

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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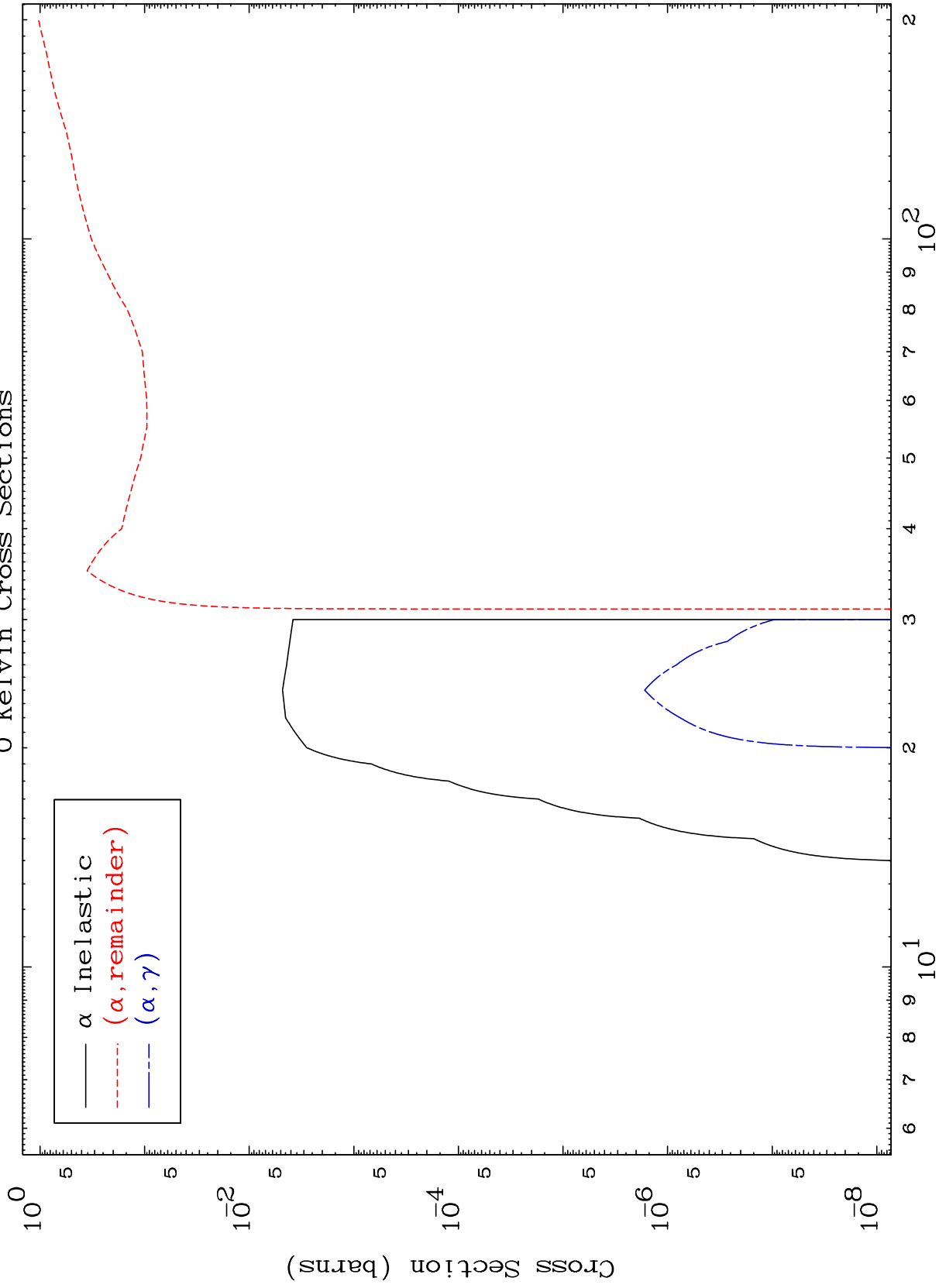
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

