

Program EVALPLOT
(Version 2018-1)

by

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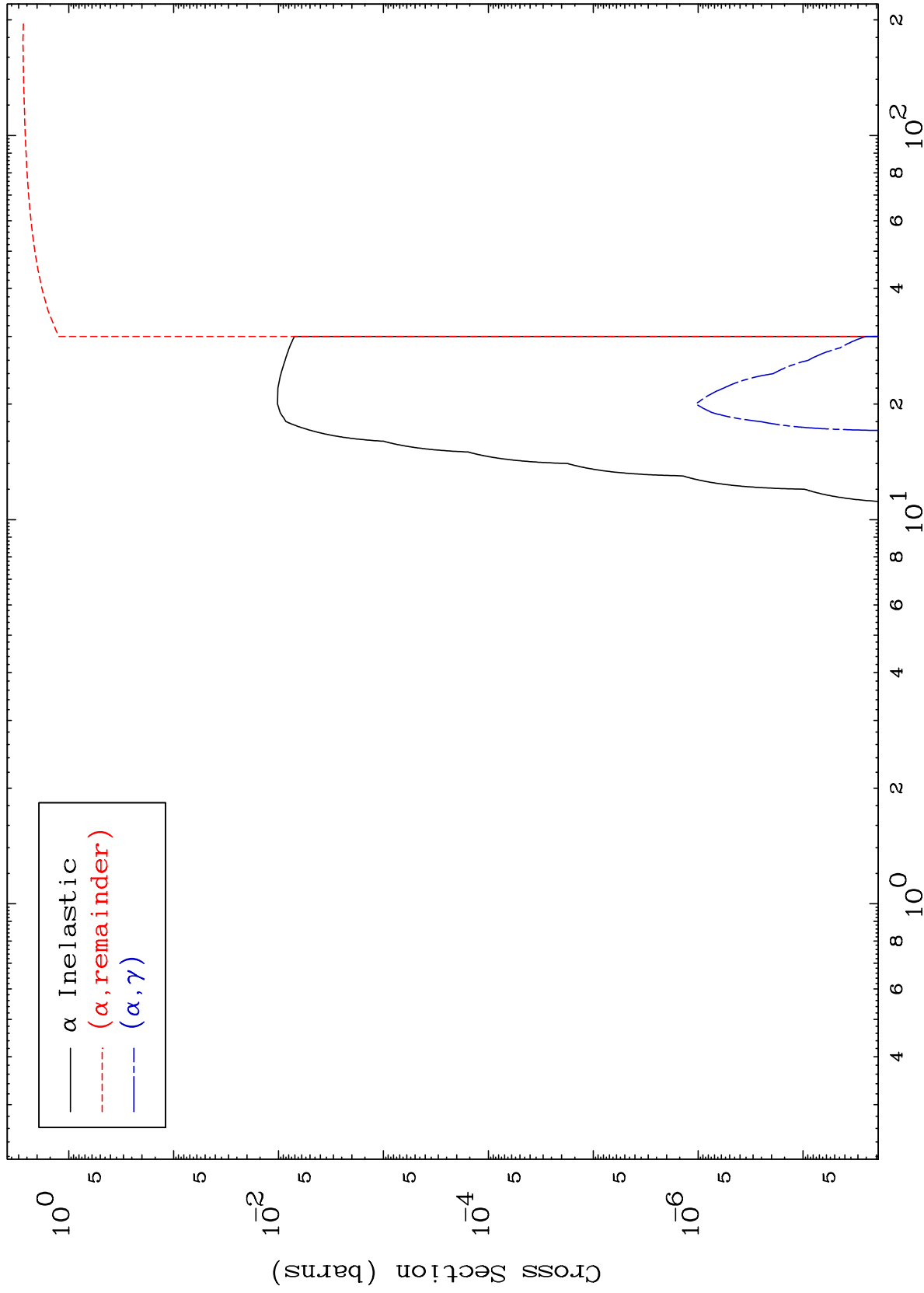
Press Mouse Button to Start

