

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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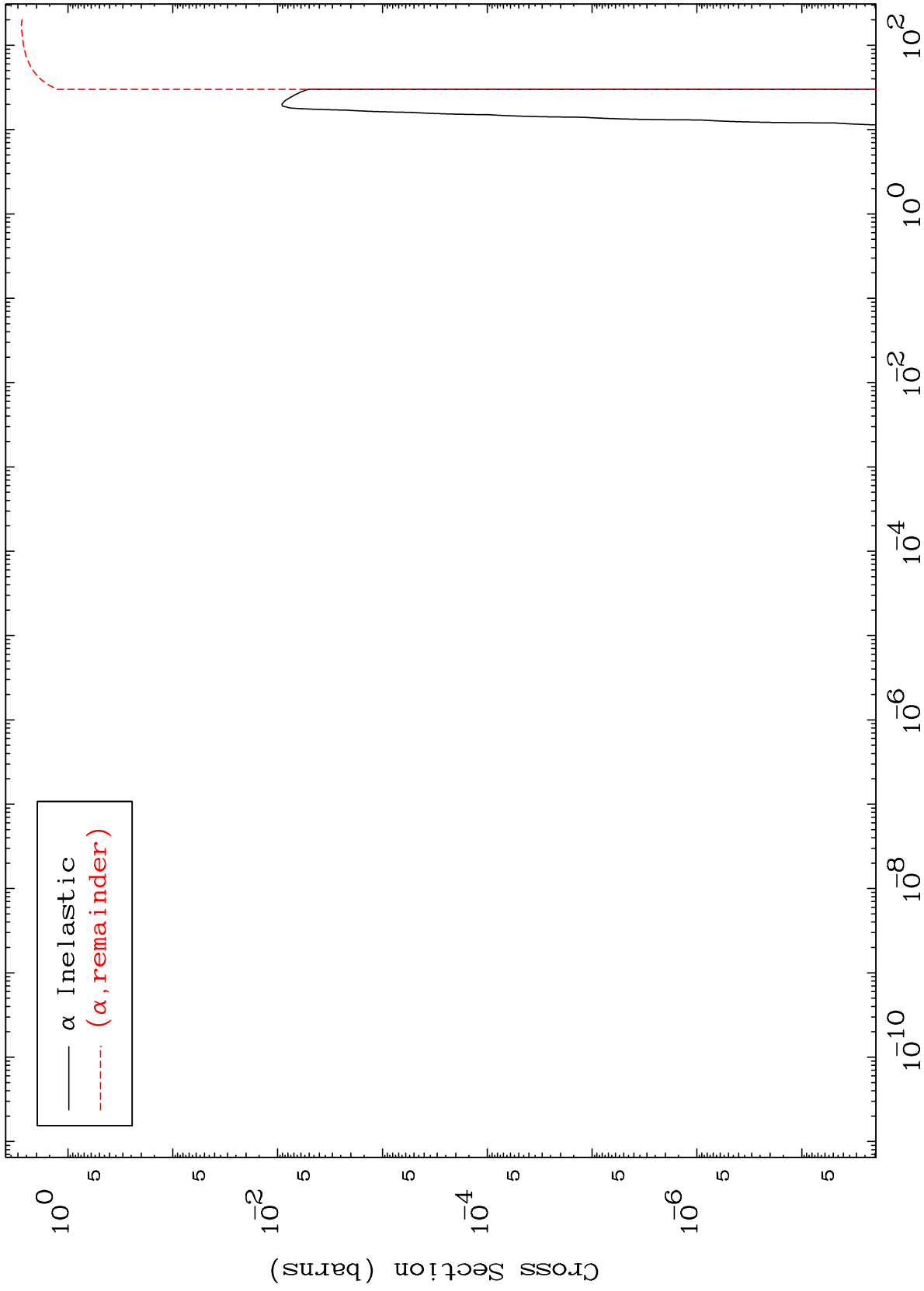
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7670

