

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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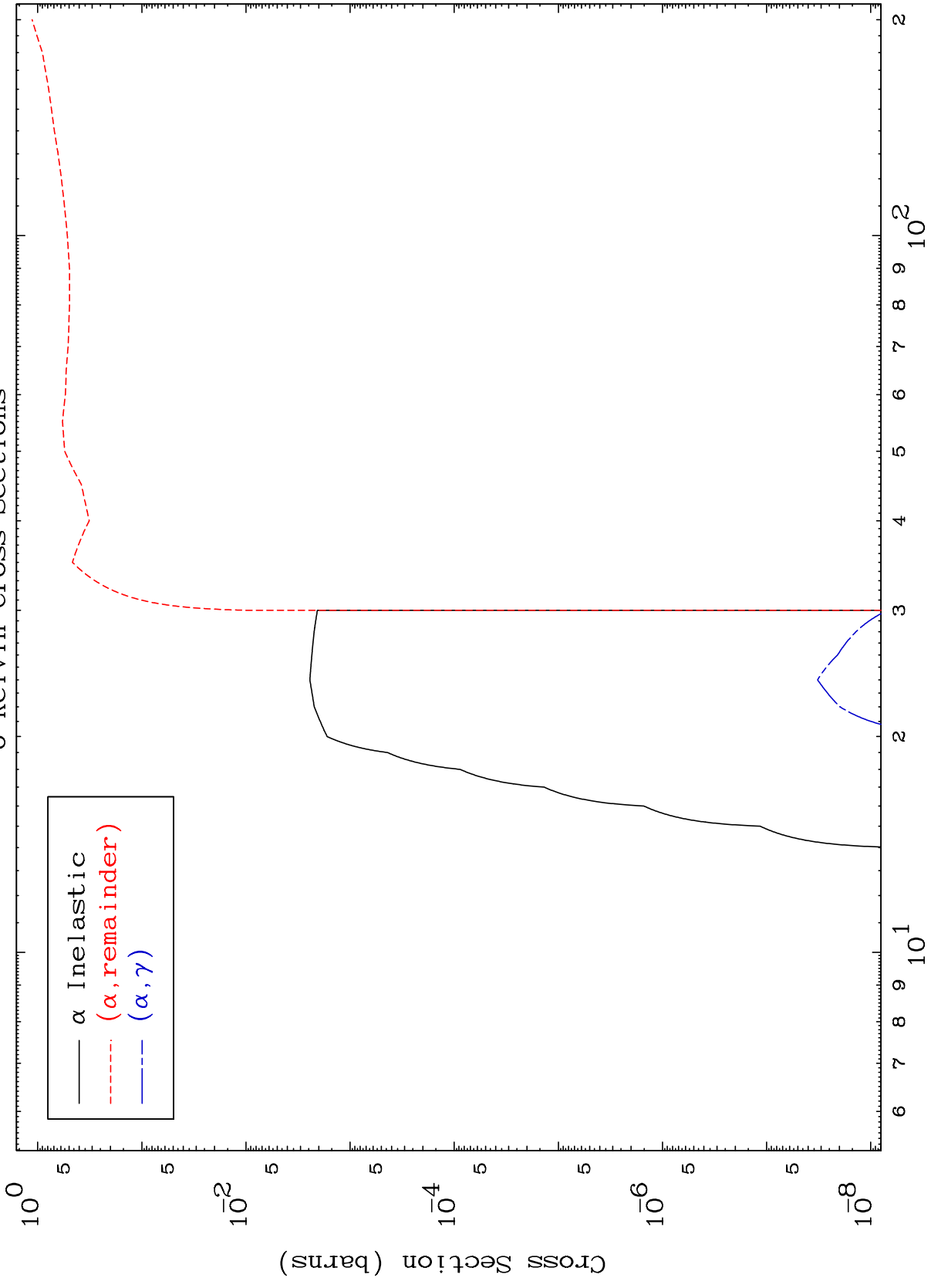
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9446

