

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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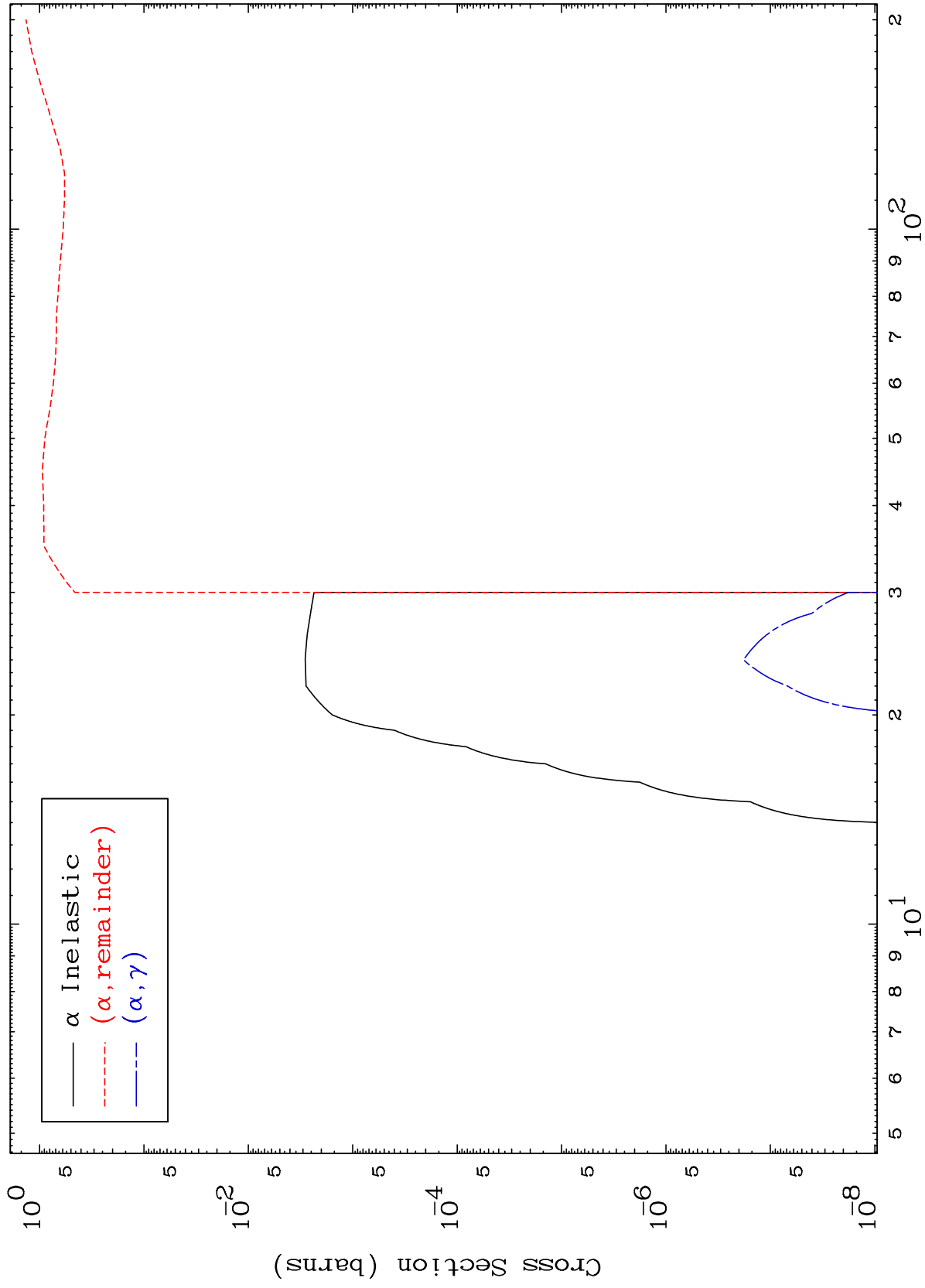
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9364

