

Program EVALPLOT  
(Version 2018-1)

by

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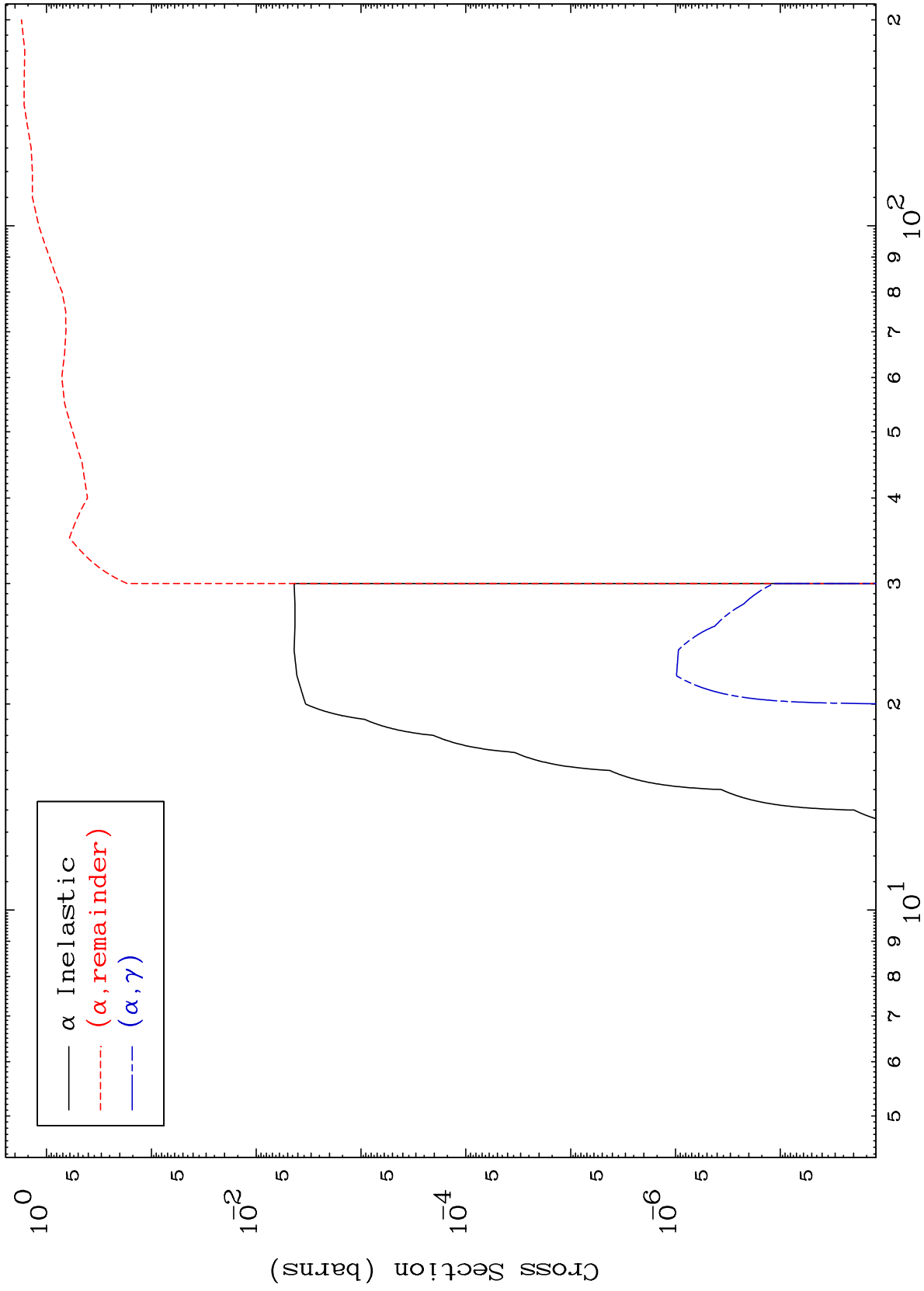
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9141

0 Kelvin  $\alpha$  Major  
Cross Sections

91-Pa-234



1

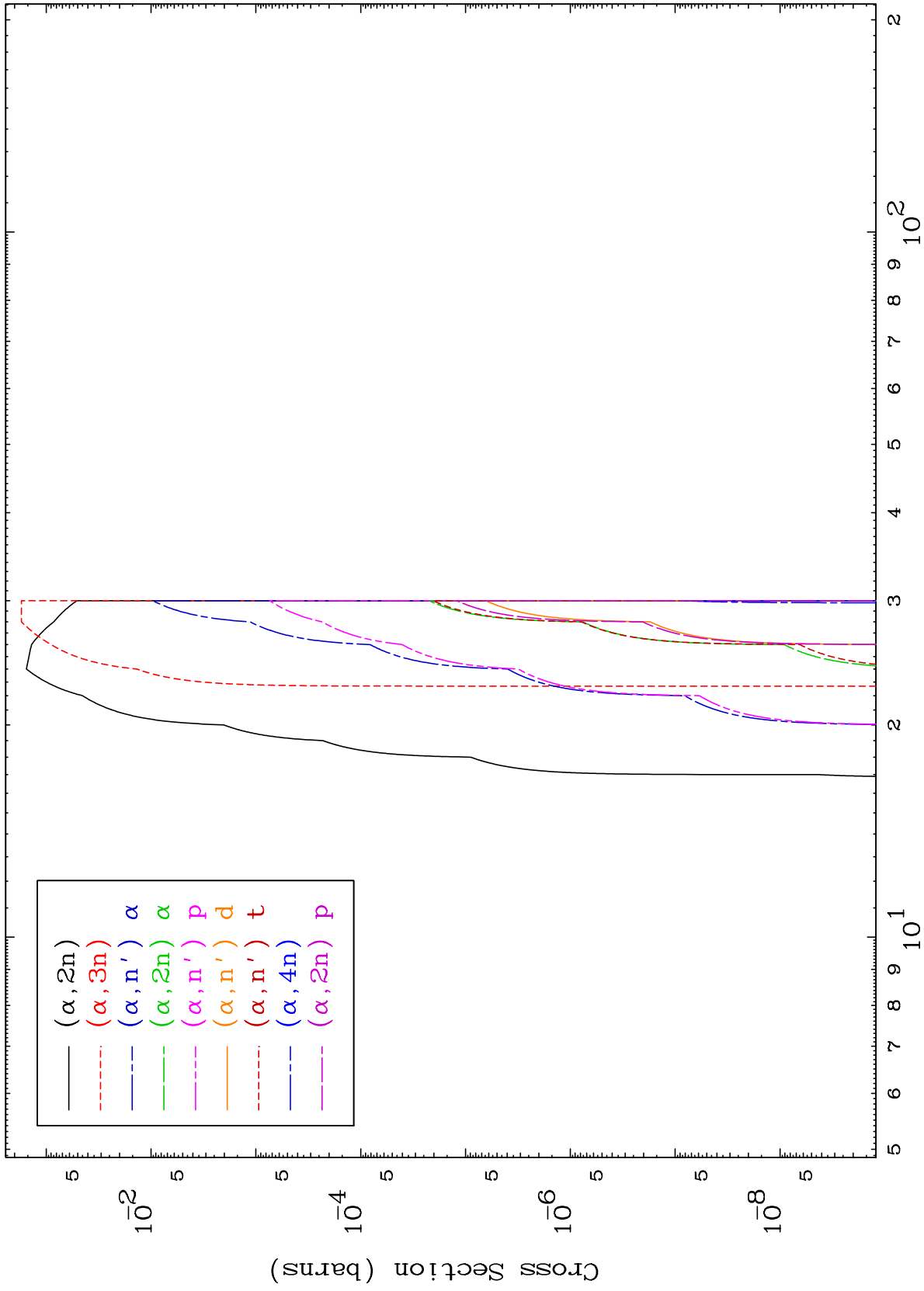
Incident Energy (MeV)