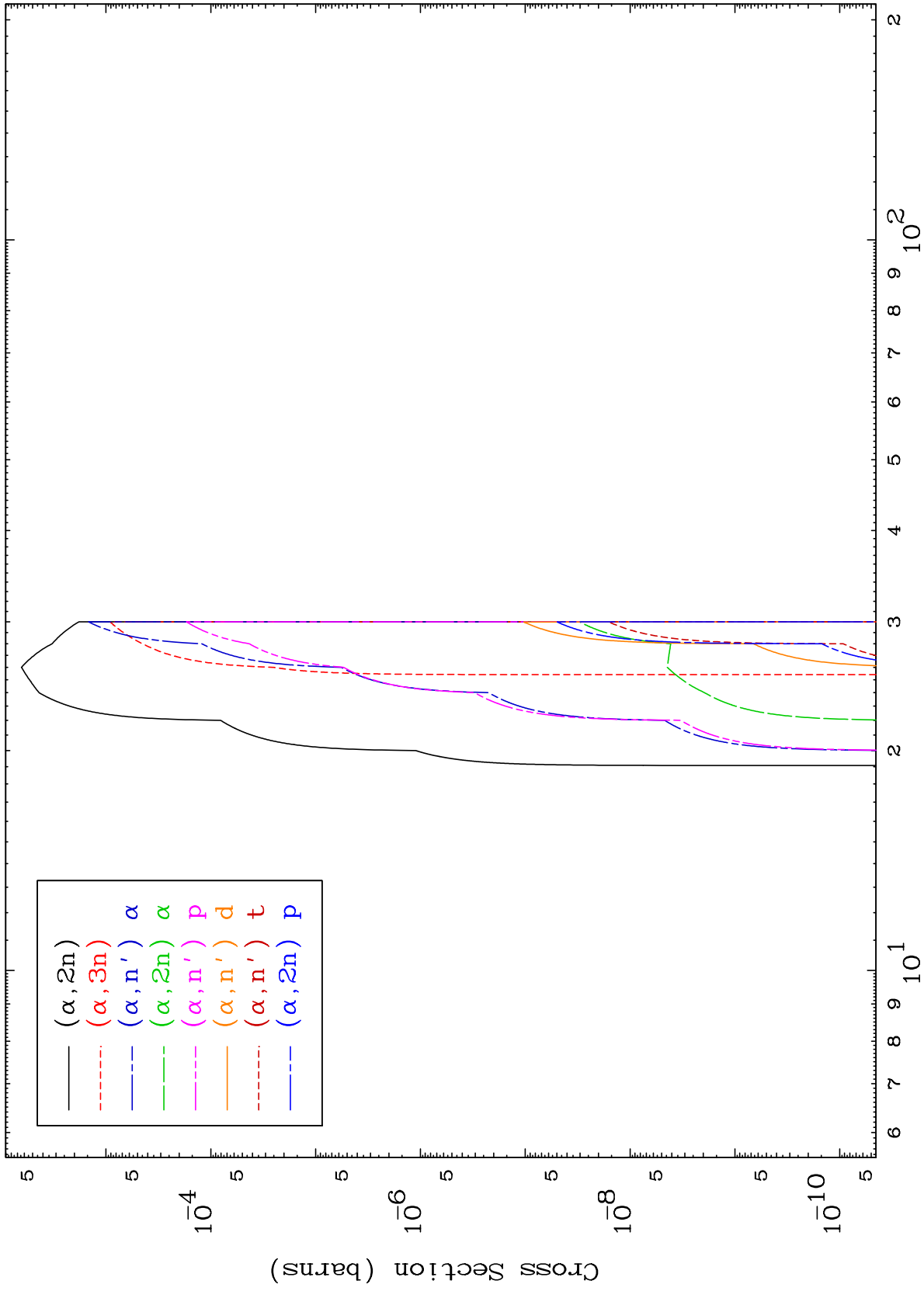
MAT 9343

0 Kelvin  $\alpha$  Major  
Cross Sections

93-Np-236



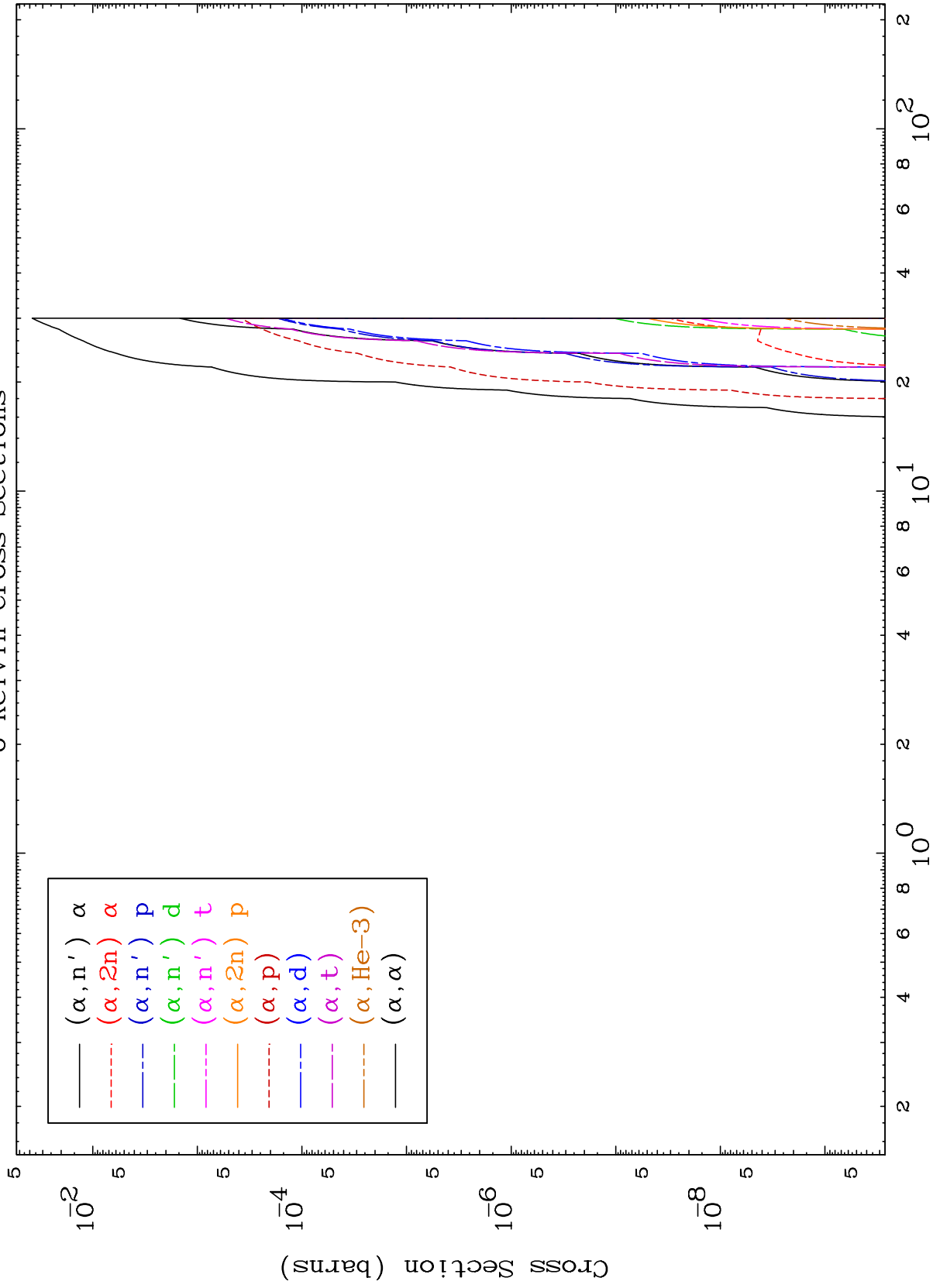
$\alpha$  Inelastic  
( $\alpha$ , remainder)  
( $\alpha$ ,  $\gamma$ )