0 Kelvin  $\alpha$  Major  
Cross Sections

94-Pu-242

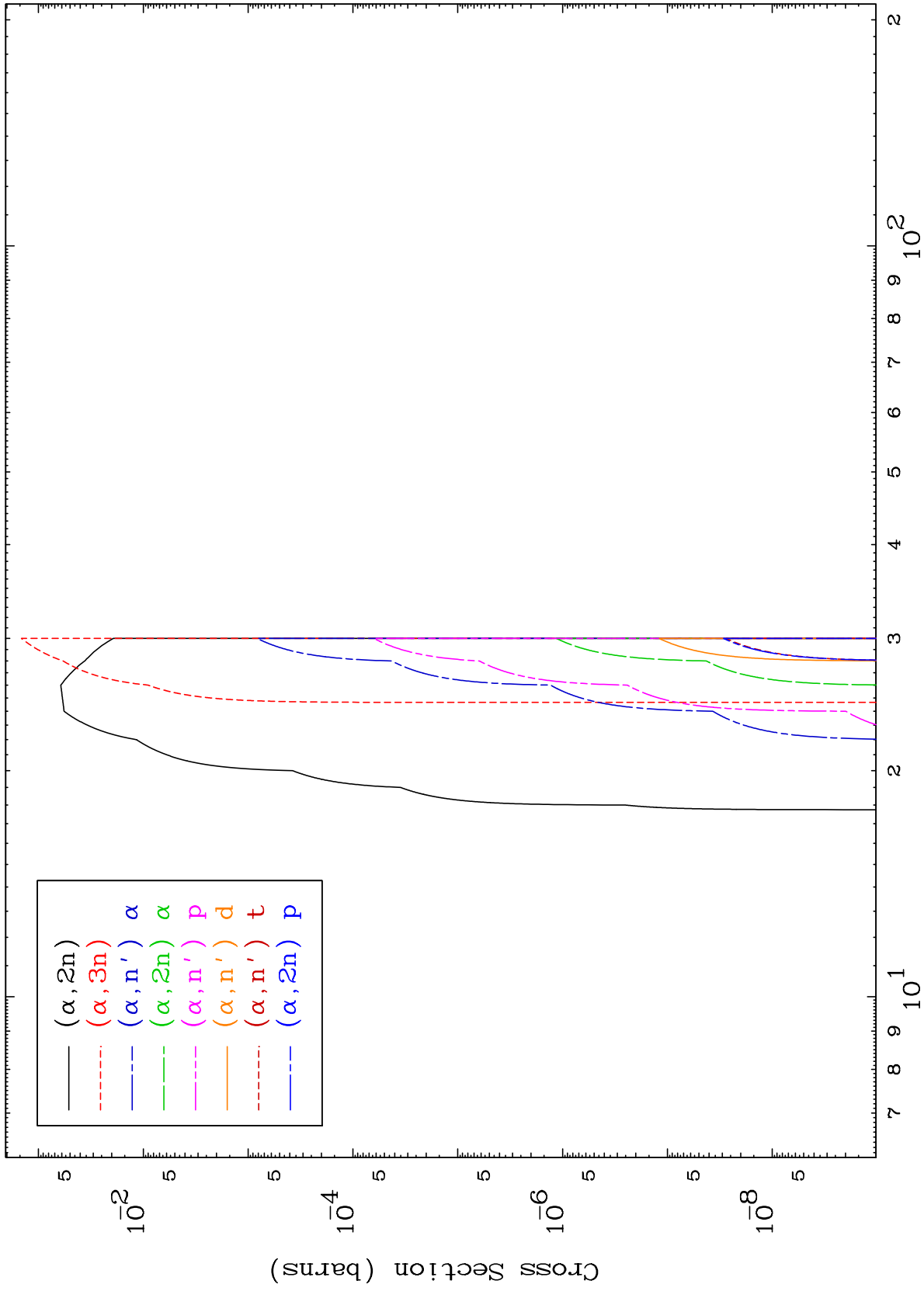


$\alpha$  Inelastic  
( $\alpha$ , remainder)  
( $\alpha$ ,  $\gamma$ )

MAT 9446

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

94-Pu-242



2

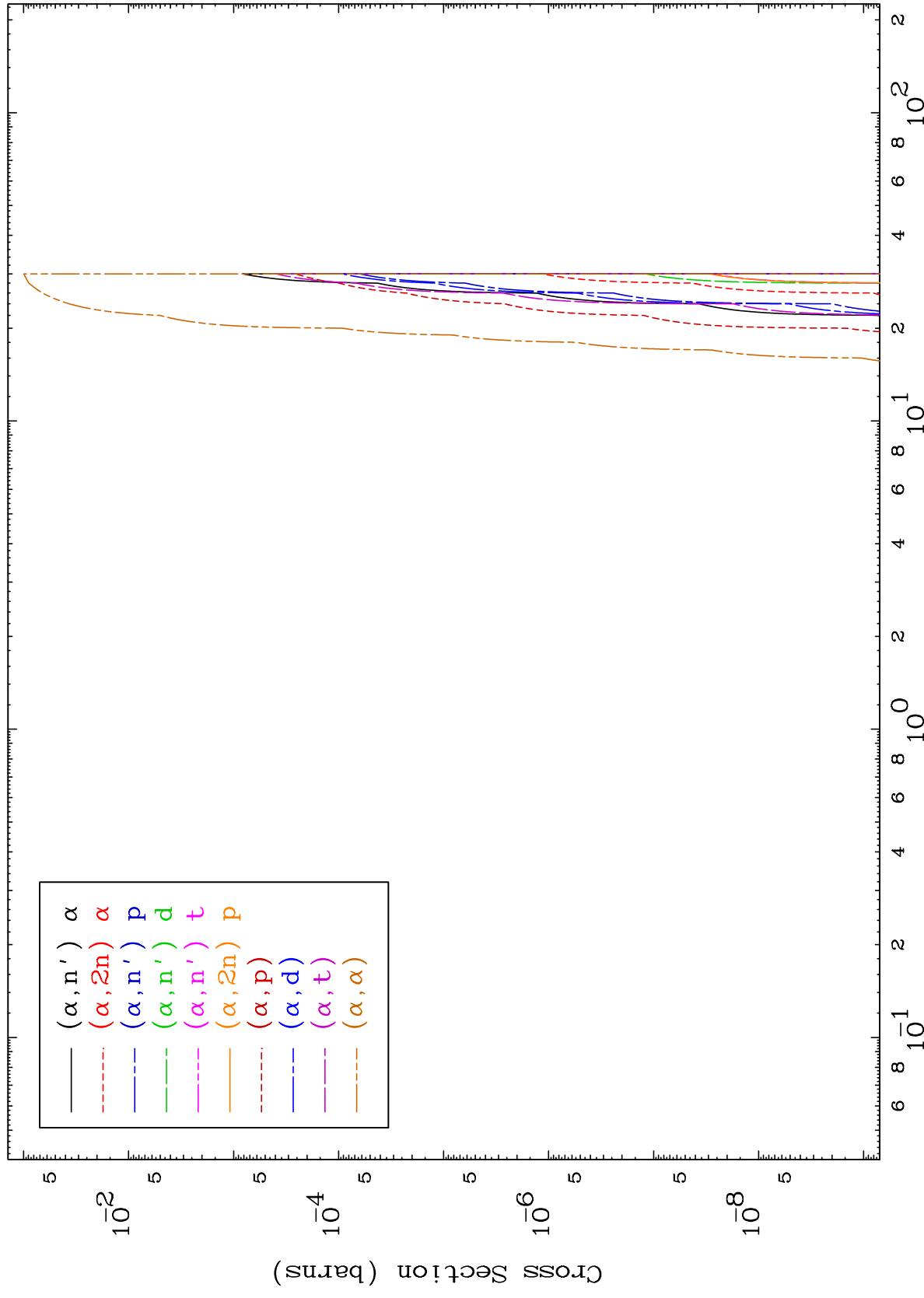
Incident Energy (MeV)