91-Pa-234

MAT 9141

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

91-Pa-234



2

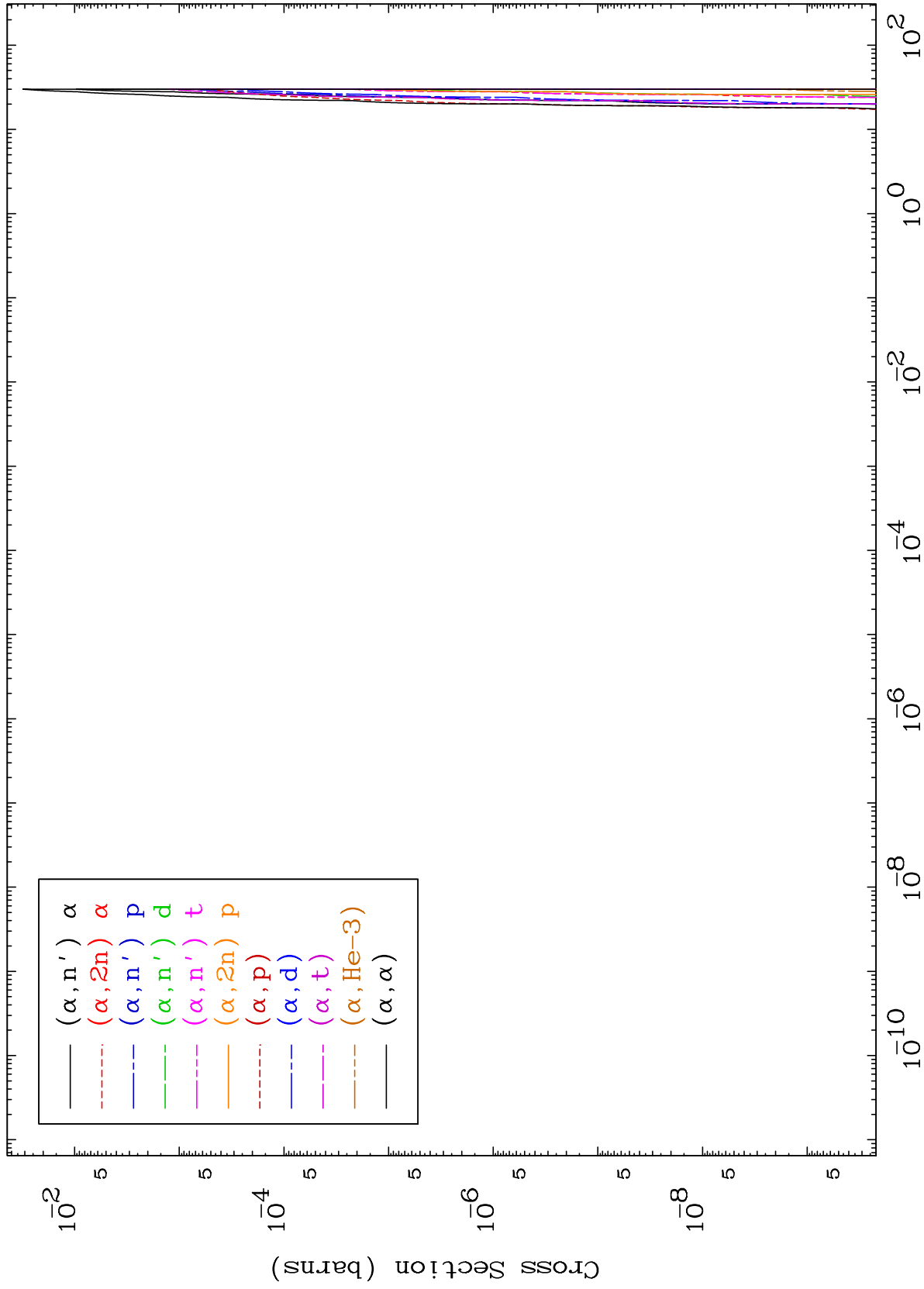
Incident Energy (MeV)