MAT 7543

α Major

75-Re-191

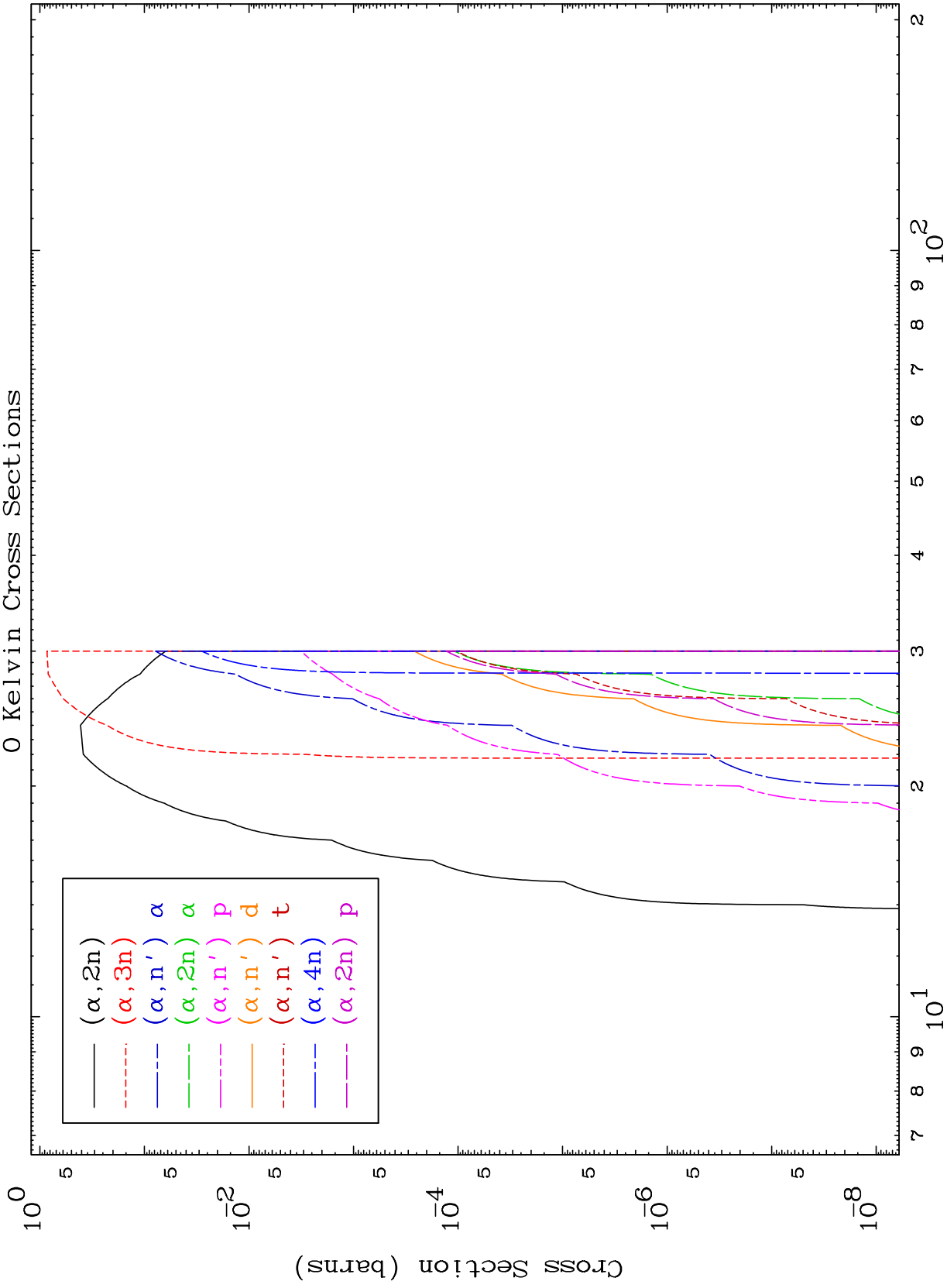
0 Kelvin Cross Sections



MAT 7543

α Neutron Production
0 Kelvin Cross Sections

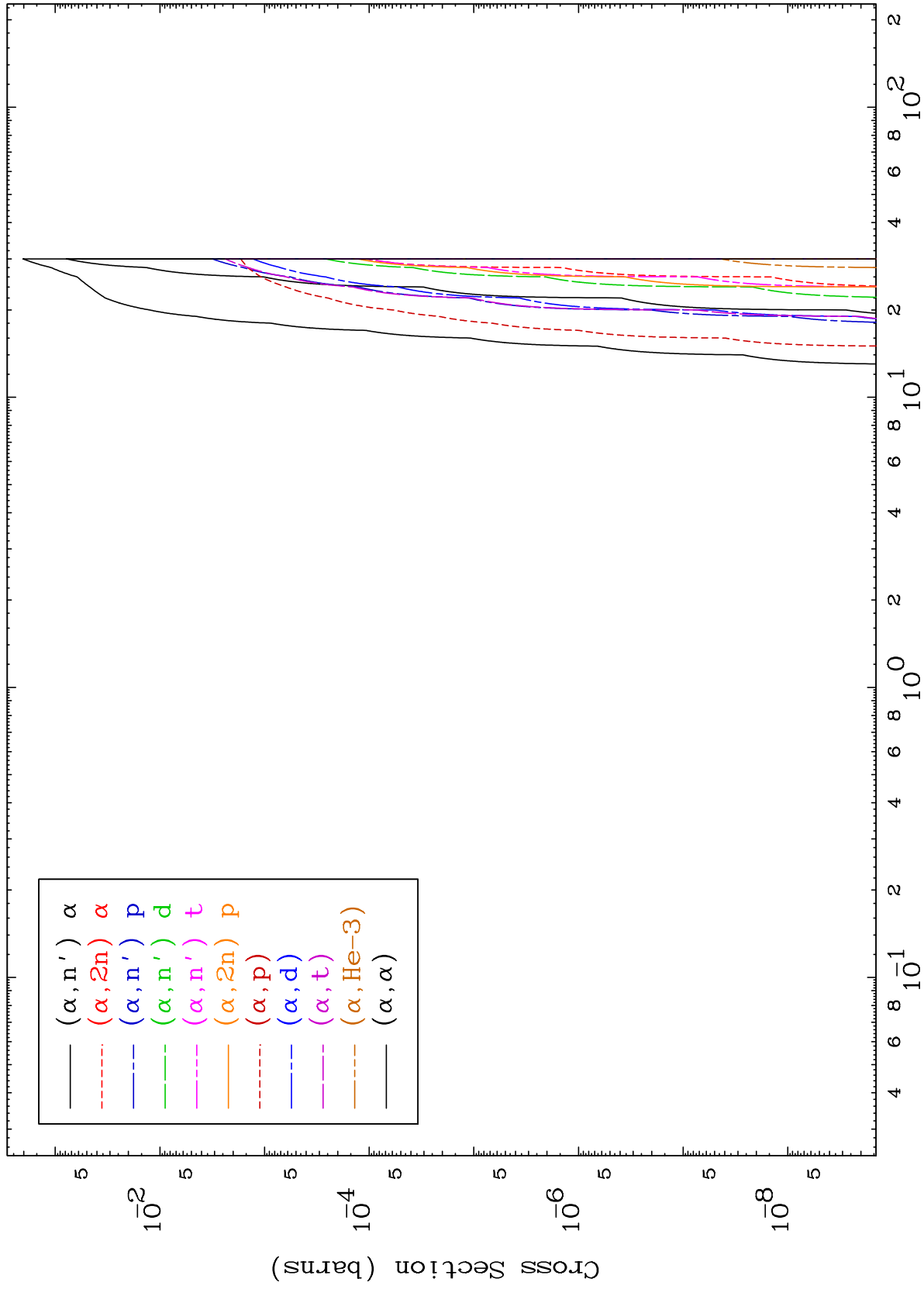
75-Re-191



2

Incident Energy (MeV)

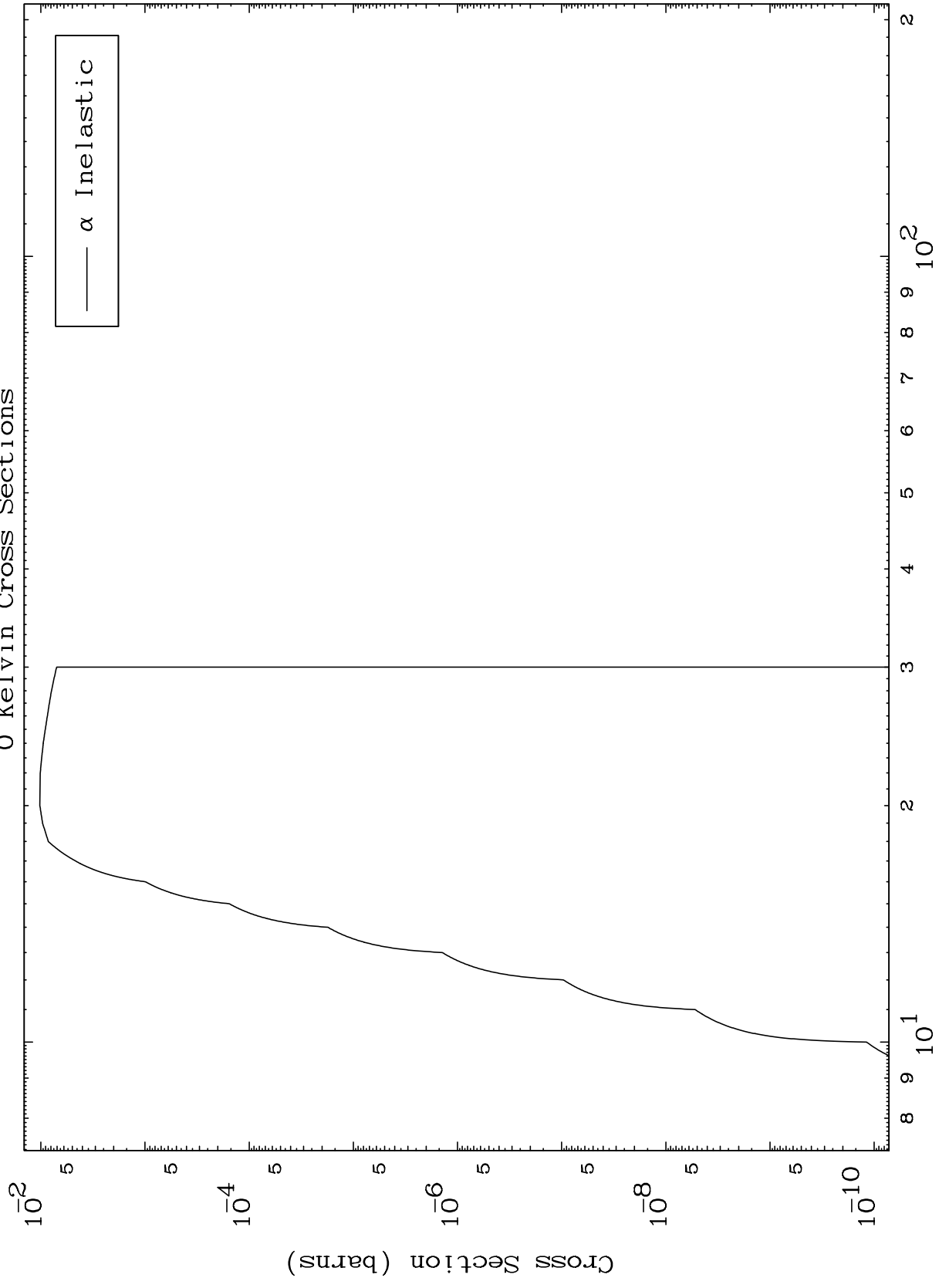
75-Re-191



MAT 7543

(α, n') Level
0 Kelvin Cross Sections

75-Re-191



4

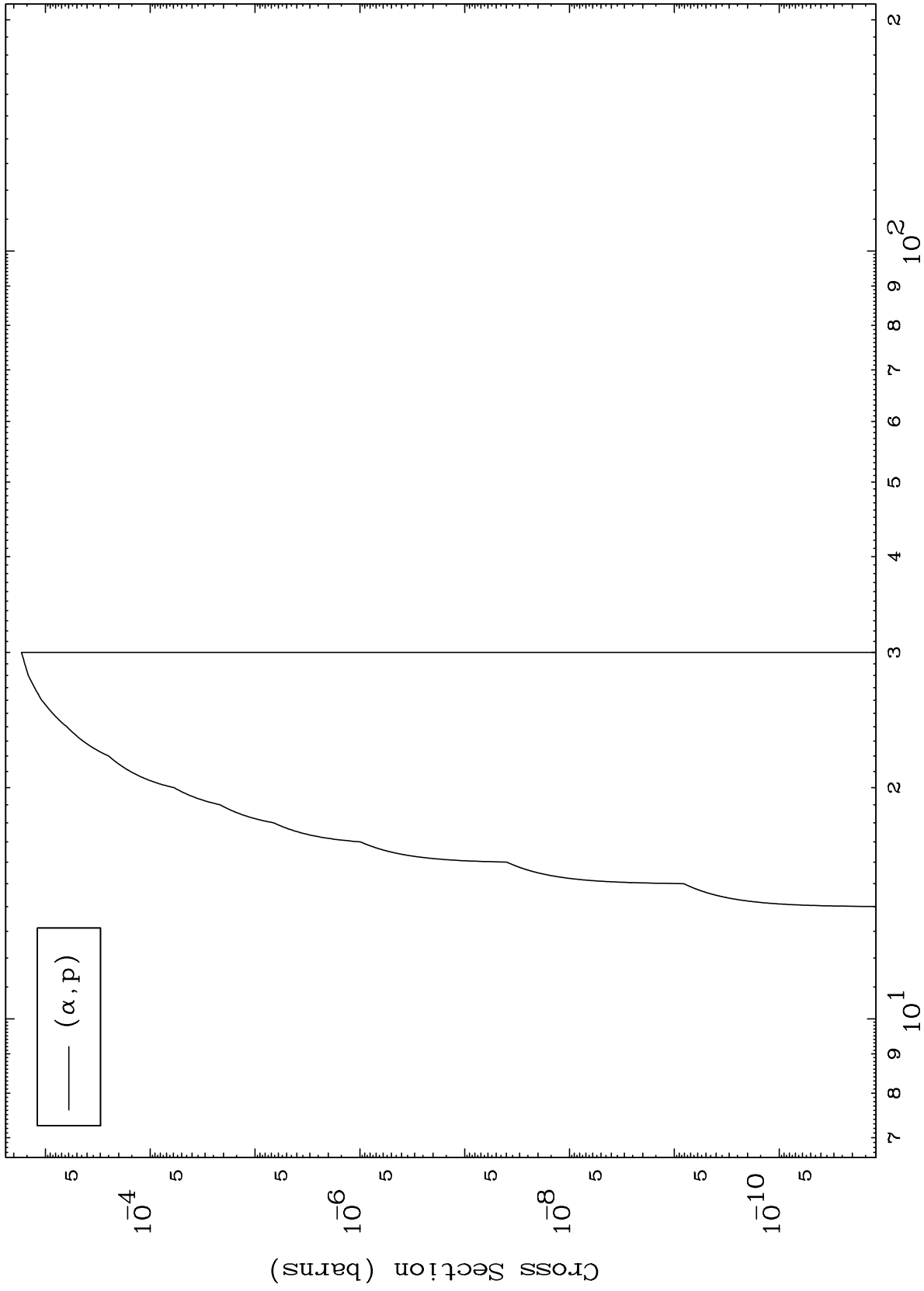
Incident Energy (MeV)