94-Pu-242

MAT 9446

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

94-Pu-242

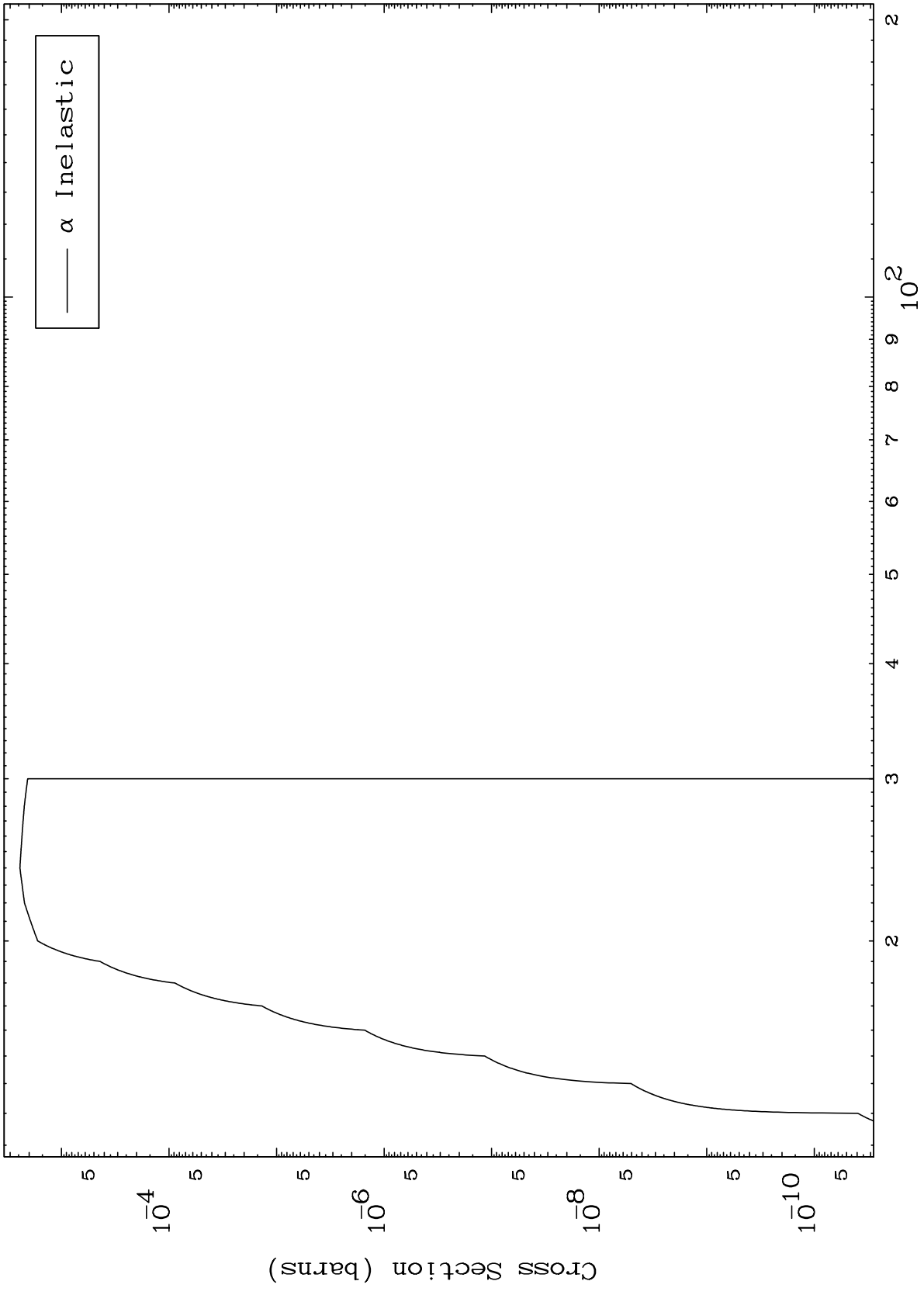


94-Pu-242

MAT 9446

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

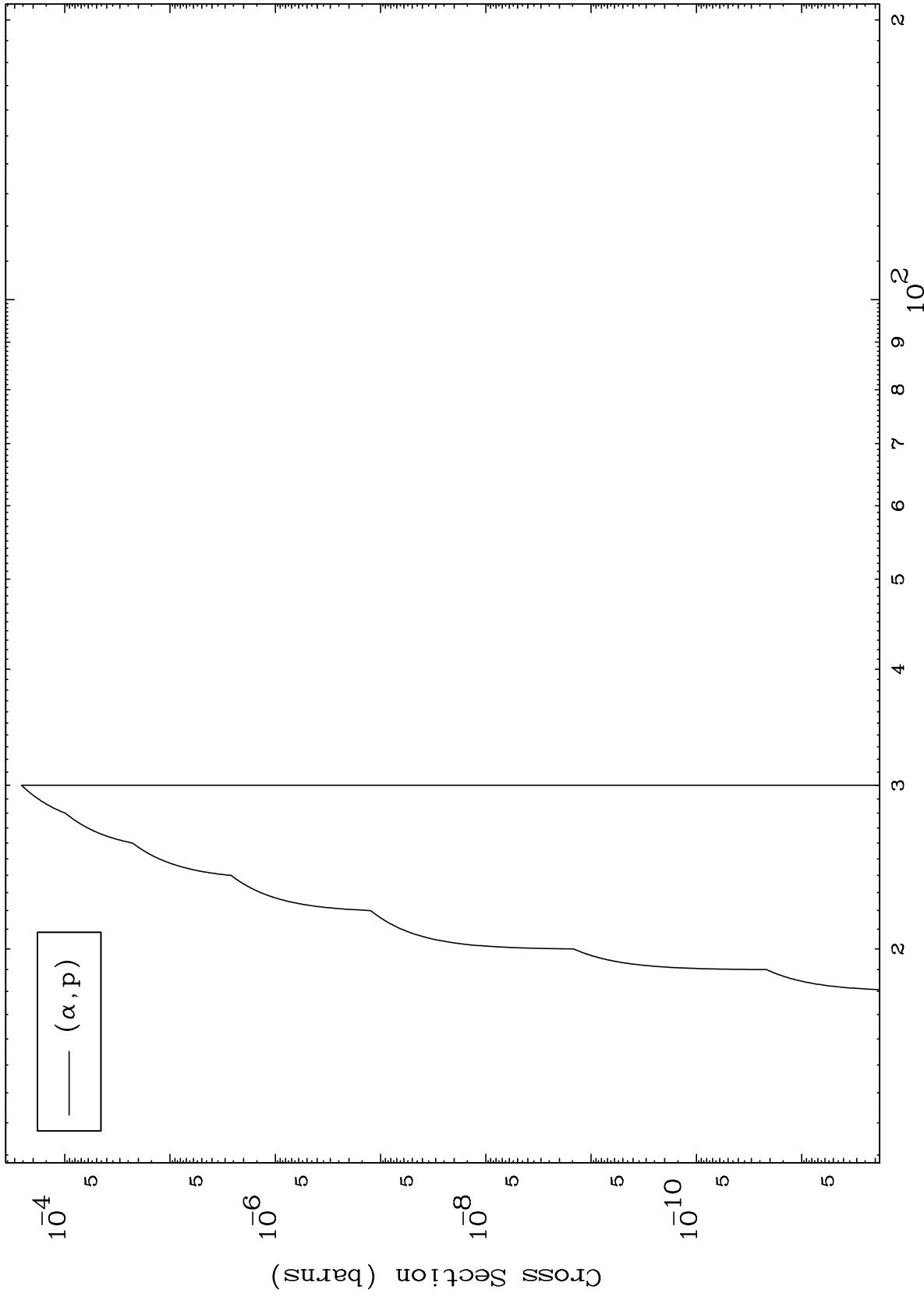
94-Pu-242



4

Incident Energy (MeV)