0 Kelvin  $\alpha$  Major  
Cross Sections

93-Np-243

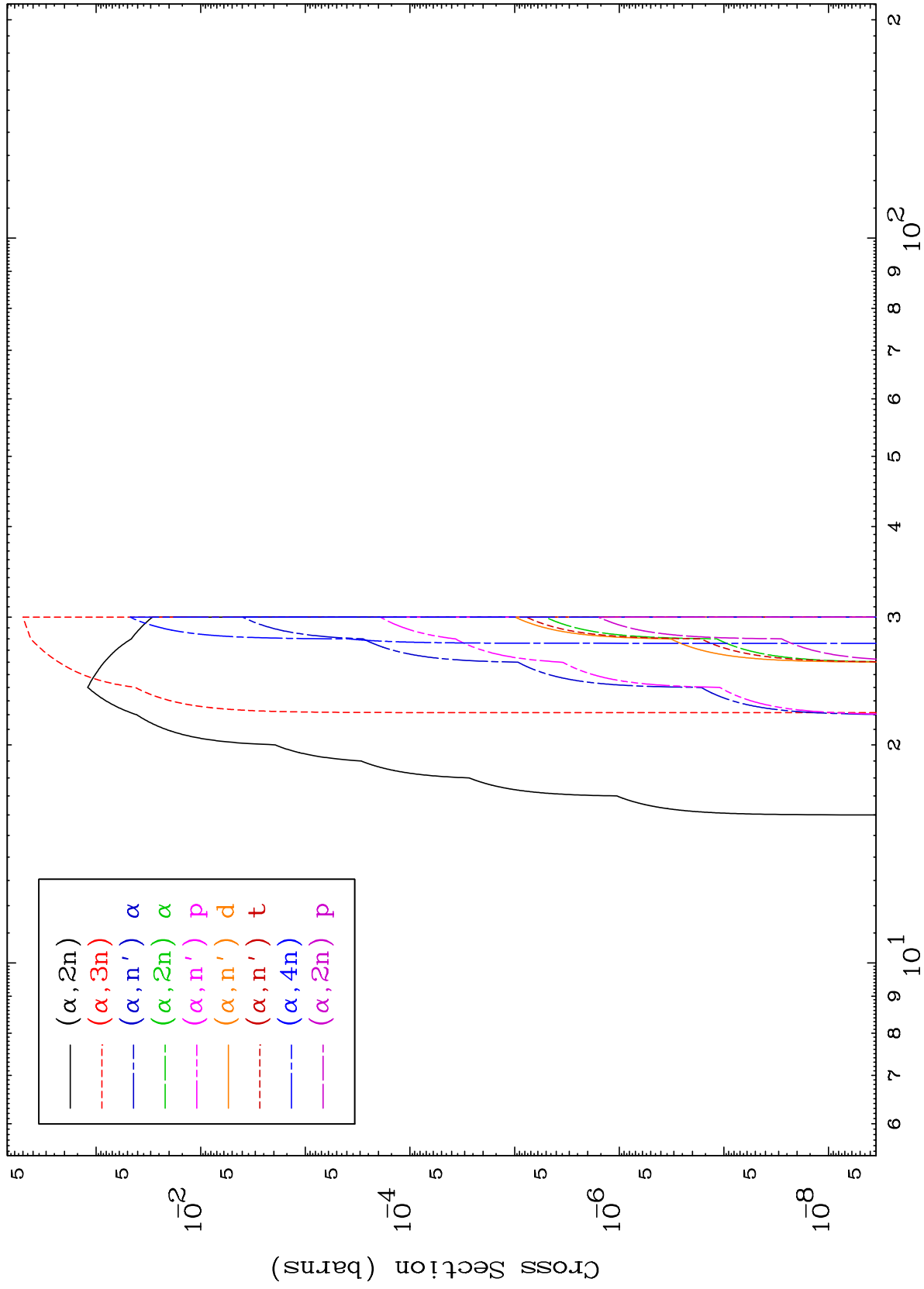


$\alpha$  Inelastic  
( $\alpha$ , remainder)  
( $\alpha$ ,  $\gamma$ )

MAT 9364

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

93-Np-243



2

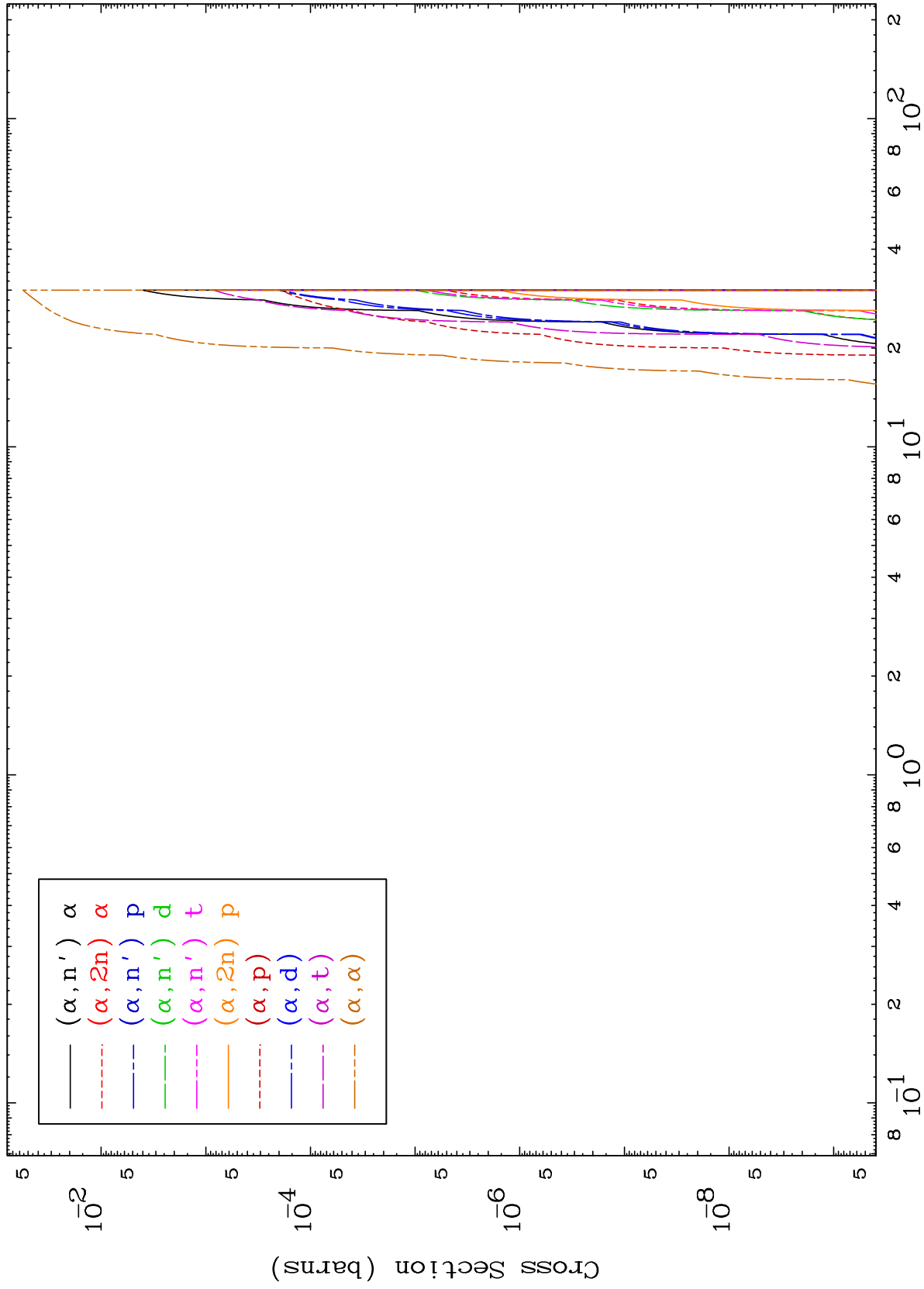
Incident Energy (MeV)