75-Re-191

MAT 7543

(α, p) Levels
0 Kelvin Cross Sections

75-Re-191



5

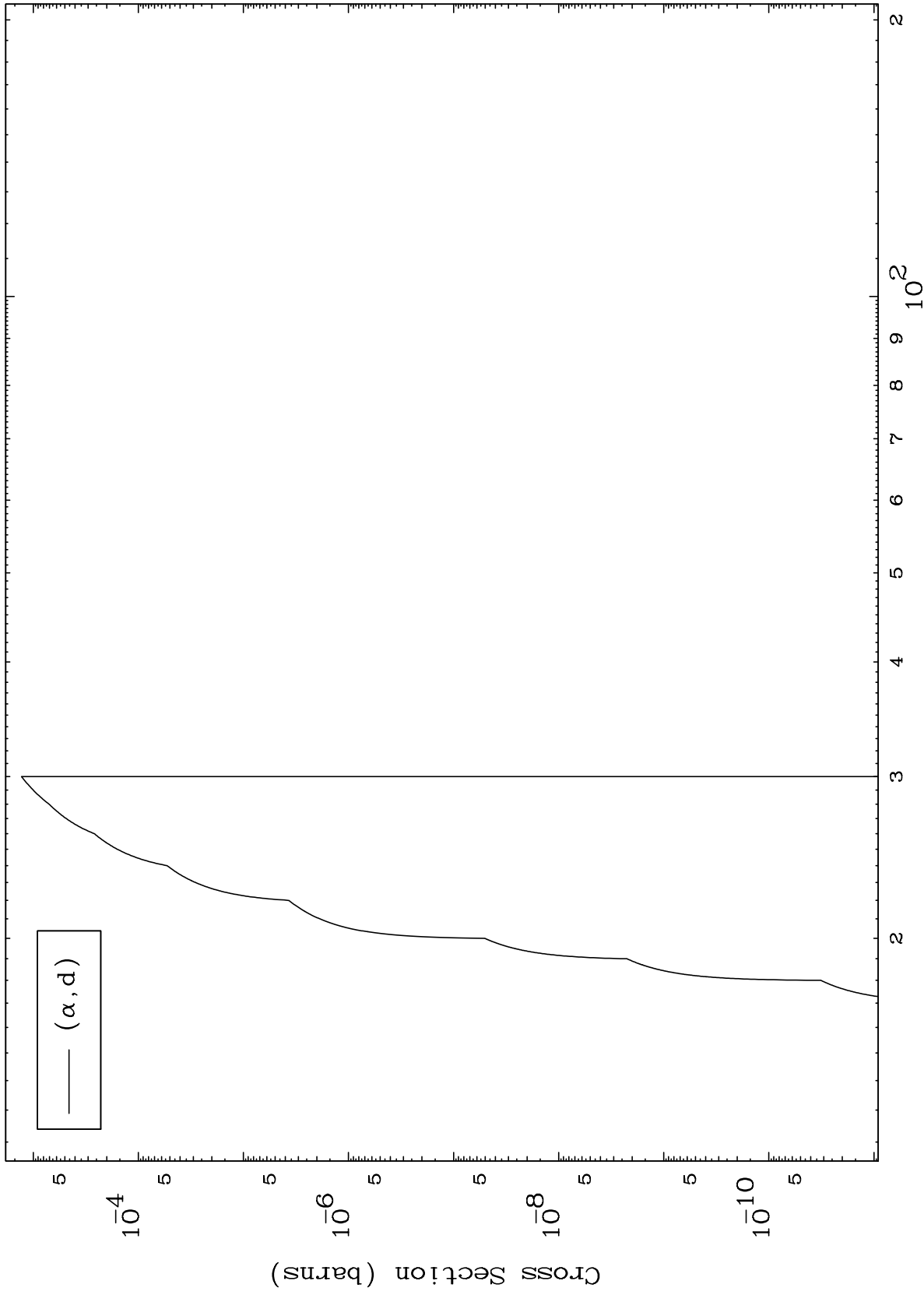
Incident Energy (MeV)

75-Re-191

MAT 7543

(α, d) Levels
0 Kelvin Cross Sections

75-Re-191



6

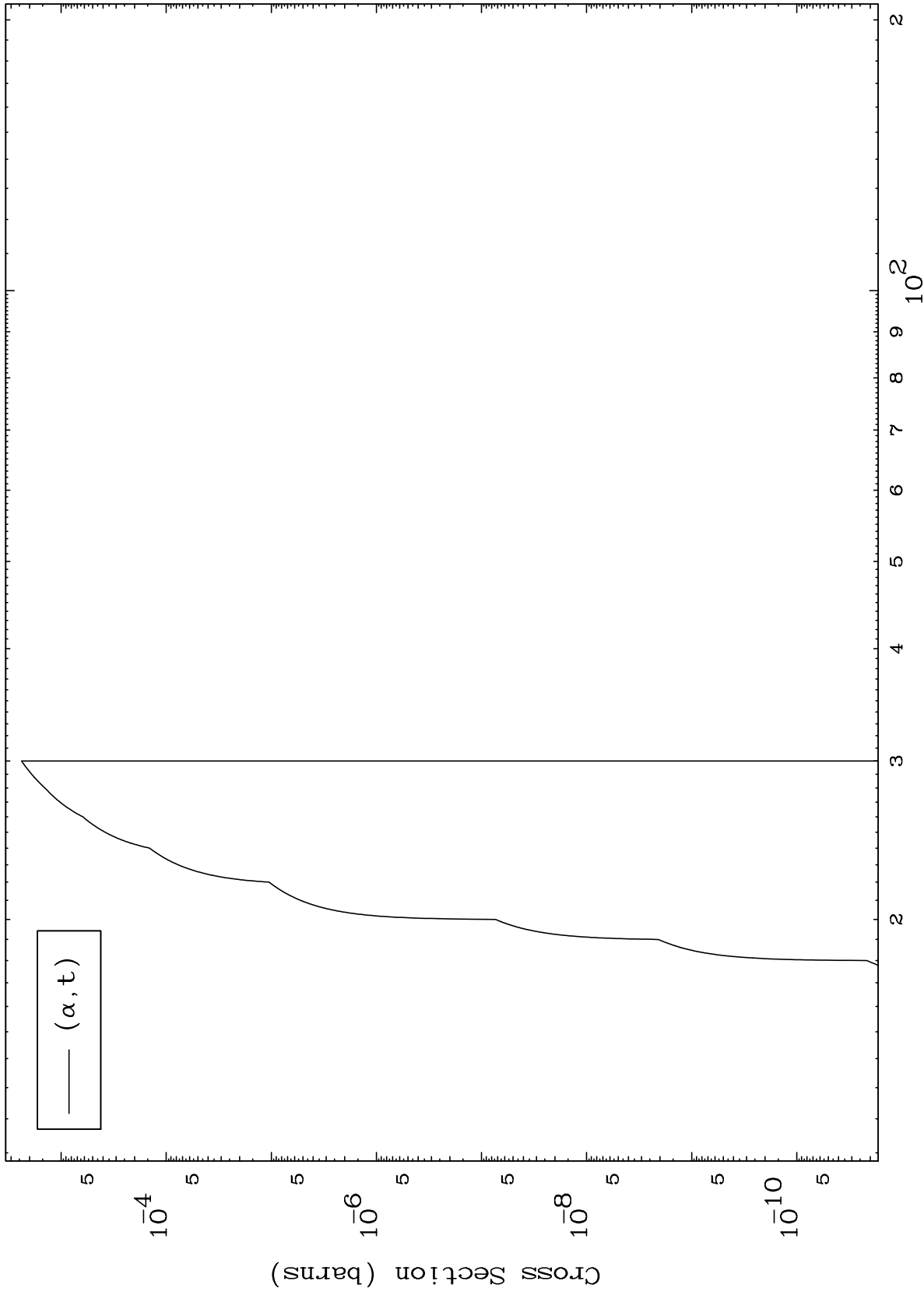
Incident Energy (MeV)