0 Kelvin  $\alpha$  Major  
Cross Sections

76-Os-199



1

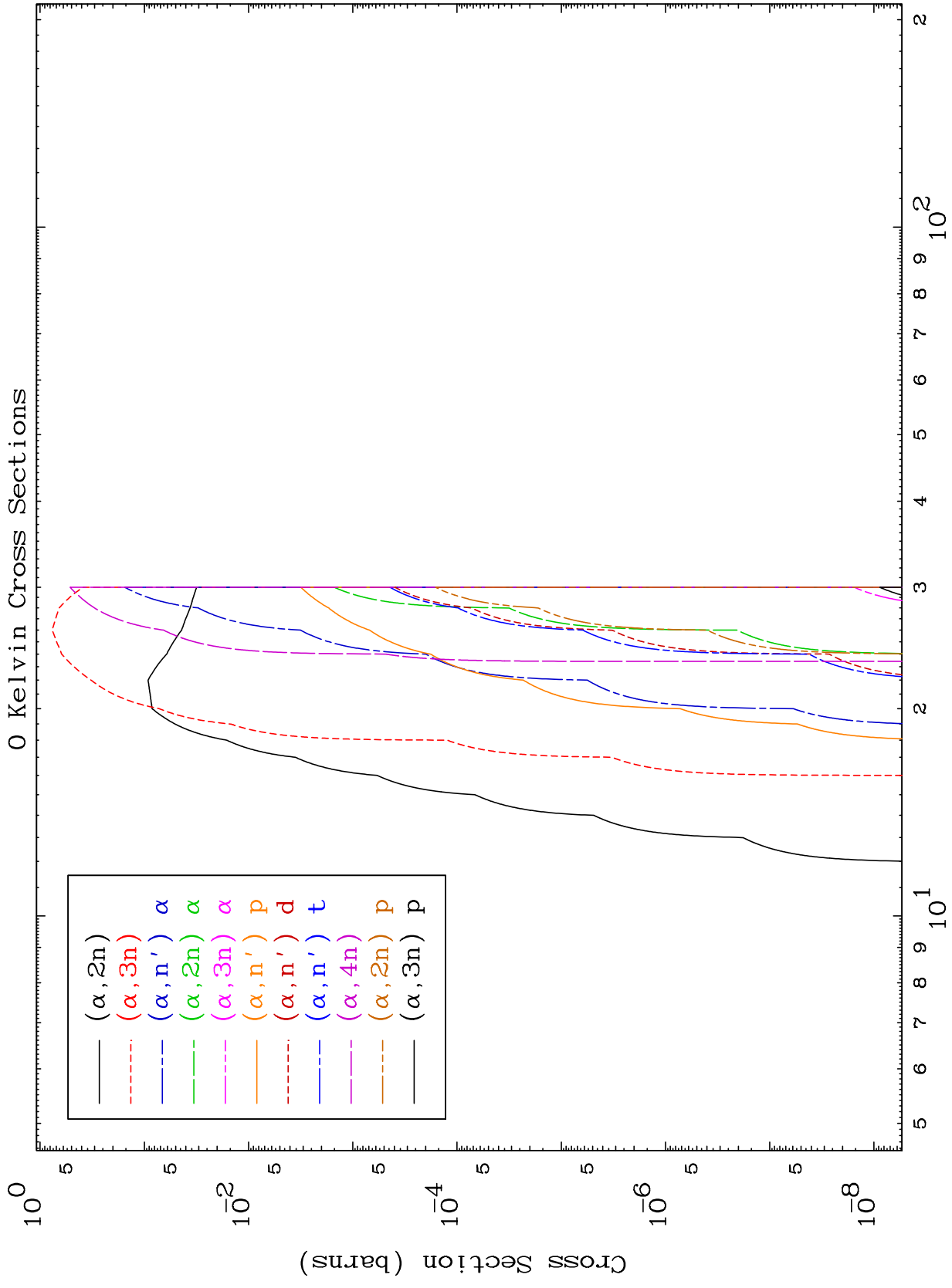
Incident Energy (MeV)

76-Os-199

MAT 7670

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

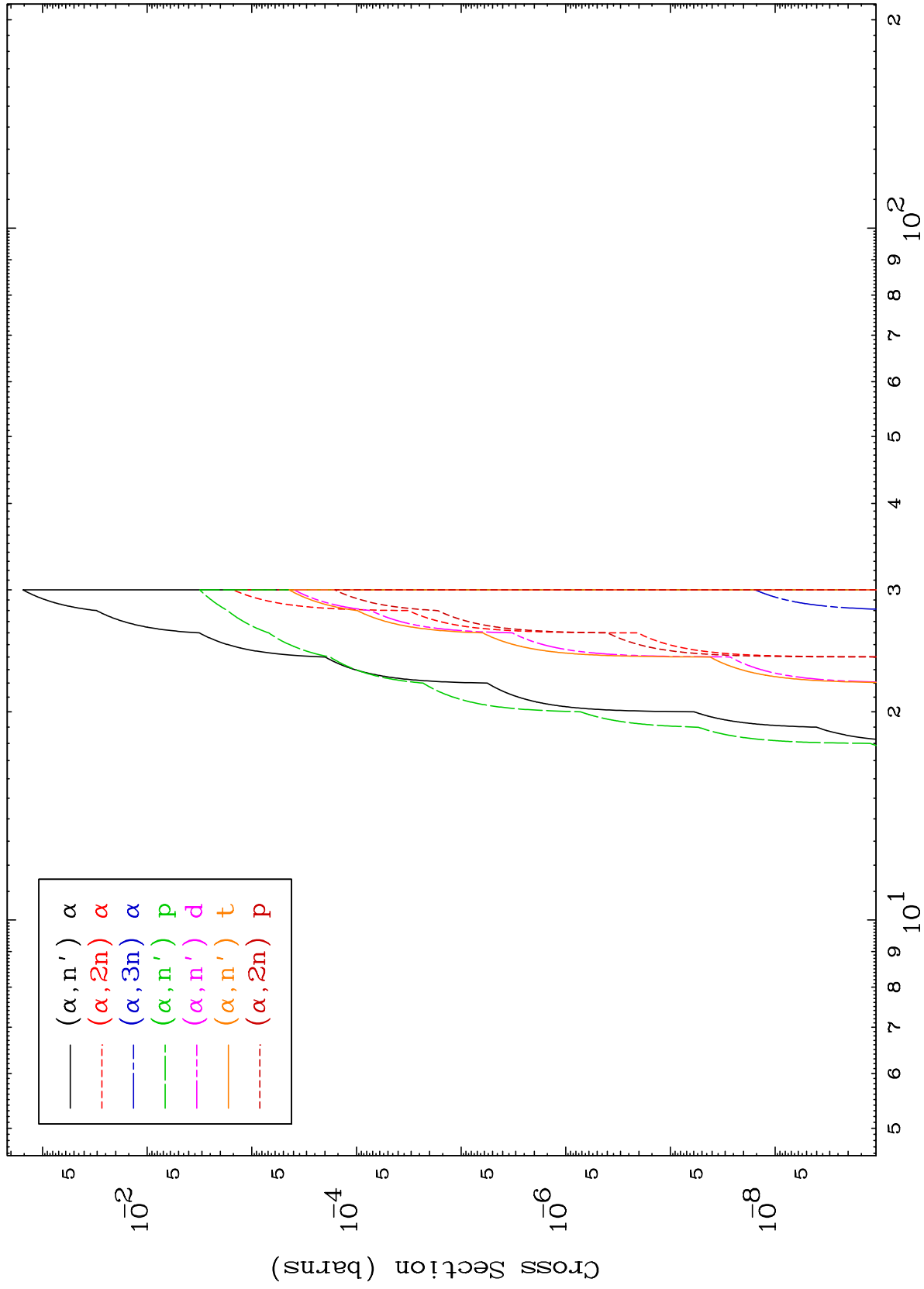
76-Os-199



2

Incident Energy (MeV)

