

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

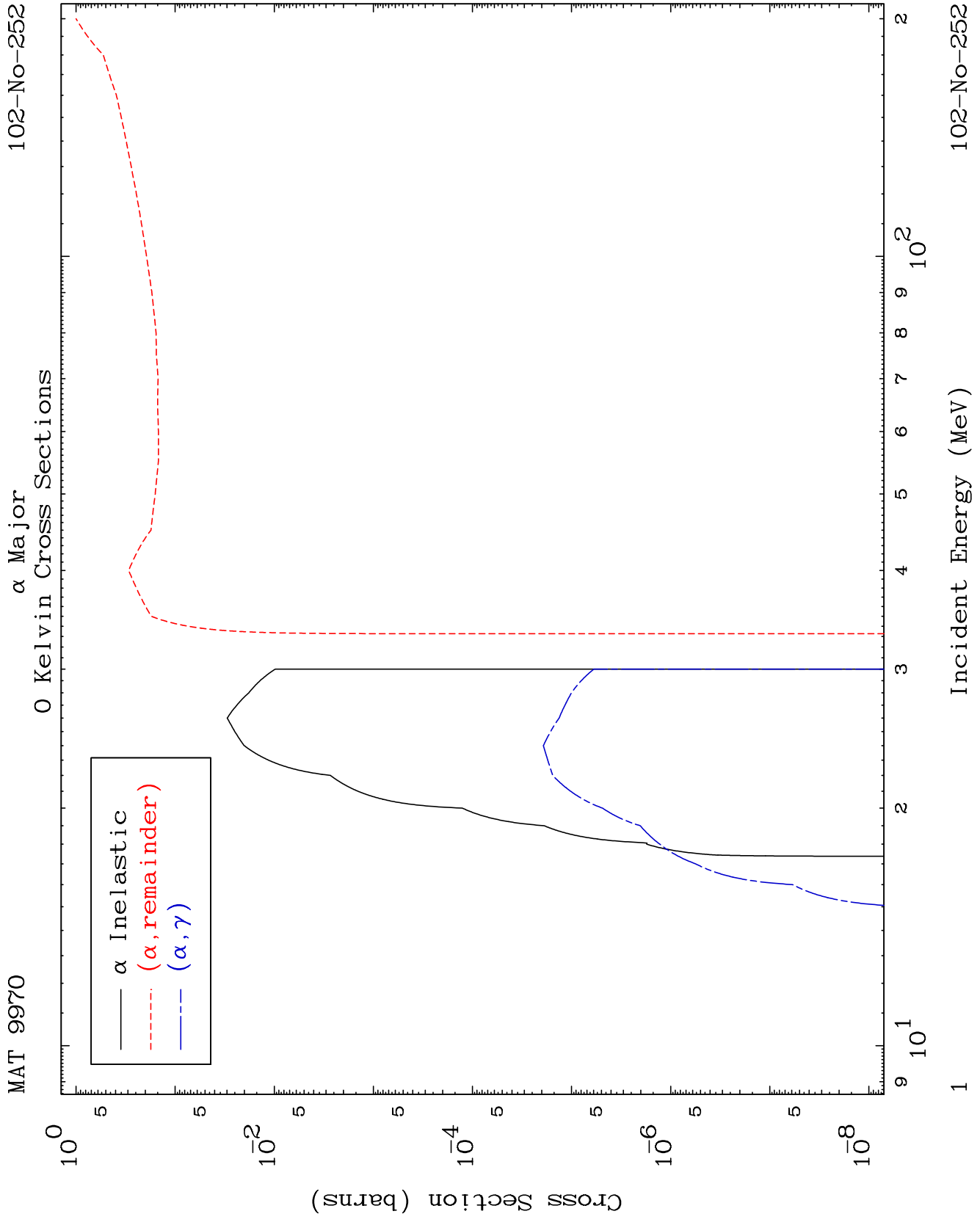
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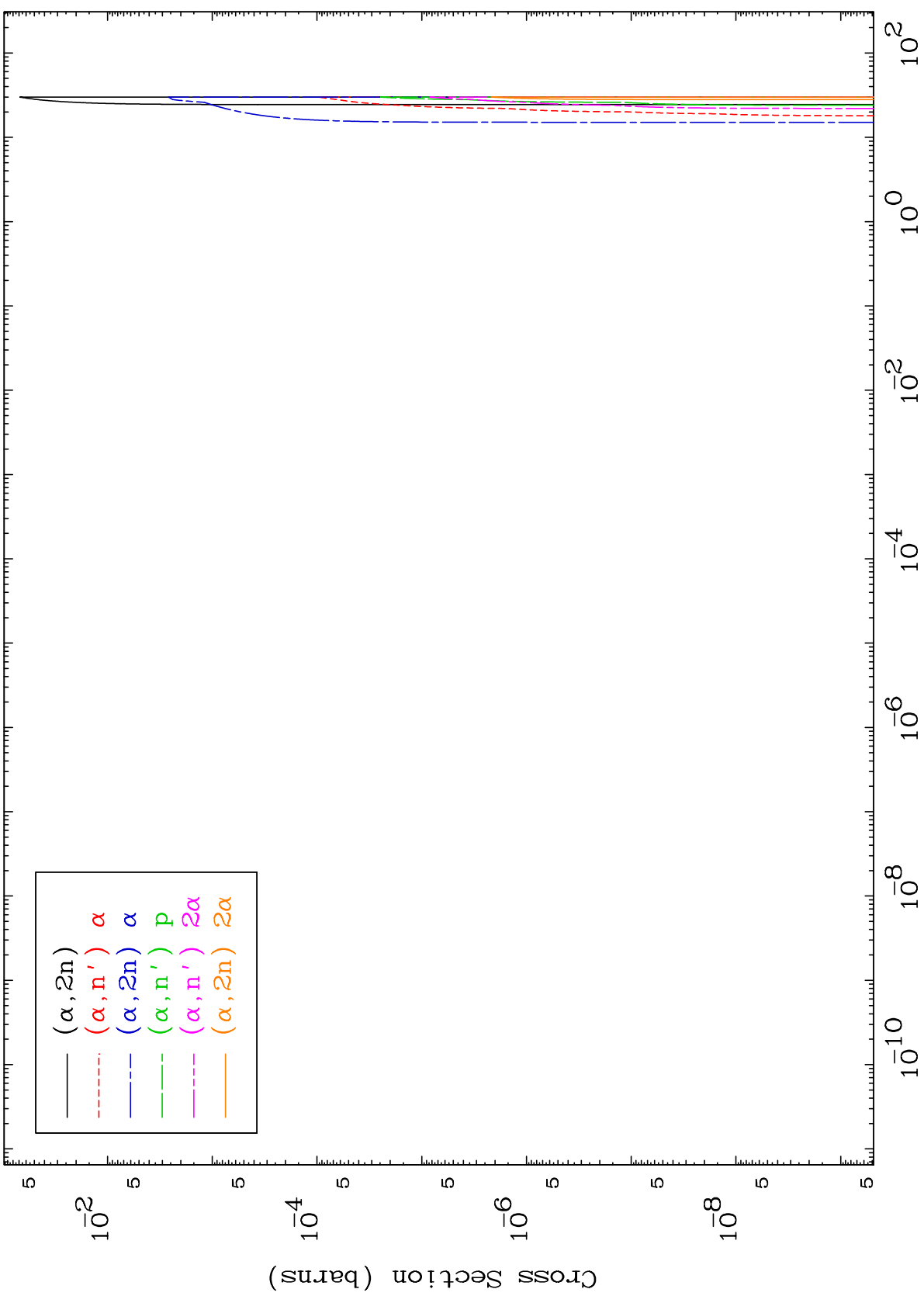
Press Mouse Button to Start