75-Re-191

MAT 7543

(α, t) Levels
0 Kelvin Cross Sections

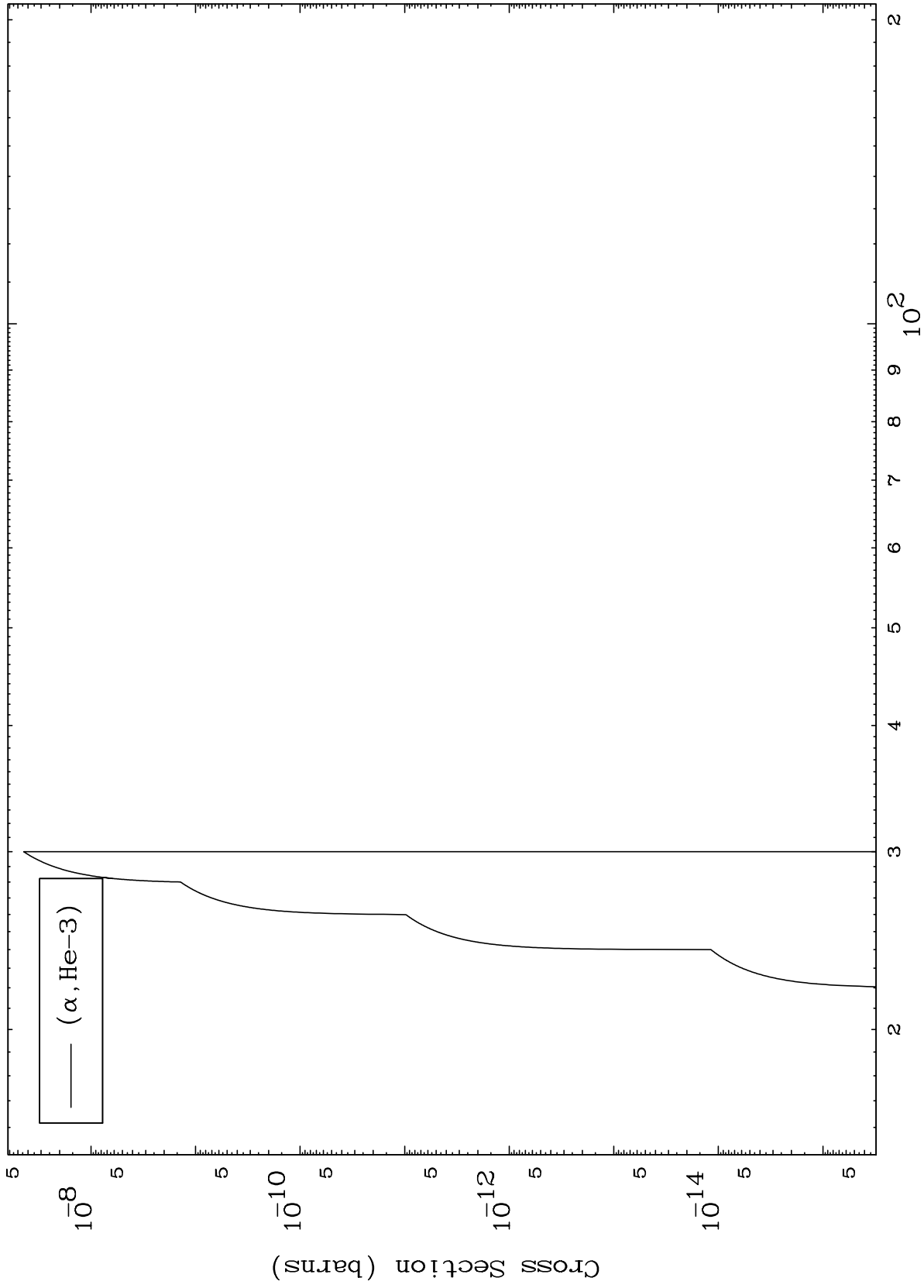
75-Re-191

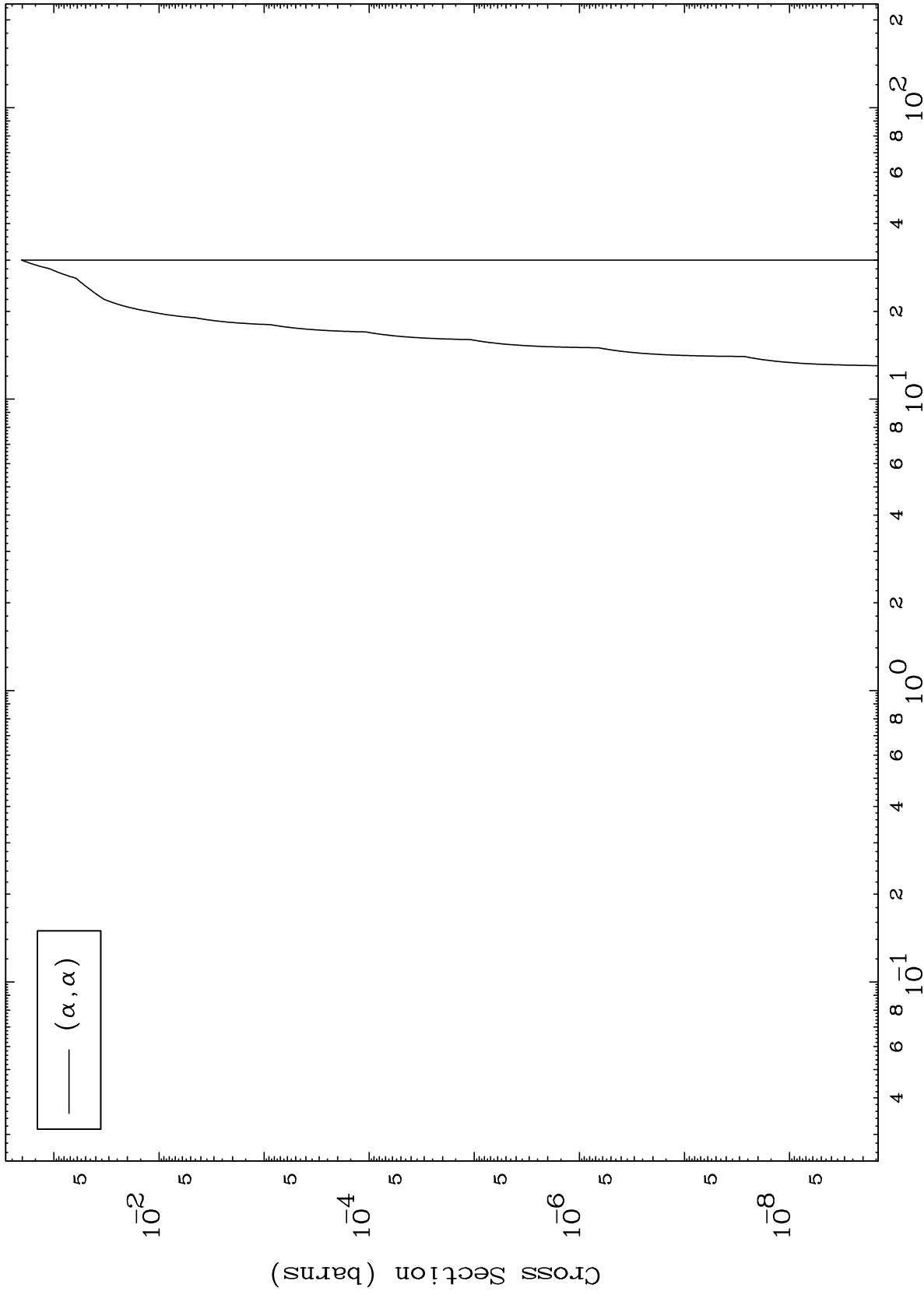


7

Incident Energy (MeV)