91-Pa-234

MAT 9141

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

91-Pa-234



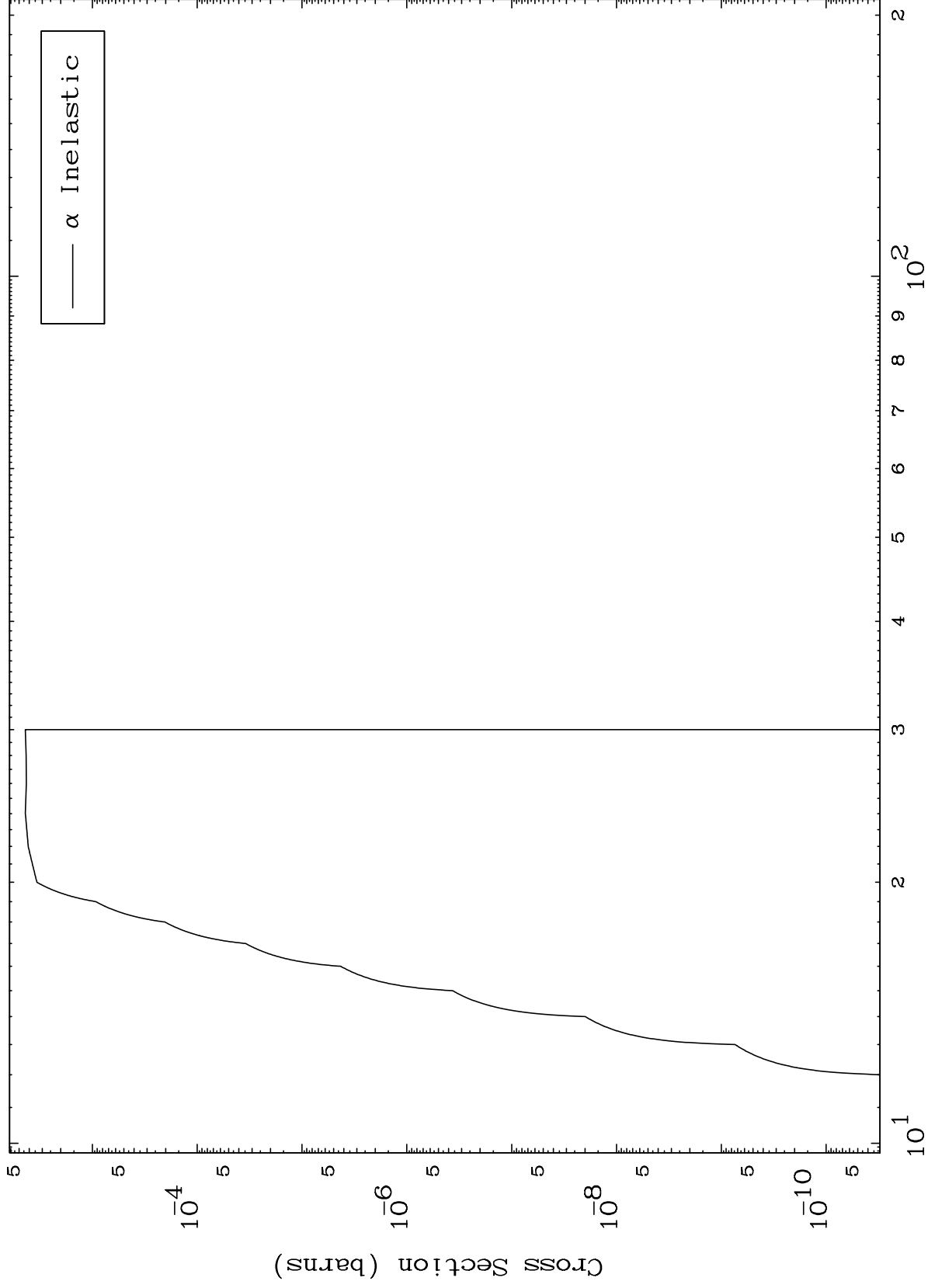
3

91-Pa-234

MAT 9141

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

91-Pa-234



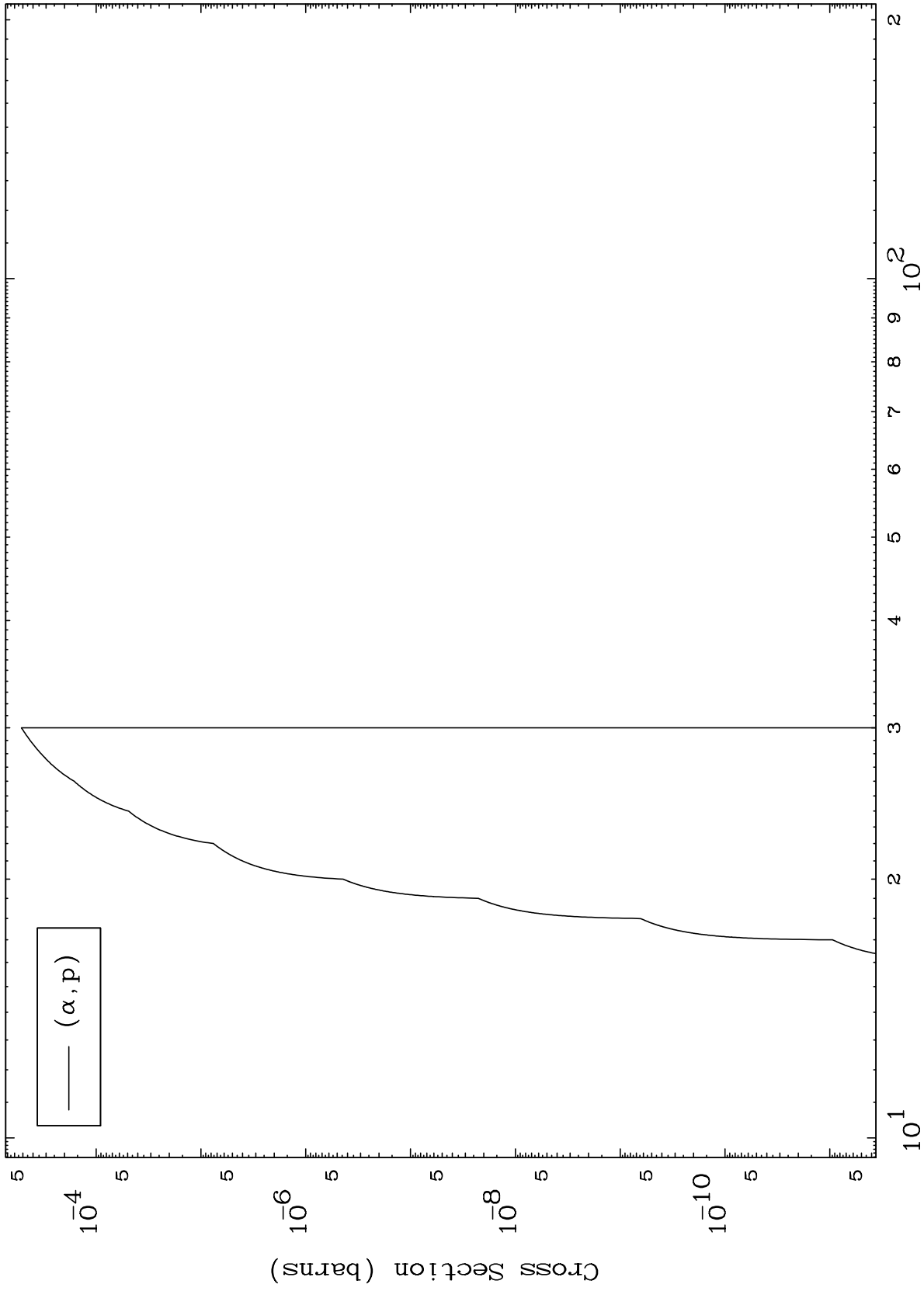
Incident Energy (MeV)