MAT 9970

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

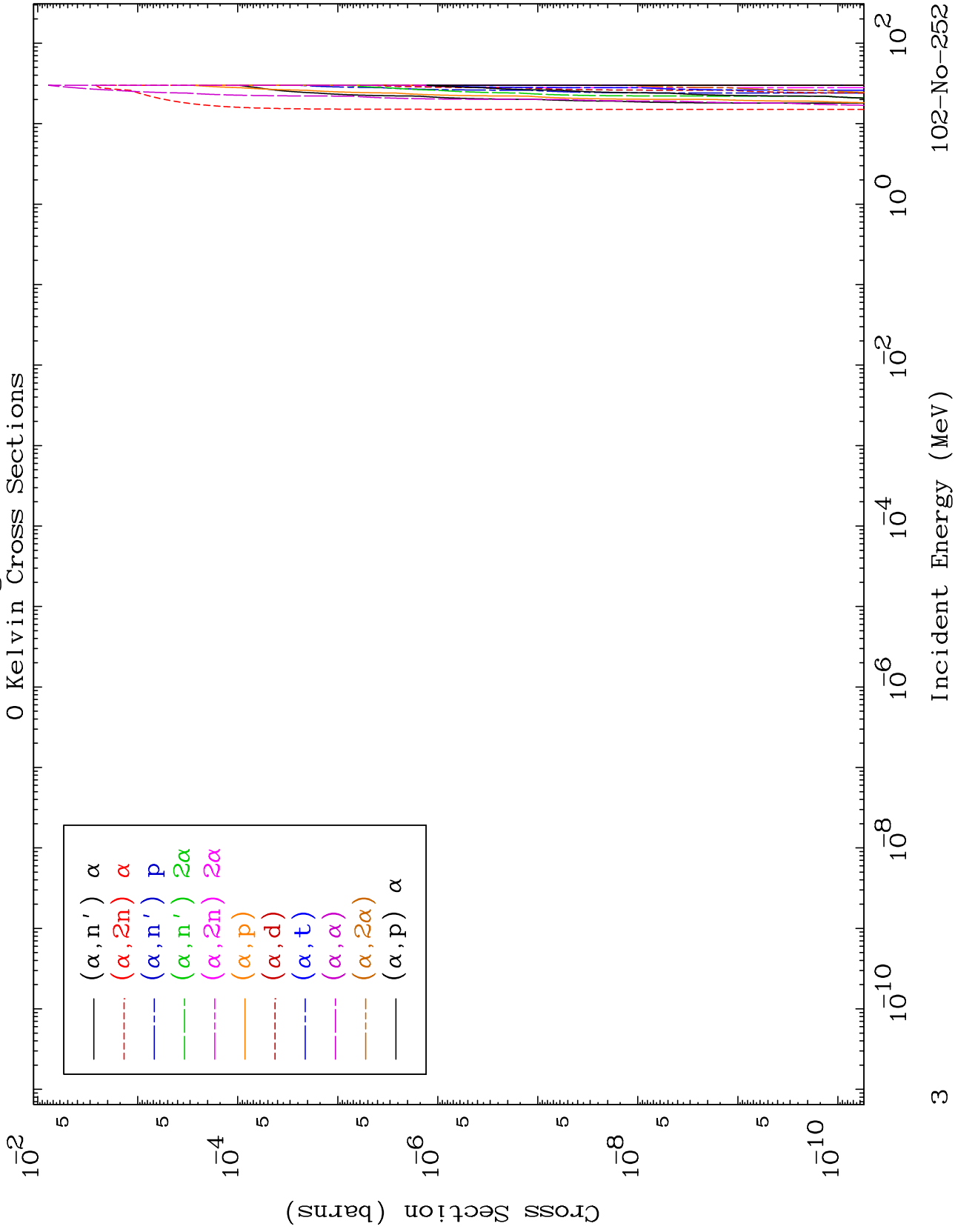
102-No-252



MAT 9970

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

