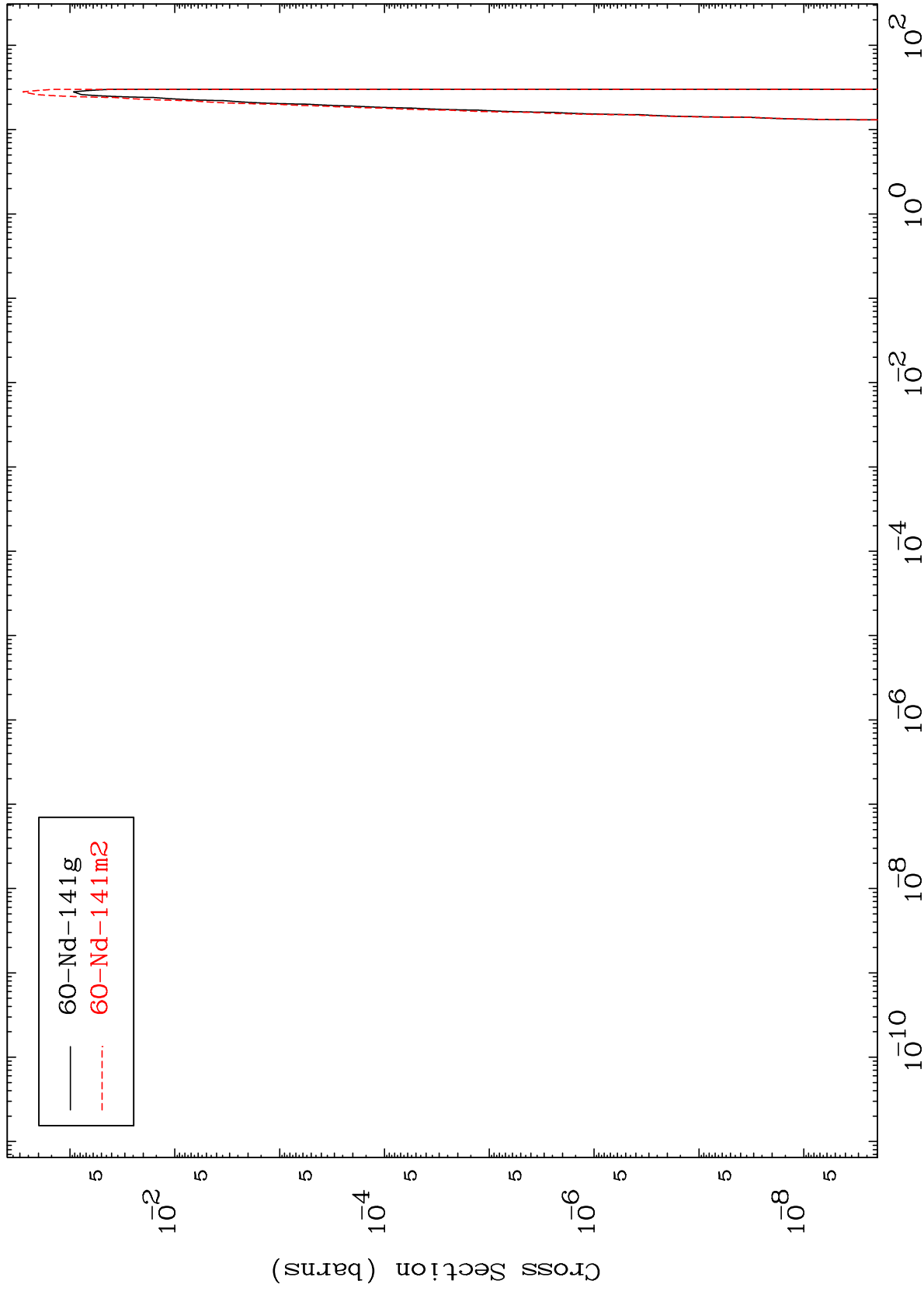


MAT 6023

(α, α)

60-Nd-141

Radionuclide Production Cross Section



13

Incident Energy (MeV)

60-Nd-141

MAT 6023

Radionuclide Production Cross Section
($\alpha, 2\alpha$)

60-Nd-141