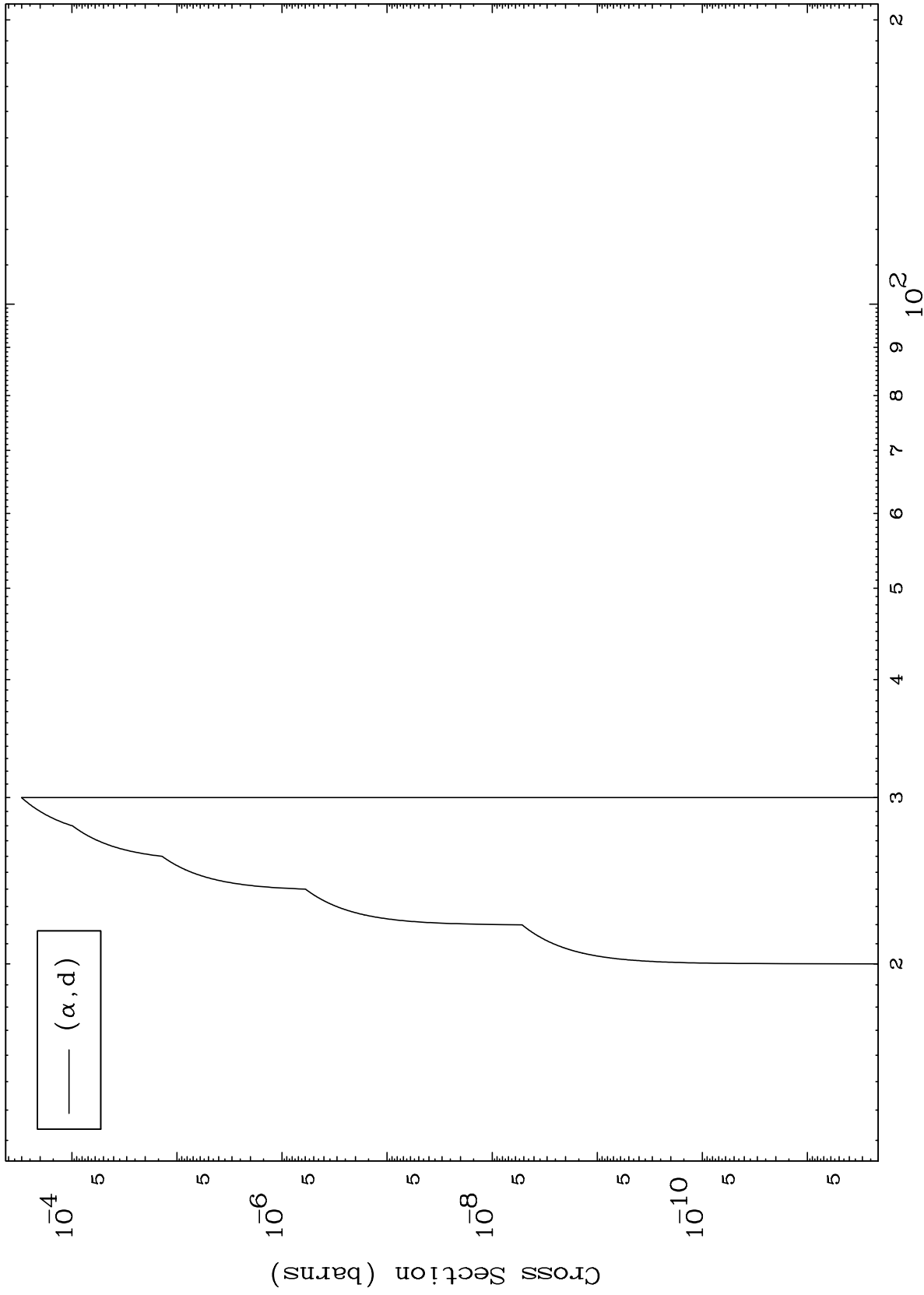
91-Pa-234

MAT 9141

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

91-Pa-234

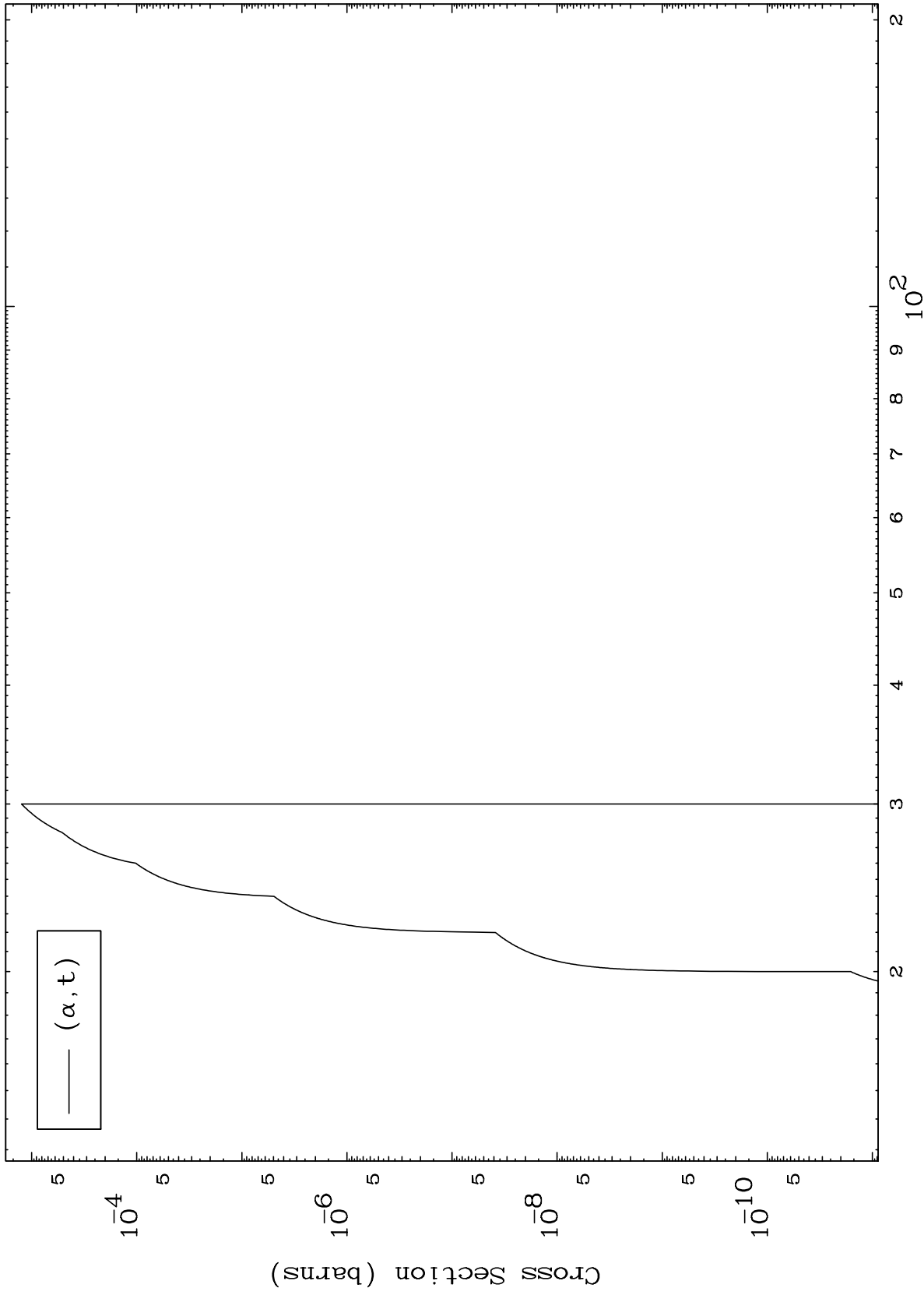




MAT 9141

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

91-Pa-234

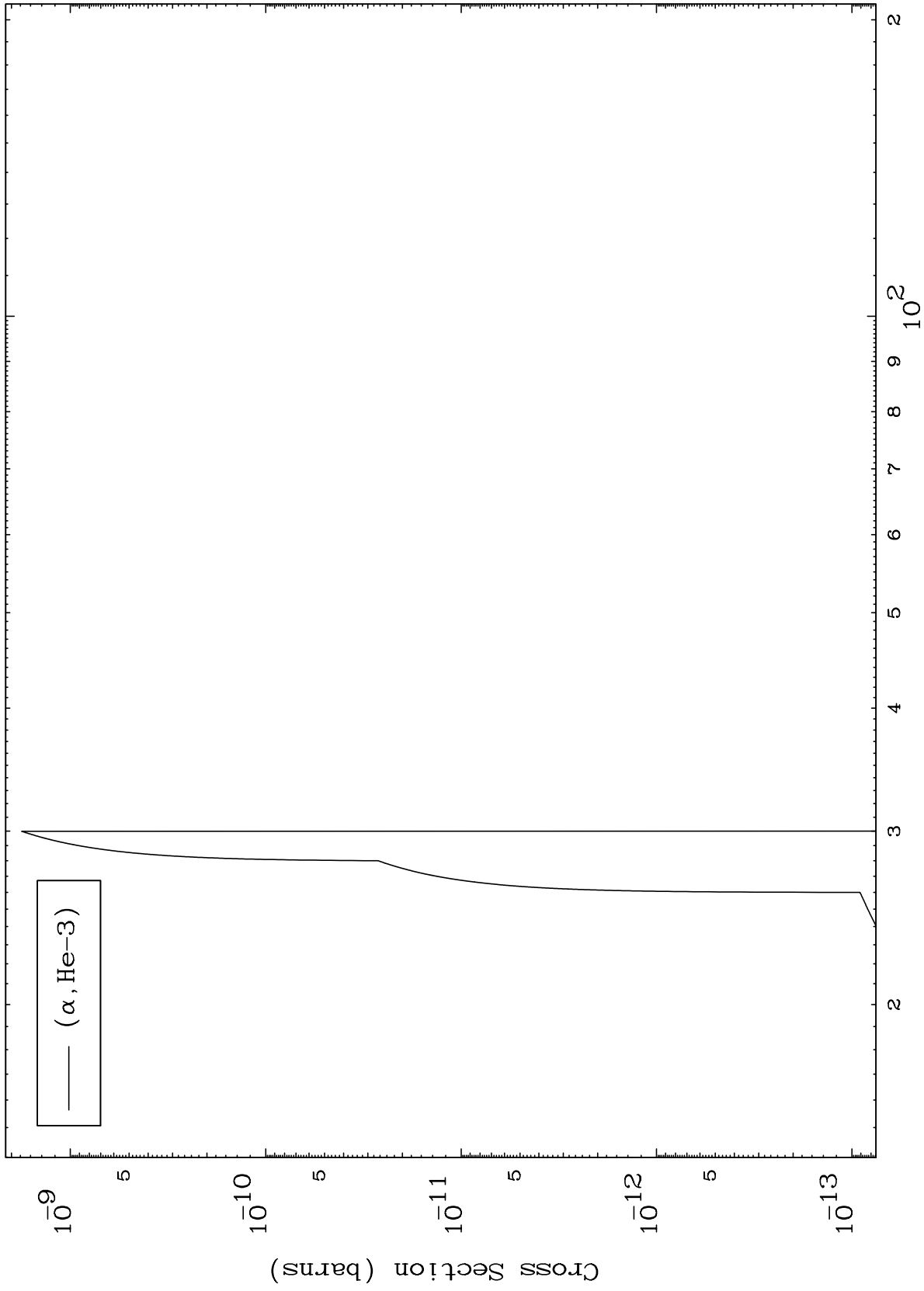


7

Incident Energy (MeV)

