

Program EVALPLOT  
(Version 2018-1)

by

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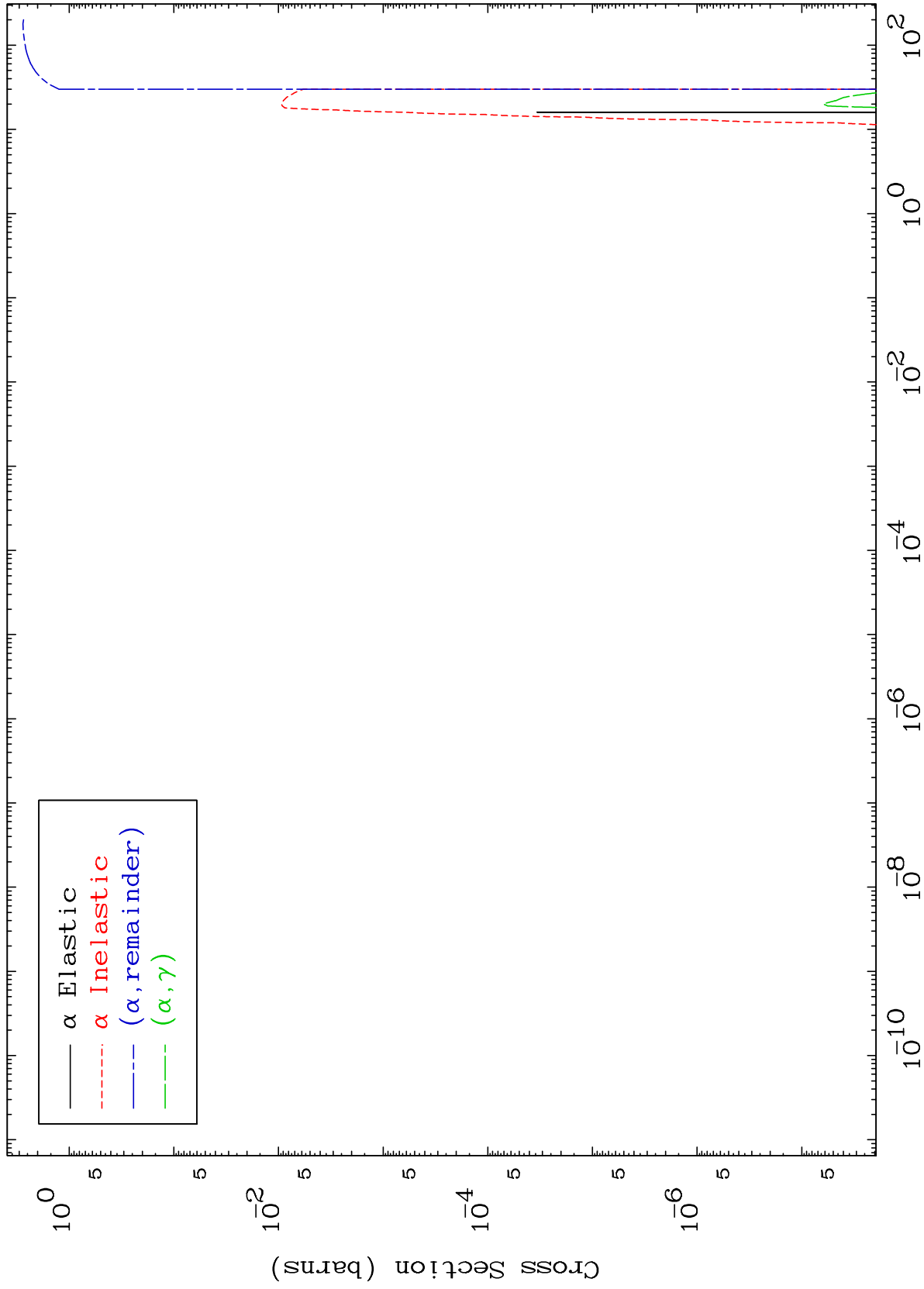
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7664