93-Np-243

MAT 9364

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

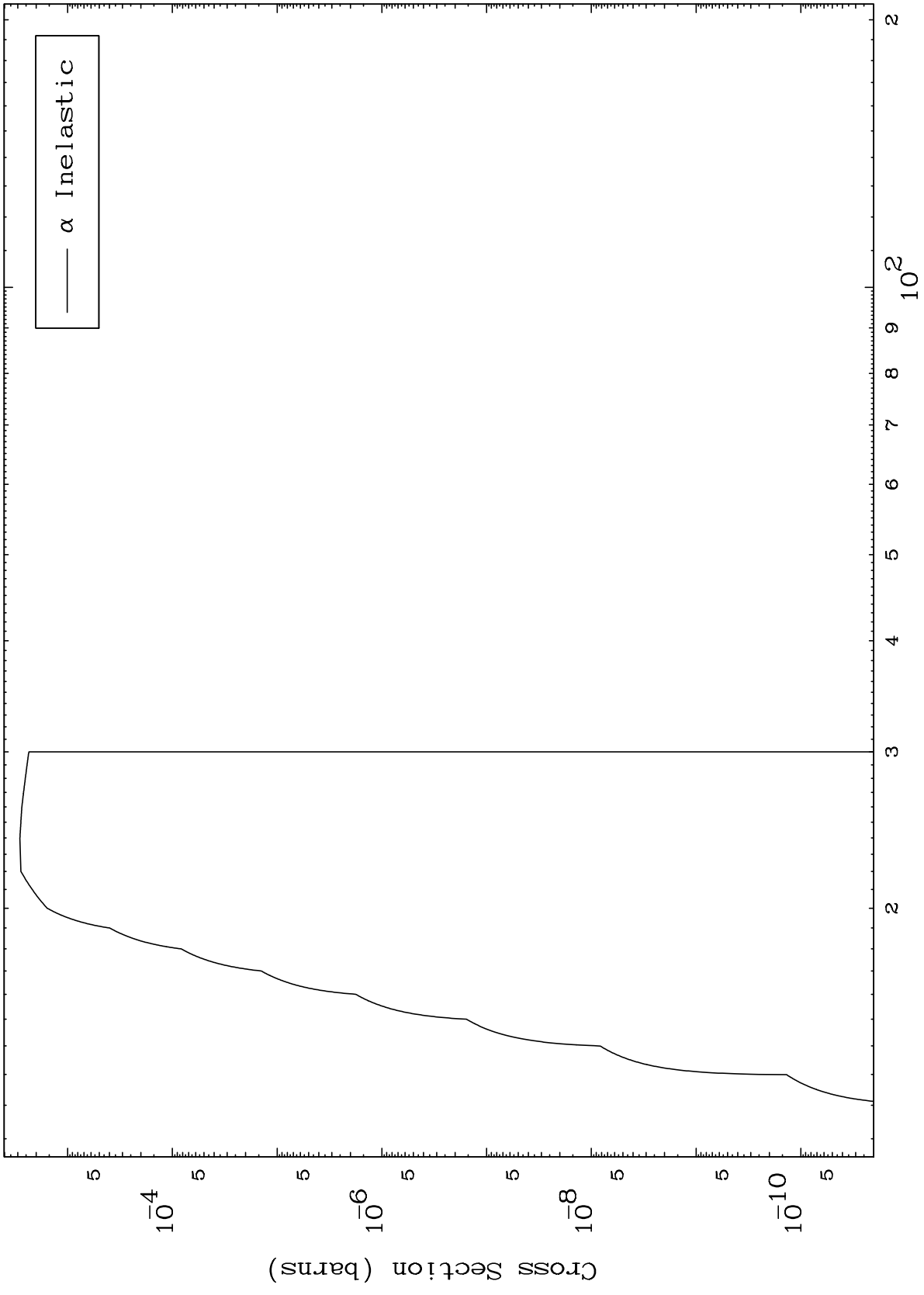
93-Np-243

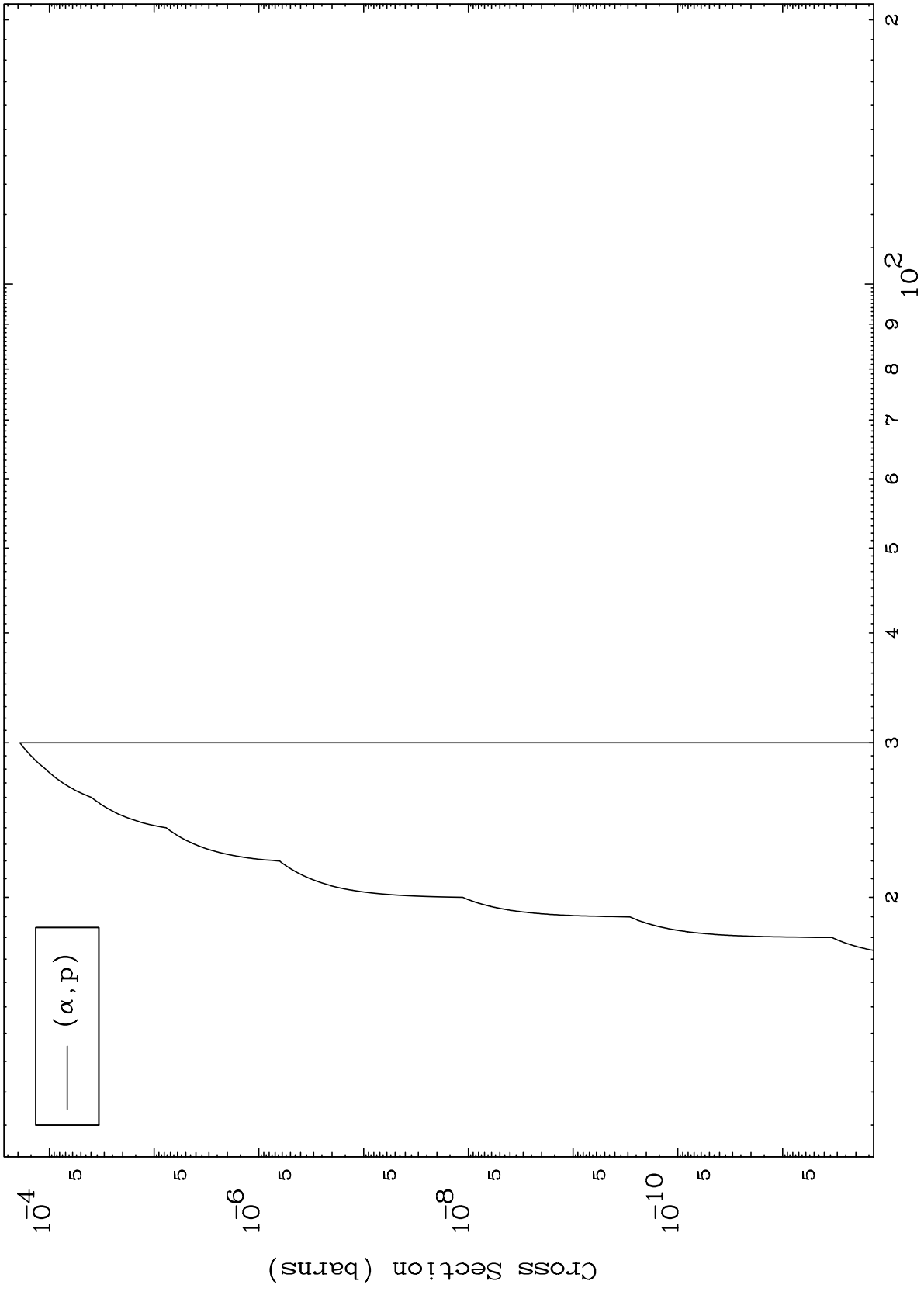