MAT 9343

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

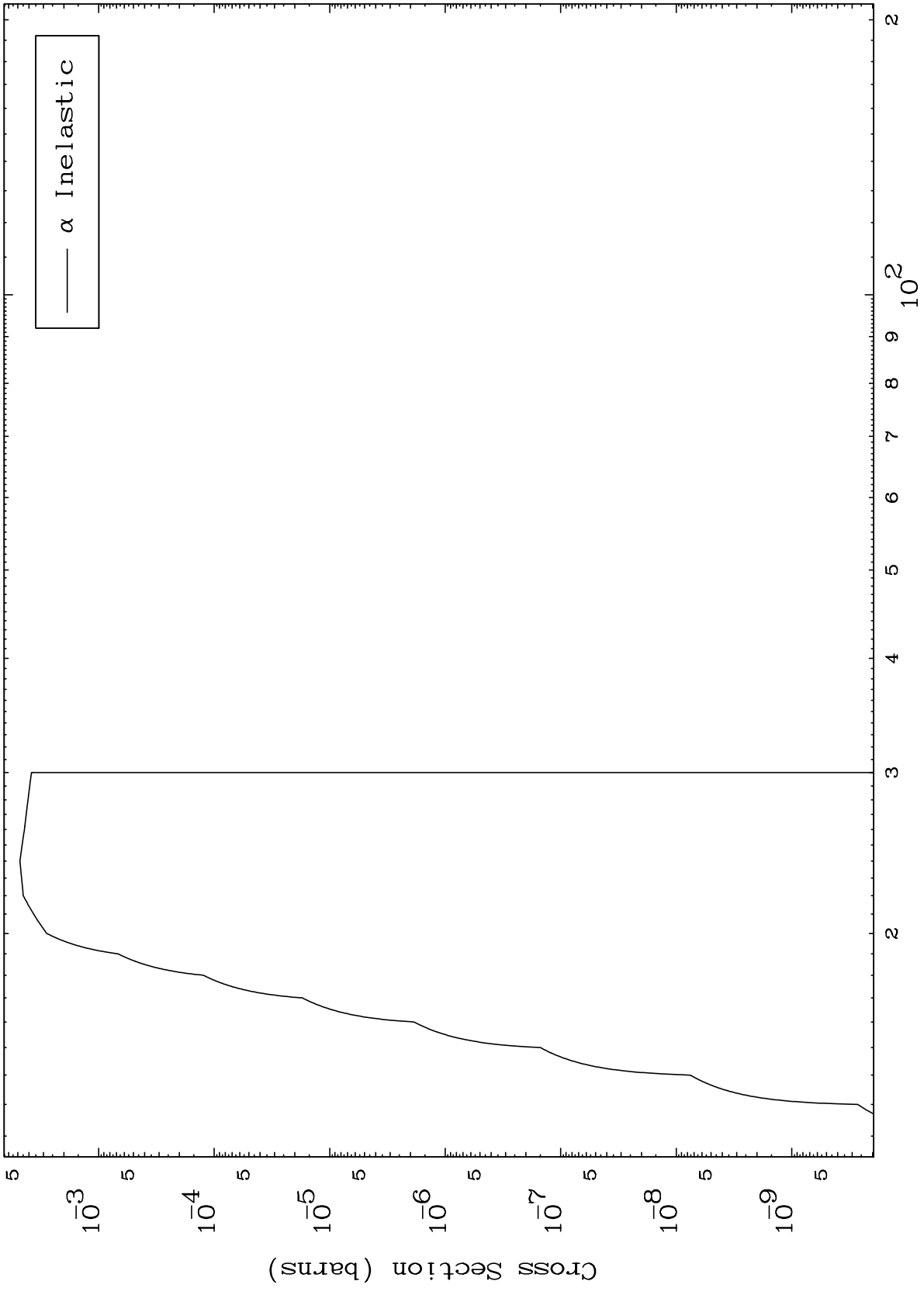
93-Np-236



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Incident Energy (MeV)

