

Program EVALPLOT  
(Version 2018-1)

by

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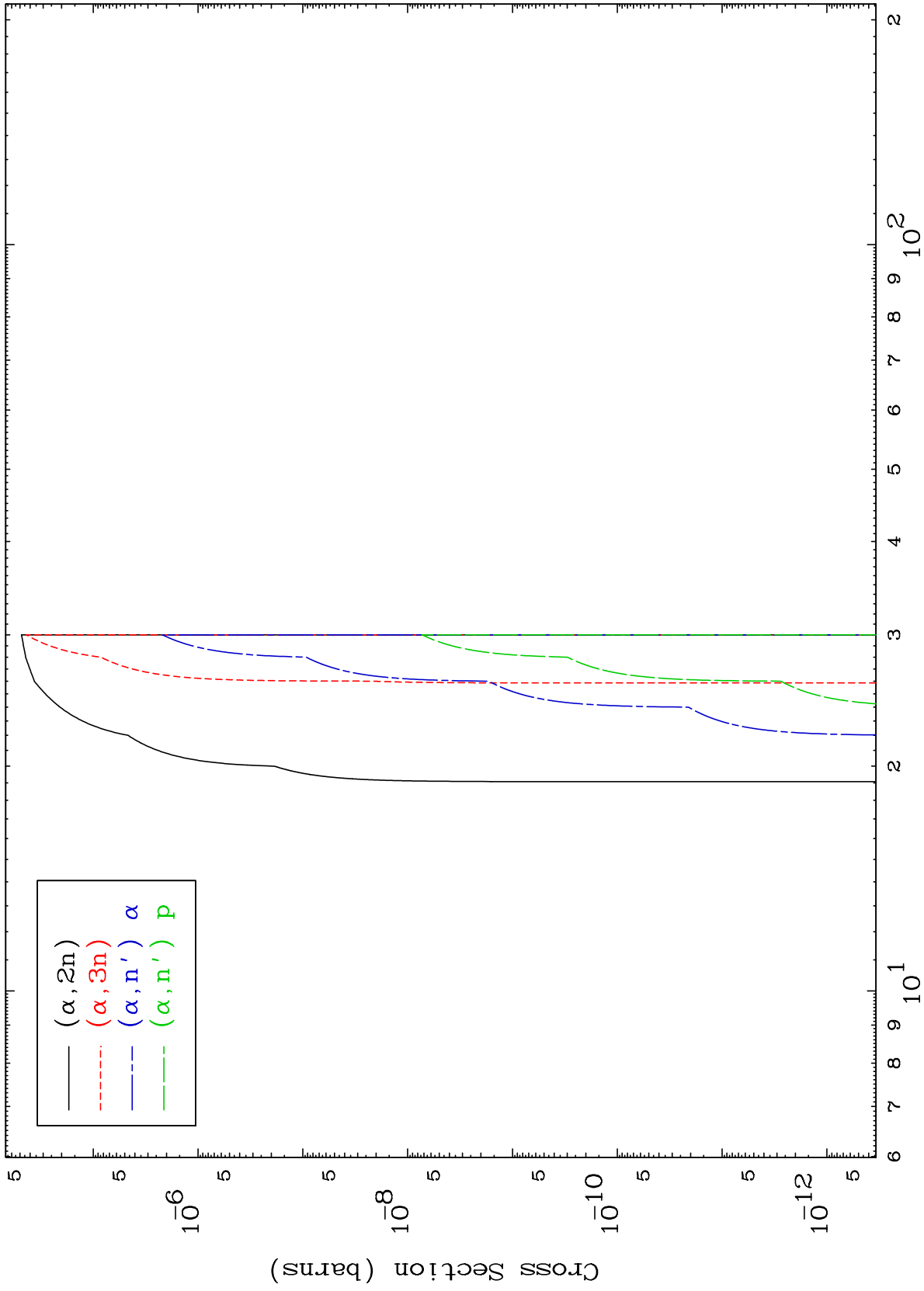
Press Mouse Button to Start