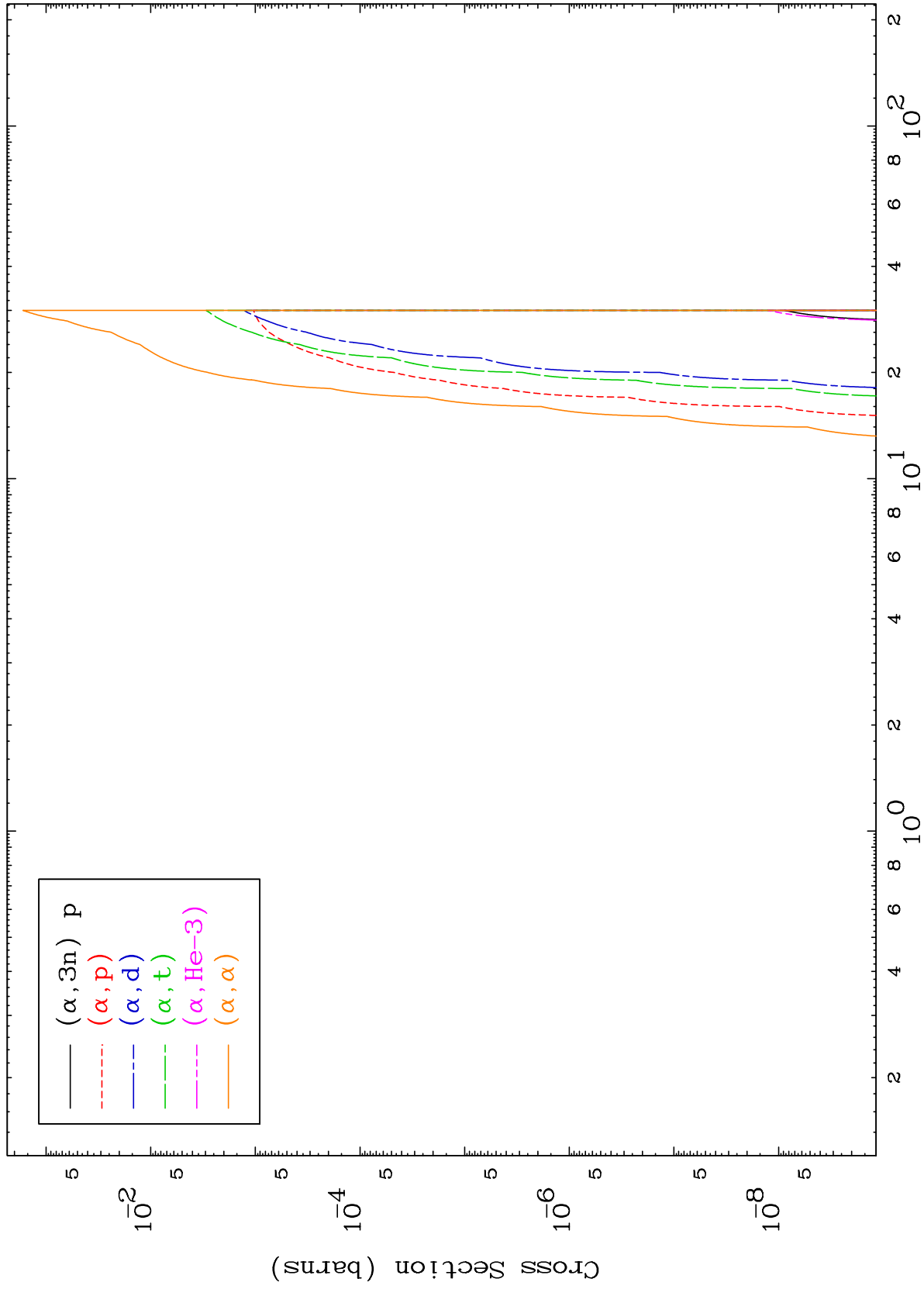
76-Os-199



MAT 7670

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

76-Os-199



4

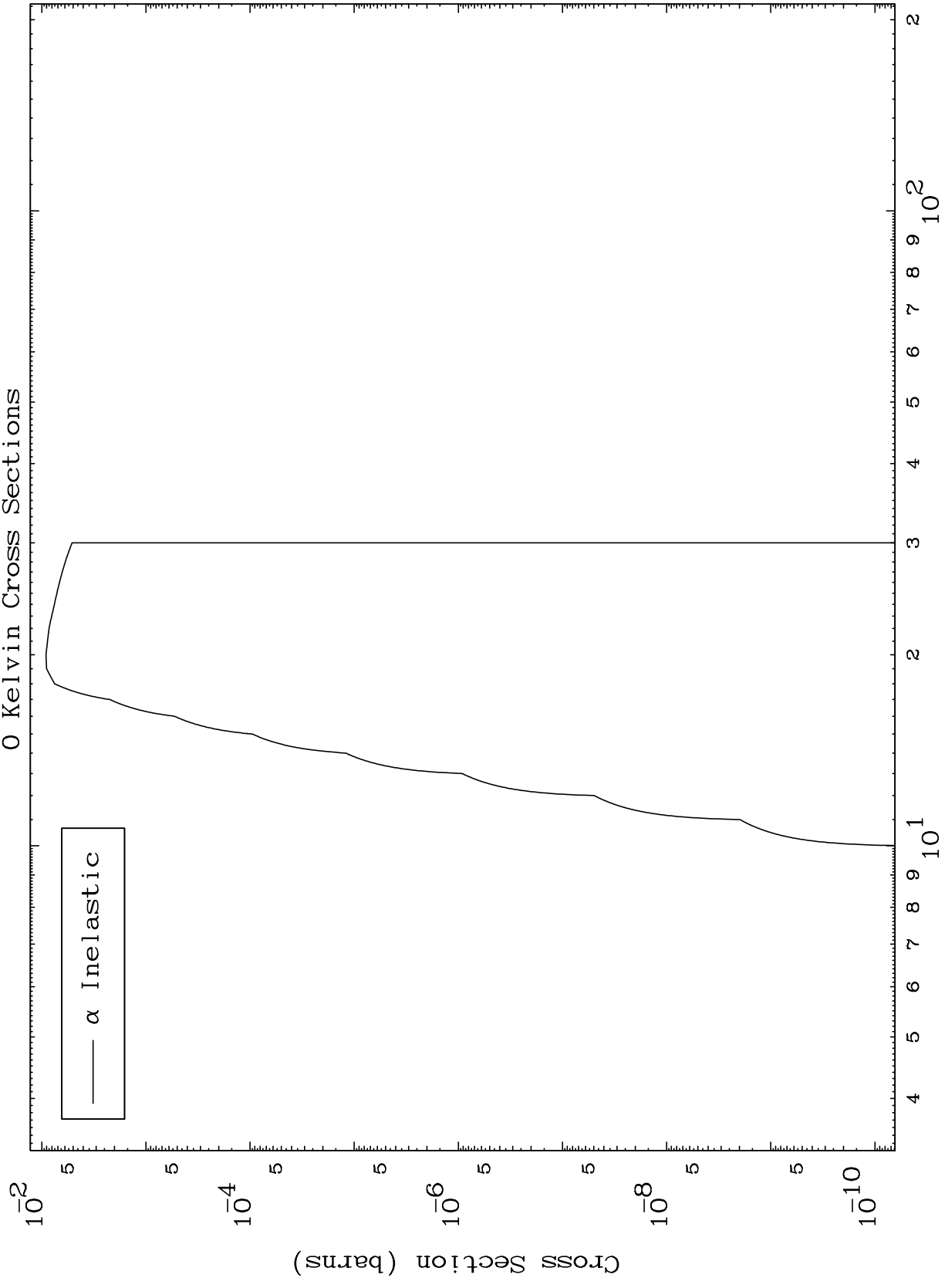
Incident Energy (MeV)