0 Kelvin  $\alpha$  Major  
Cross Sections

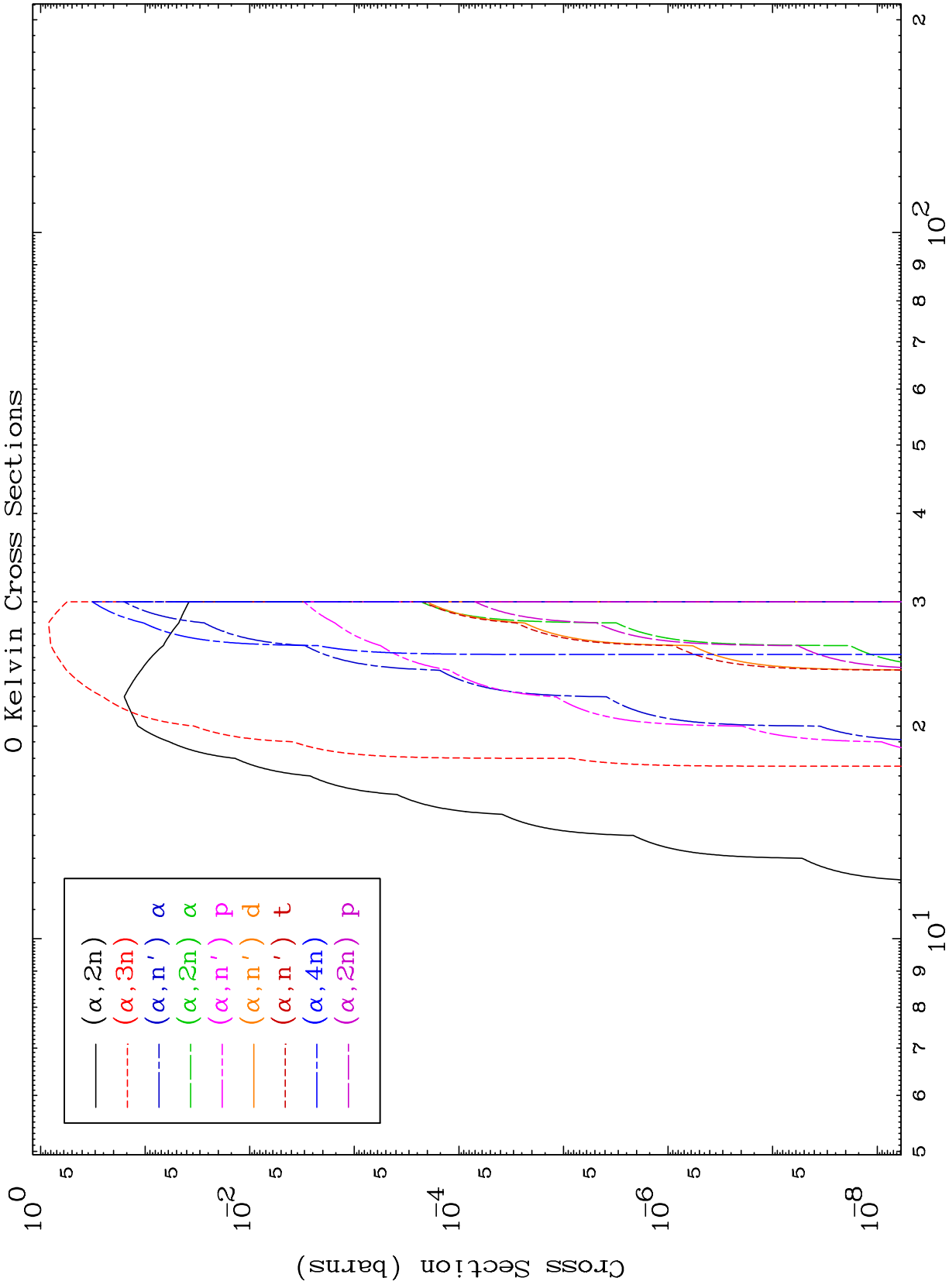
76-Os-197



MAT 7664

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

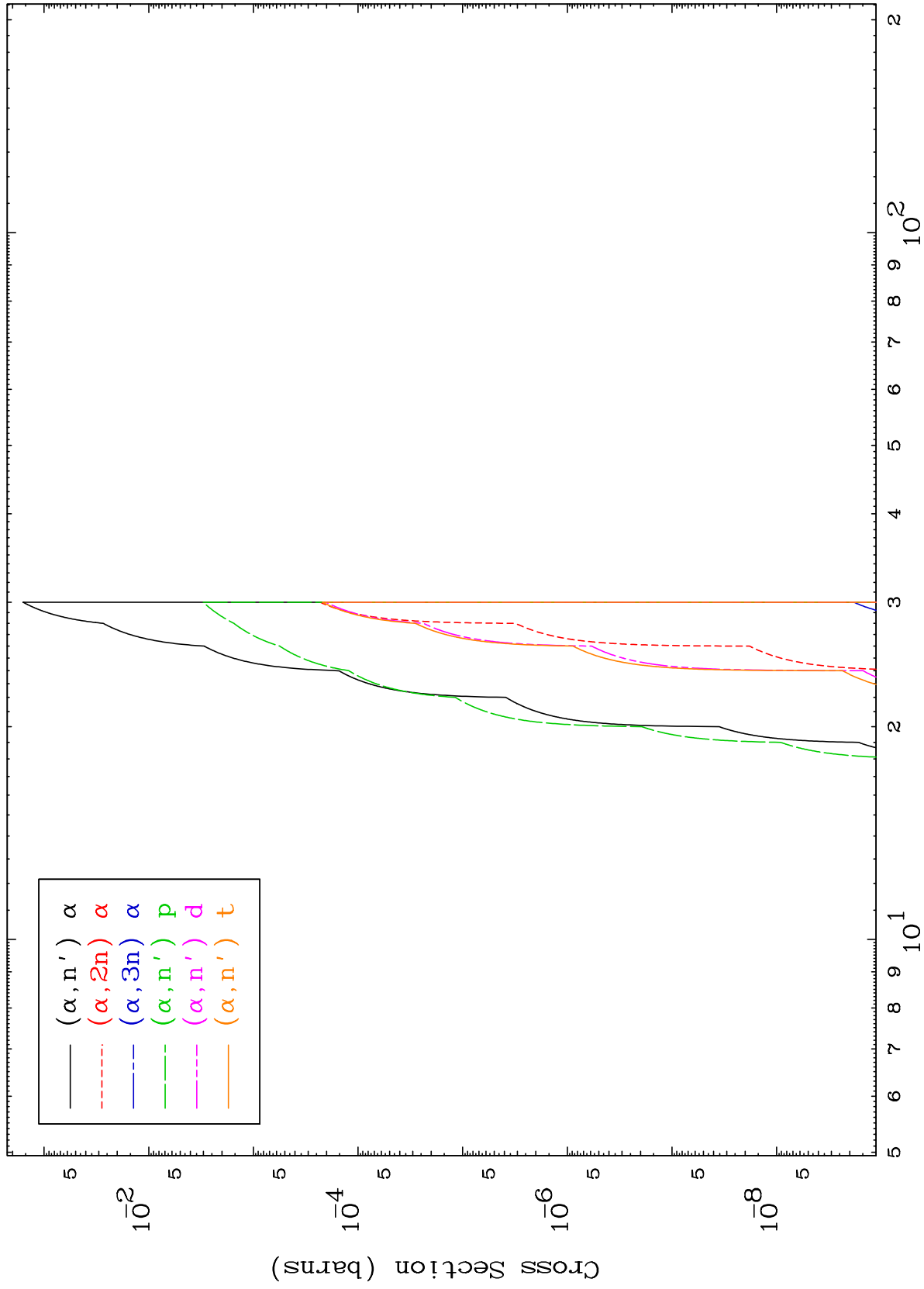
76-Os-197

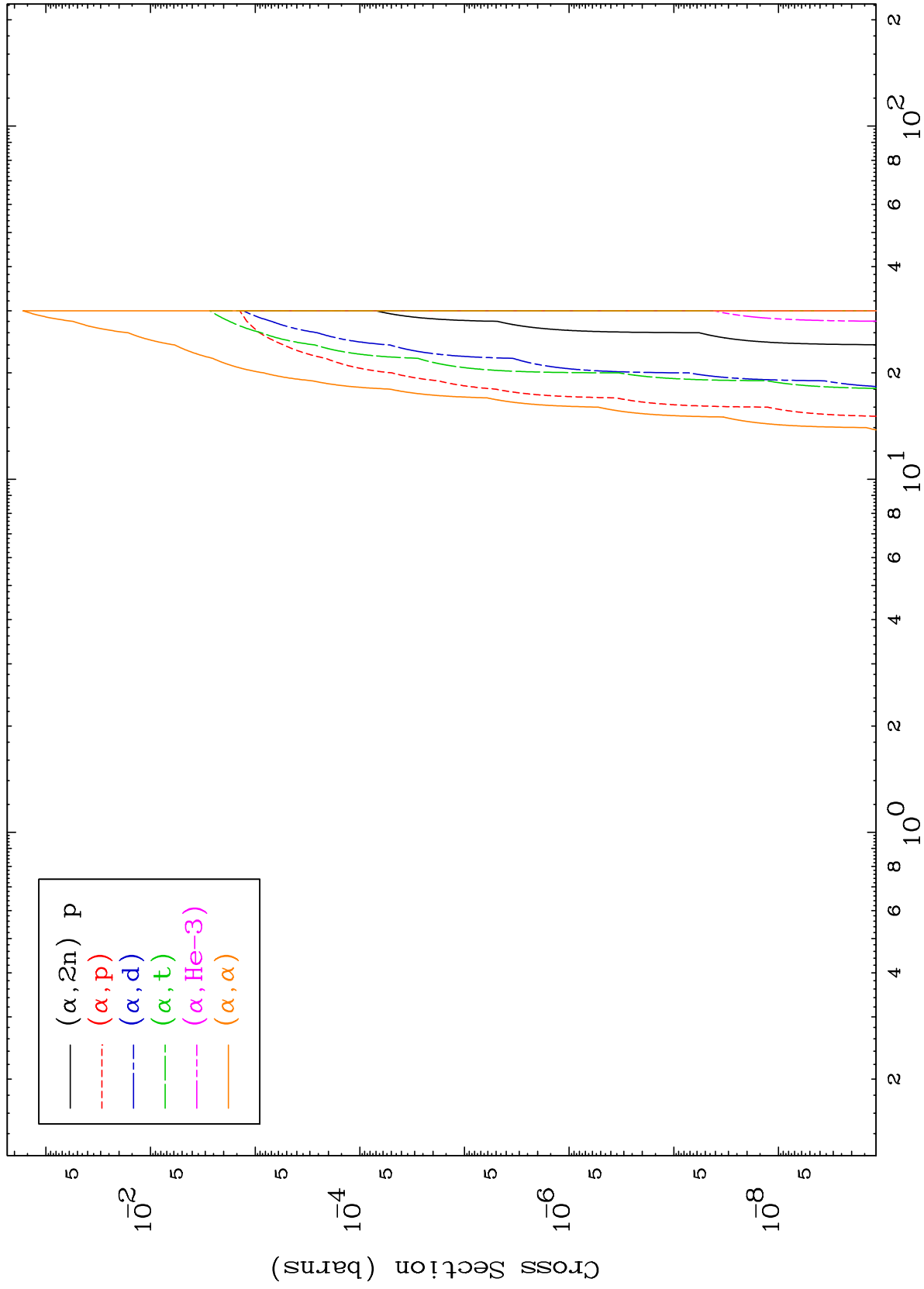


2

Incident Energy (MeV)