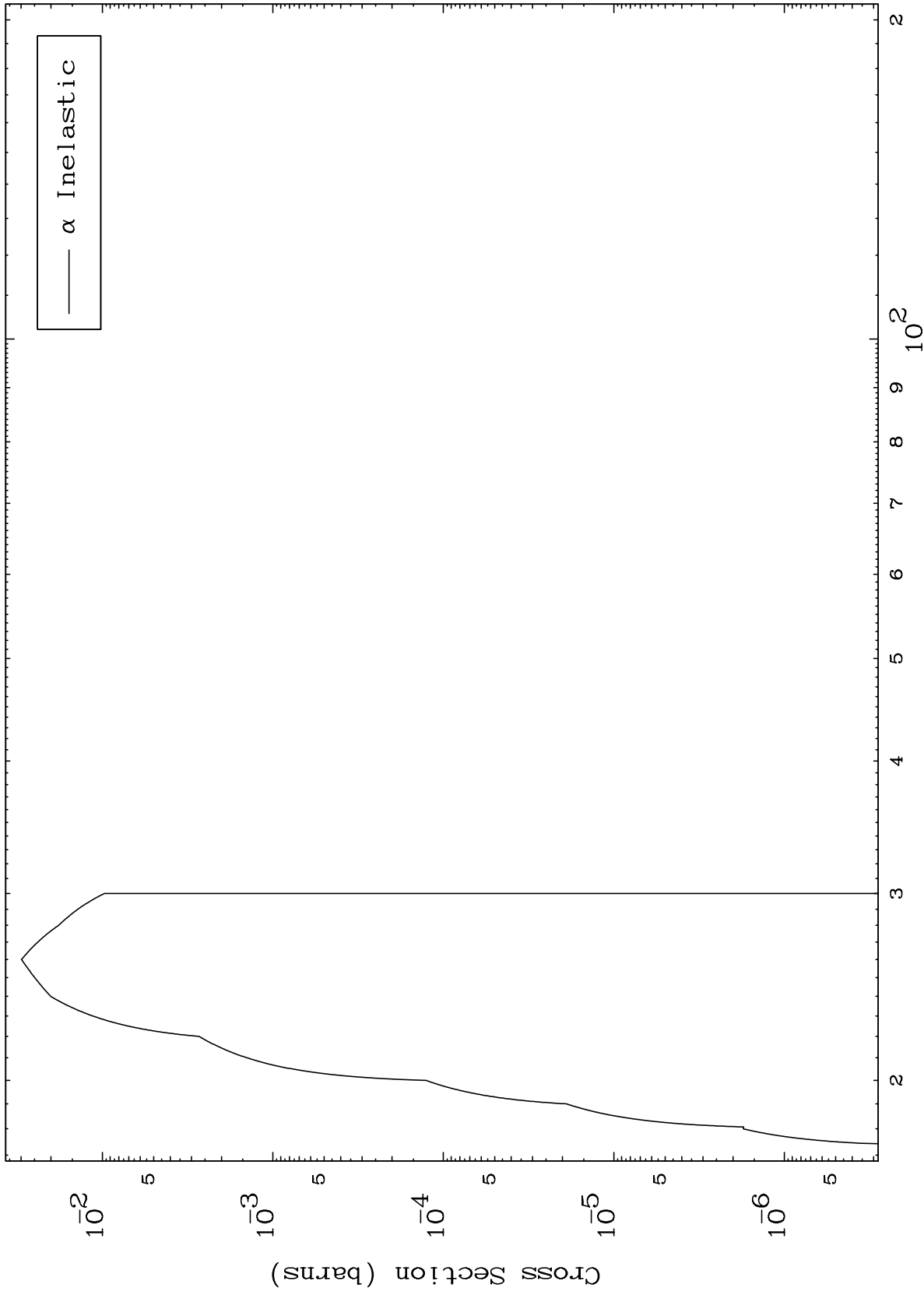
102-No-252



MAT 9970

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

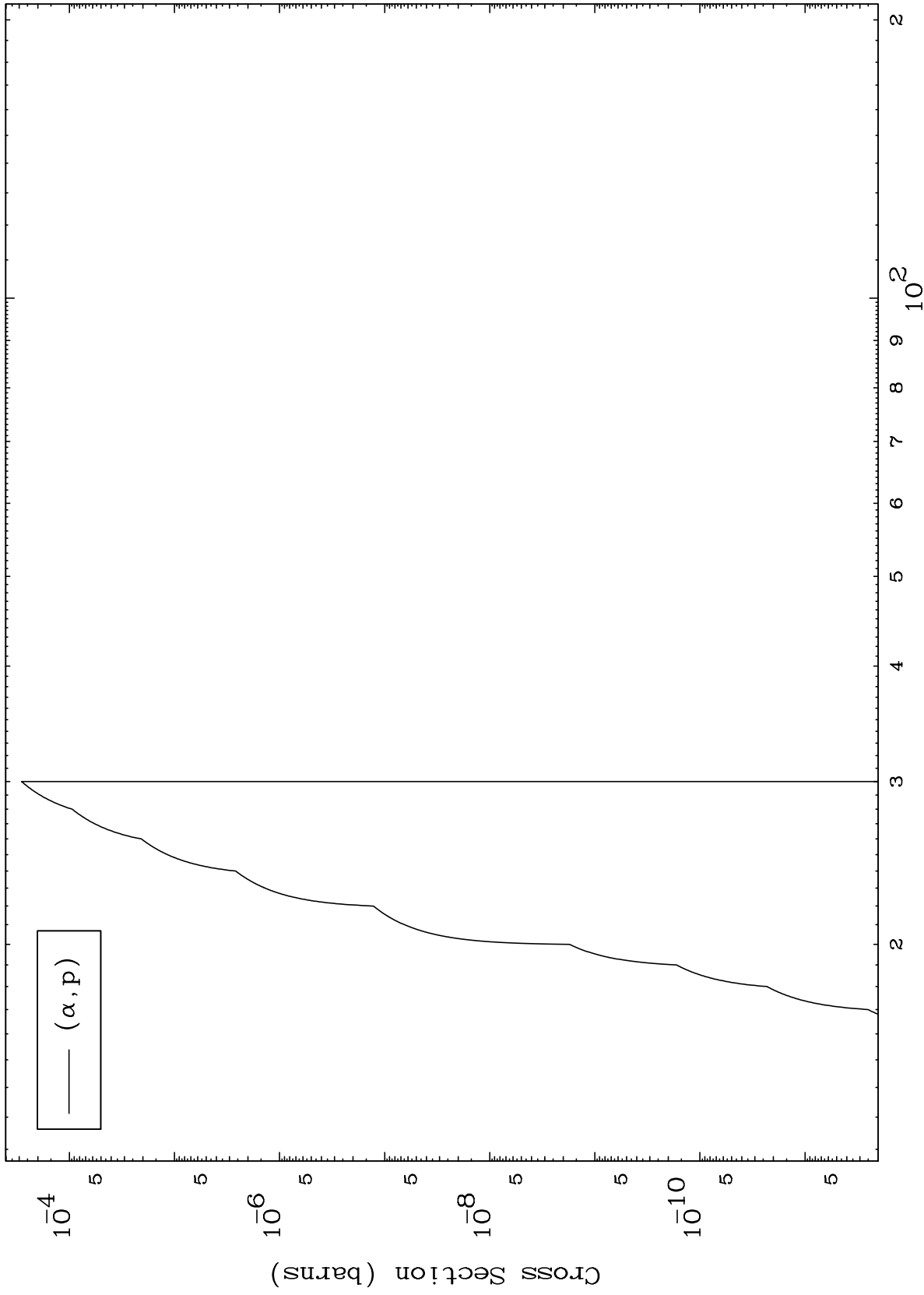
102-No-252



4

Incident Energy (MeV)