76-Os-199

MAT 7670

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

76-0s-199



5

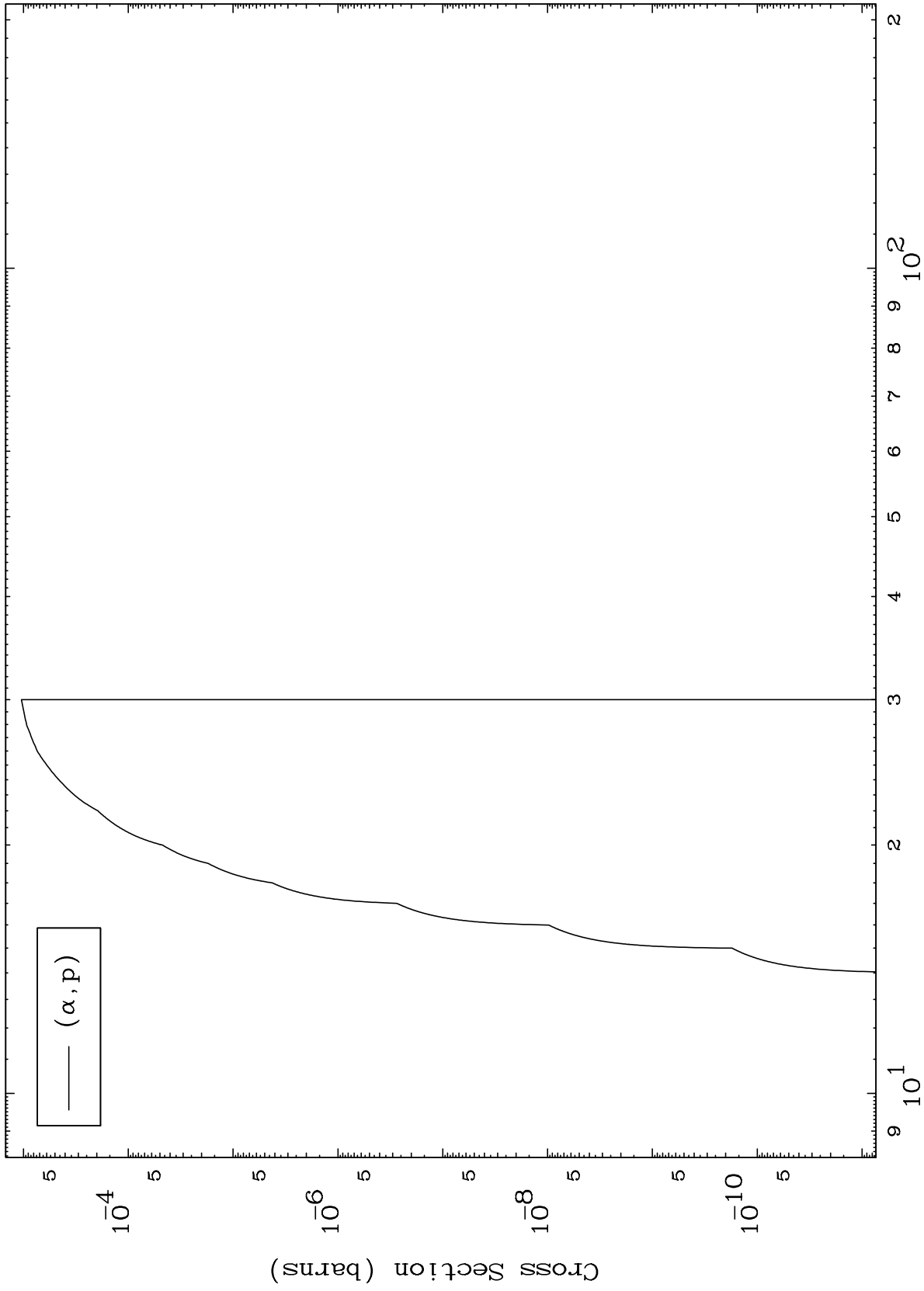
Incident Energy (MeV)