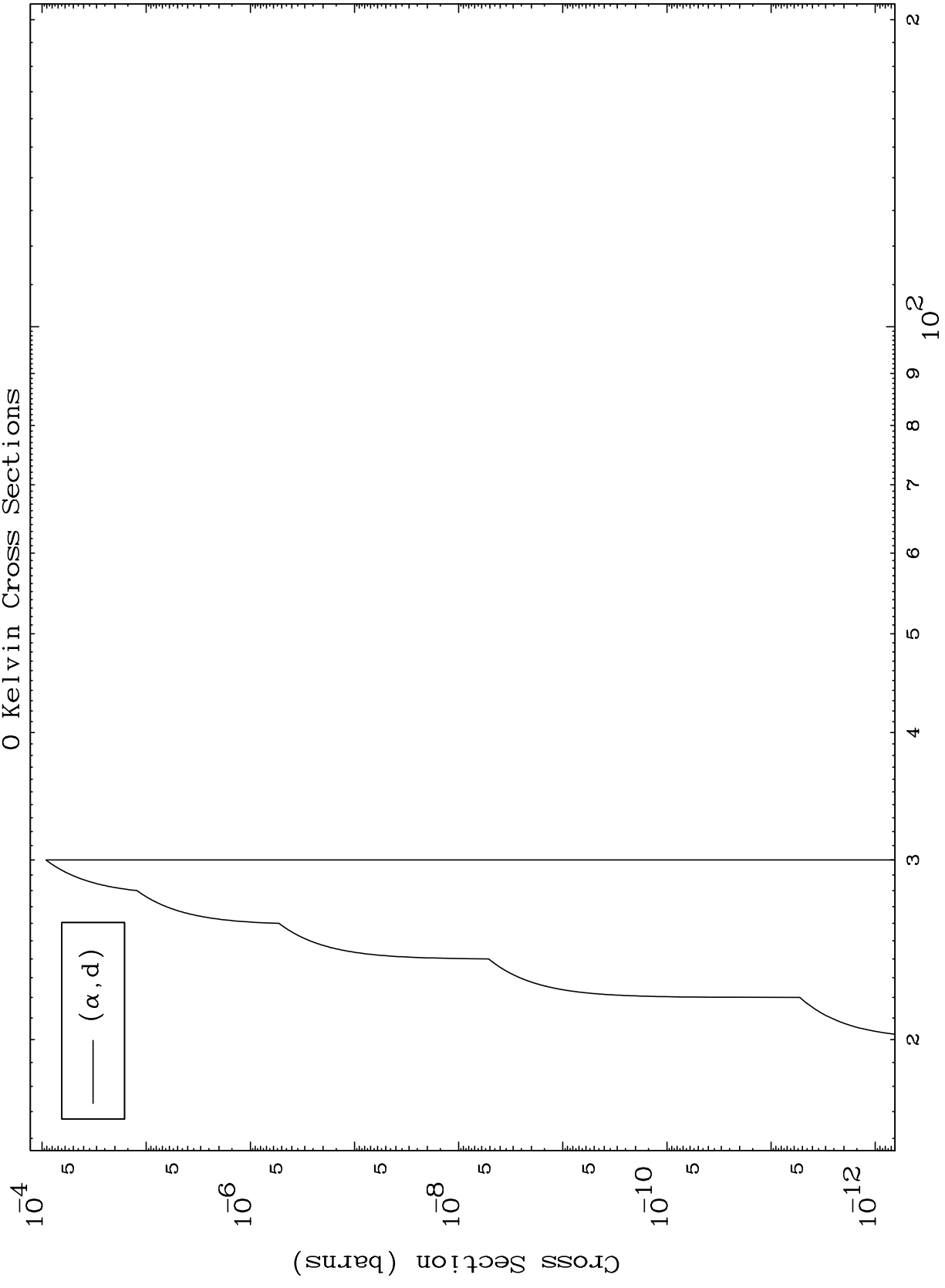
94-Pu-242



MAT 9446

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

94-Pu-242



6

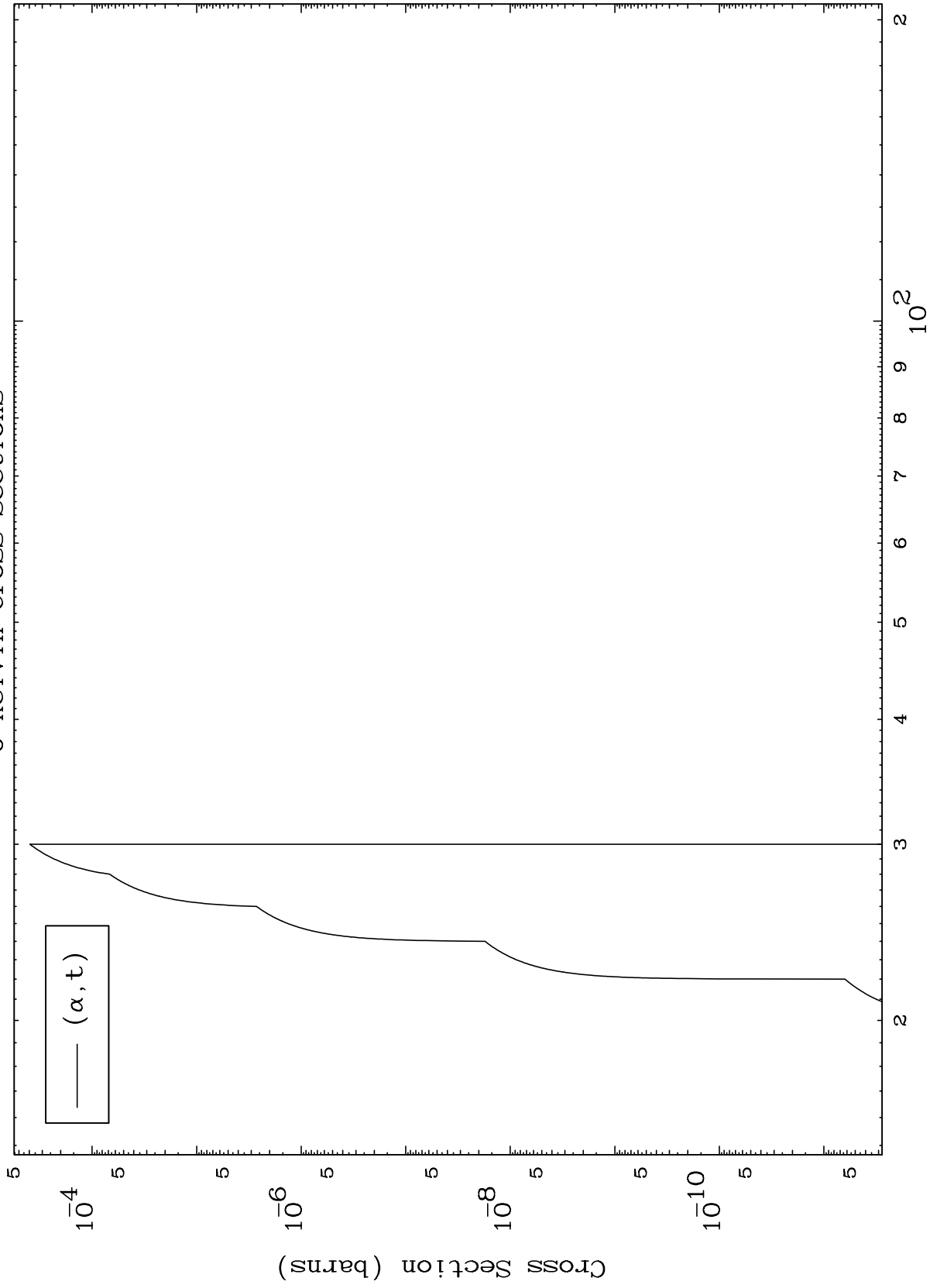
Incident Energy (MeV)

94-Pu-242

MAT 9446

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

94-Pu-242



7

Incident Energy (MeV)