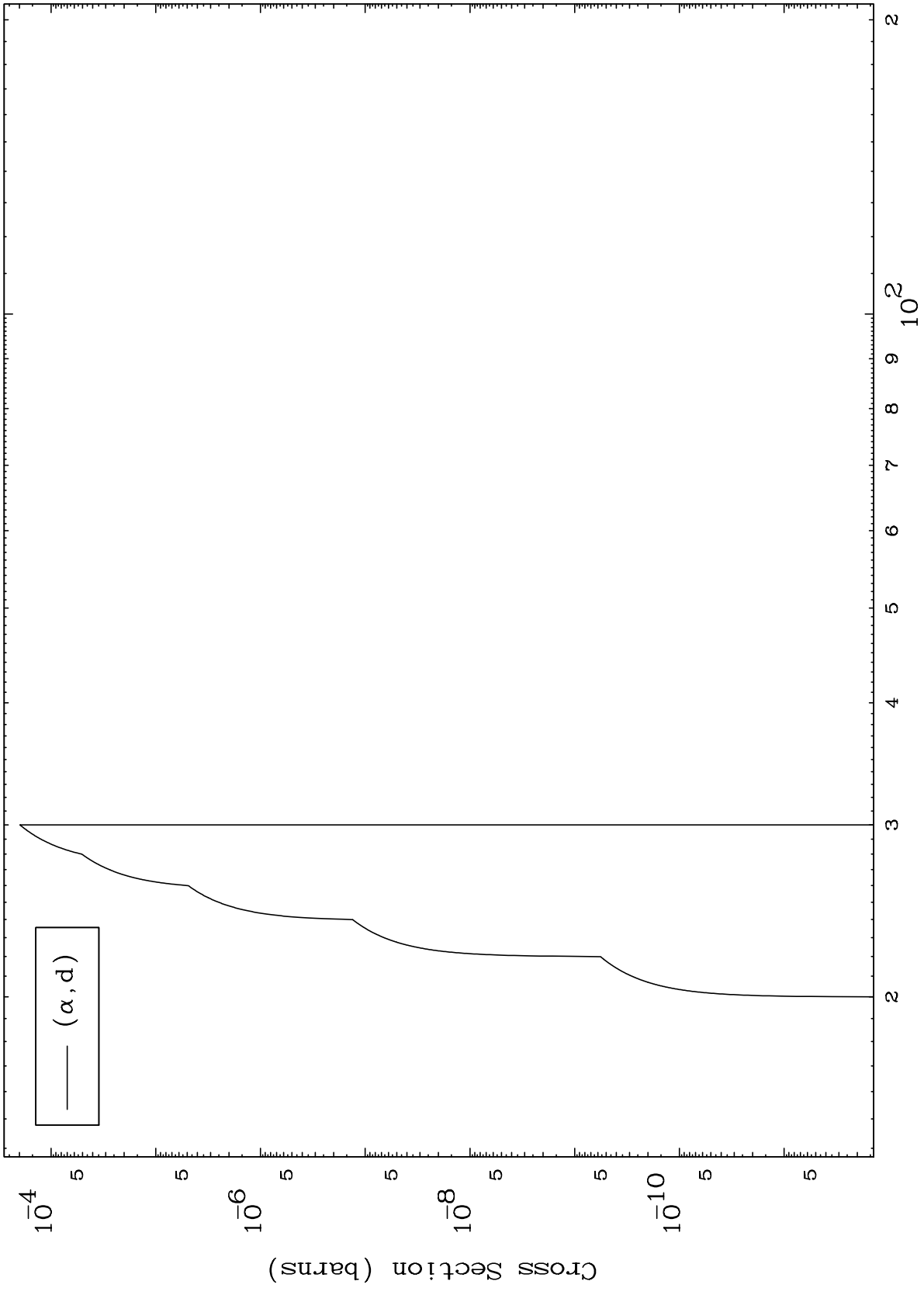
3

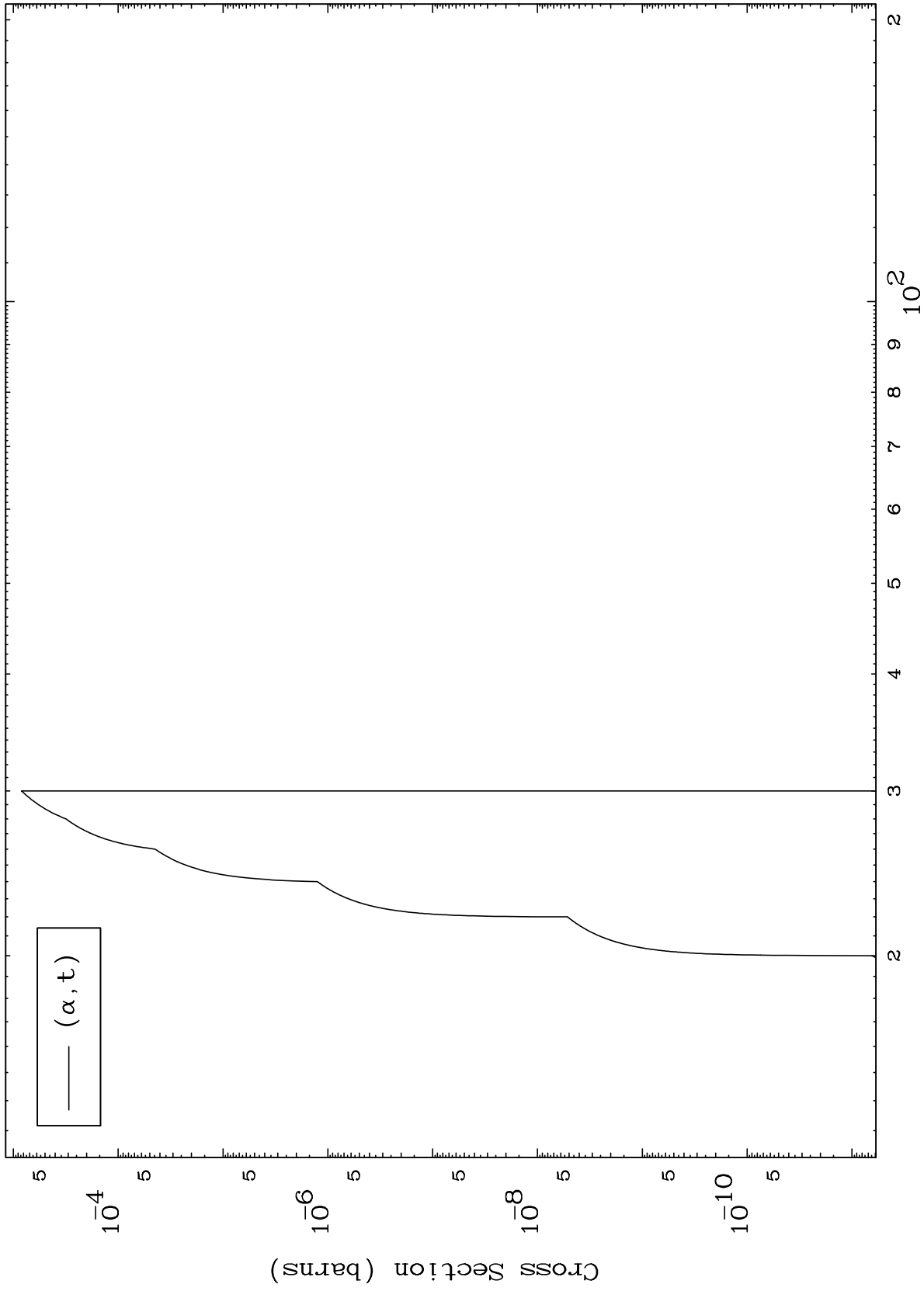
Incident Energy (MeV)