76-0s-199

MAT 7670

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

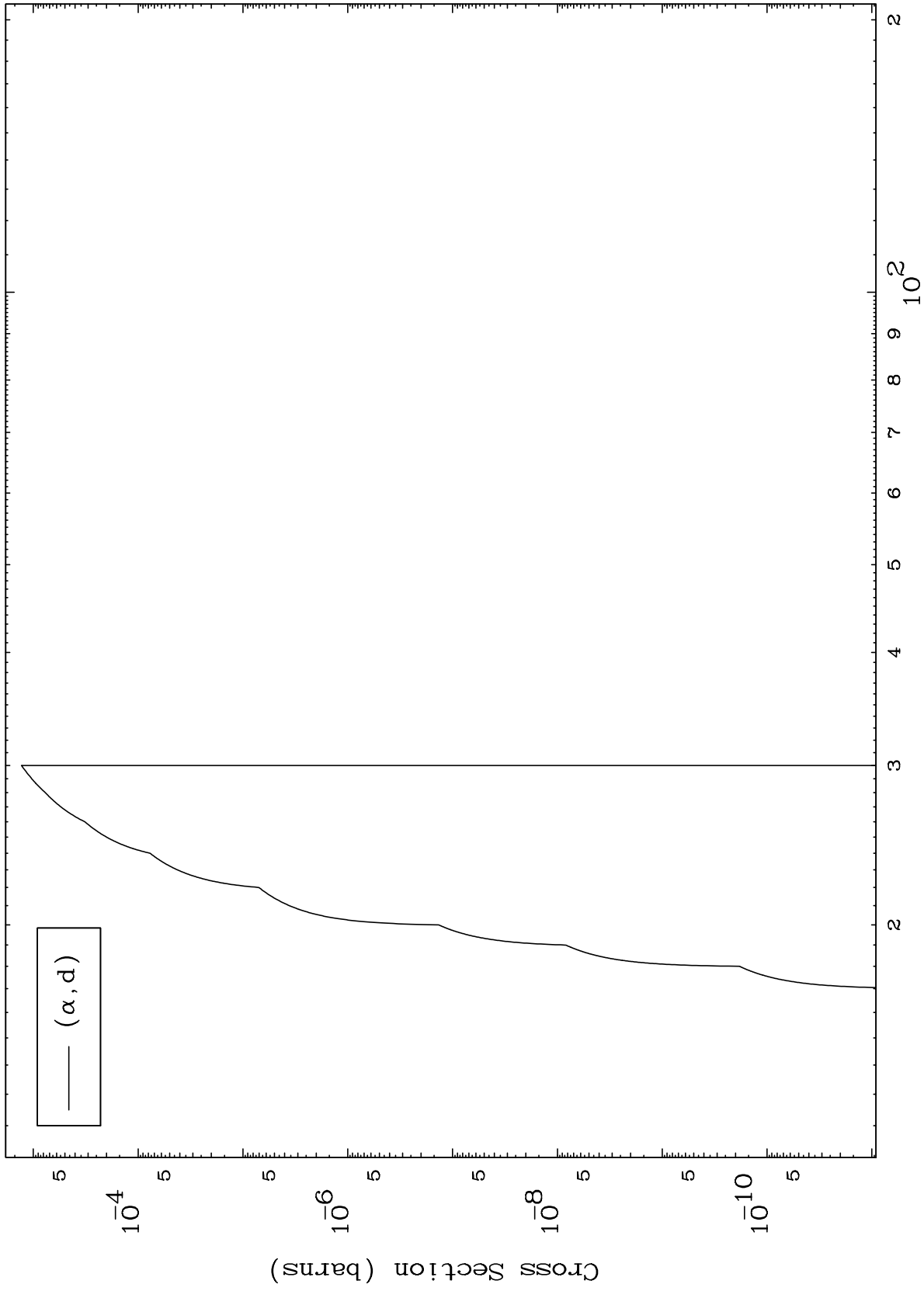