75-Re-191

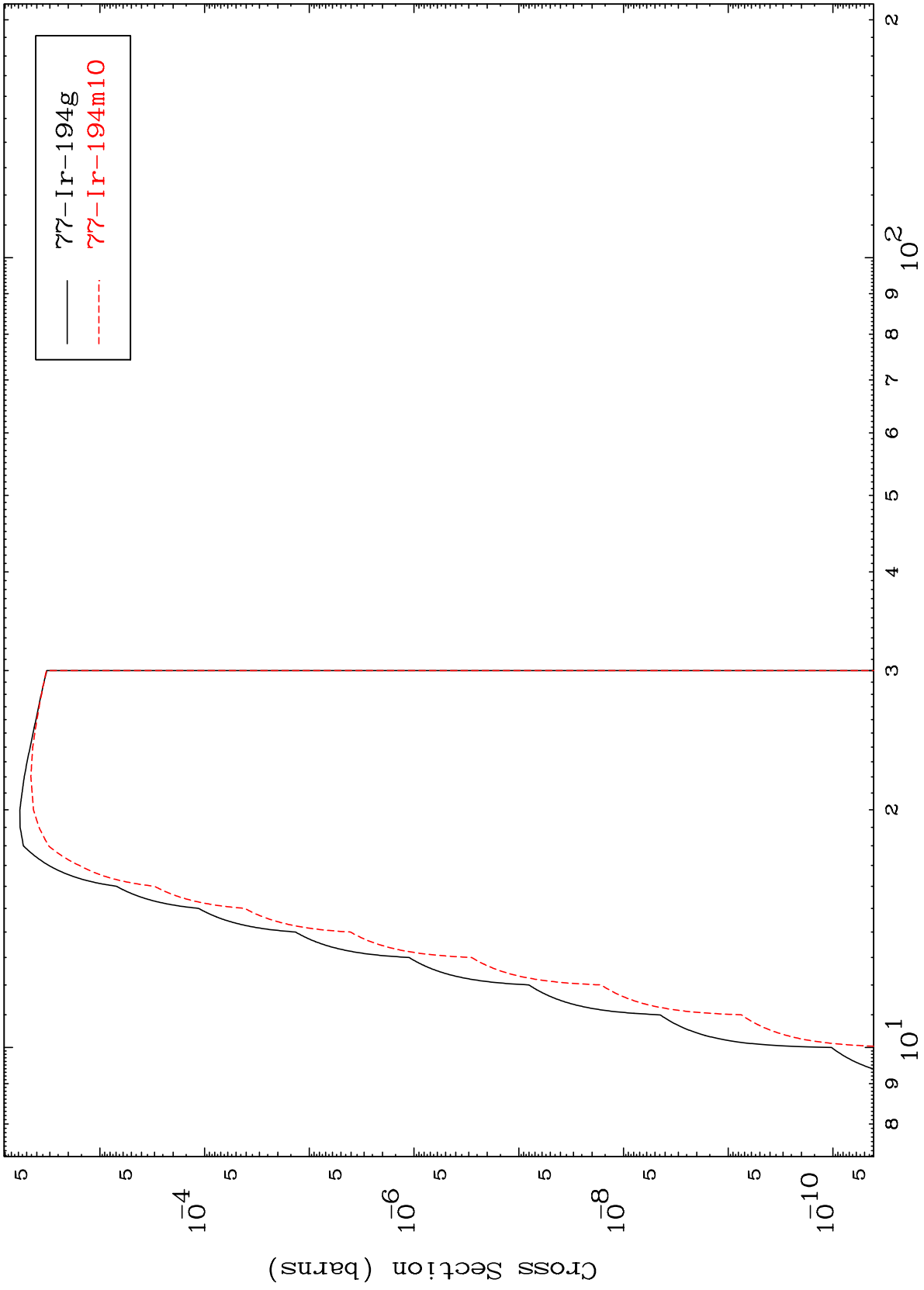




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Radionuclide Production Cross Section
 α Inelastic

75-Re-191



10

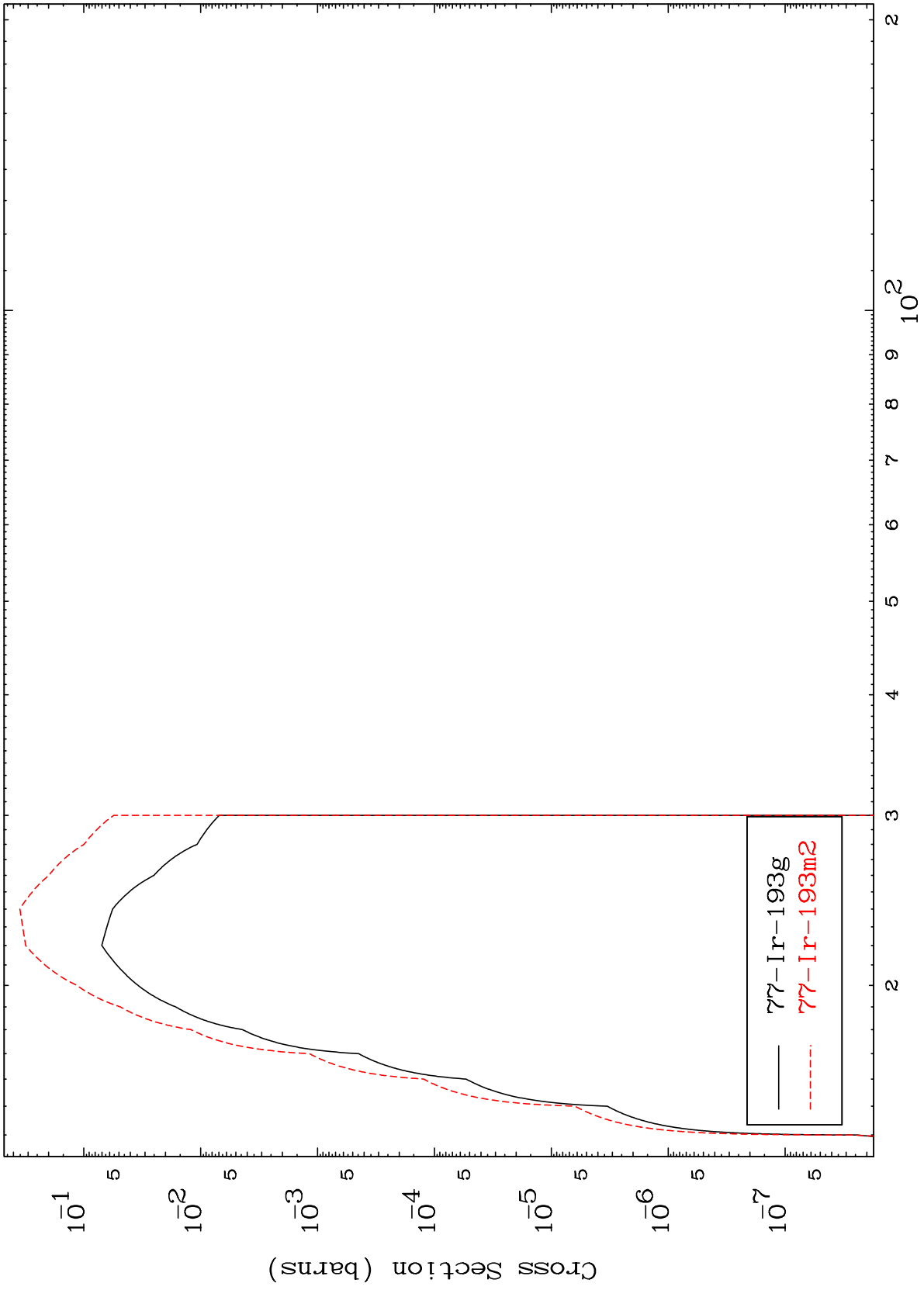
Incident Energy (MeV)

75-Re-191

MAT 7543

75-Re-191

($\alpha, 2n$)
Radionuclide Production Cross Section



11

Incident Energy (MeV)

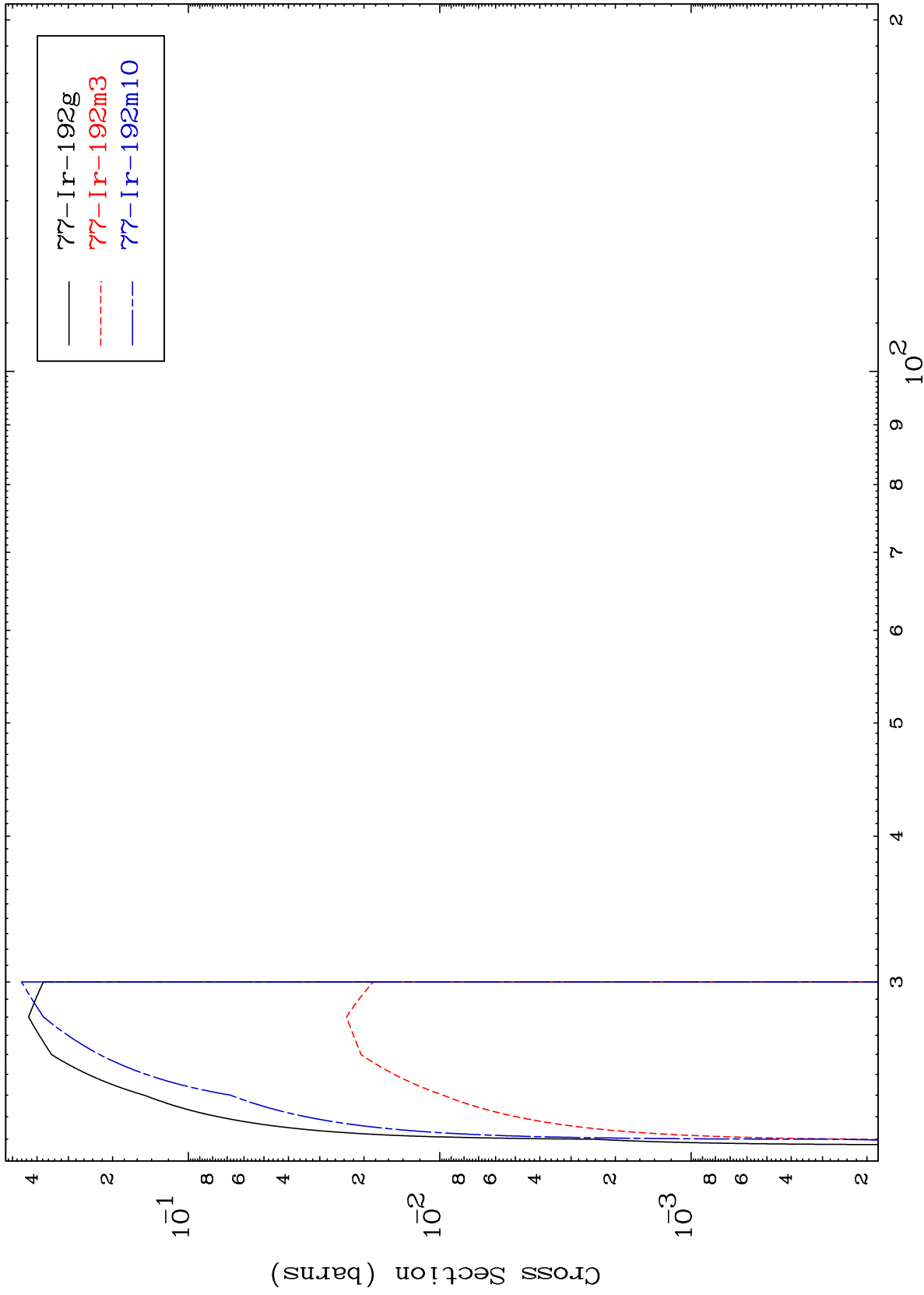
75-Re-191

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($\alpha, 3n$)