76-Os-197

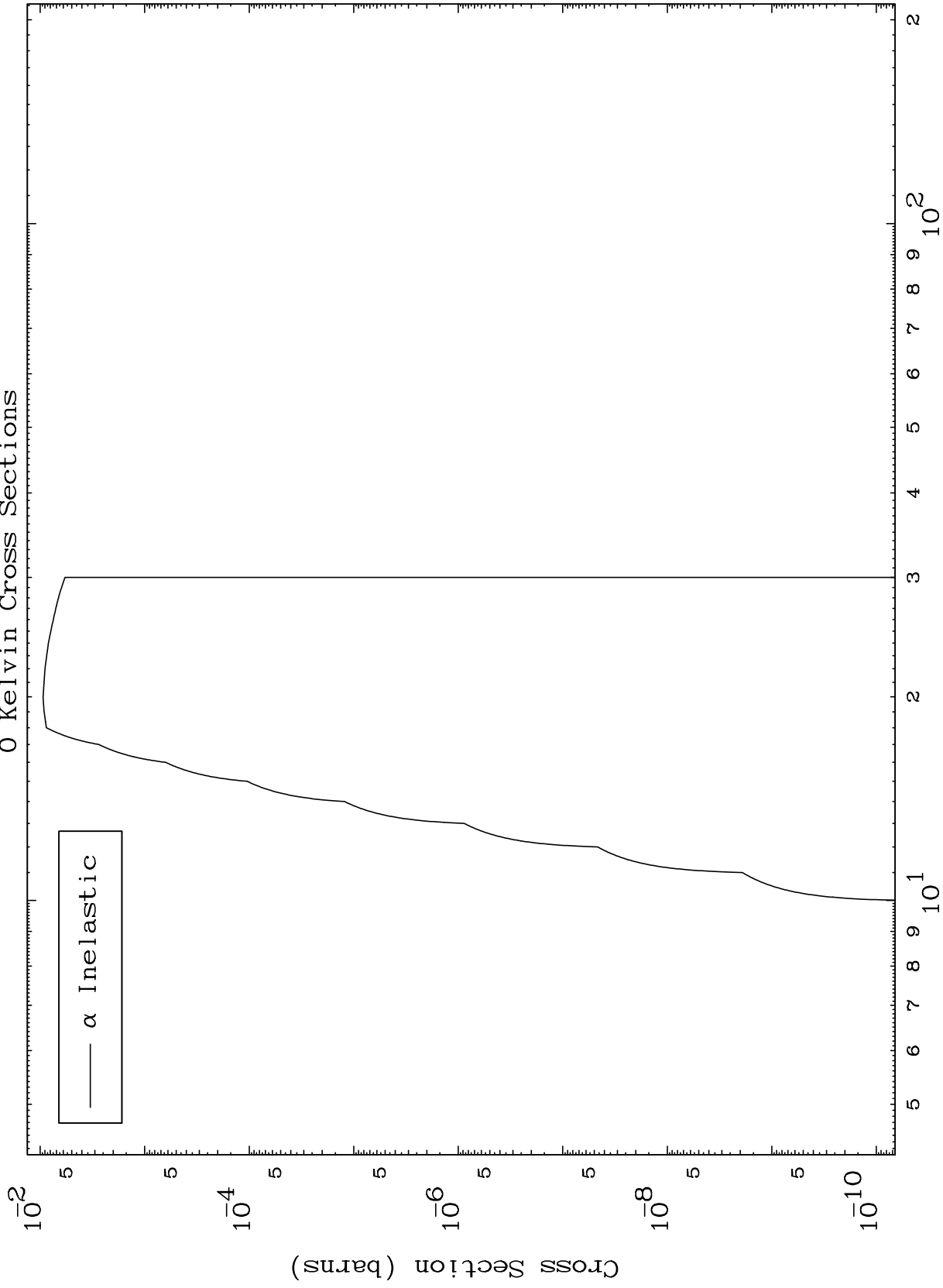




MAT 7664

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

76-0s-197



5

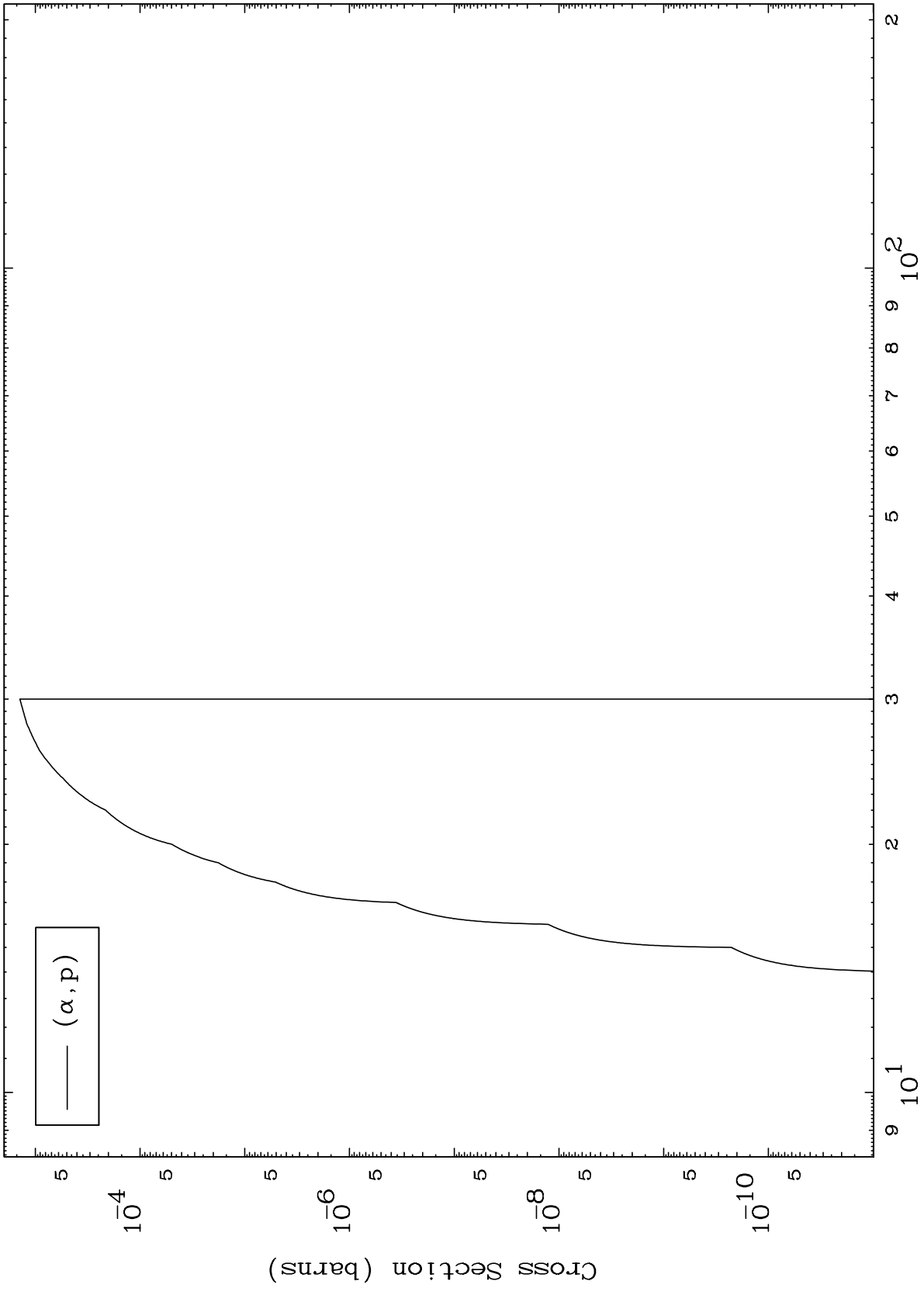
Incident Energy (MeV)