MAT 9982

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

102-No-264



2

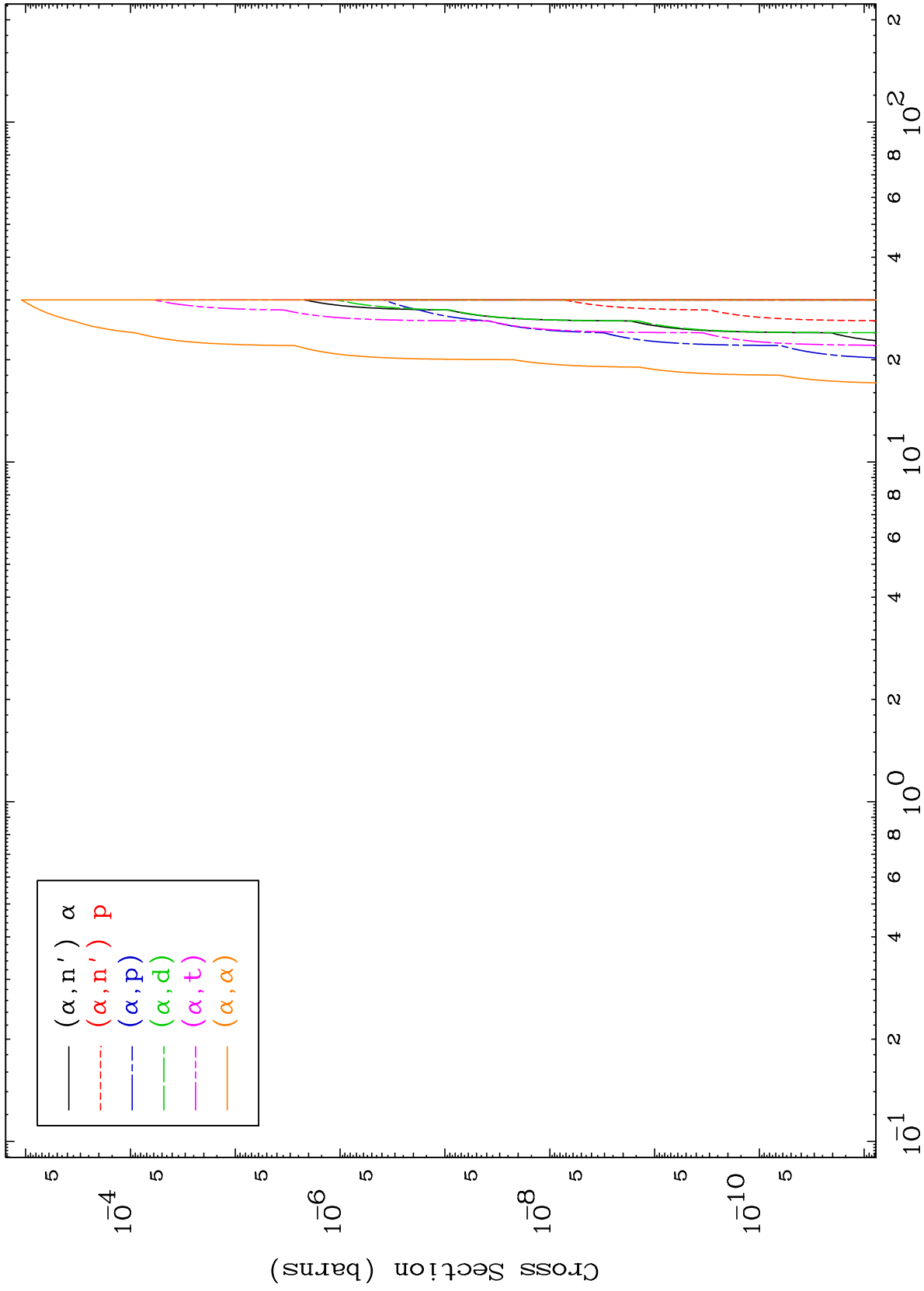
Incident Energy (MeV)