75-Re-191

Radionuclide Production Cross Section



12

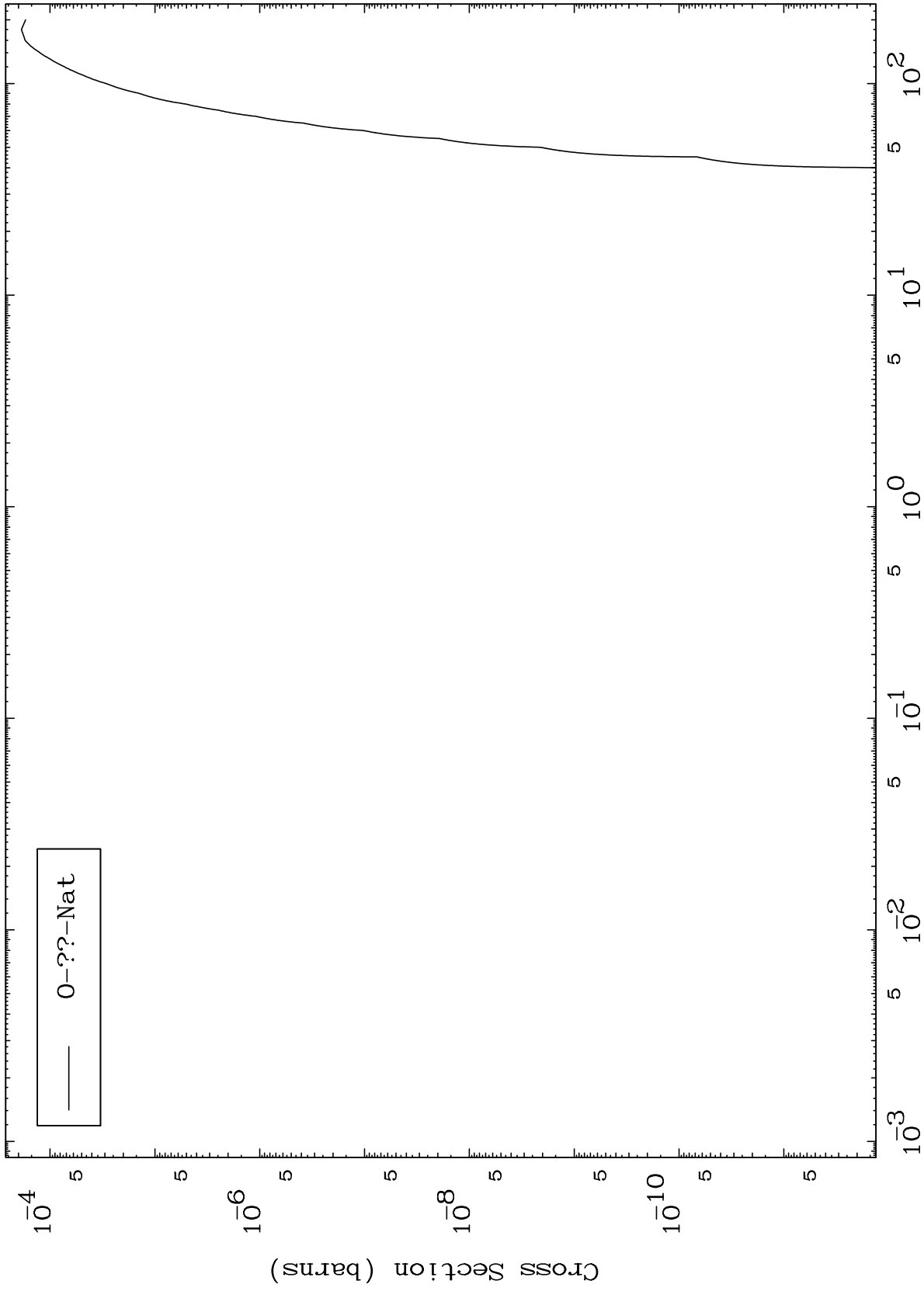
Incident Energy (MeV)

75-Re-191

MAT 7543

α Fission
Radionuclide Production Cross Section

75-Re-191



13

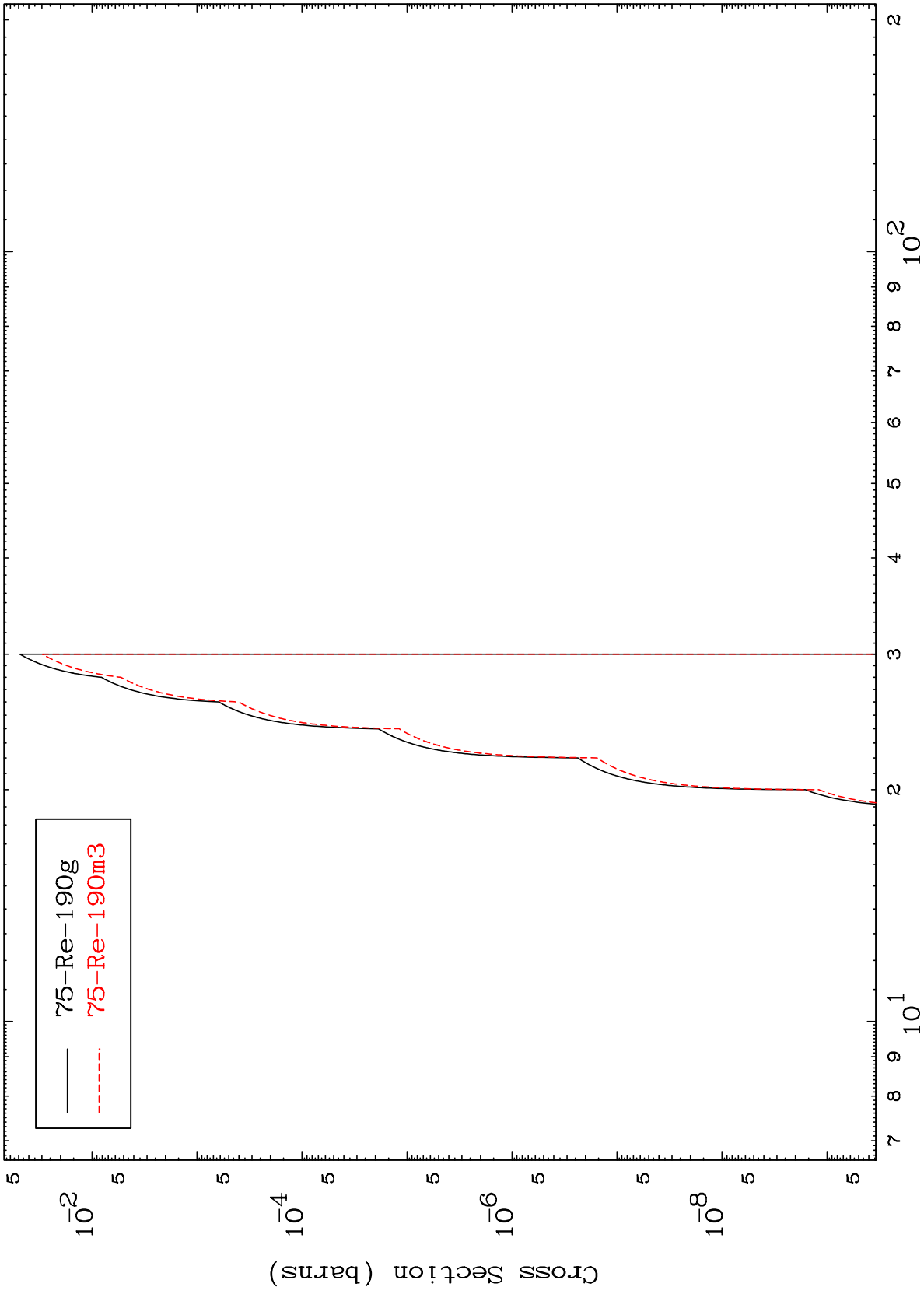
Incident Energy (MeV)

75-Re-191

MAT 7543

75-Re-191

(α, n') α
Radionuclide Production Cross Section



14

Incident Energy (MeV)