76-0s-197

MAT 7664

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

76-0s-197



6

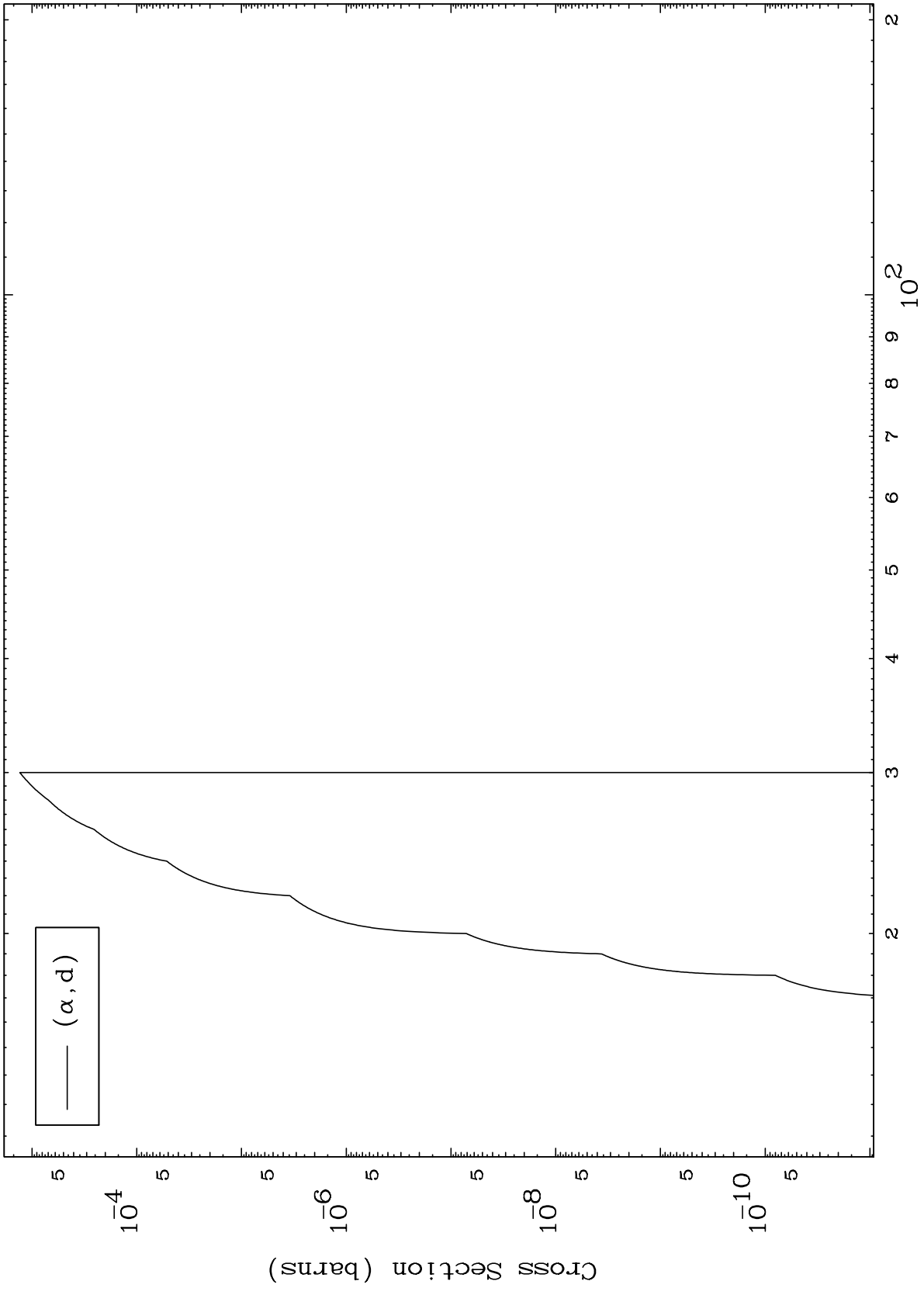
Incident Energy (MeV)