91-Pa-234

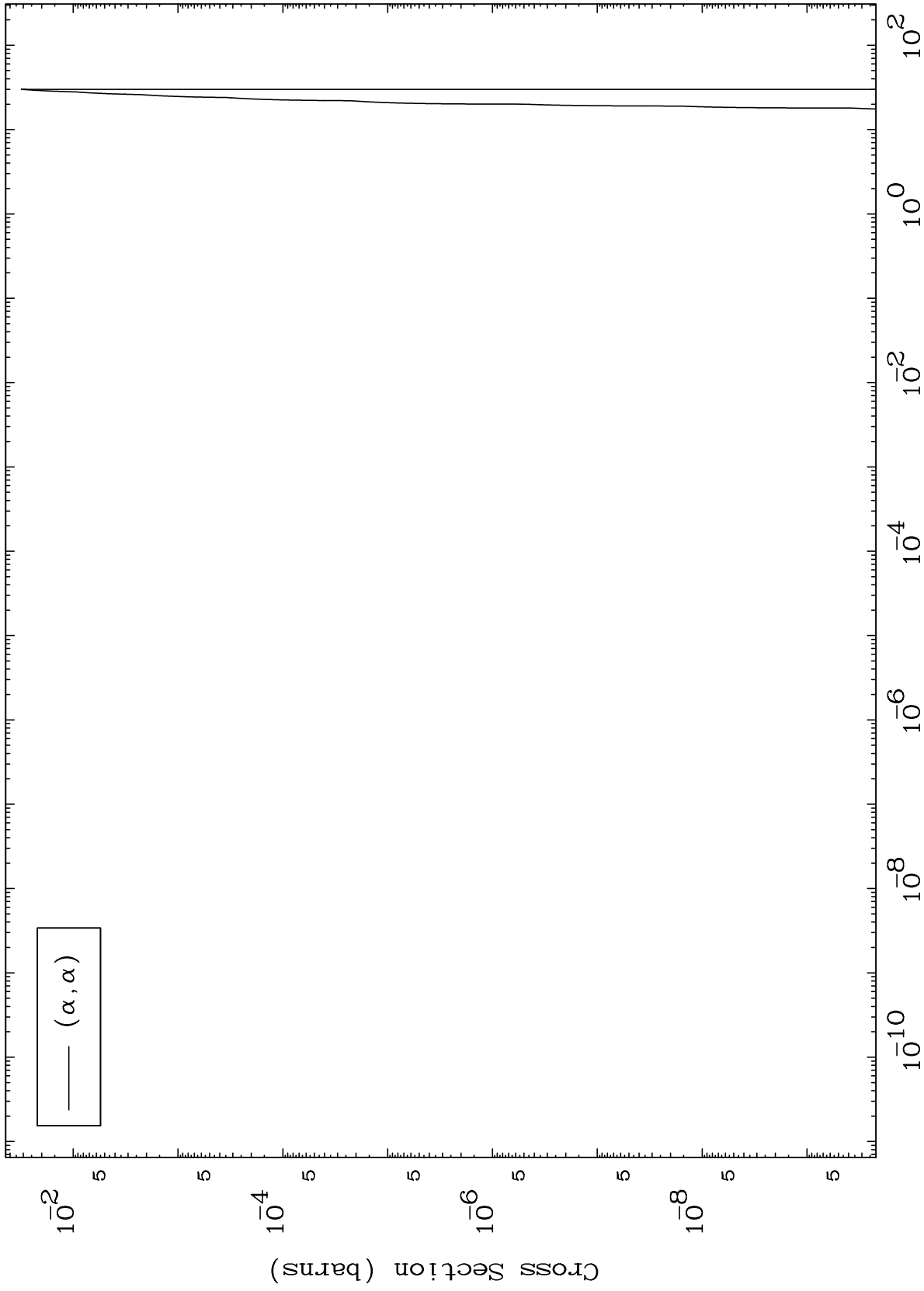




MAT 9141

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

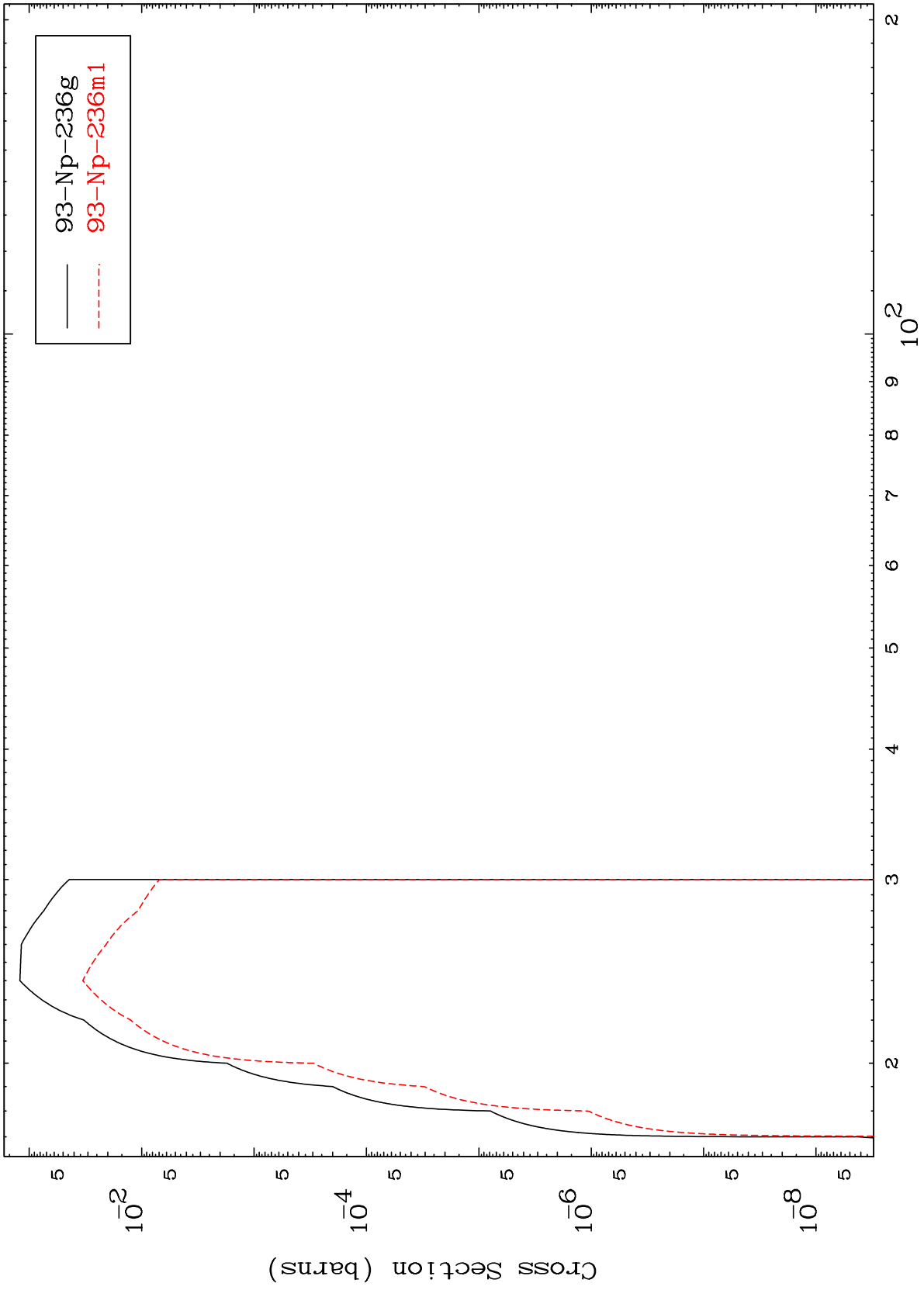