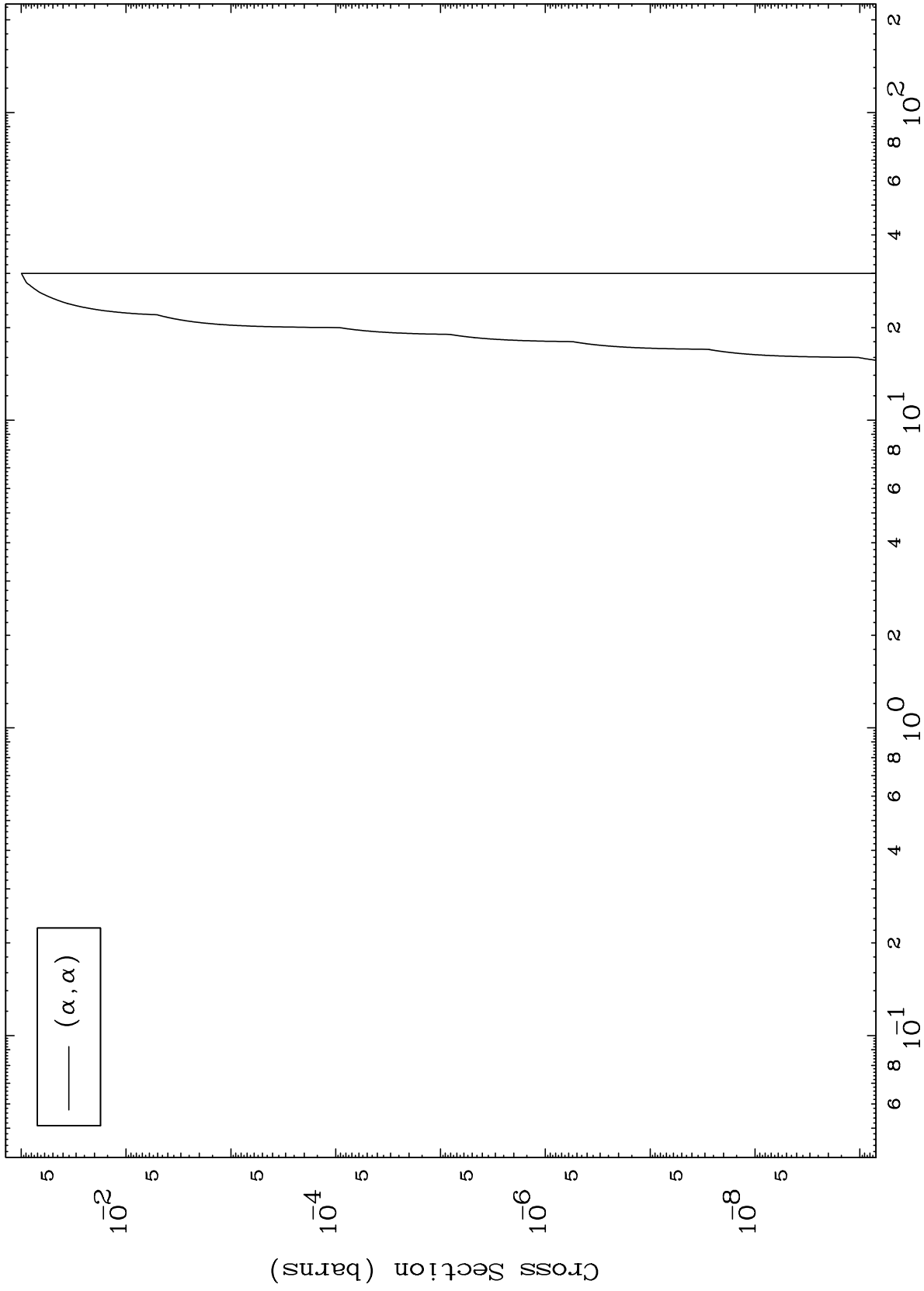
94-Pu-242



MAT 9446

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

94-Pu-242



8

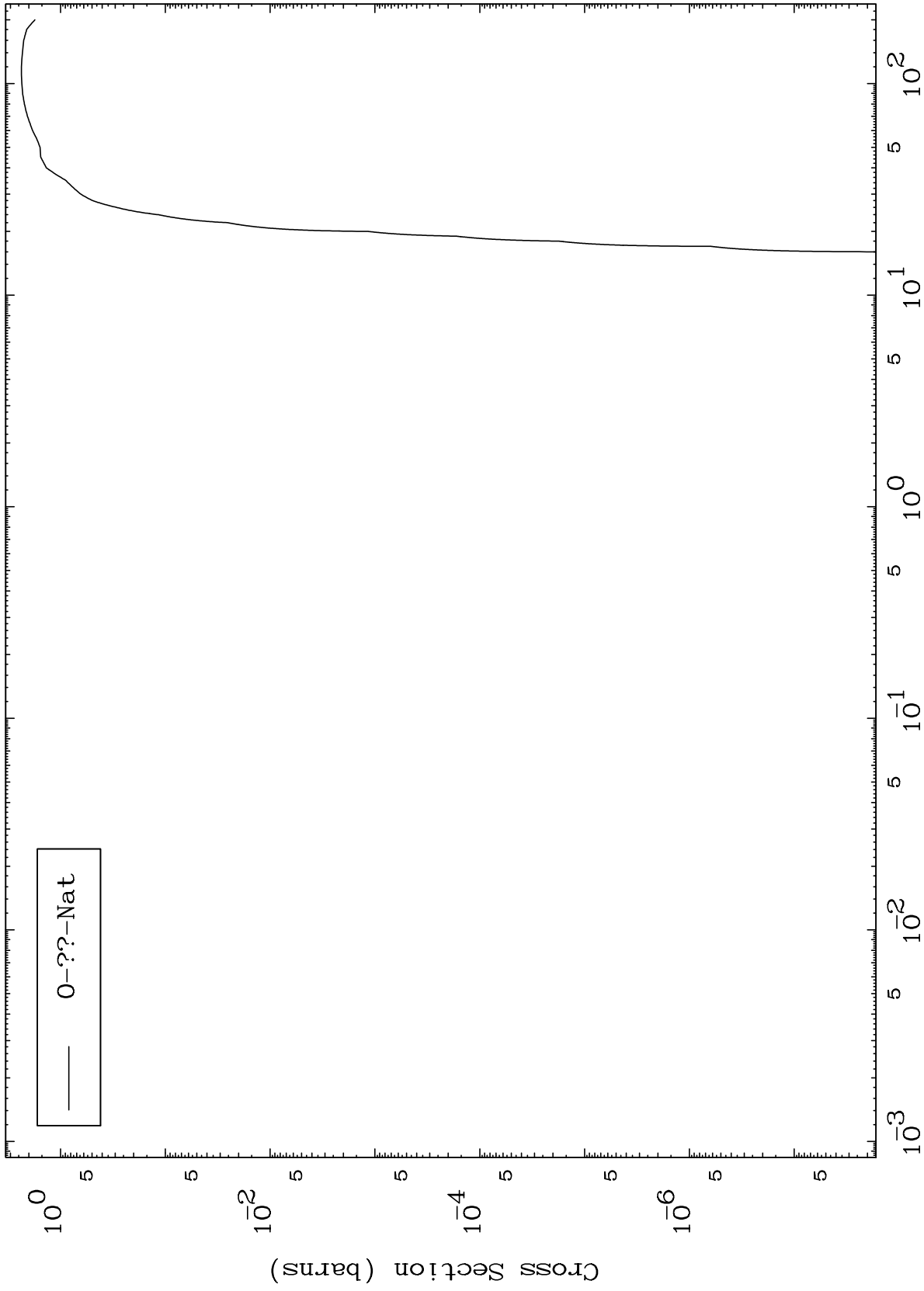
Incident Energy (MeV)

94-Pu-242

MAT 9446

Radionuclide Production Cross Section  
 $\alpha$  Fission

94-Pu-242



Incident Energy (MeV)