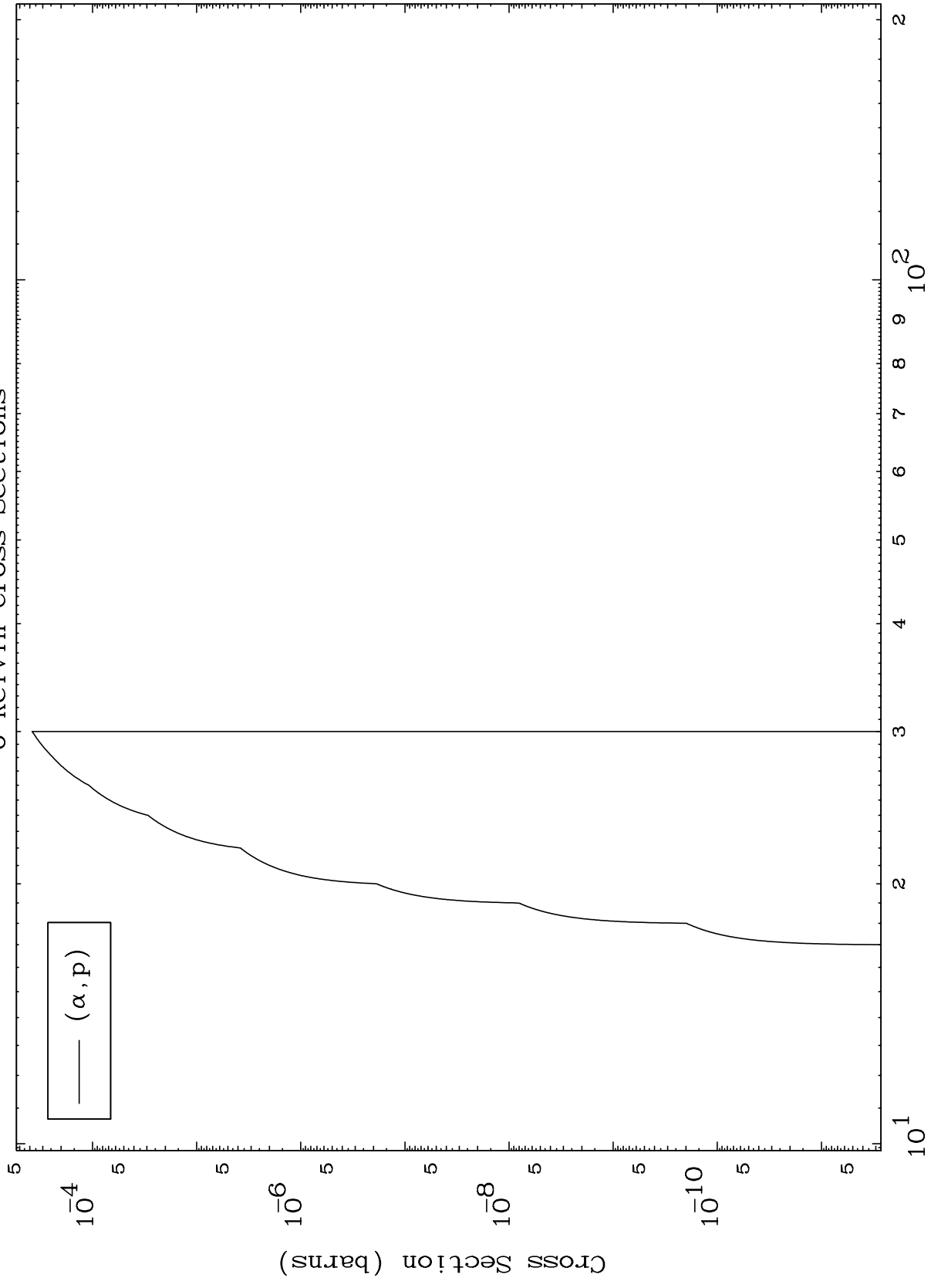
93-Np-236



MAT 9343

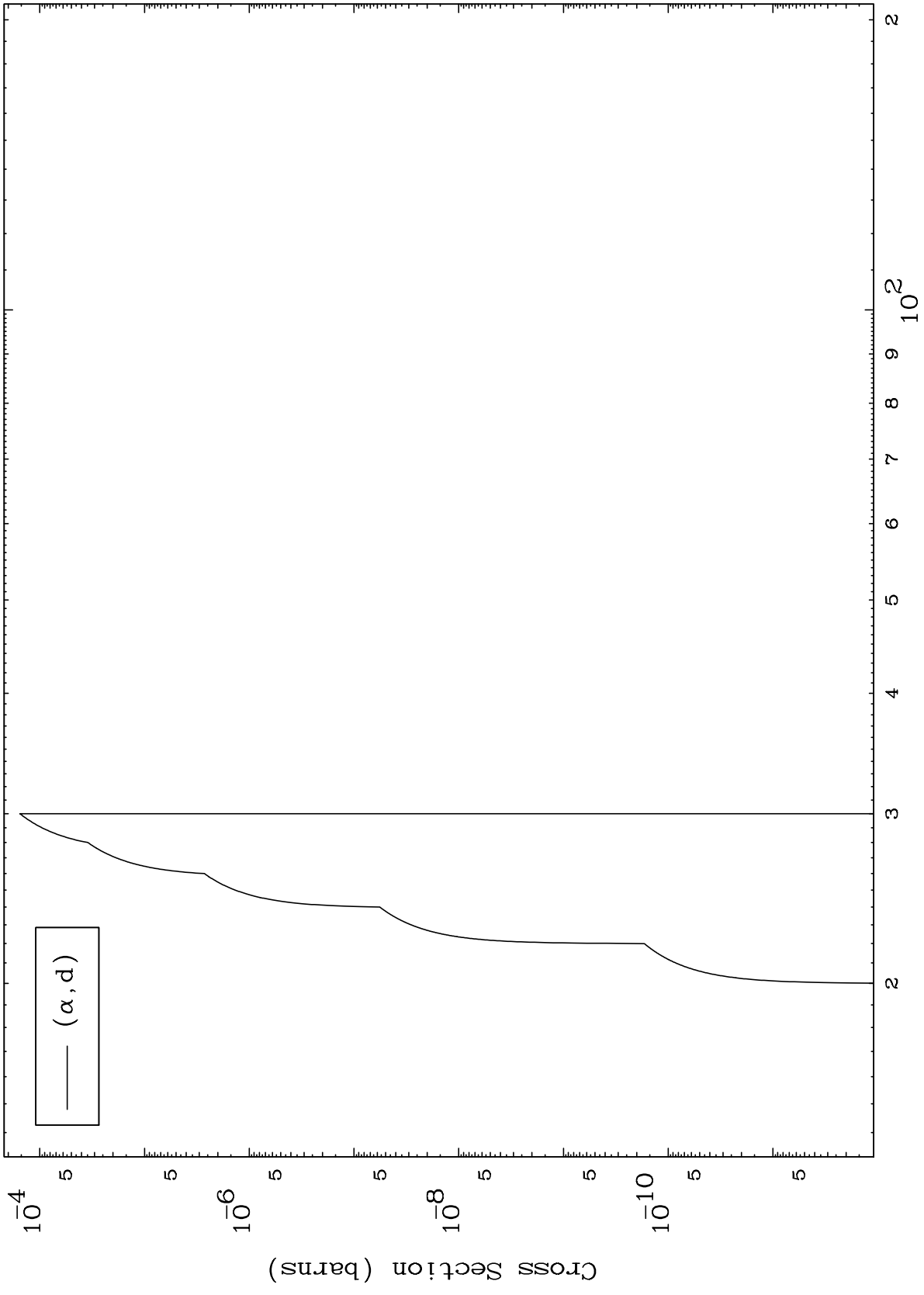
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

93-Np-236



Incident Energy (MeV)