93-Np-243







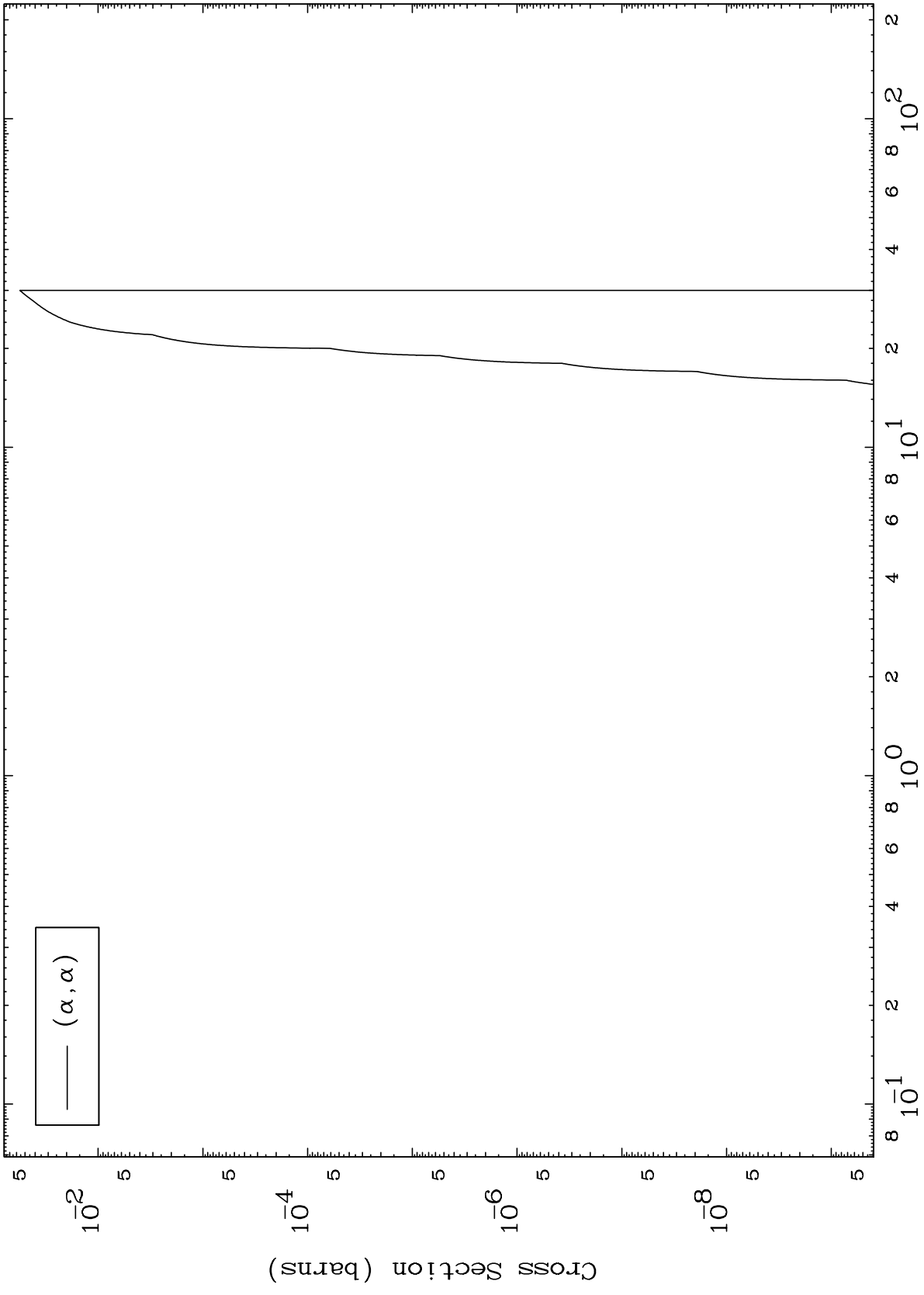




MAT 9364

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

93-Np-243



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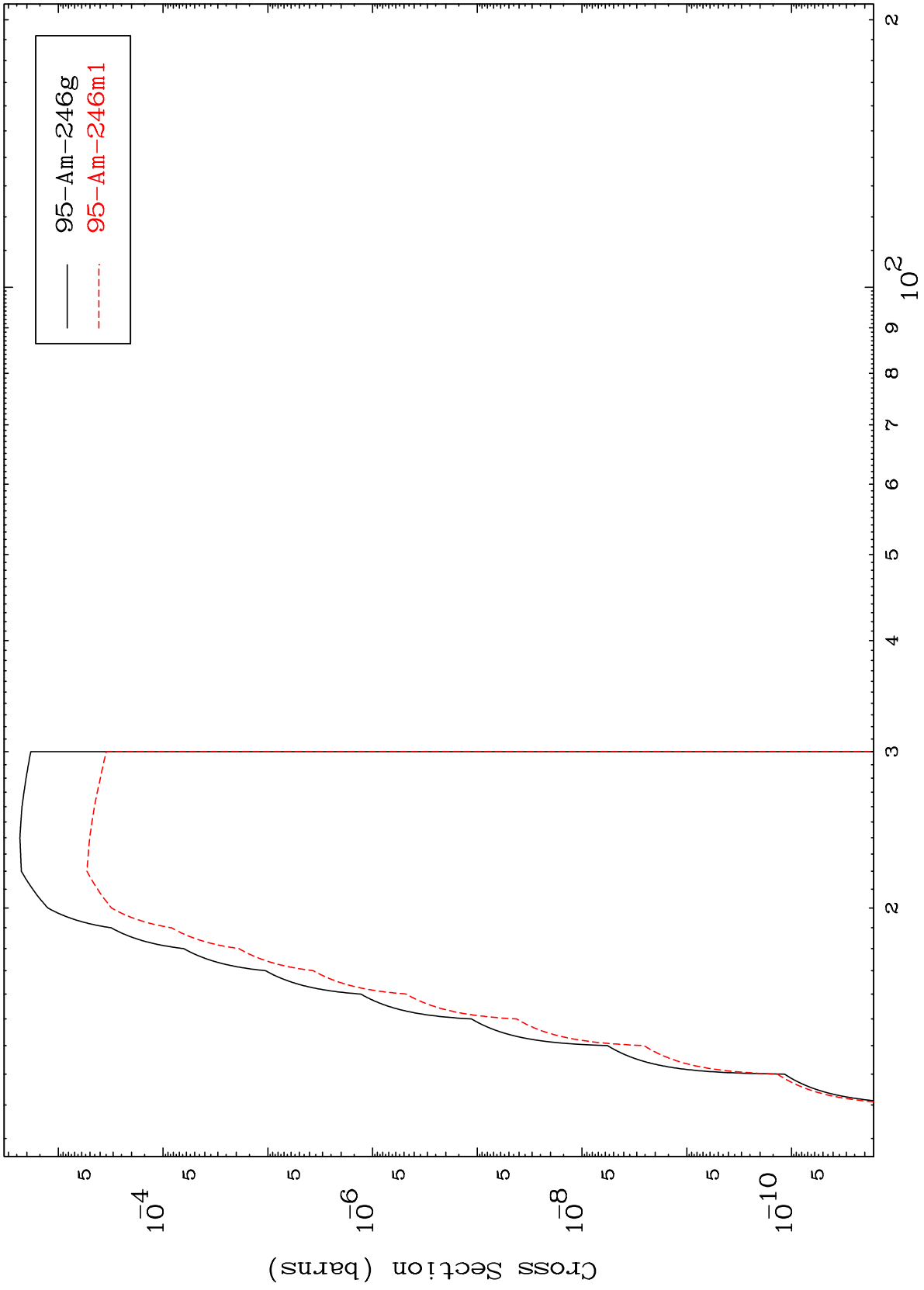
Incident Energy (MeV)