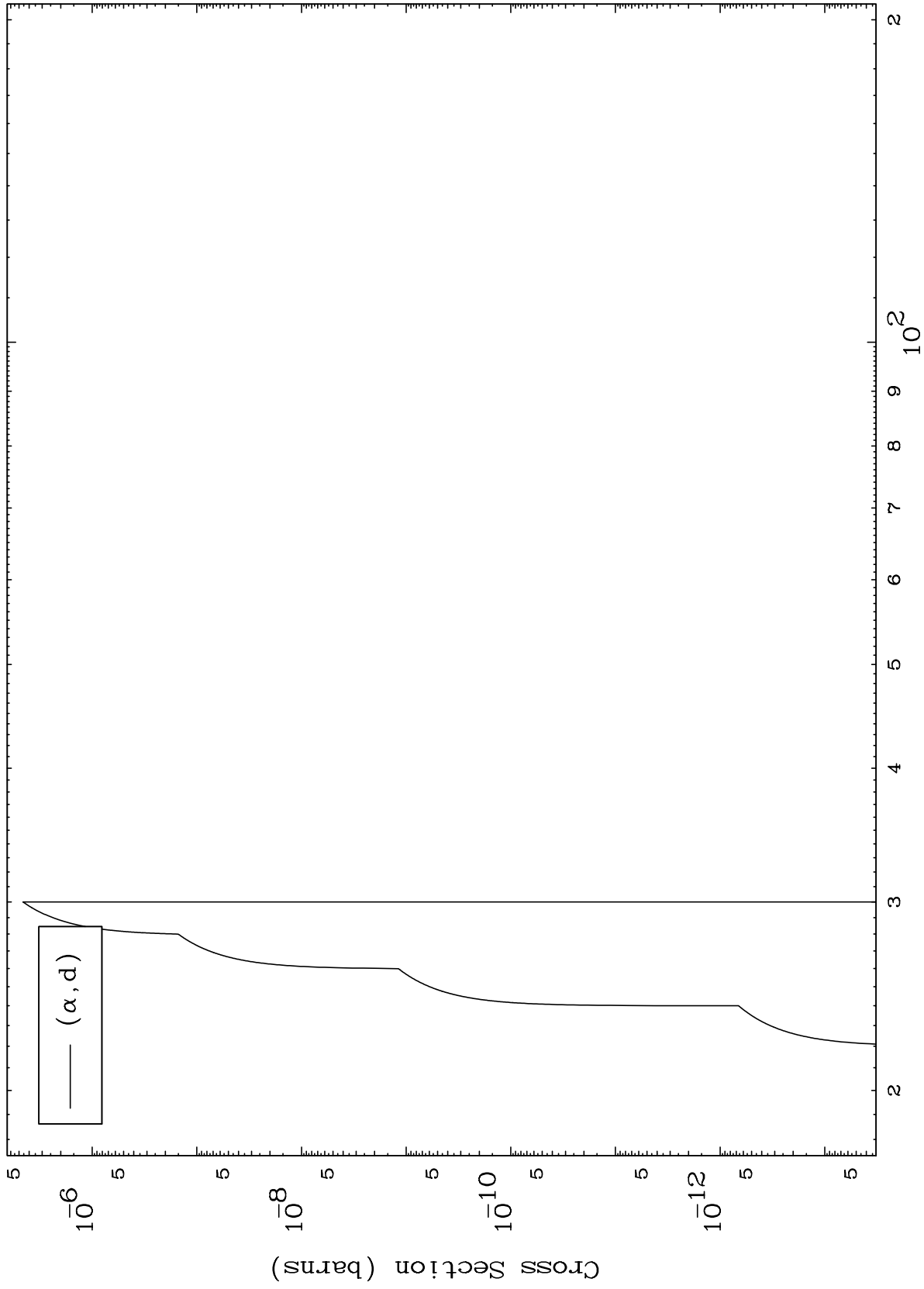
102-No-252



MAT 9970

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

102-No-252



6

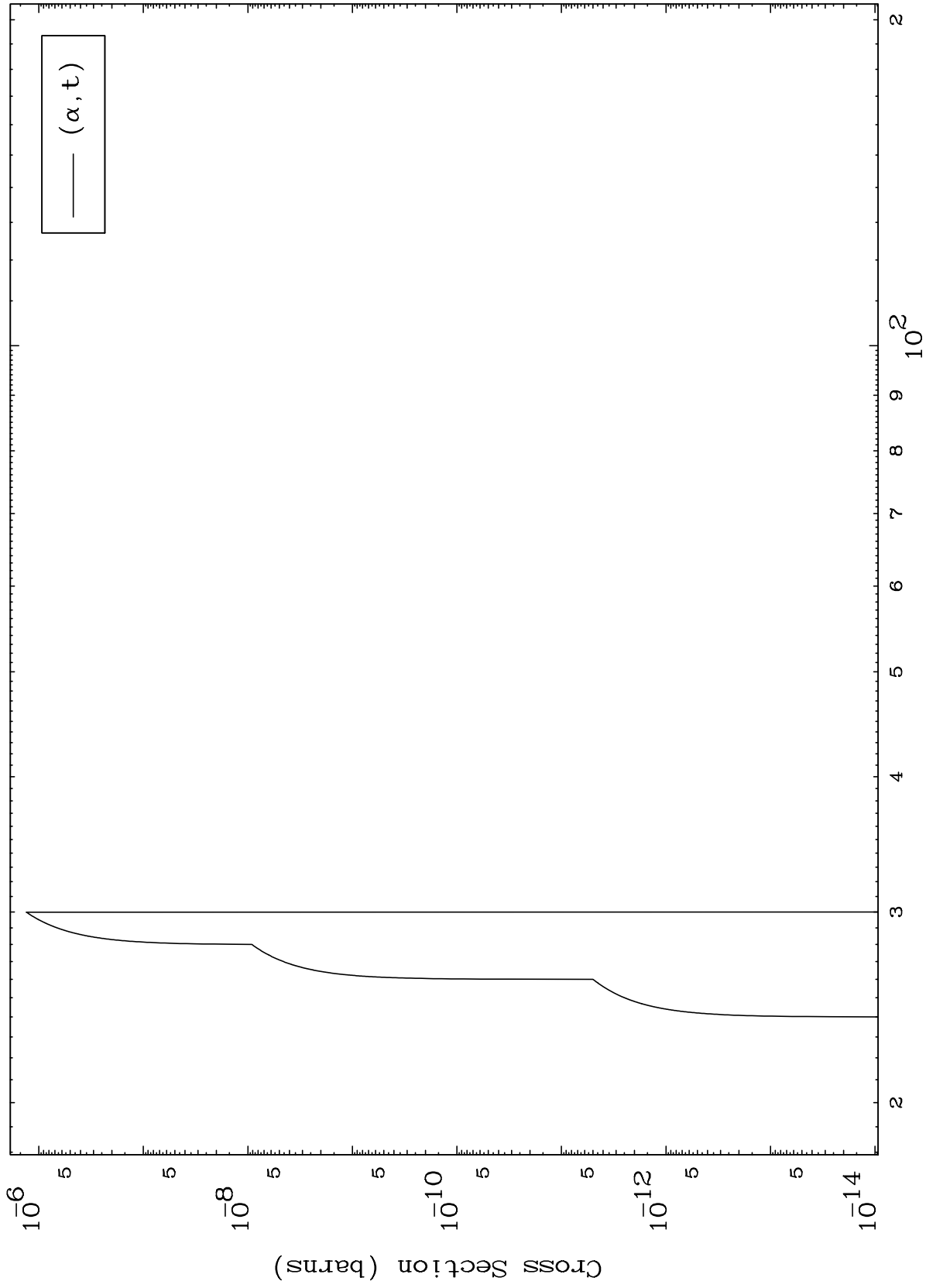
Incident Energy (MeV)