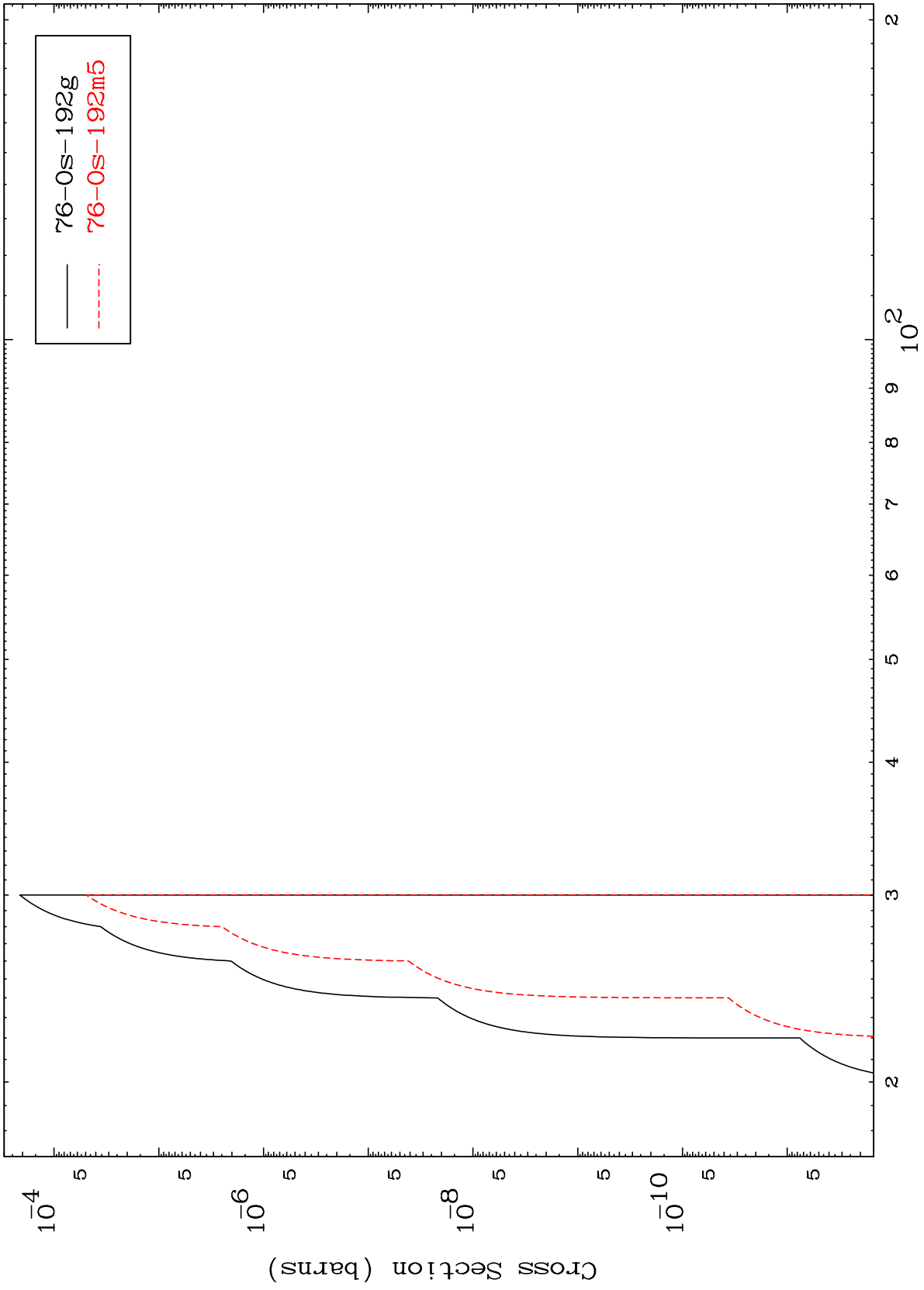
75-Re-191

MAT 7543

(α, n') d

75-Re-191

Radionuclide Production Cross Section



15

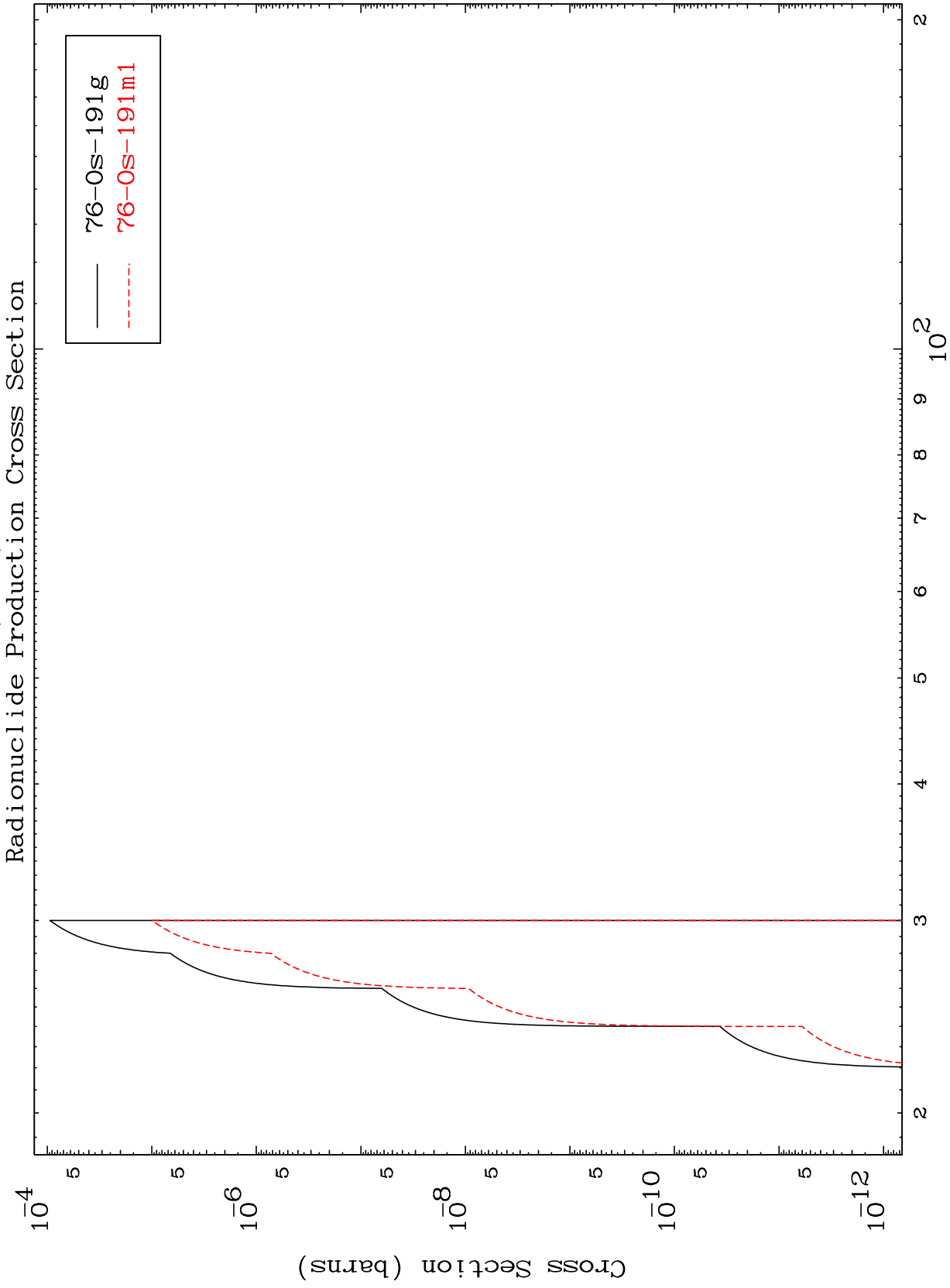
Incident Energy (MeV)

75-Re-191

MAT 7543

(α, n') t

75-Re-191



16

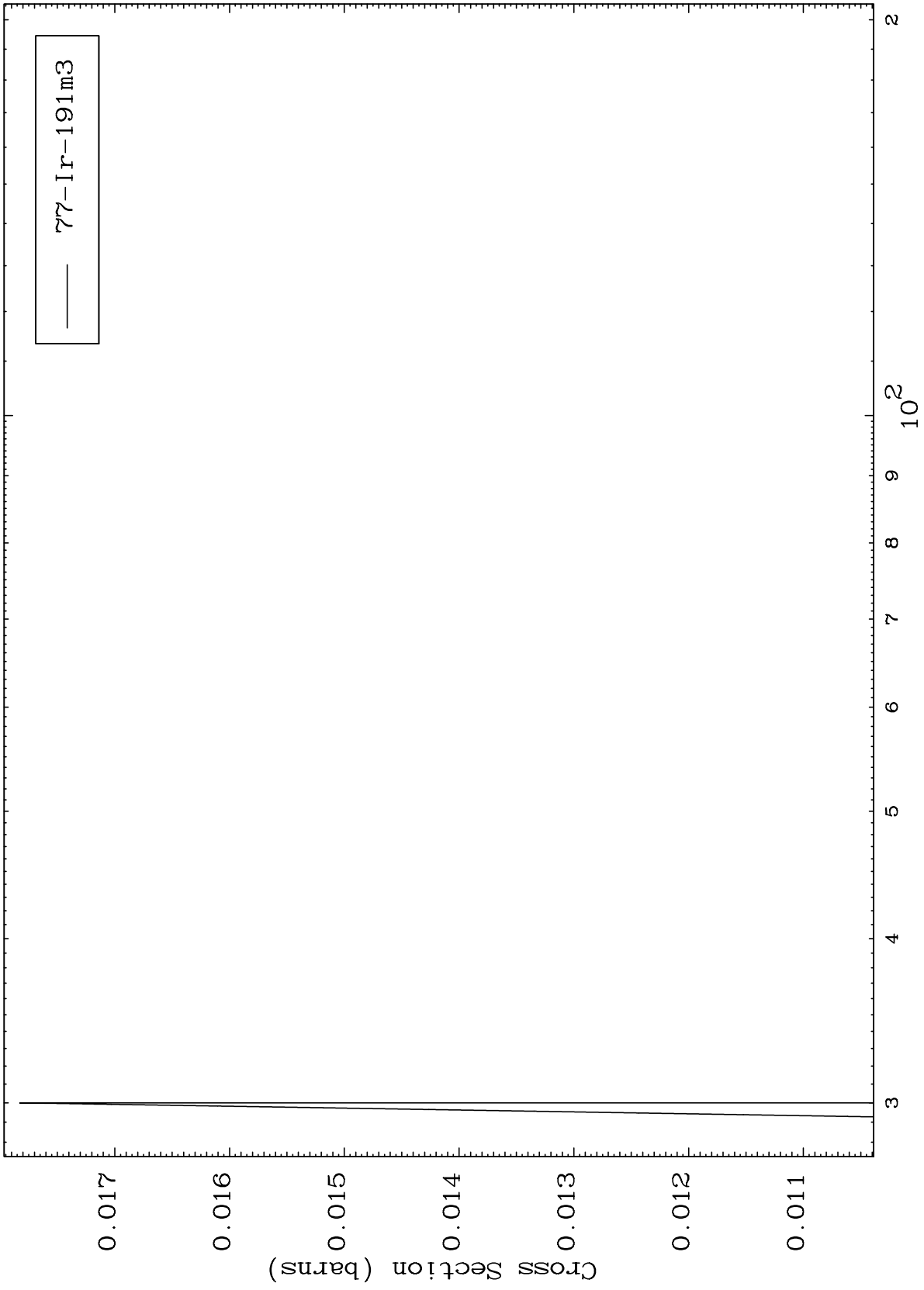
Incident Energy (MeV)