76-0s-197

MAT 7664

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

76-0s-197

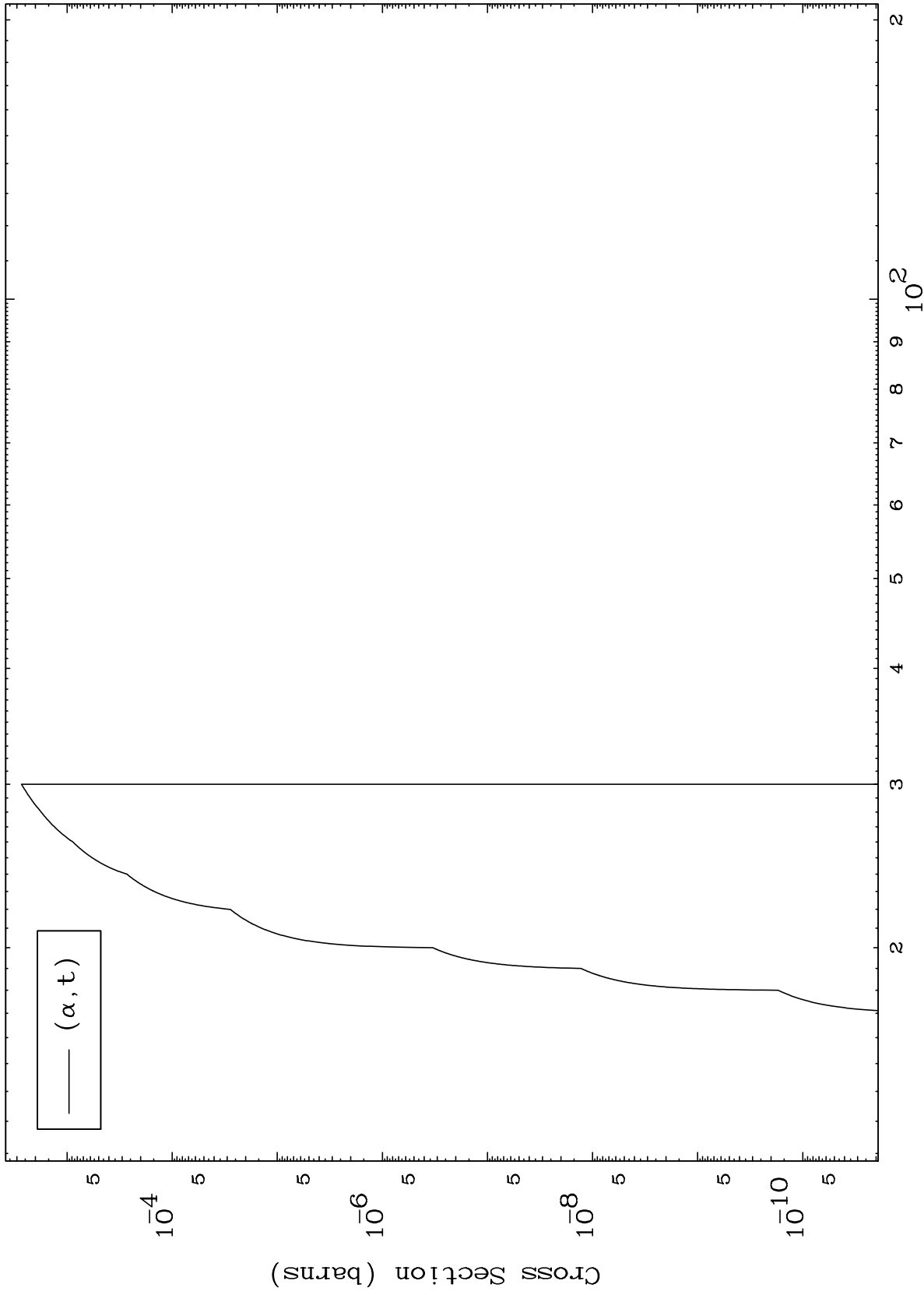


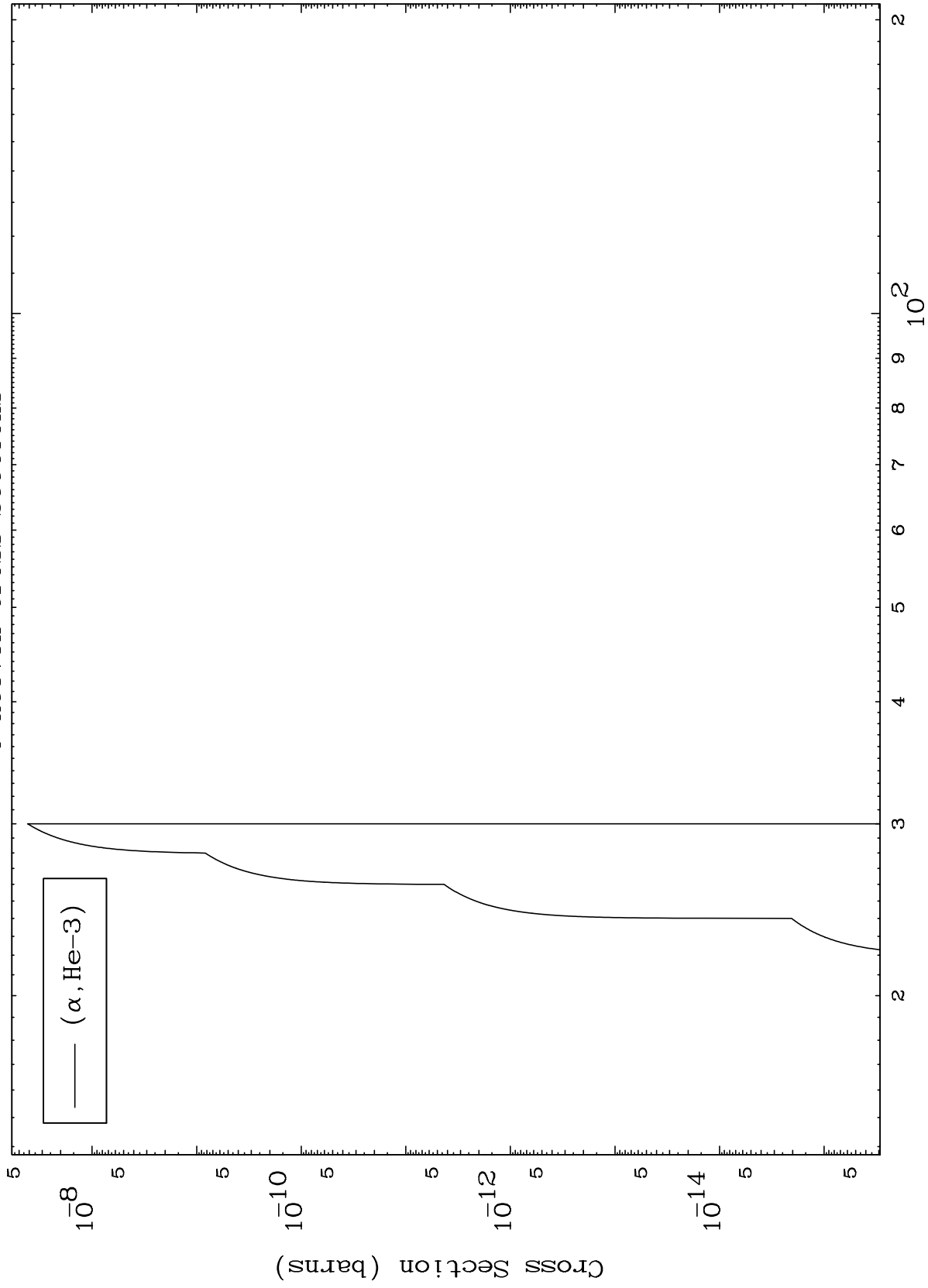
7

Incident Energy (MeV)