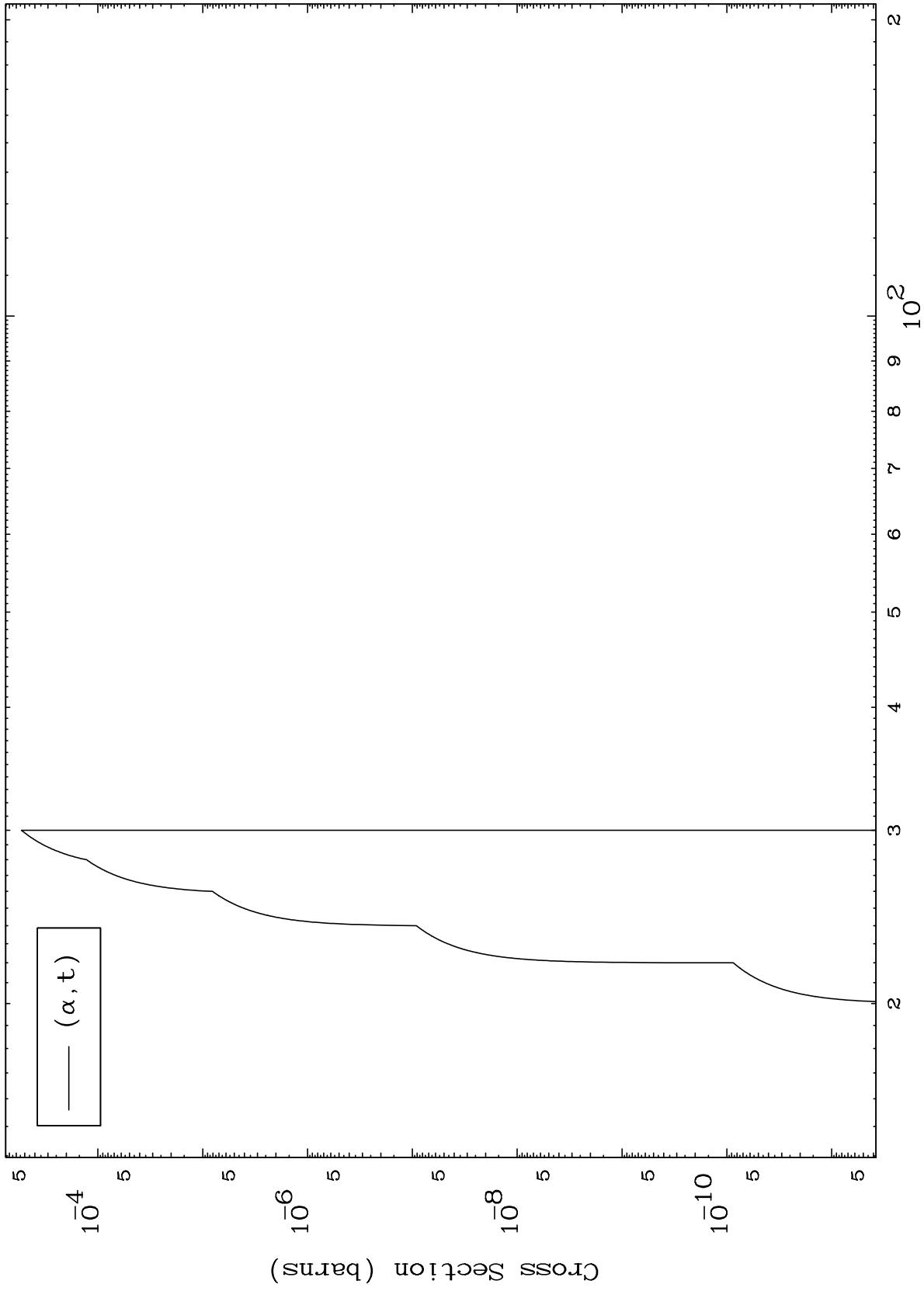
93-Np-236



MAT 9343

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

93-Np-236

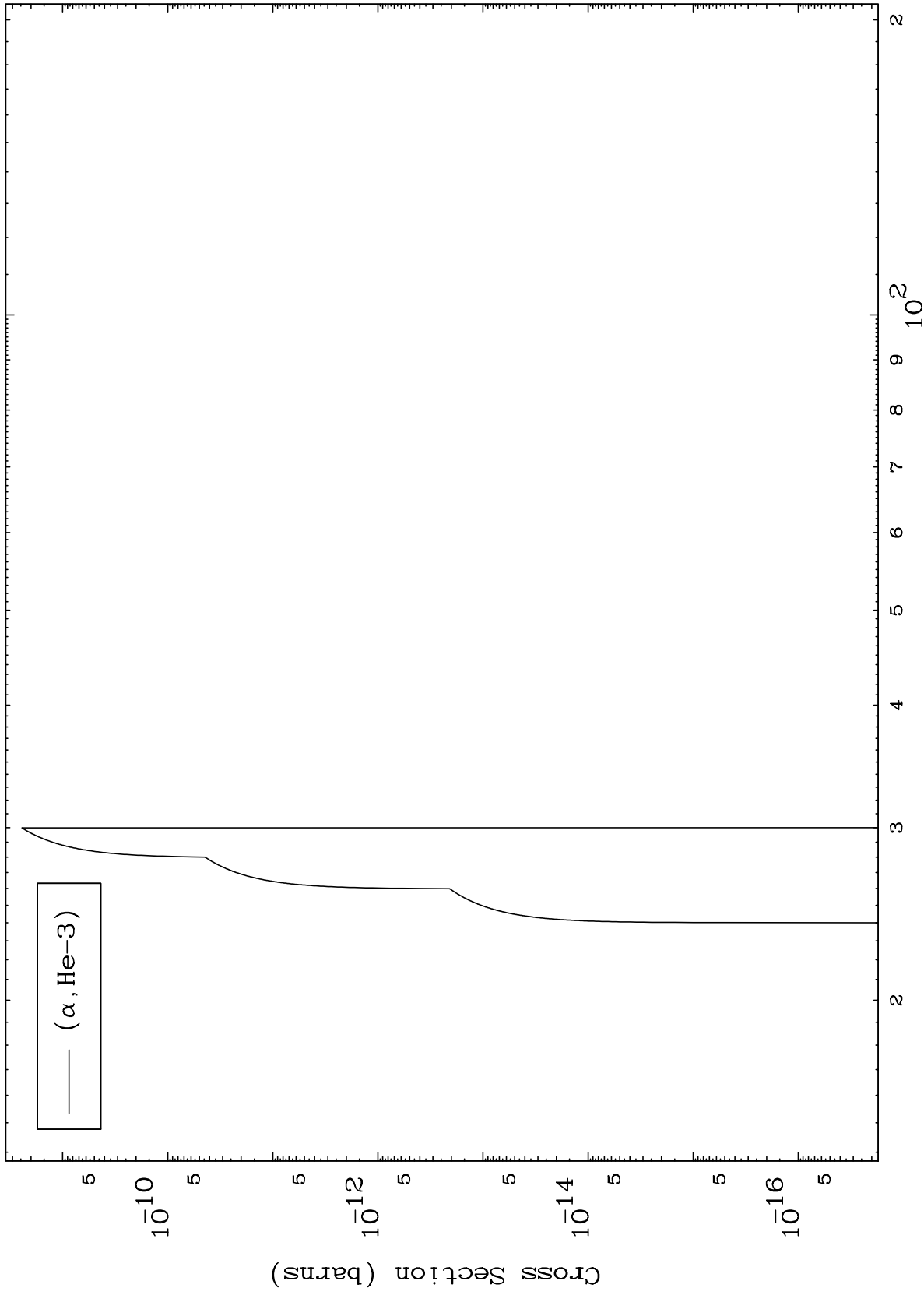


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Incident Energy (MeV)