91-Pa-234



MAT 9141

91-Pa-234

( $\alpha, 2n$ )  
Radionuclide Production Cross Section



10

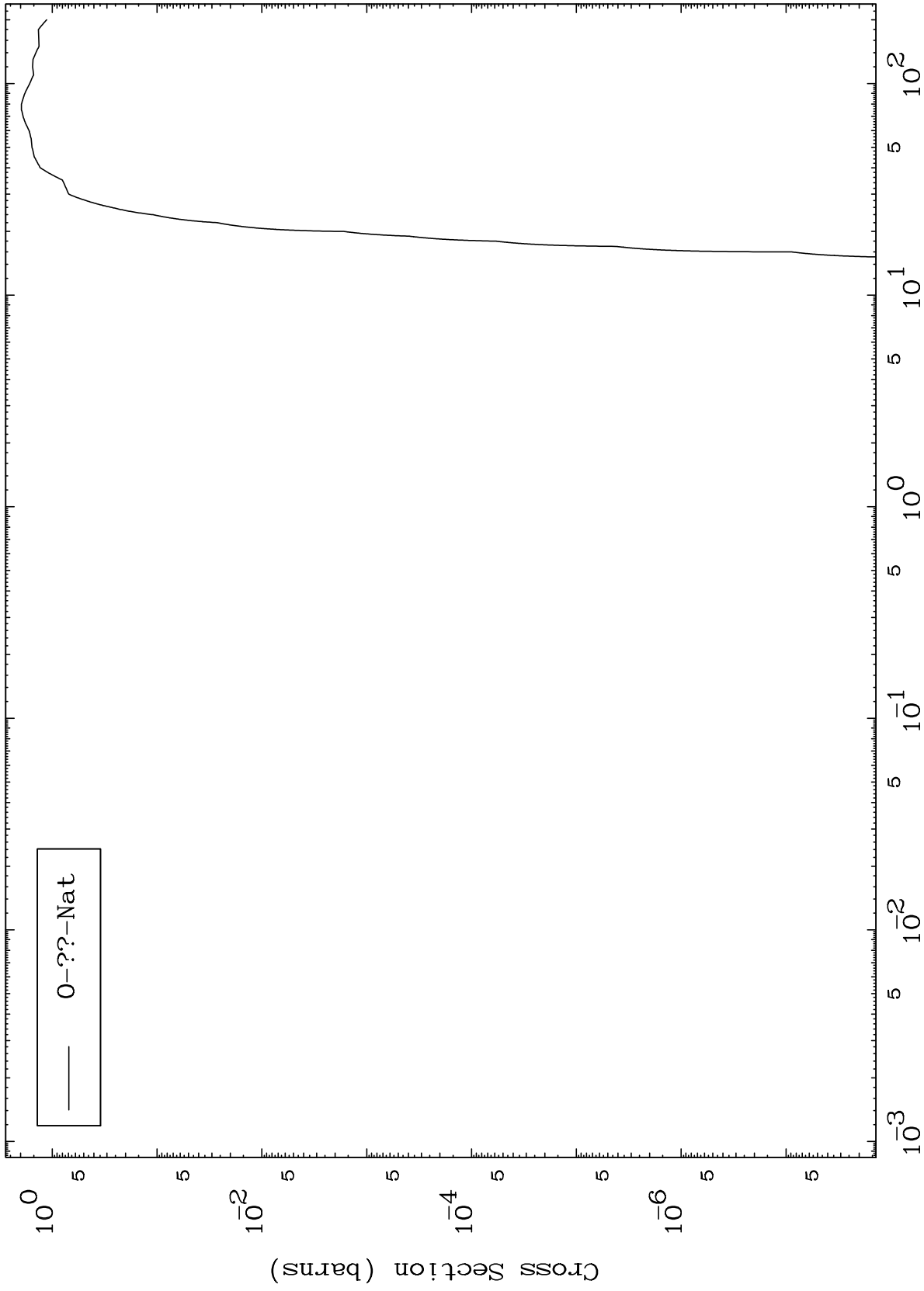
Incident Energy (MeV)