102-No-252

MAT 9970

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

102-No-252



7

Incident Energy (MeV)

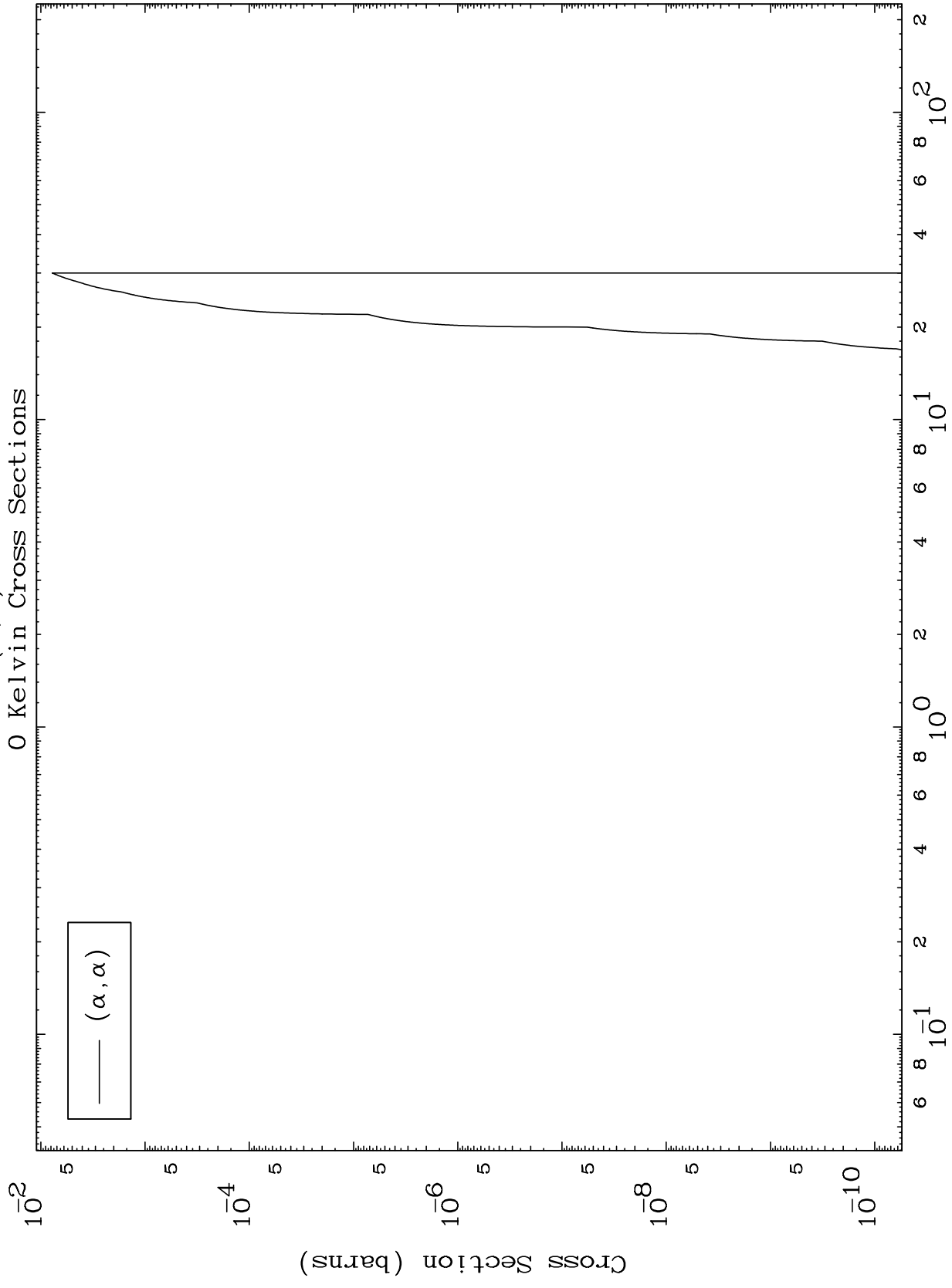
102-No-252



MAT 9970

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

102-No-252



8

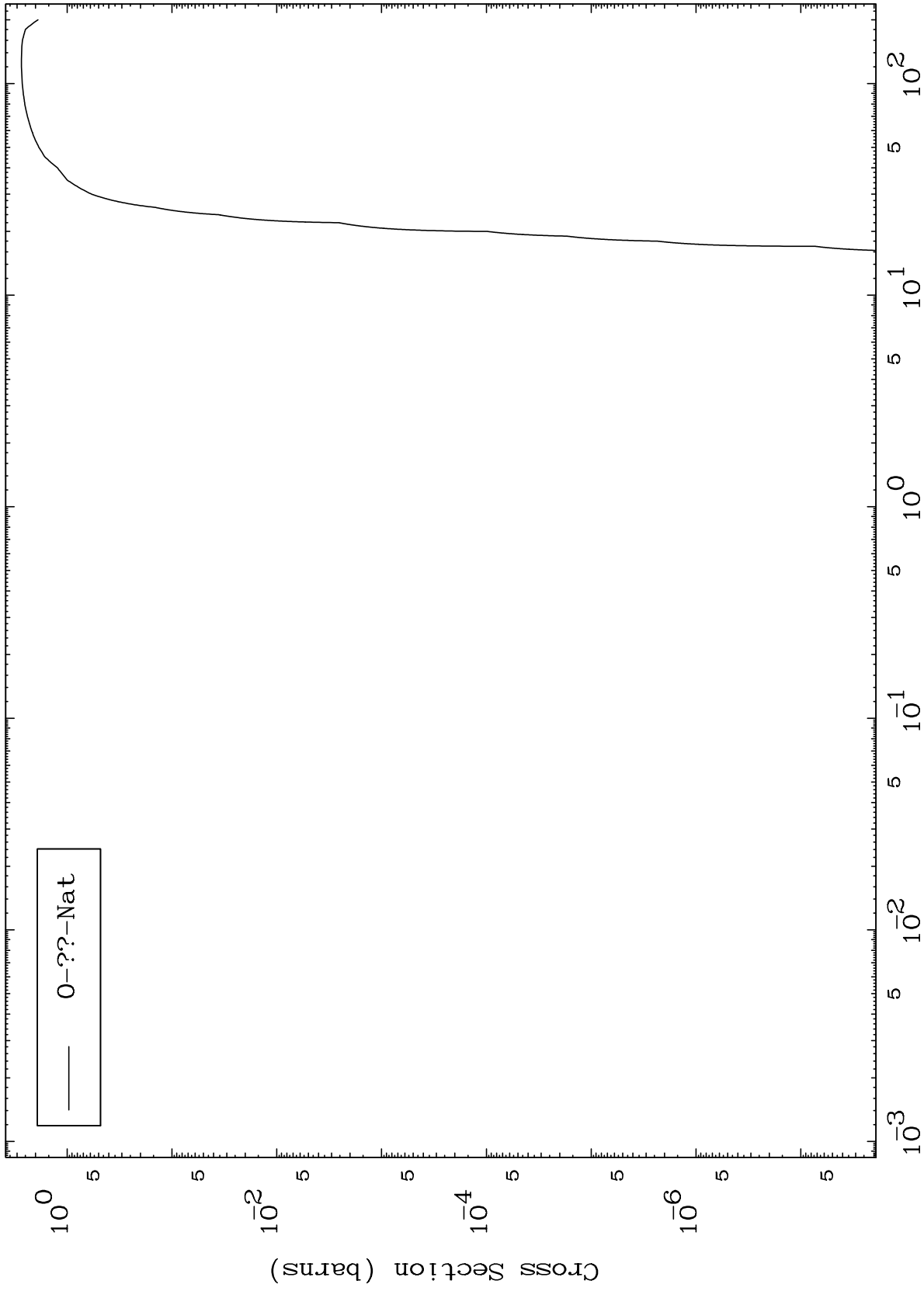