93-Np-236

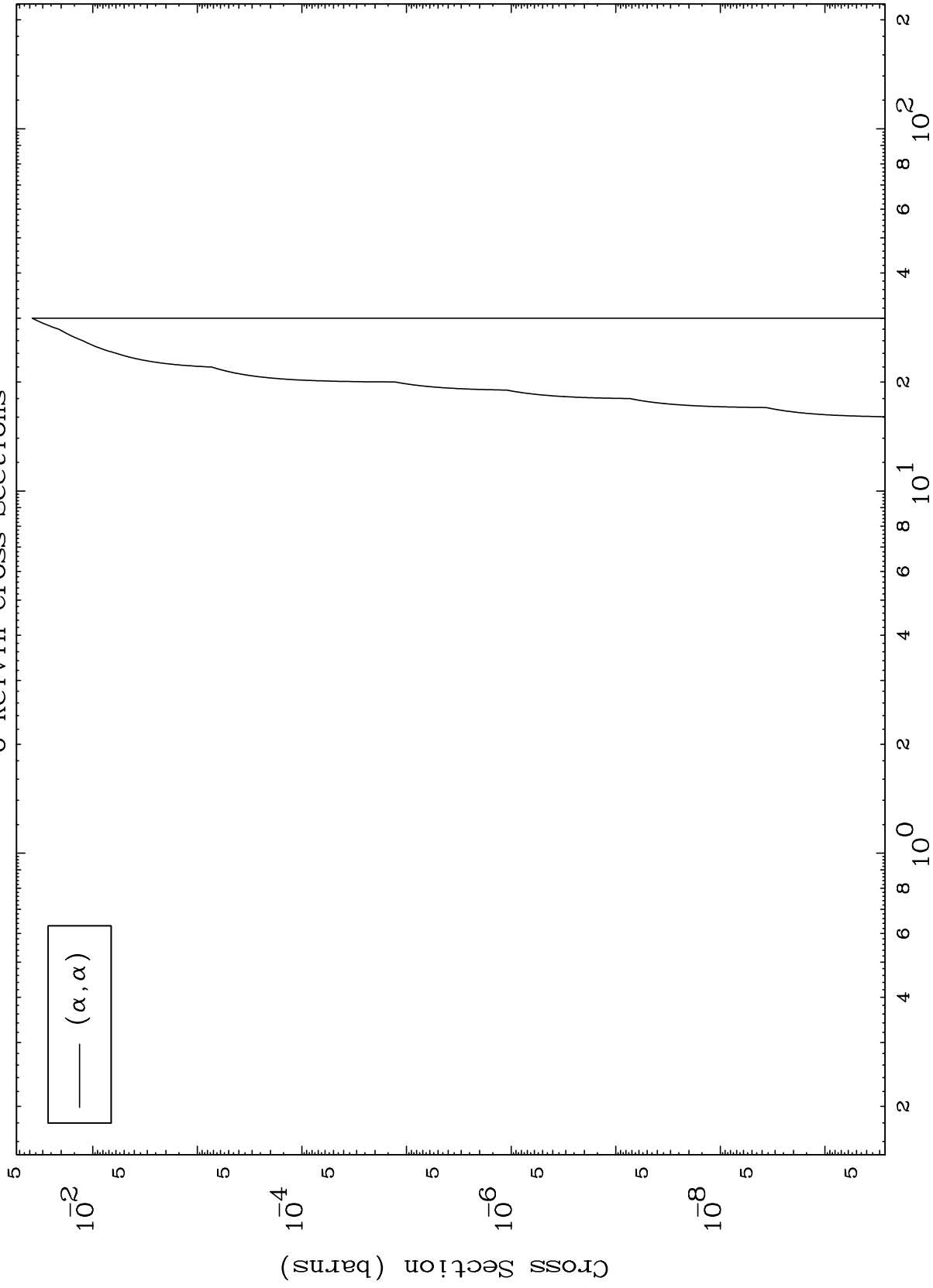




MAT 9343

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

93-Np-236



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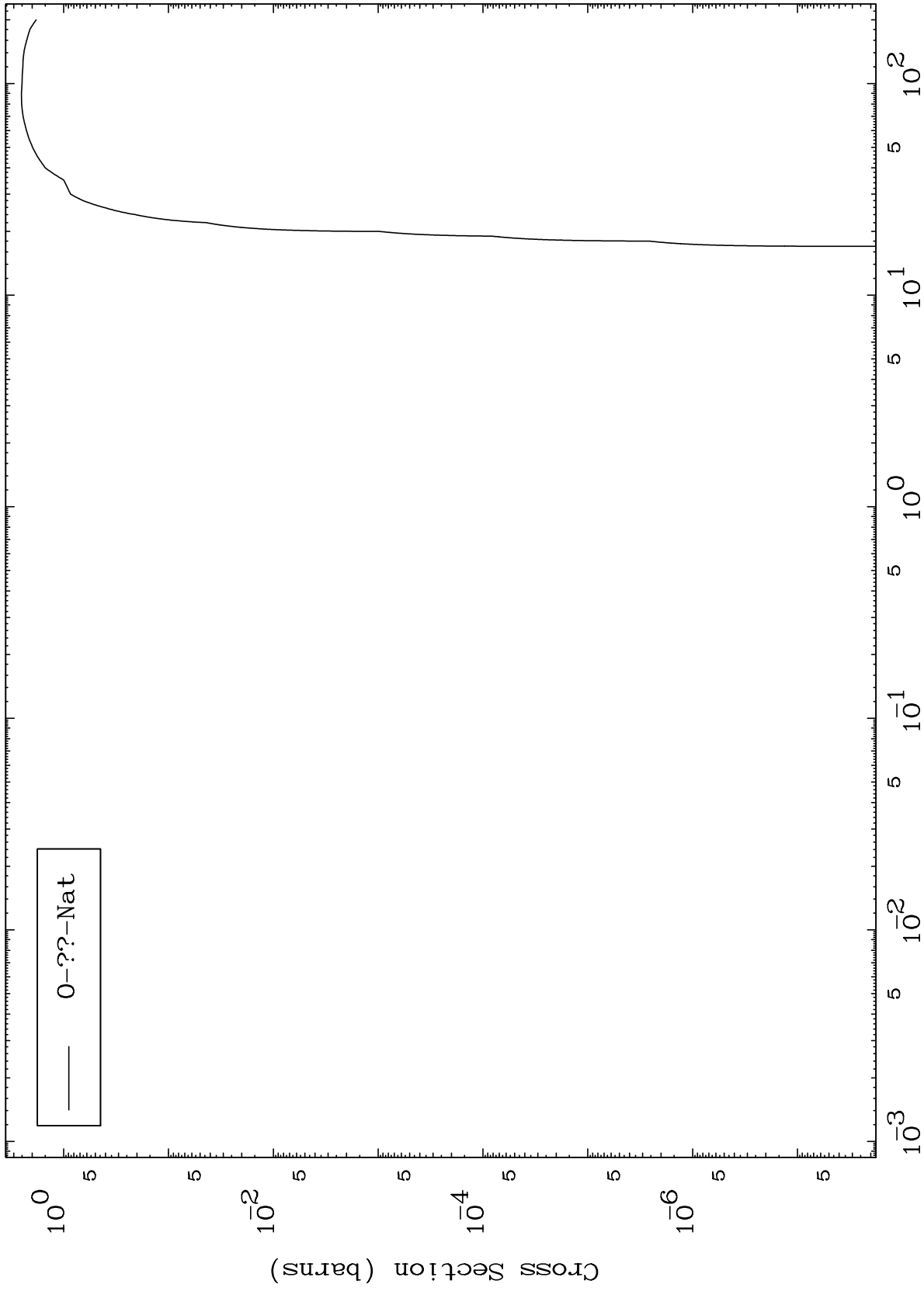
Incident Energy (MeV)

93-Np-236

MAT 9343

$\alpha$  Fission  
Radionuclide Production Cross Section

93-Np-236



Incident Energy (MeV)