102-No-264

MAT 9982

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

102-No-264



3

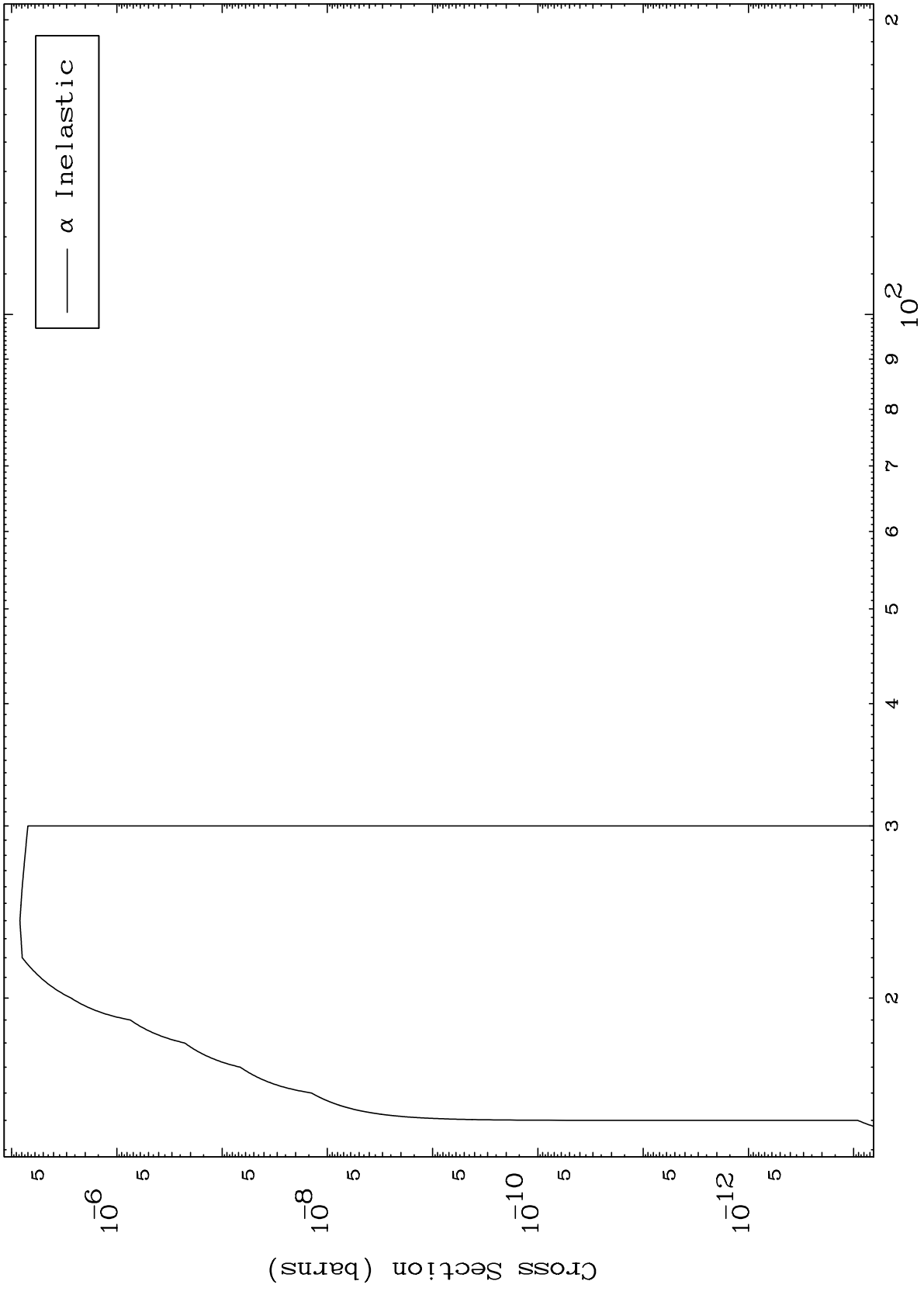
Incident Energy (MeV)