94-Pu-242

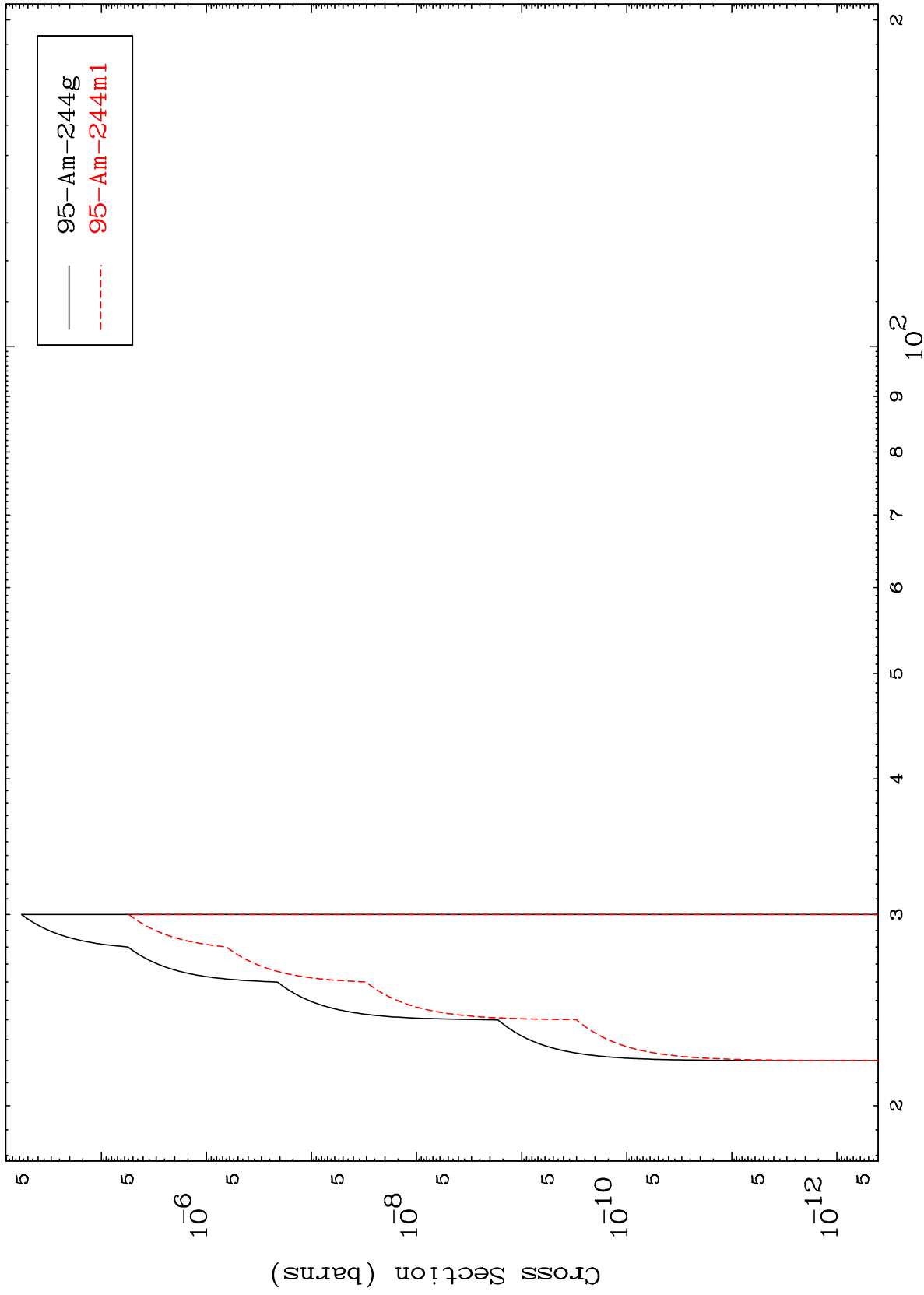
9

MAT 9446

( $\alpha, n'$ ) p

94-Pu-242

Radionuclide Production Cross Section



10

Incident Energy (MeV)