93-Np-243

MAT 9364

93-Np-243

Radionuclide Production Cross Section



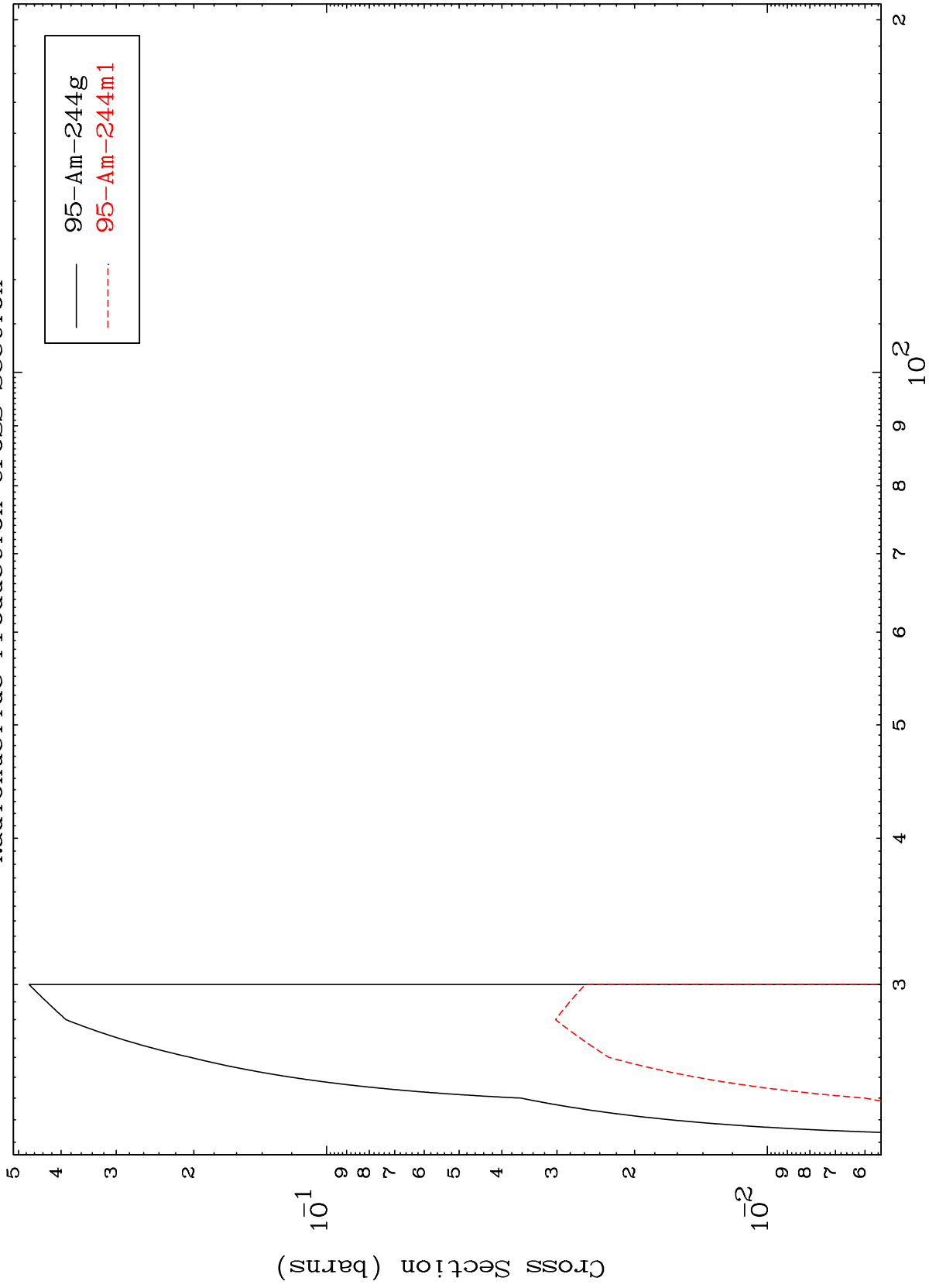
9

93-Np-243

MAT 9364

93-Np-243

( $\alpha, 3n$ )  
Radionuclide Production Cross Section



10

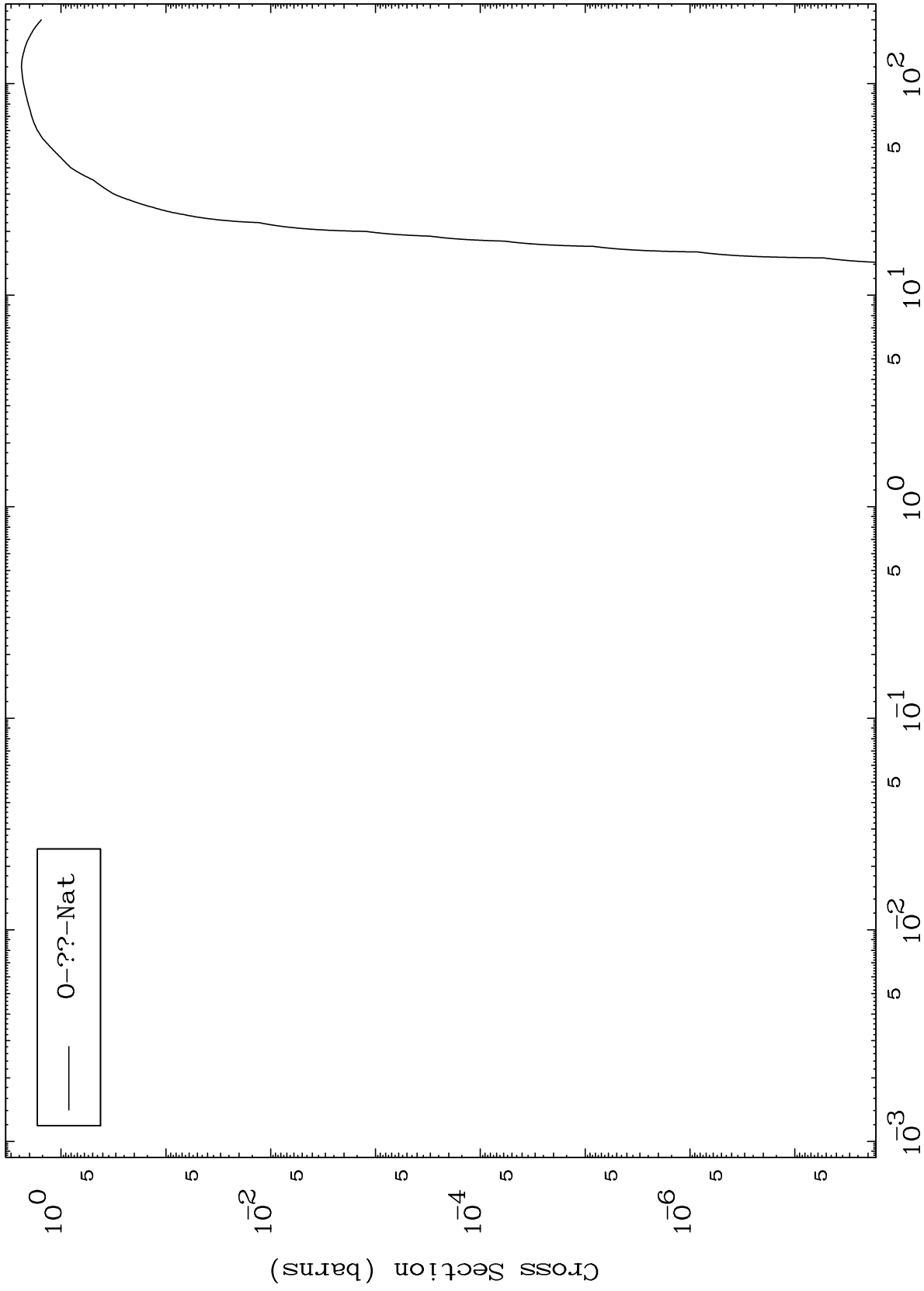
Incident Energy (MeV)