102-No-264

MAT 9982

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

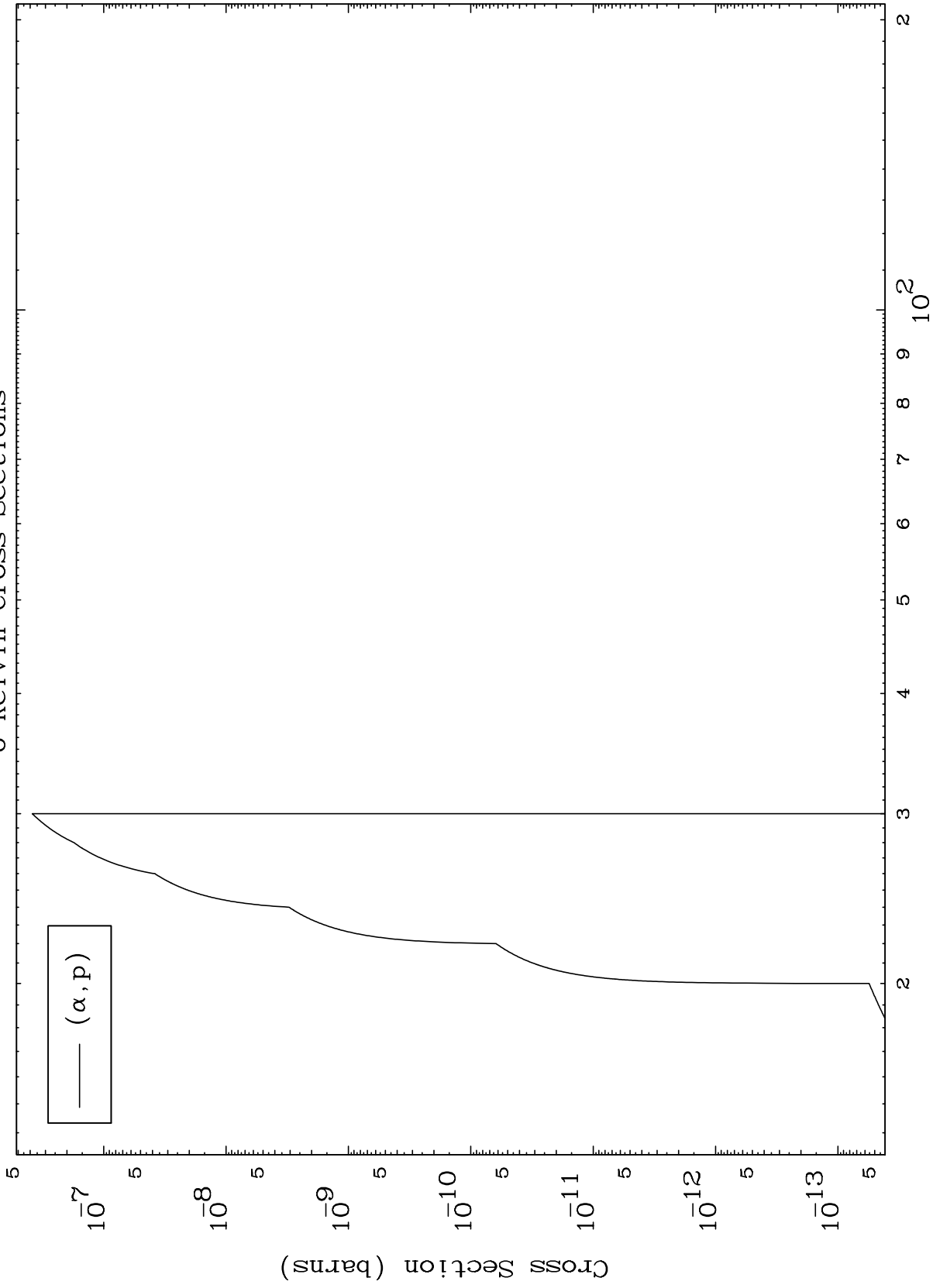
102-No-264



4

Incident Energy (MeV)