76-0s-197



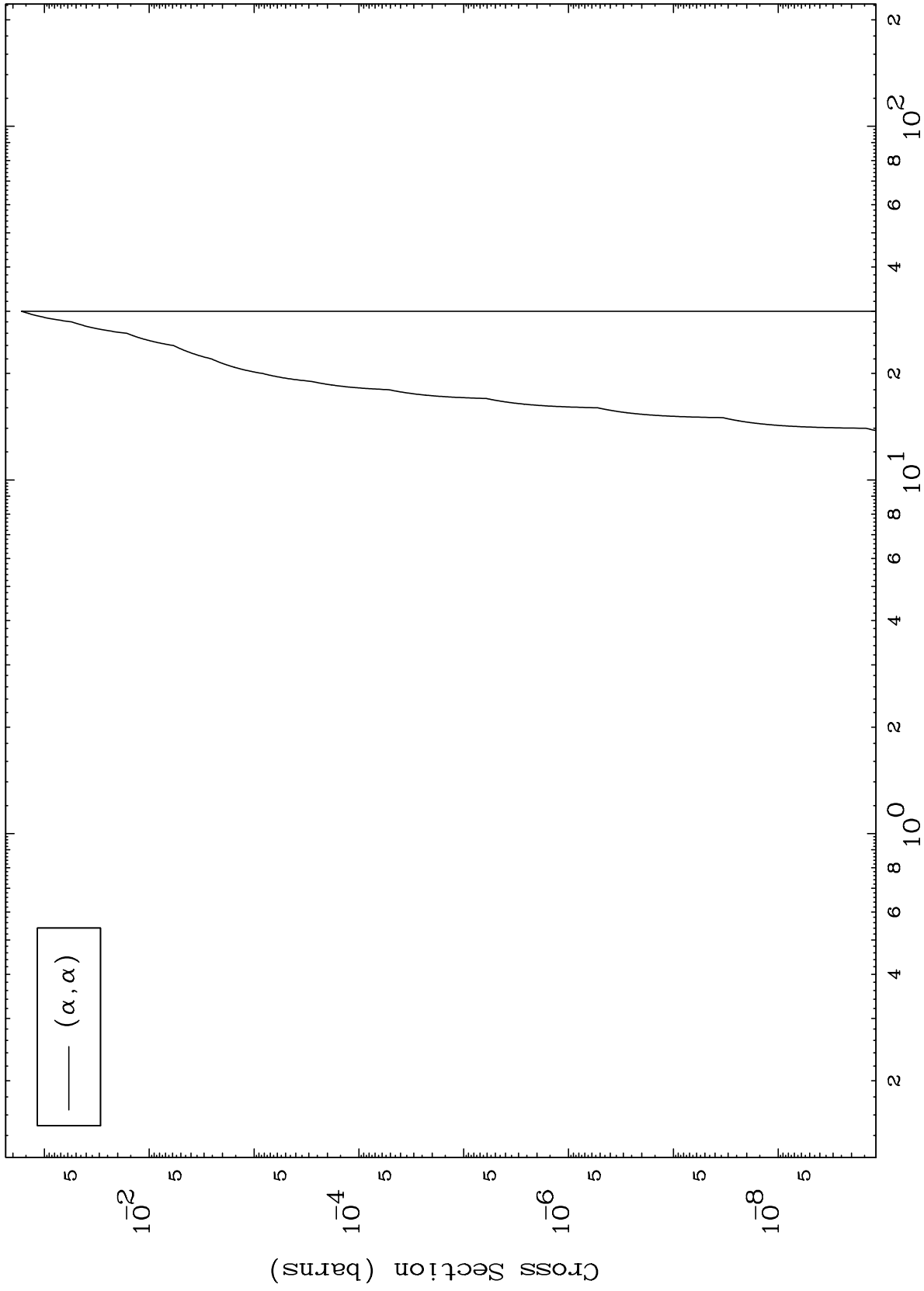




MAT 7664

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

76-0s-197



10

Incident Energy (MeV)