76-0s-199



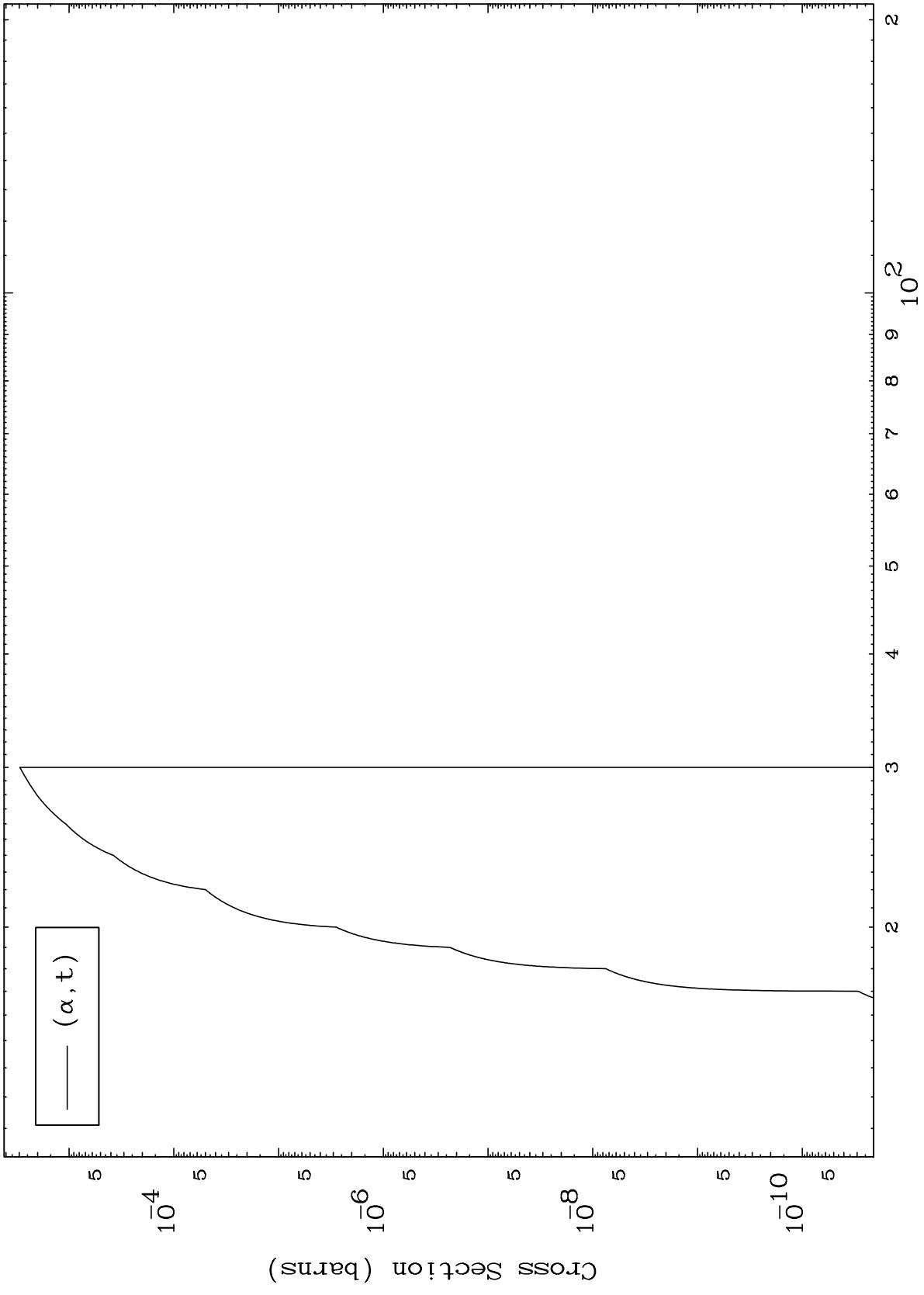
6

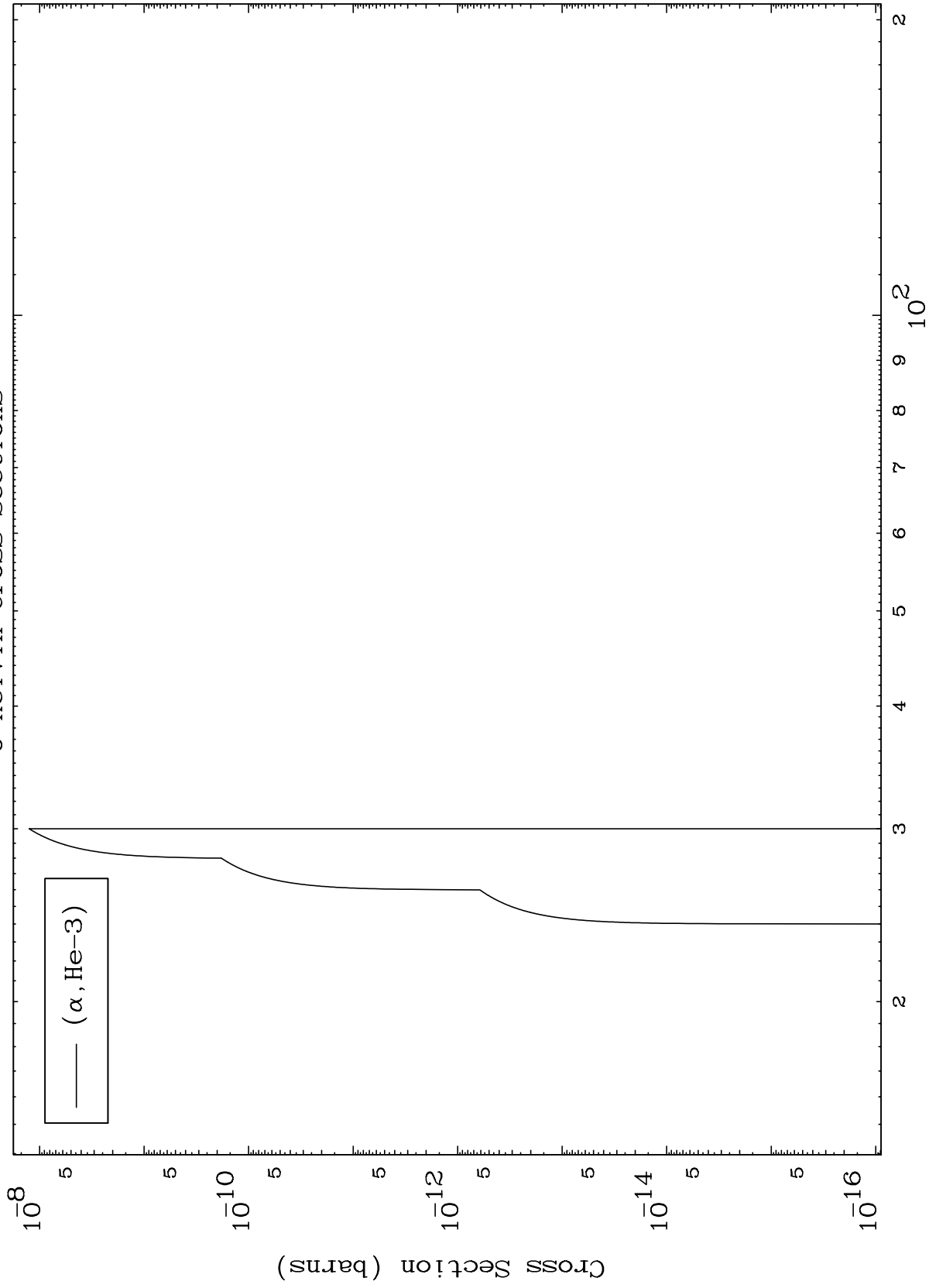
Incident Energy (MeV)