102-No-264

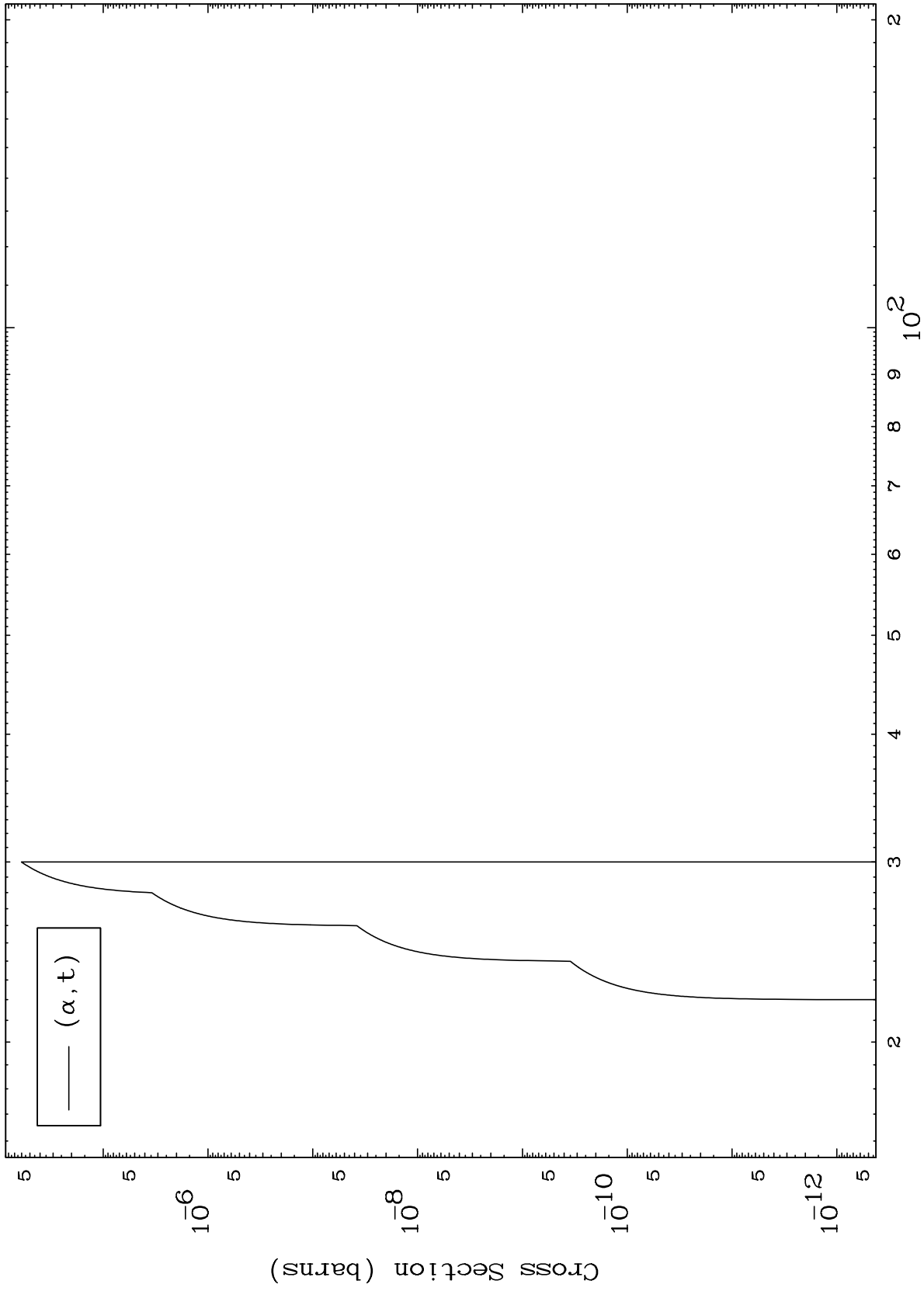




MAT 9982

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

102-No-264



7

Incident Energy (MeV)