Incident Energy (MeV)

102-No-252

MAT 9970

102-No-252

$\alpha$  Fission  
Radionuclide Production Cross Section



Incident Energy (MeV)

102-No-252

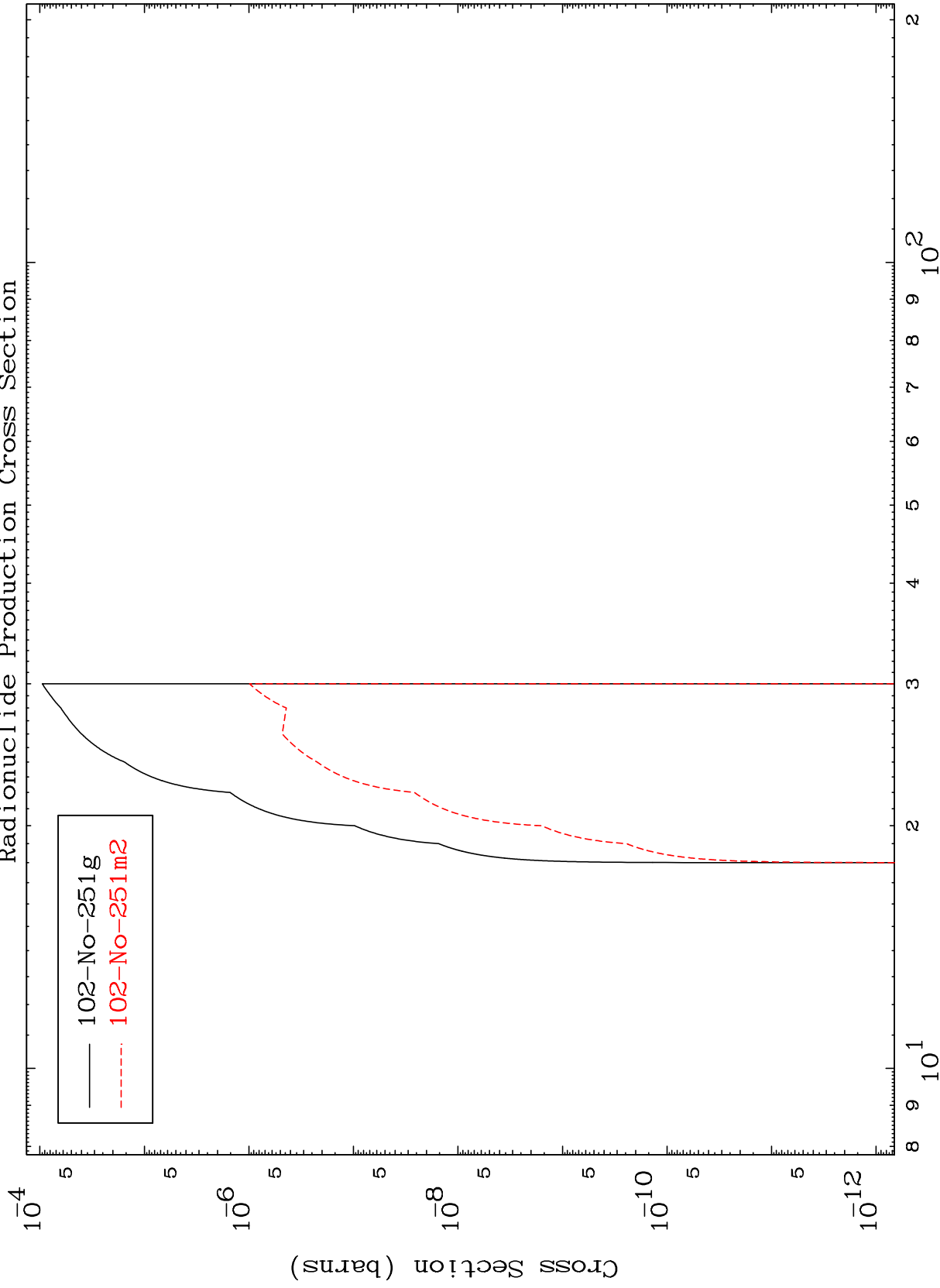
9

MAT 9970

$(\alpha, n')$   $\alpha$

102-No-252

Radionuclide Production Cross Section



10

Incident Energy (MeV)