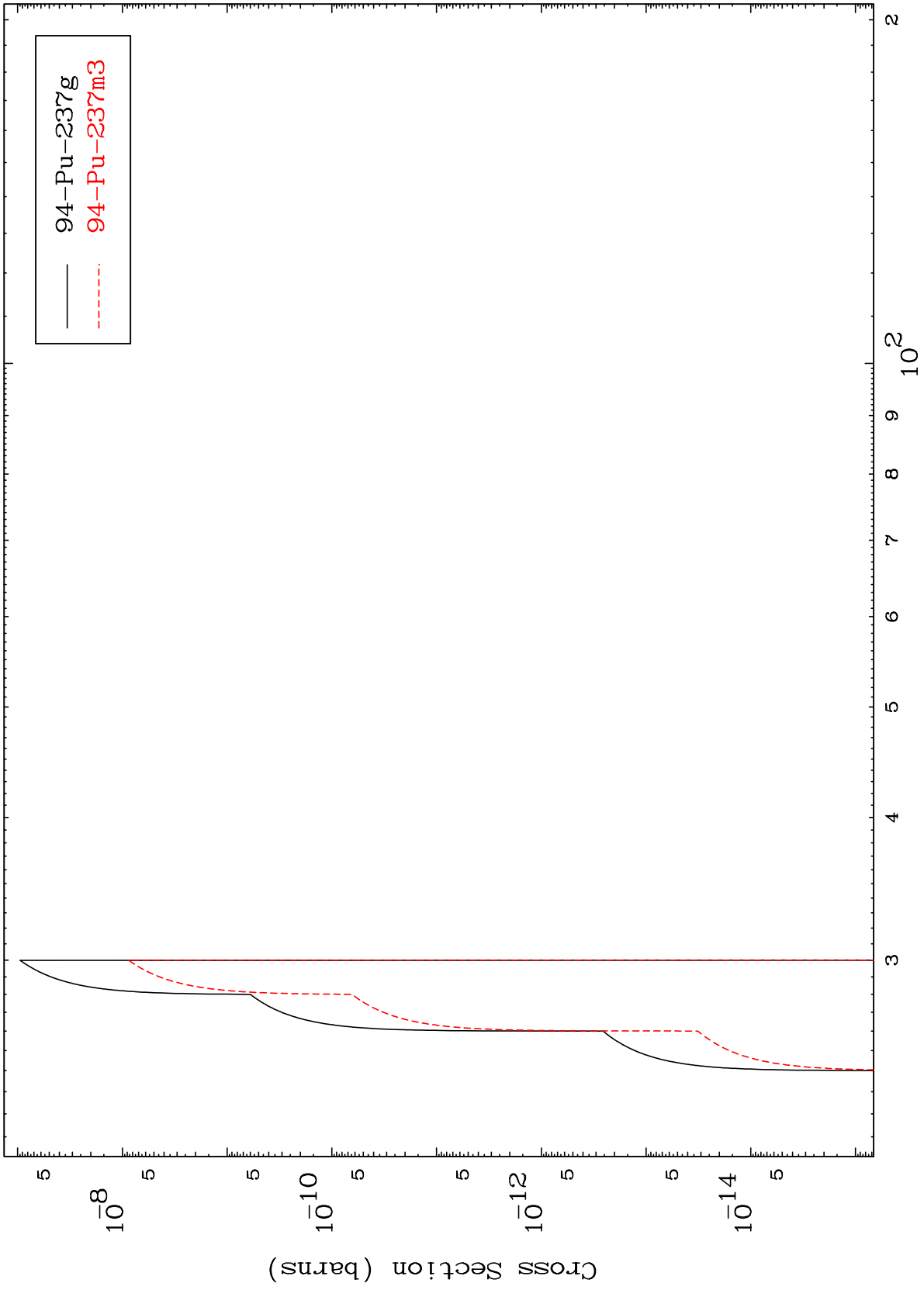
93-Np-236

MAT 9343

( $\alpha, n'$ ) d

93-Np-236

Radionuclide Production Cross Section



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Incident Energy (MeV)