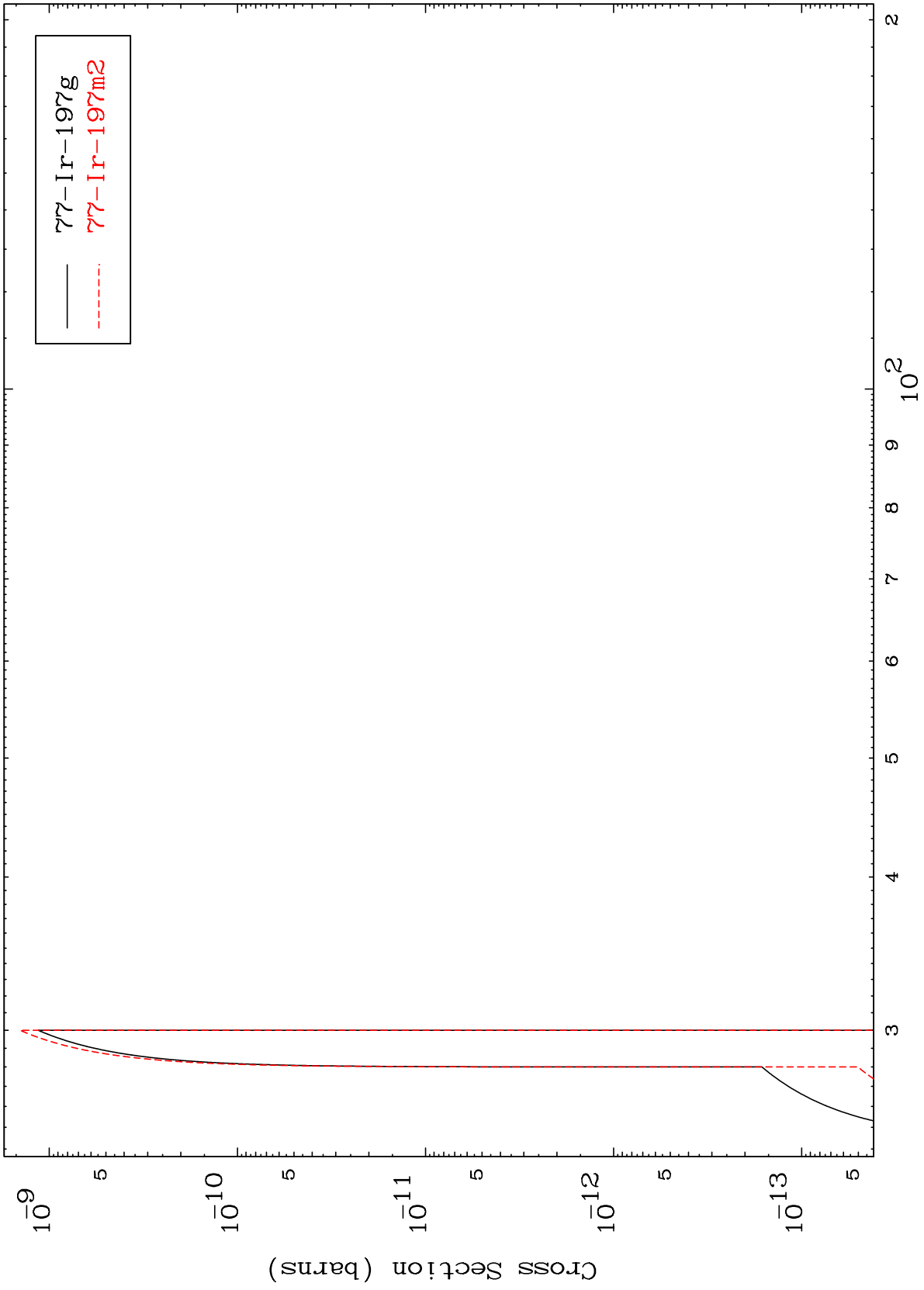
76-0s-197

MAT 7664

( $\alpha, 2n$ ) d

76-0s-197

Radionuclide Production Cross Section



11

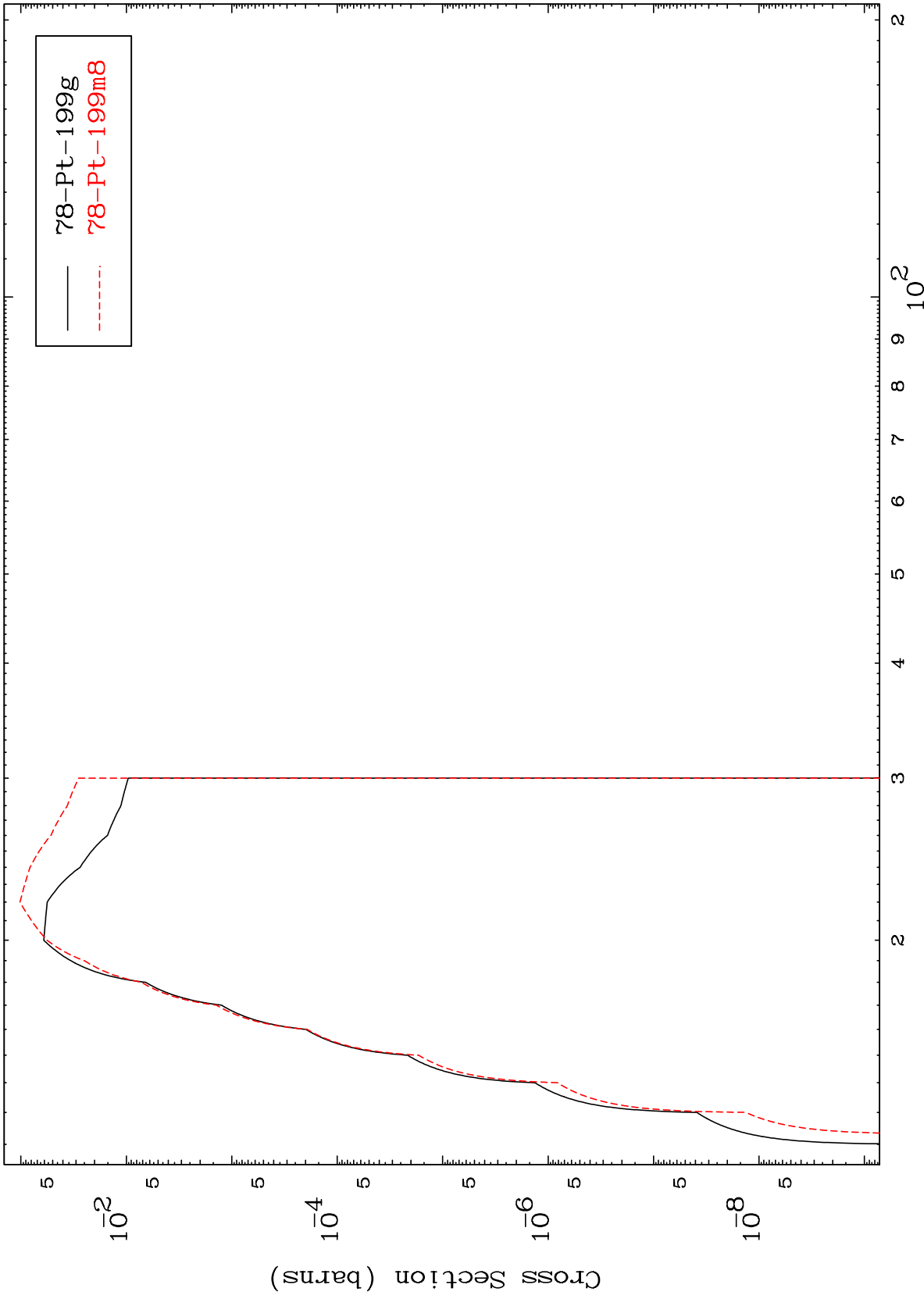
Incident Energy (MeV)

76-0s-197

MAT 7664

76-0s-197

Radionuclide Production Cross Section  
( $\alpha, 2n$ )



12

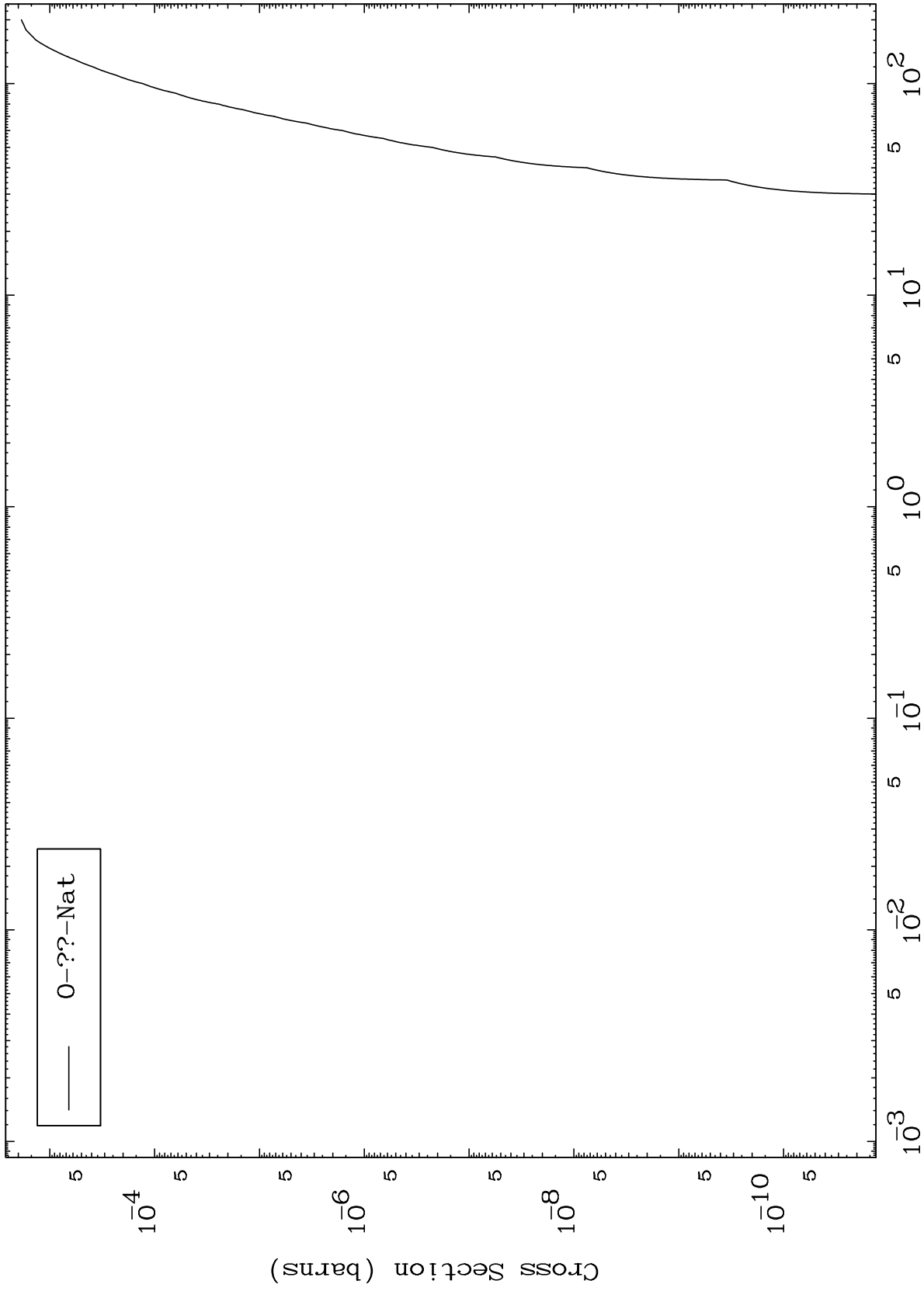
Incident Energy (MeV)

76-0s-197

MAT 7664

$\alpha$  Fission  
Radionuclide Production Cross Section

76-0s-197



13

Incident Energy (MeV)