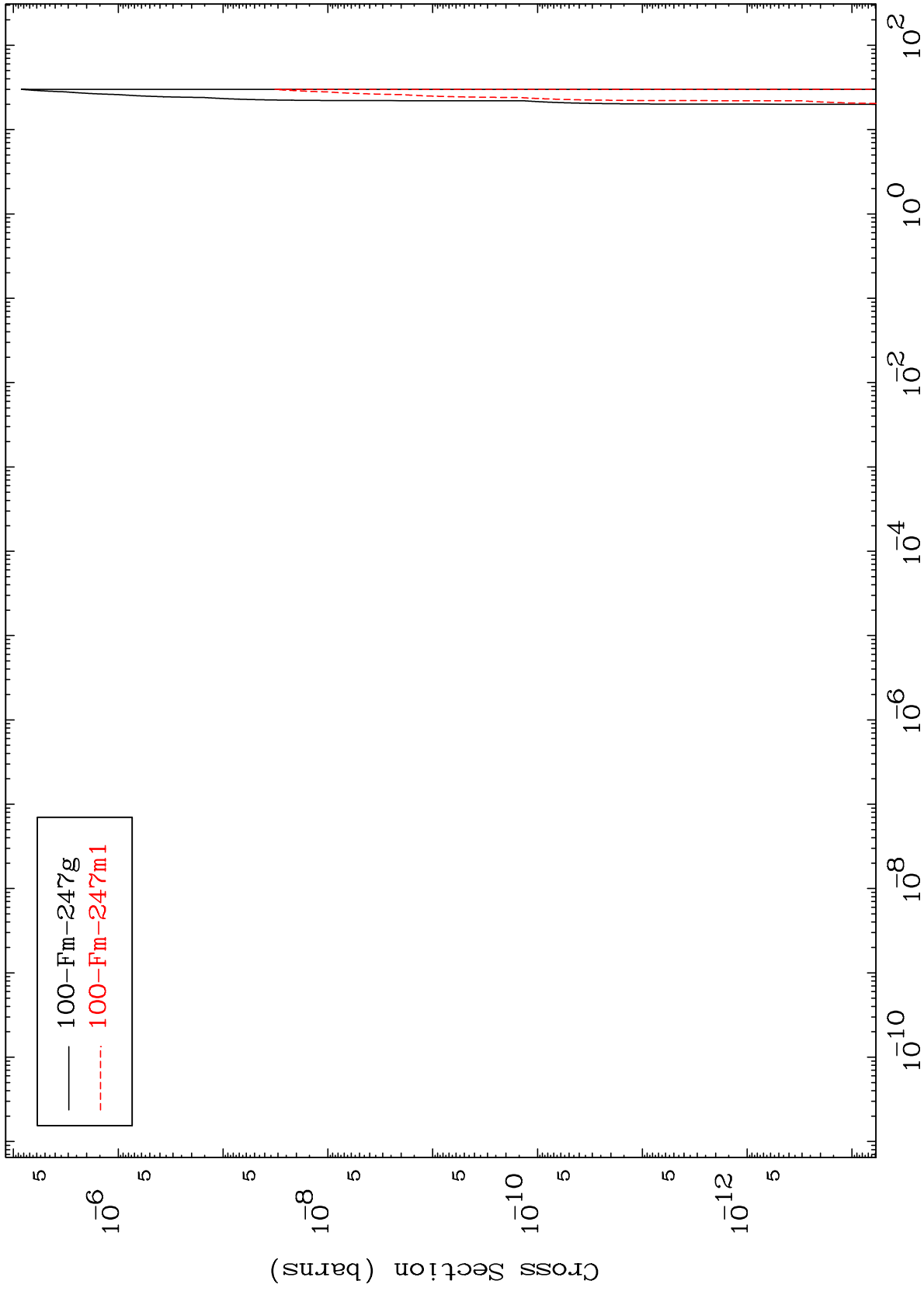
102-No-252

MAT 9970

( $\alpha, n'$ )  $2\alpha$

102-No-252

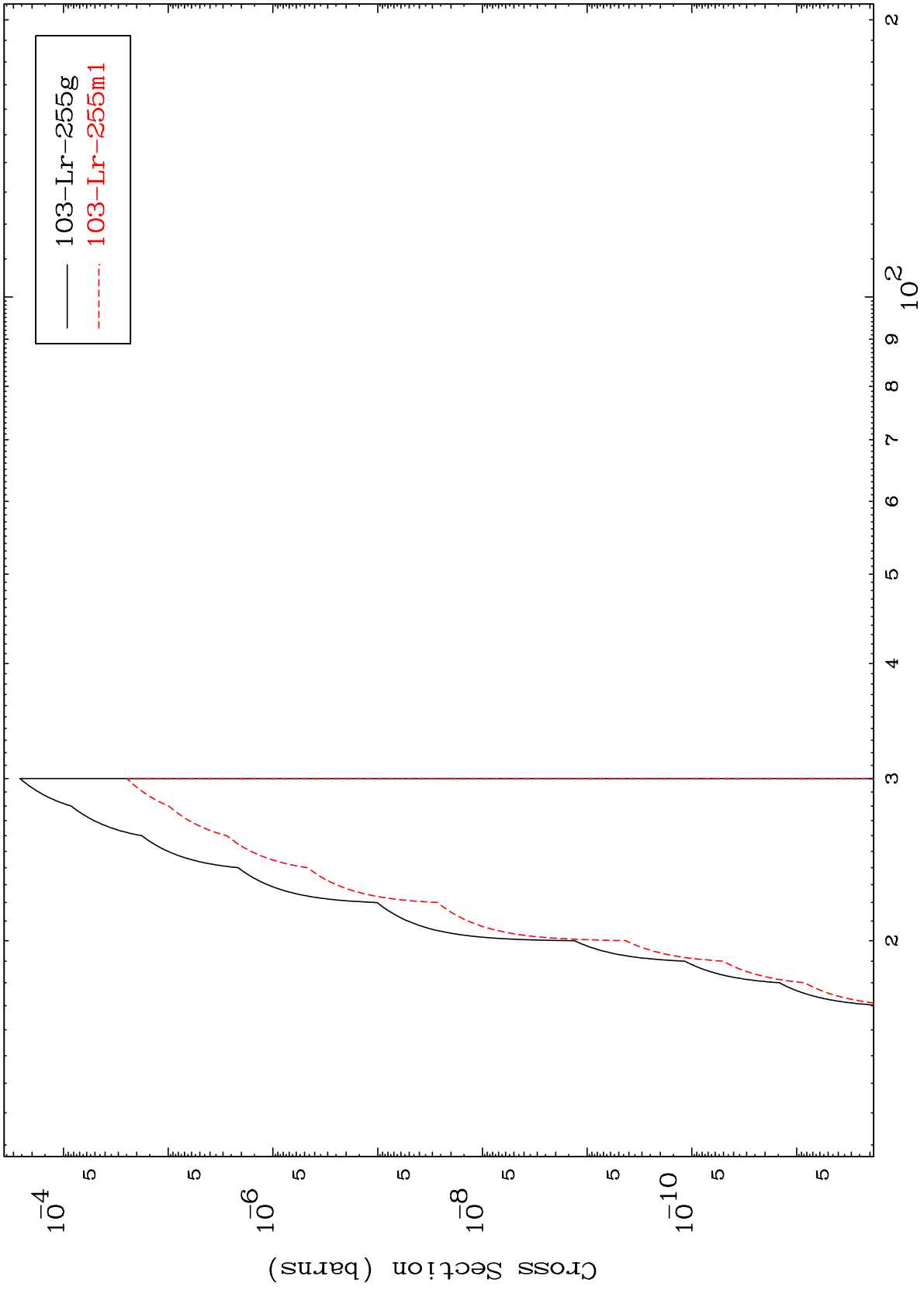
Radionuclide Production Cross Section



MAT 9970

102-No-252

( $\alpha, p$ )  