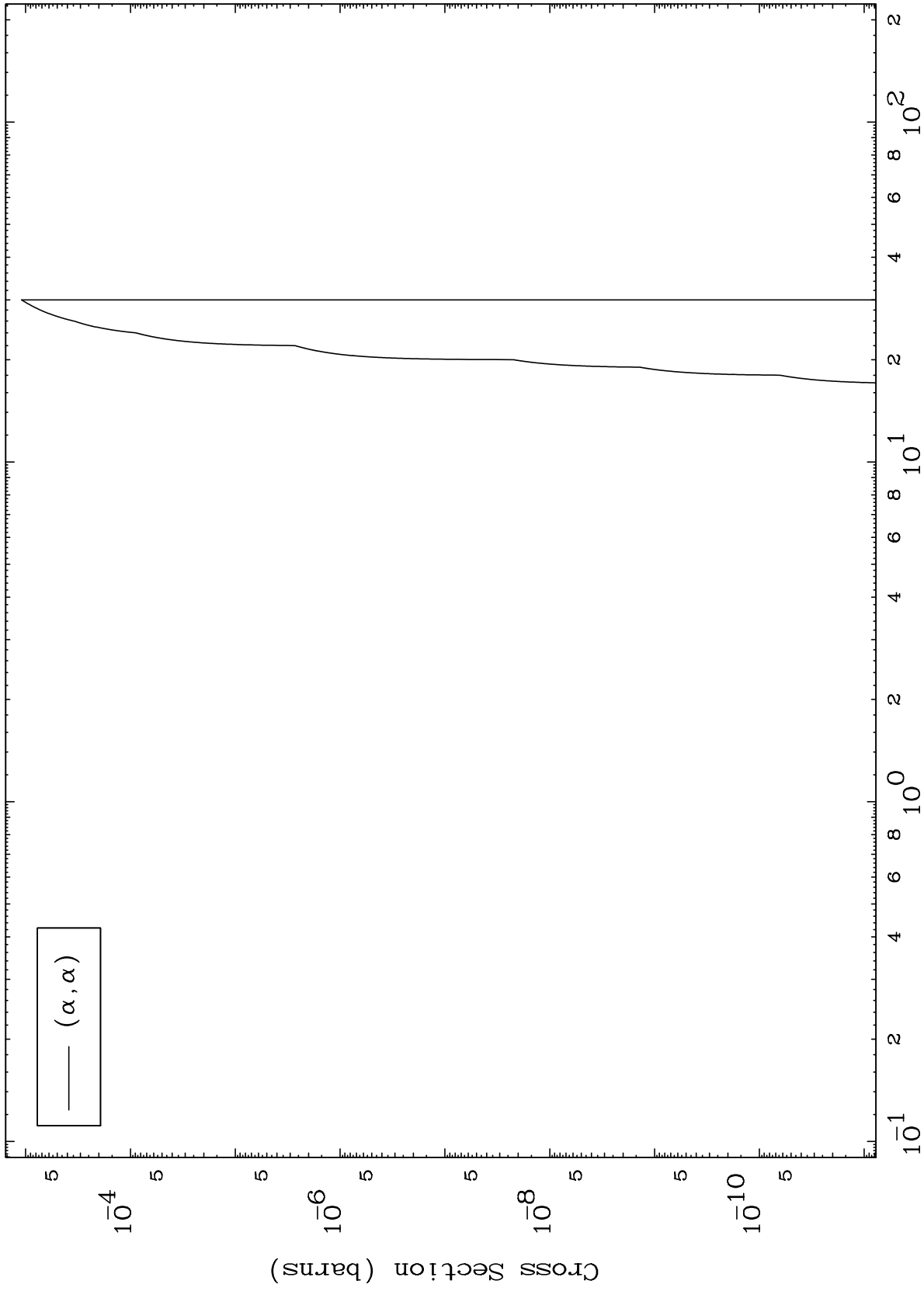
102-No-264



MAT 9982

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

102-No-264



8

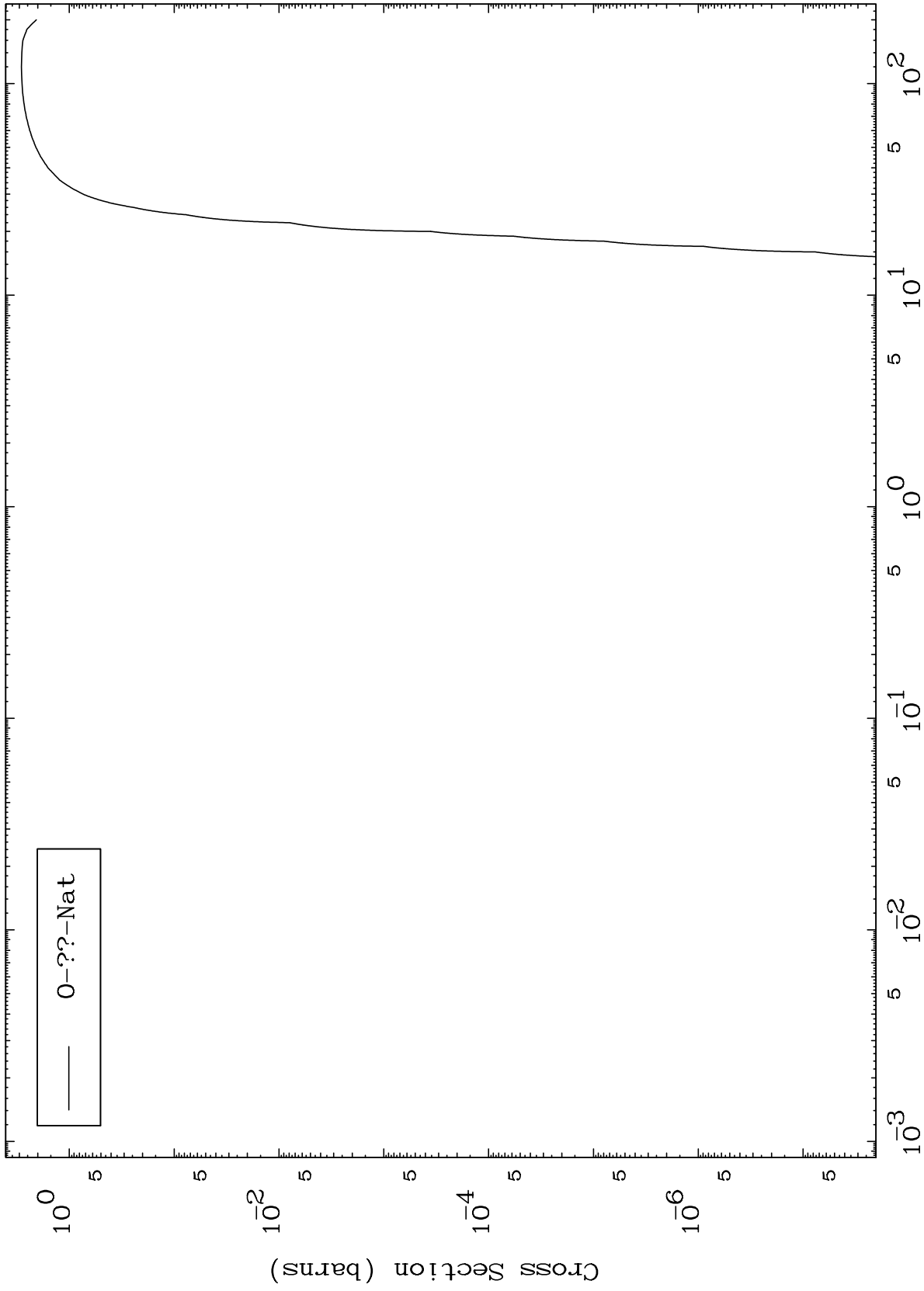
Incident Energy (MeV)

102-No-264

MAT 9982

102-No-264

$\alpha$  Fission  
Radionuclide Production Cross Section



102-No-264

Incident Energy (MeV)

9