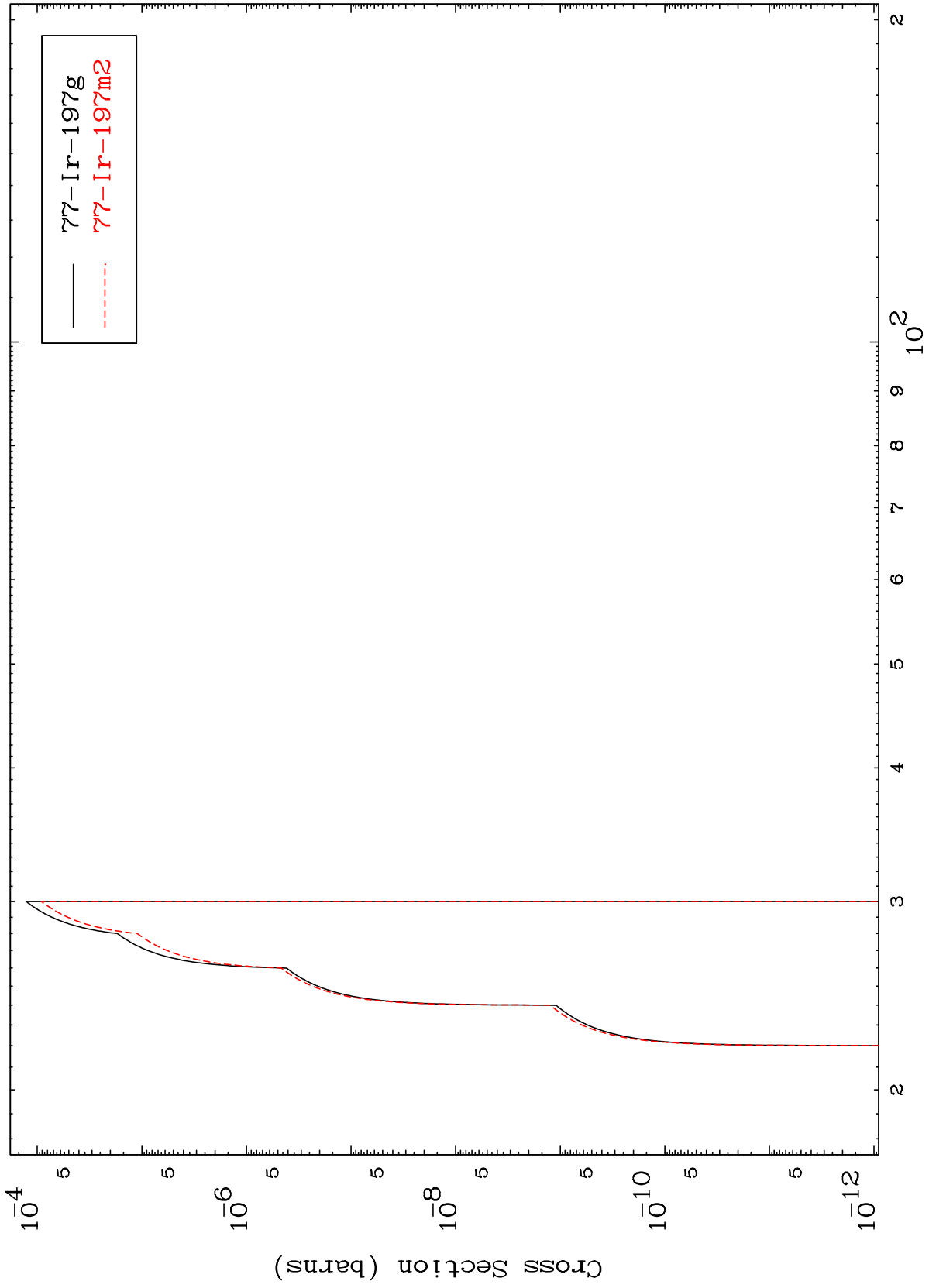
76-0s-197

MAT 7664

( $\alpha, n'$ ) t

76-0s-197

Radionuclide Production Cross Section



14

Incident Energy (MeV)

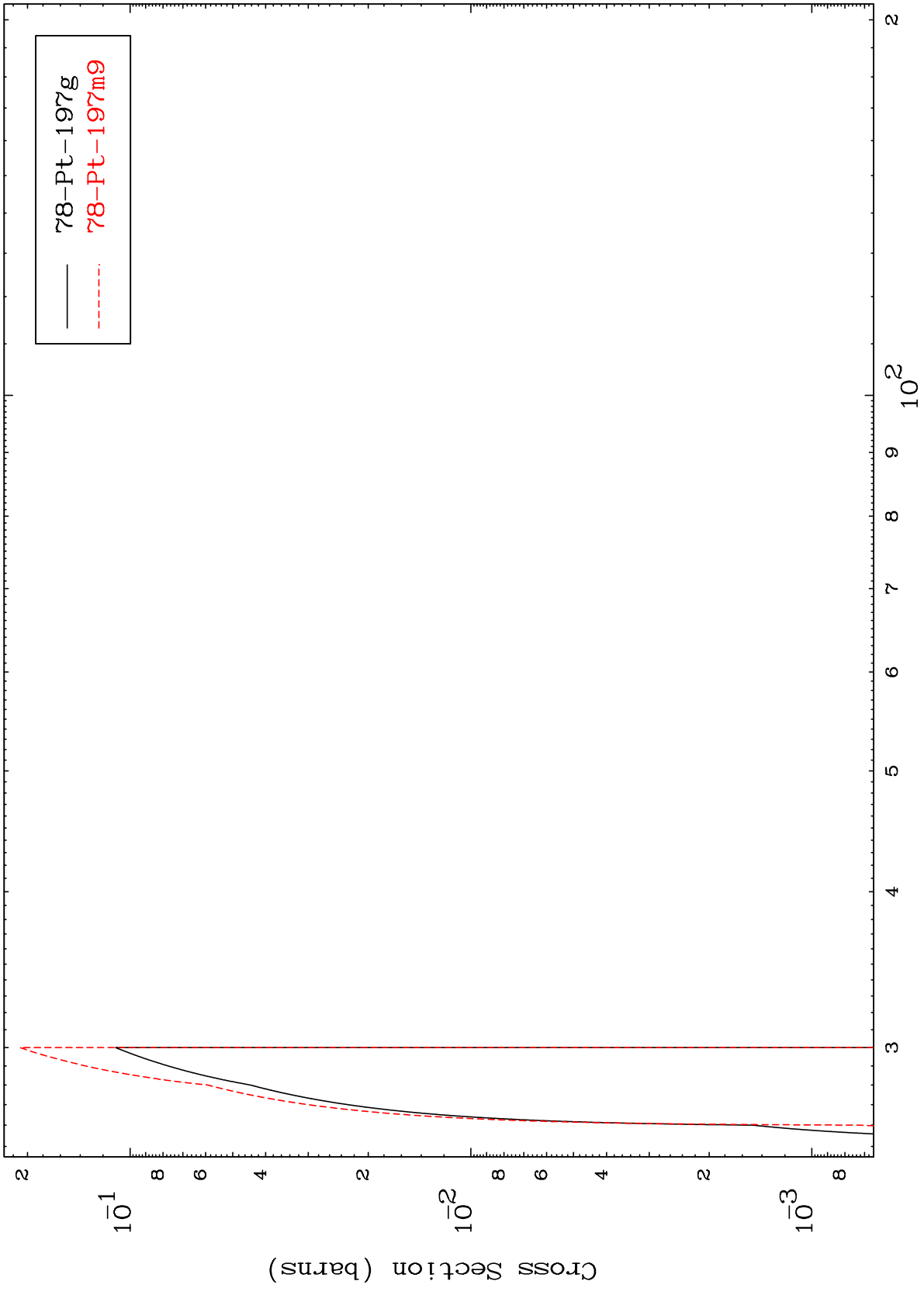
76-0s-197

MAT 7664

( $\alpha, 4n$ )

76-0s-197

Radionuclide Production Cross Section



78-Pt-197g  
78-Pt-197m9

15

Incident Energy (MeV)

76-0s-197