91-Pa-234

MAT 9141

$\alpha$  Fission  
Radionuclide Production Cross Section

91-Pa-234



11

Incident Energy (MeV)

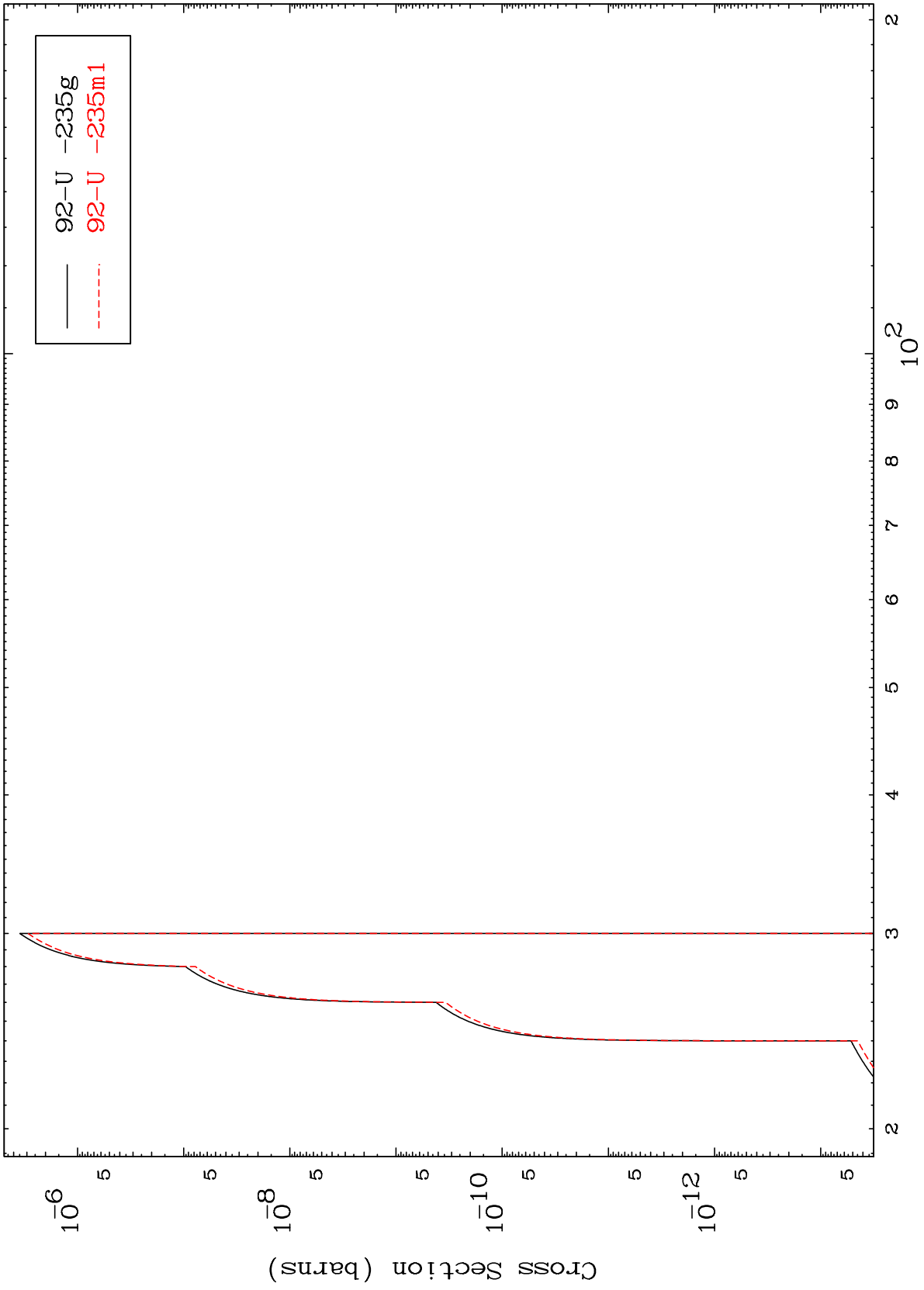
91-Pa-234

MAT 9141

( $\alpha, n'$ ) d

91-Pa-234

Radionuclide Production Cross Section



12

Incident Energy (MeV)