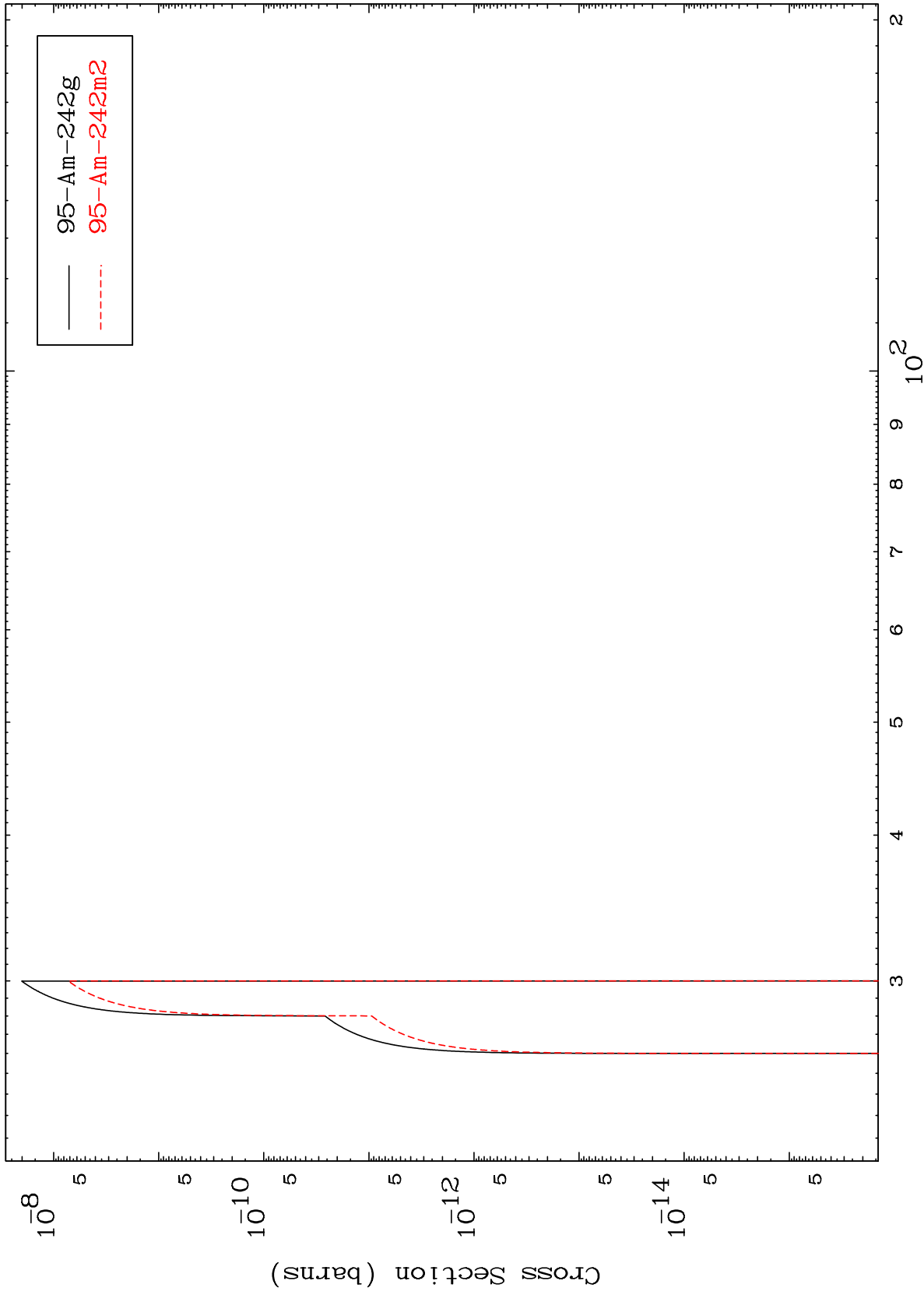
94-Pu-242

MAT 9446

( $\alpha, n'$ ) t

94-Pu-242

Radionuclide Production Cross Section



11