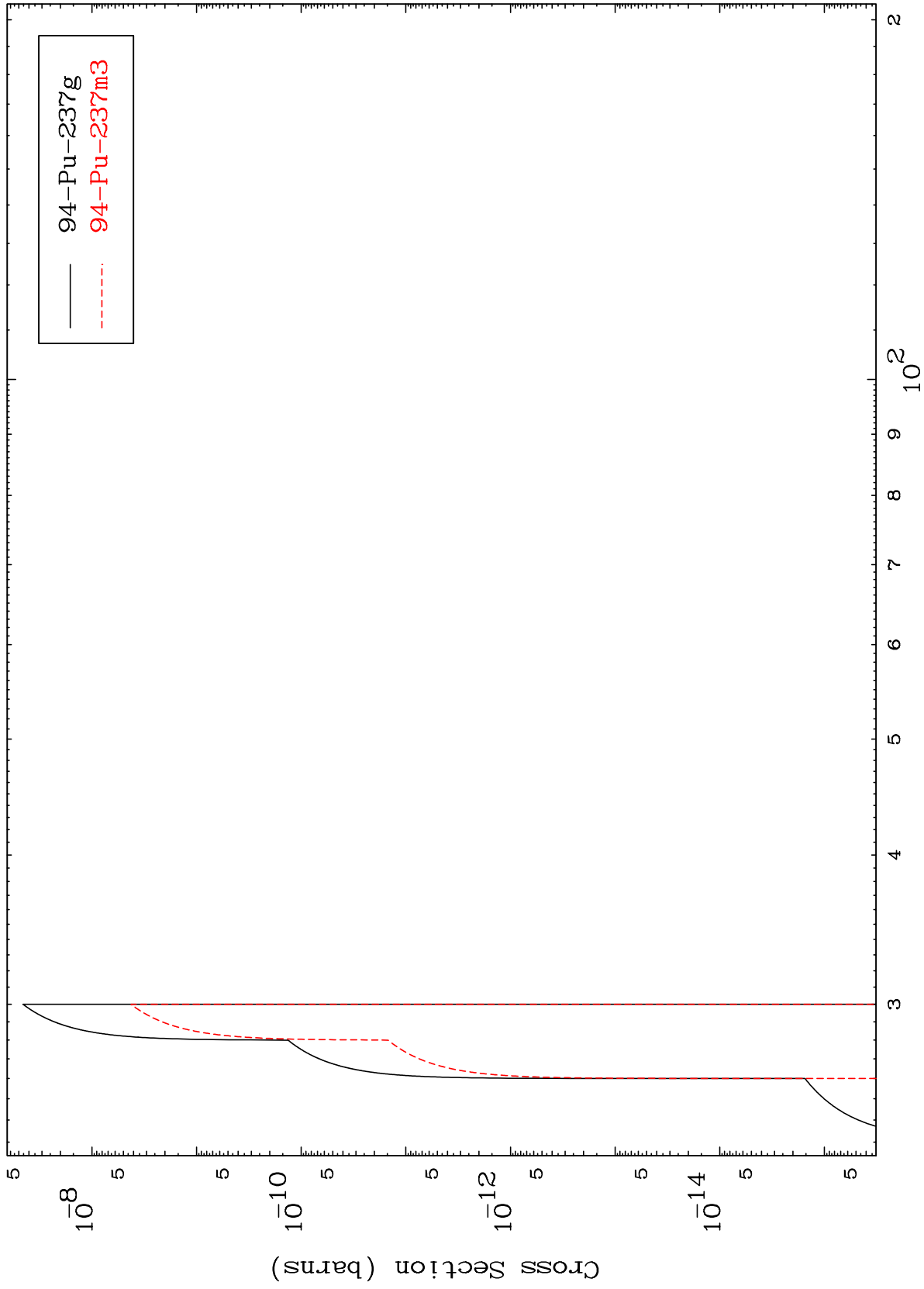
93-Np-236

MAT 9343

( $\alpha, 2n$ ) p

93-Np-236

Radionuclide Production Cross Section



12

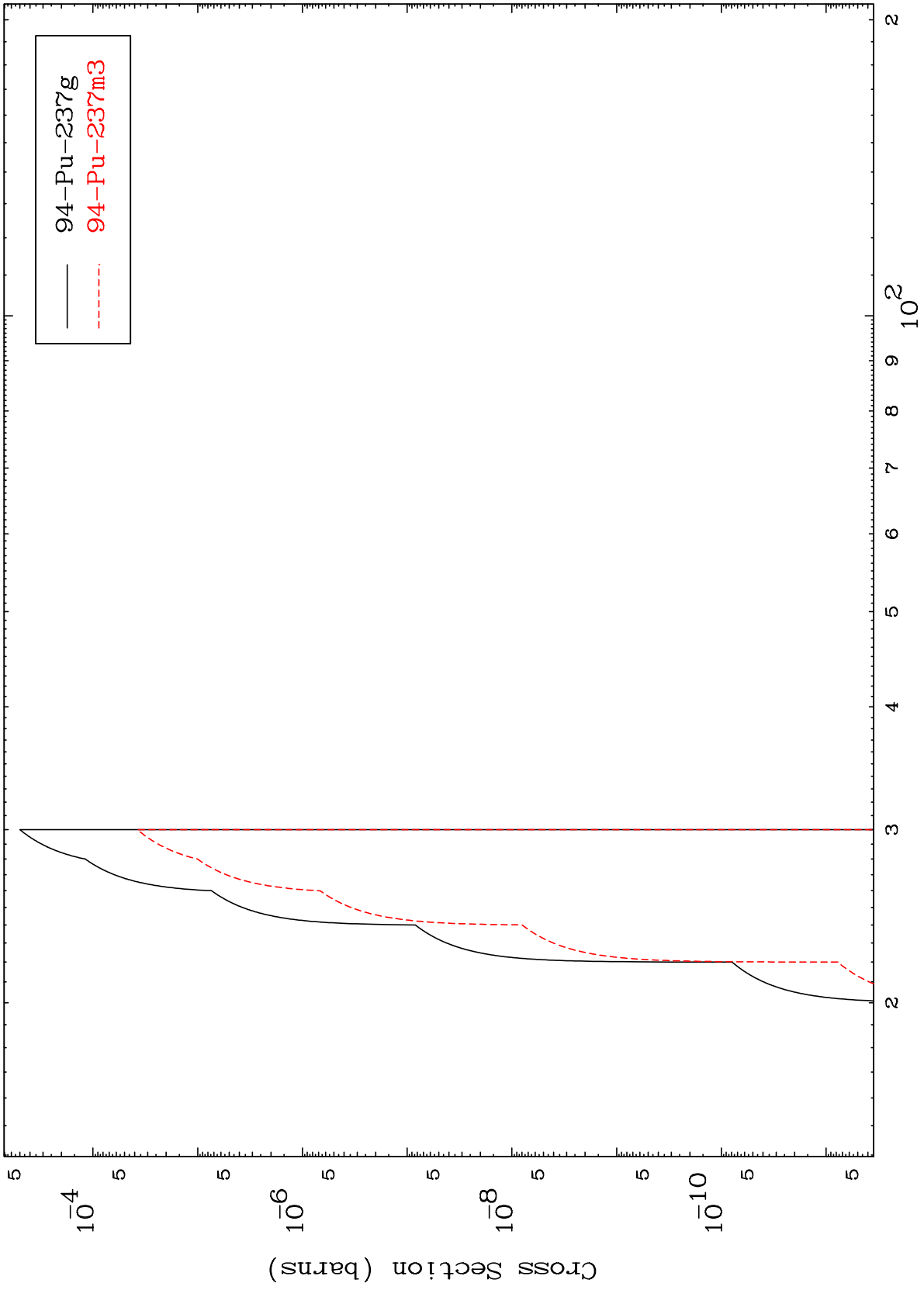
Incident Energy (MeV)

93-Np-236

MAT 9343

93-Np-236

( $\alpha, t$ )  
Radionuclide Production Cross Section



13

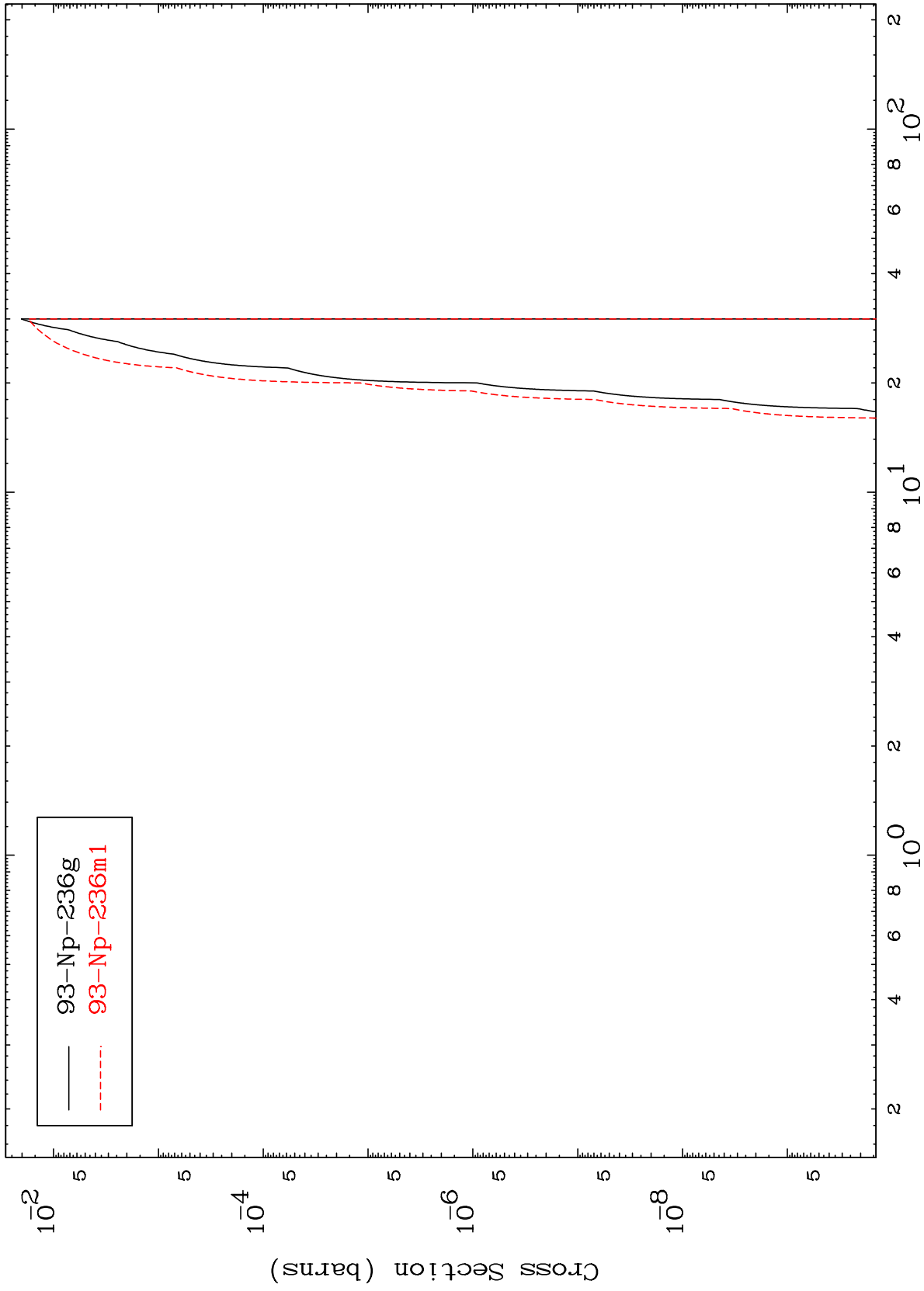
93-Np-236

MAT 9343

( $\alpha, \alpha$ )

93-Np-236

Radionuclide Production Cross Section



14

Incident Energy (MeV)

93-Np-236