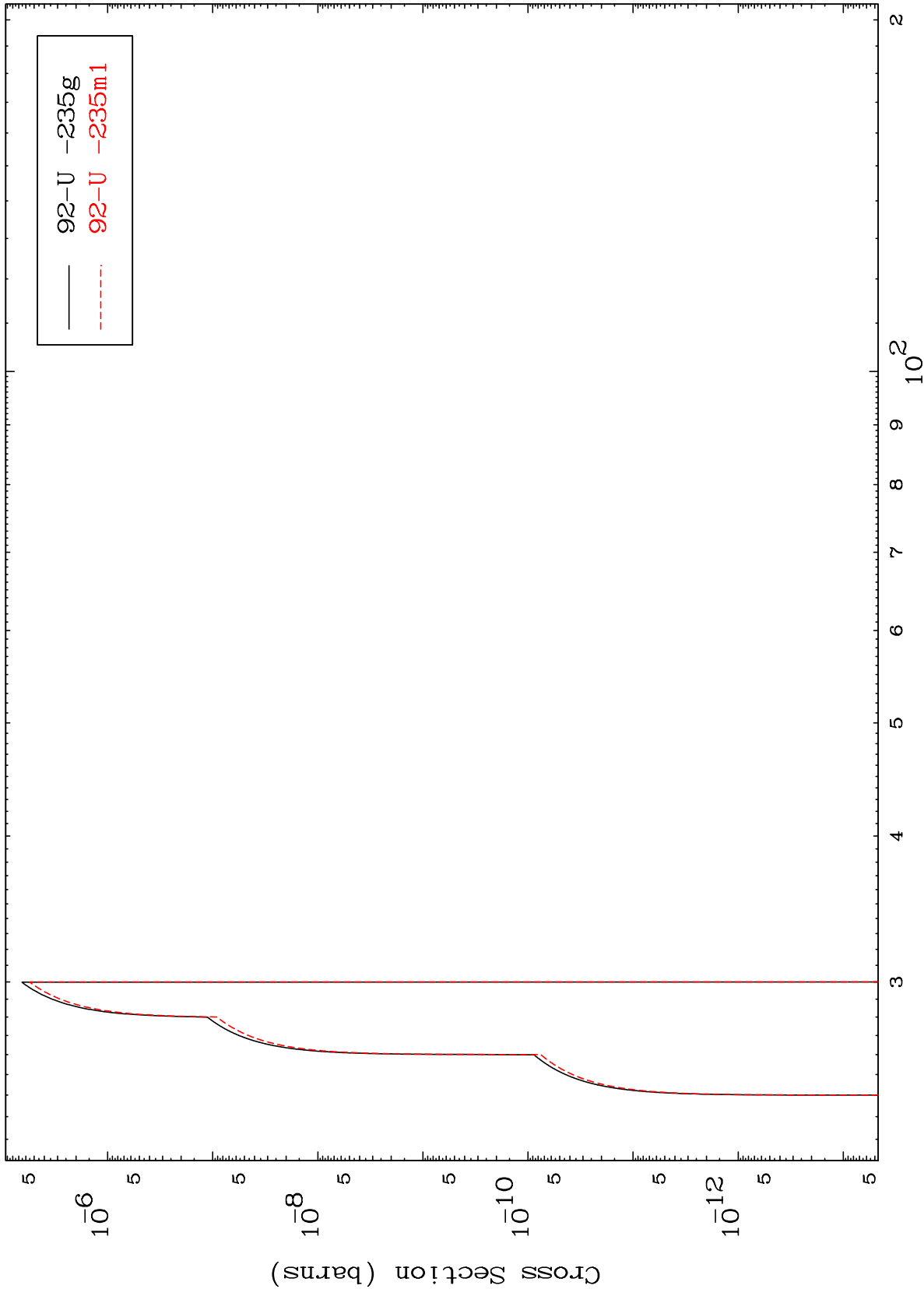
91-Pa-234

MAT 9141

( $\alpha, 2n$ ) p

91-Pa-234

Radionuclide Production Cross Section



13

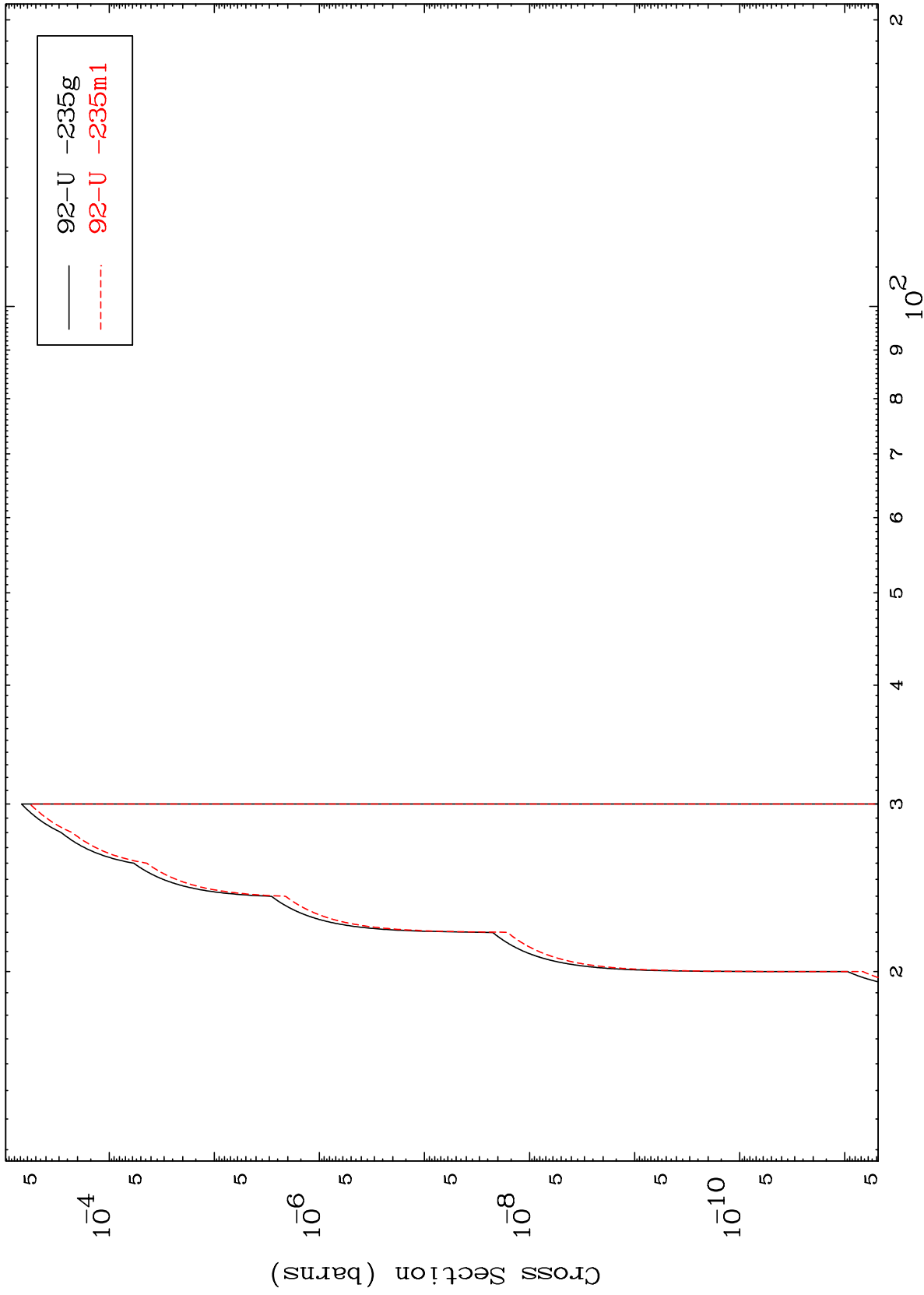
Incident Energy (MeV)

91-Pa-234

MAT 9141

91-Pa-234

( $\alpha, t$ )  
Radionuclide Production Cross Section



14

91-Pa-234

MAT 9141

( $\alpha, \alpha$ )

91-Pa-234

Radionuclide Production Cross Section