76-0s-199





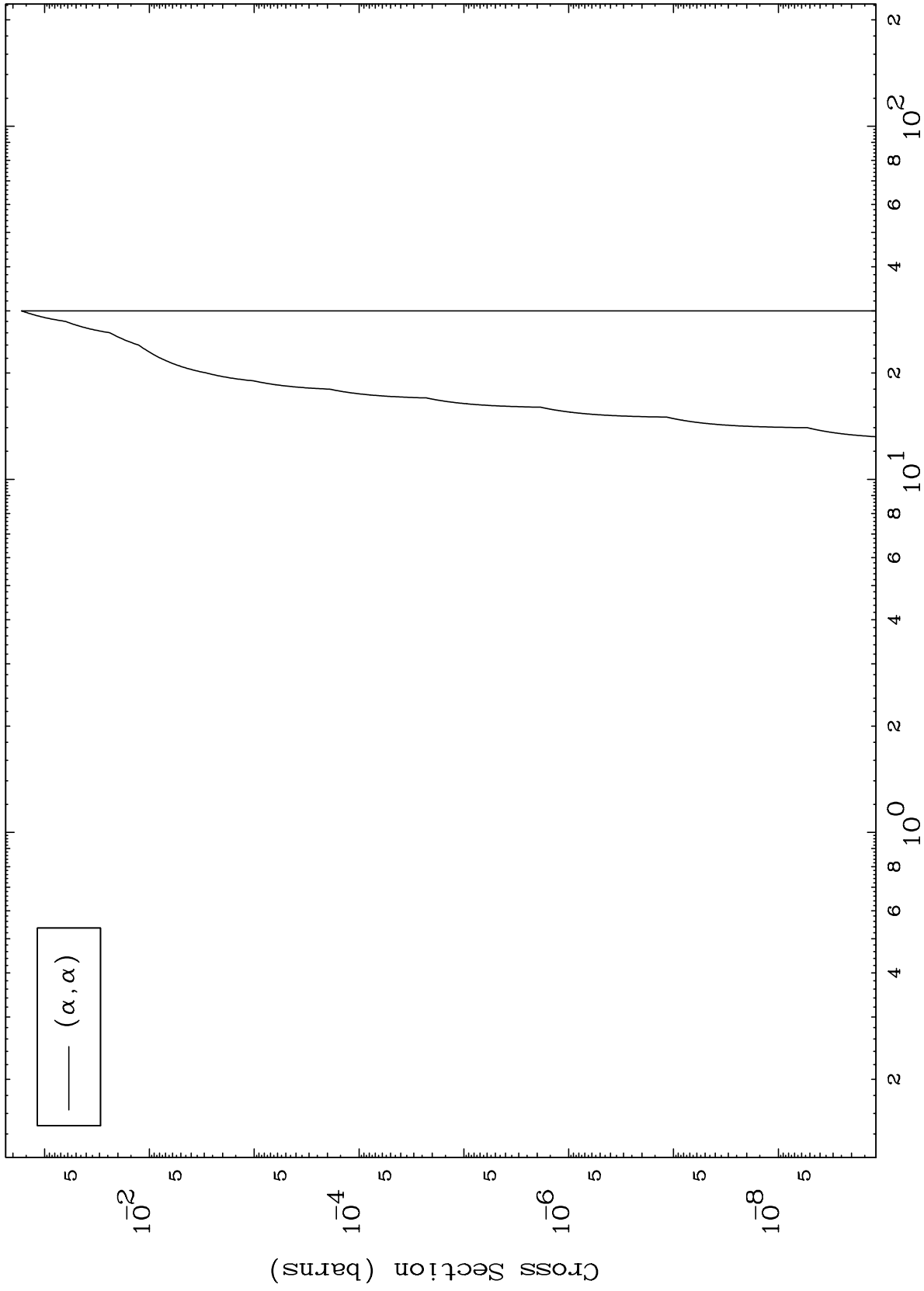




MAT 7670

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

76-0s-199



10

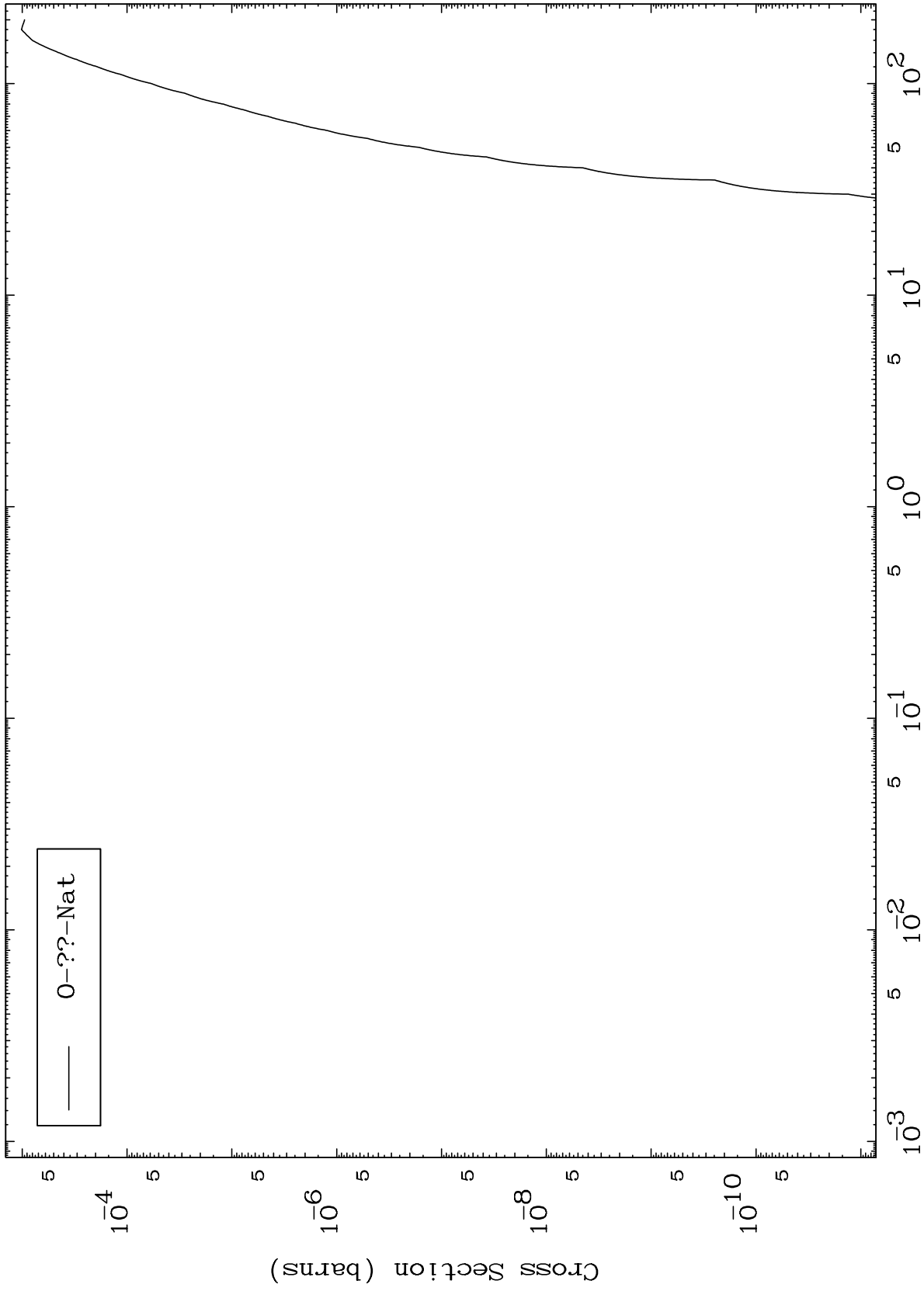
Incident Energy (MeV)