Radionuclide Production Cross Section



12

Incident Energy (MeV)

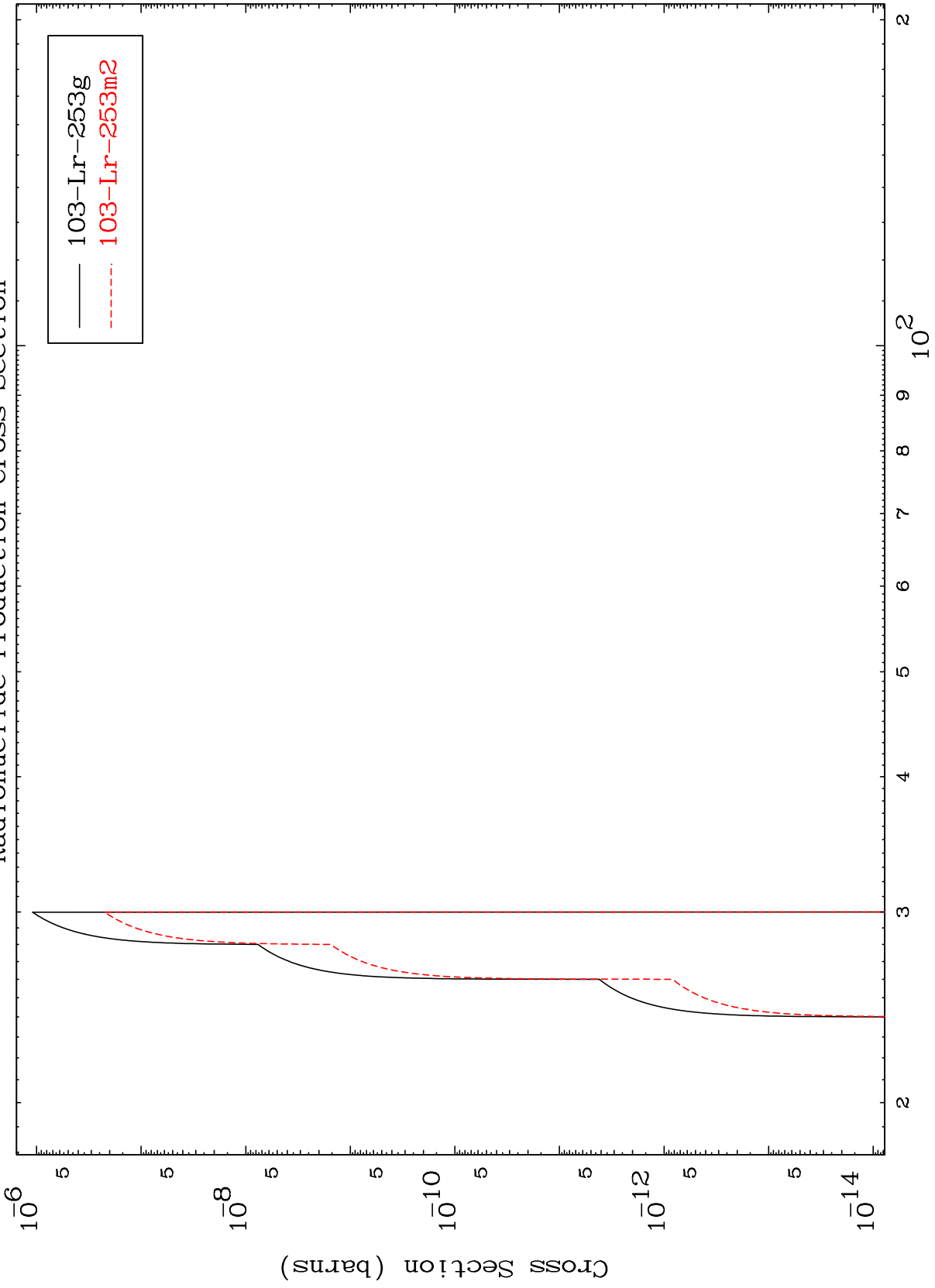
102-No-252

MAT 9970

( $\alpha, t$ )

102-No-252

Radionuclide Production Cross Section



13

Incident Energy (MeV)

102-No-252