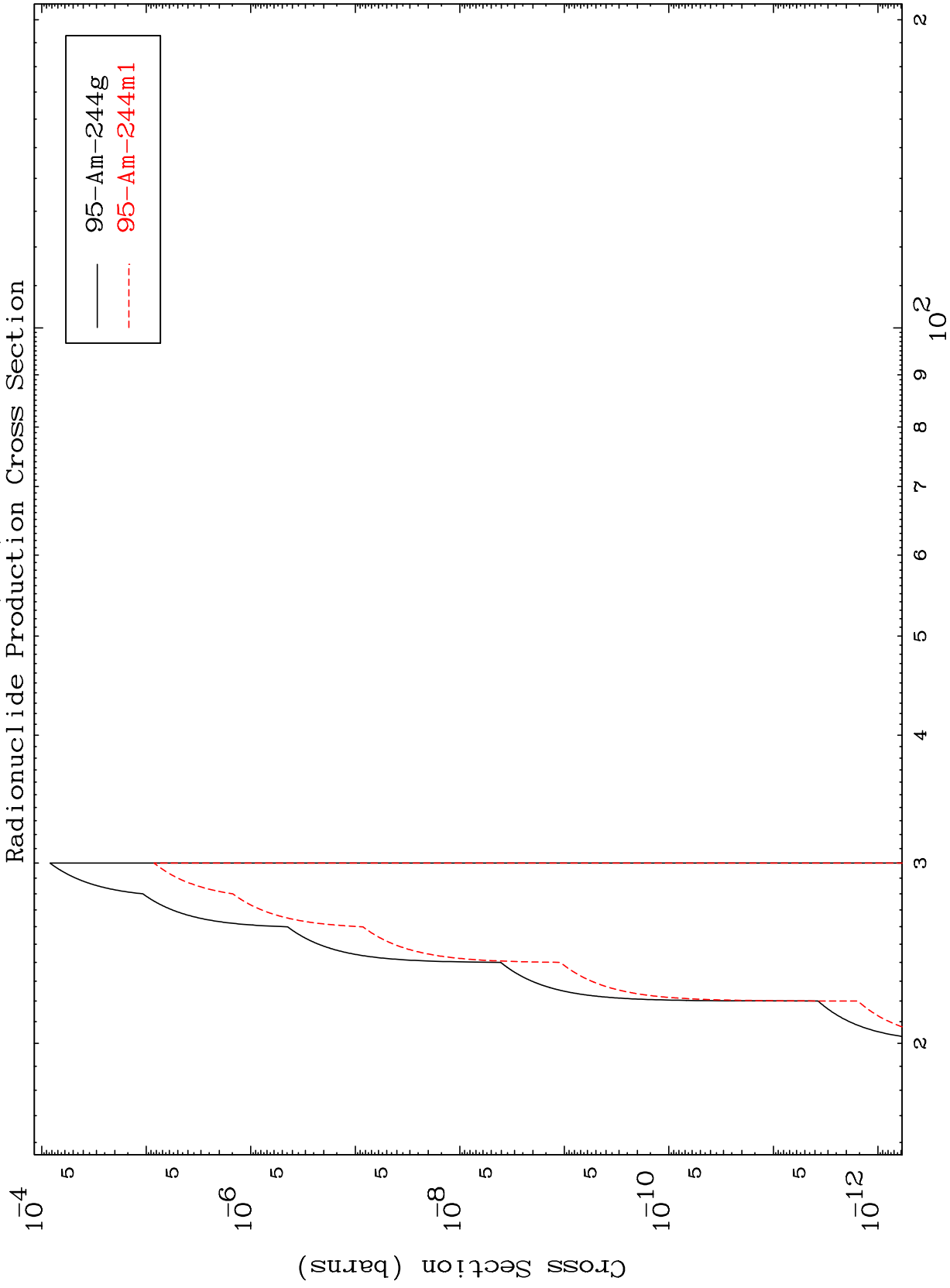
Incident Energy (MeV)

94-Pu-242

MAT 9446

( $\alpha, d$ )

94-Pu-242



12

Incident Energy (MeV)

94-Pu-242