93-Np-243

MAT 9364

$\alpha$  Fission  
Radionuclide Production Cross Section

93-Np-243



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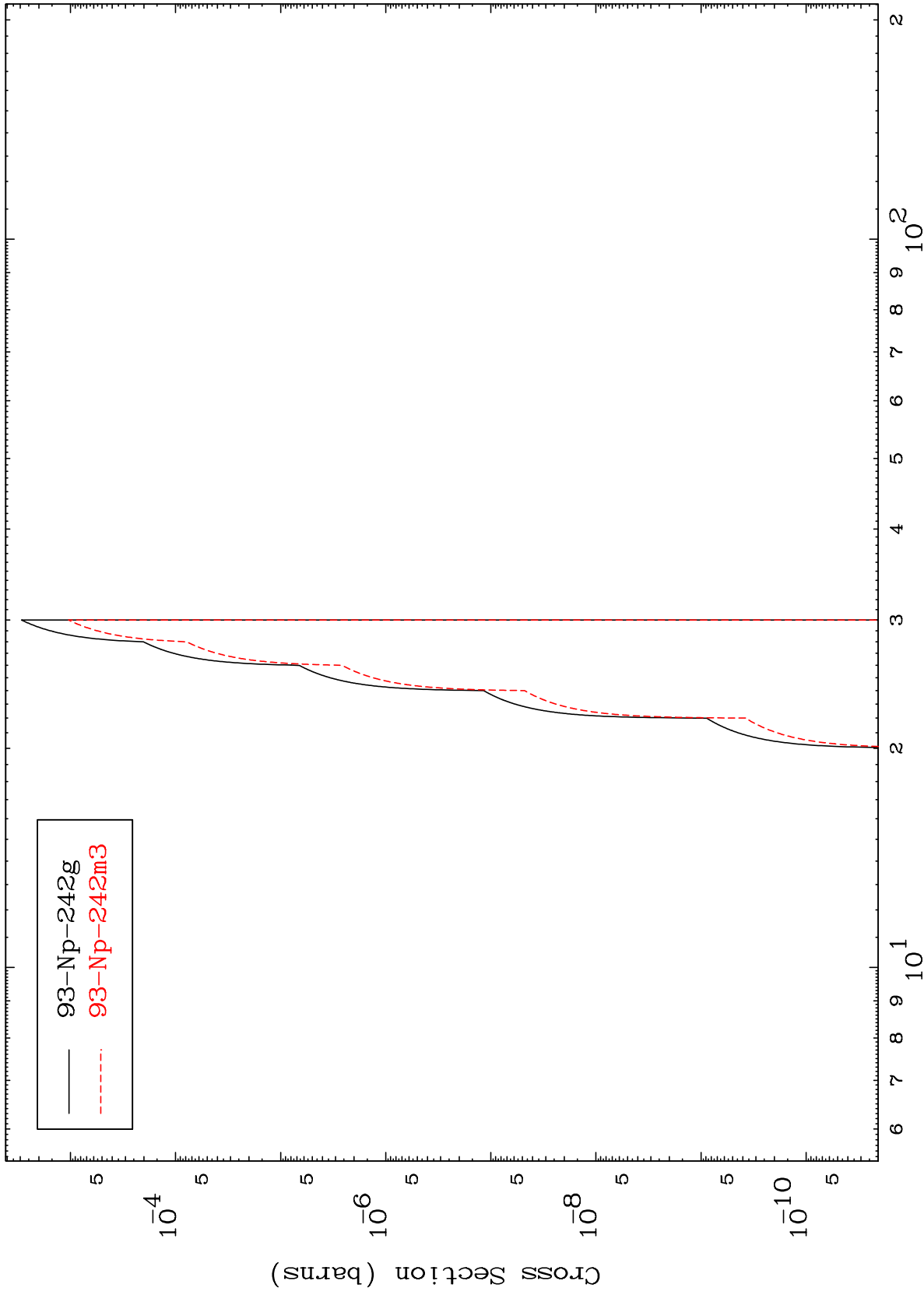
Incident Energy (MeV)

93-Np-243

MAT 9364

93-Np-243

$(\alpha, n')$   $\alpha$   
Radionuclide Production Cross Section



12

Incident Energy (MeV)

93-Np-243