76-0s-199

MAT 7670

$\alpha$  Fission  
Radionuclide Production Cross Section

76-0s-199



11

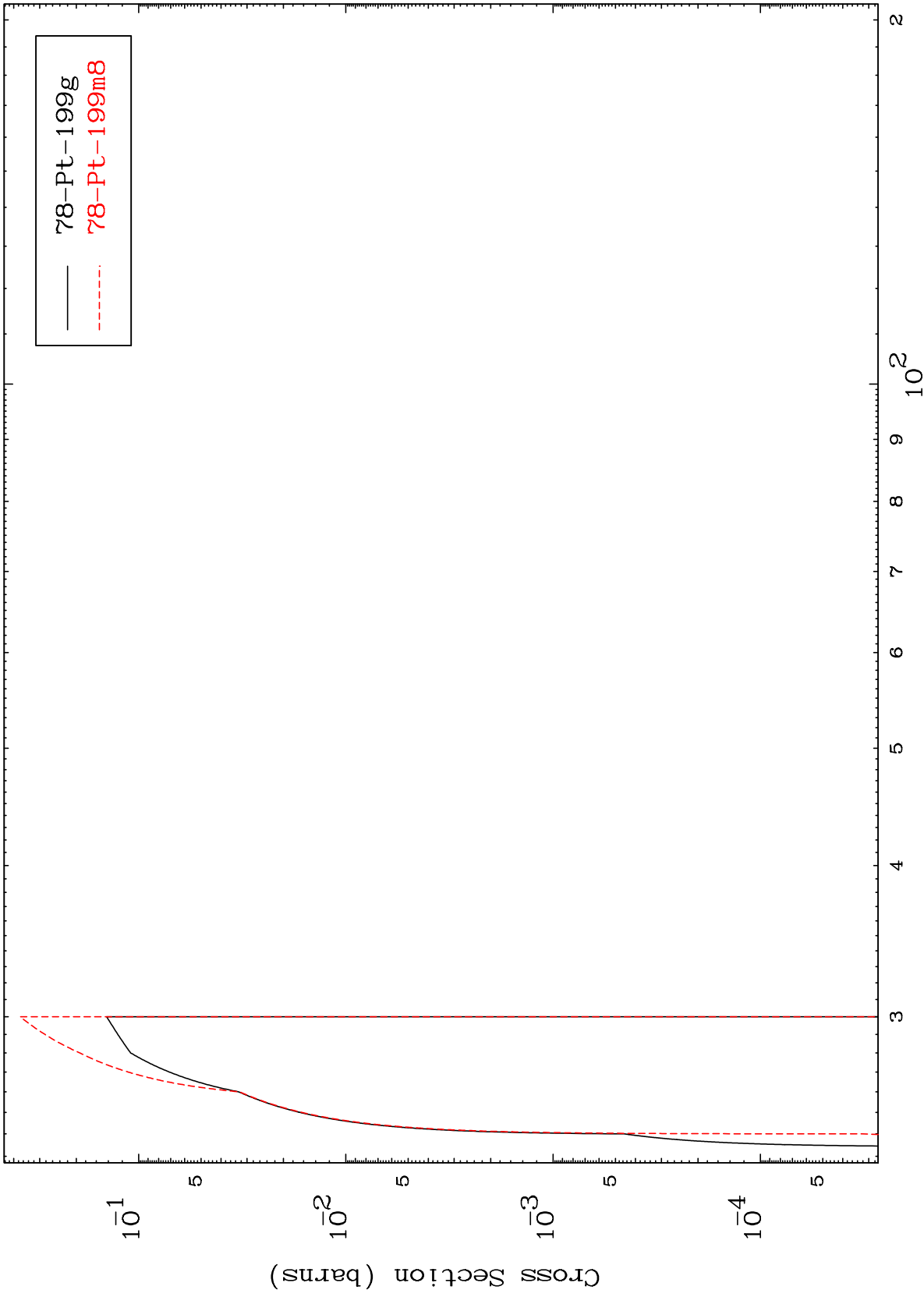
Incident Energy (MeV)

76-0s-199

MAT 7670

76-0s-199

( $\alpha, 4n$ )  
Radionuclide Production Cross Section



12

Incident Energy (MeV)

76-0s-199