75-Re-191

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75-Re-191

($\alpha, 4n$)
Radionuclide Production Cross Section



17

75-Re-191

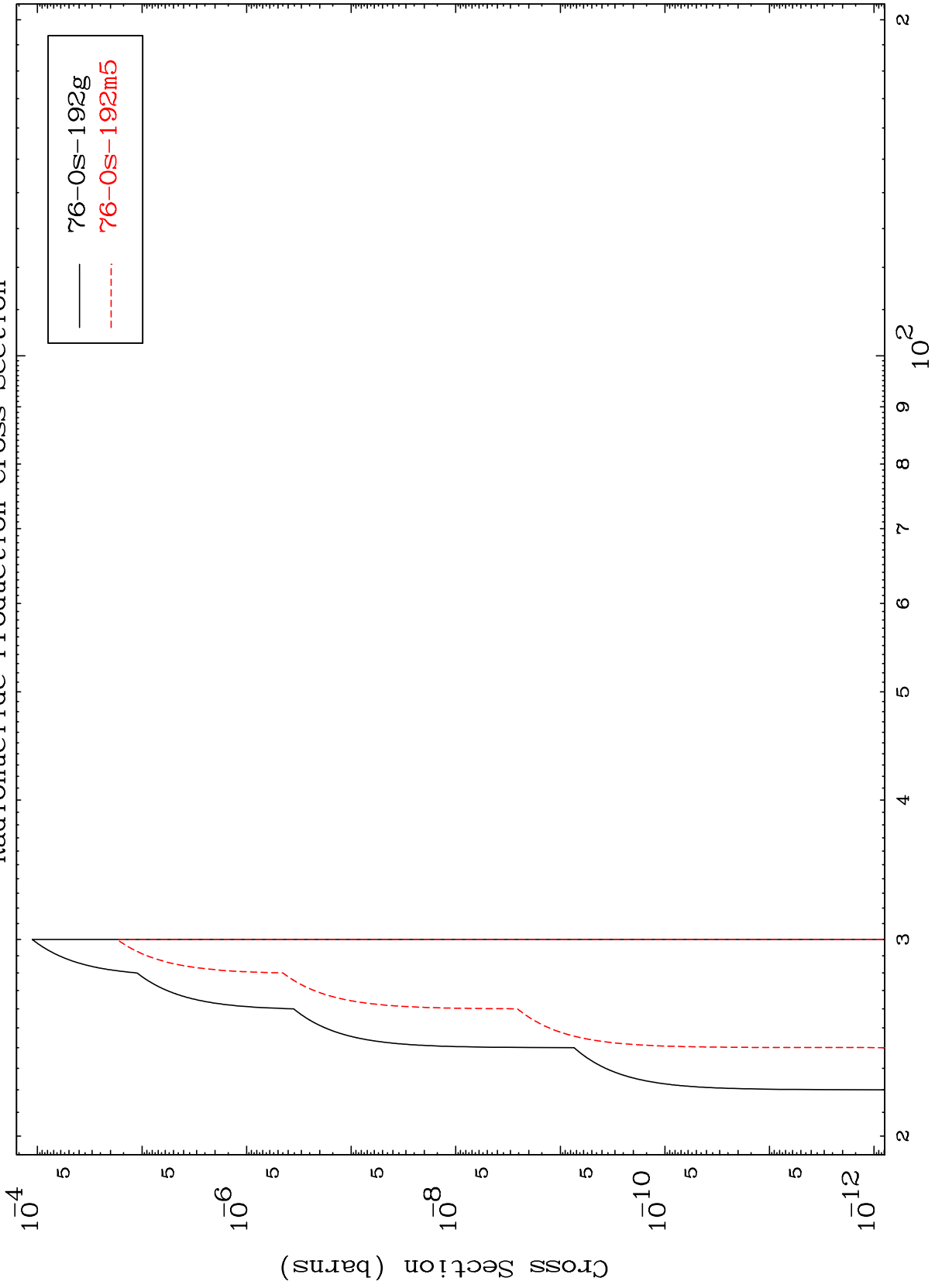
Incident Energy (MeV)

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($\alpha, 2n$) p

75-Re-191

Radionuclide Production Cross Section



18

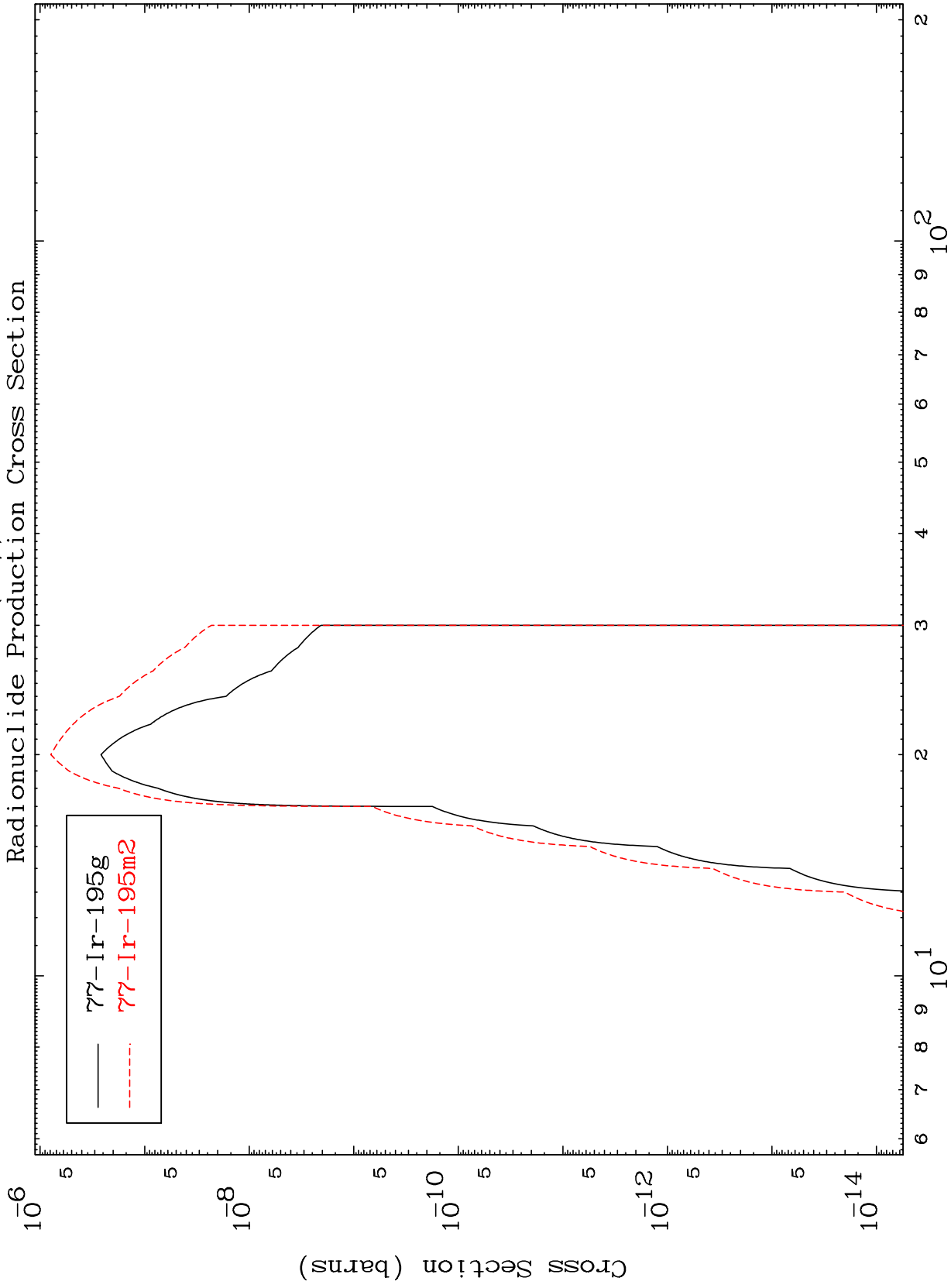
Incident Energy (MeV)

75-Re-191

MAT 7543

75-Re-191

(α, γ)
Radionuclide Production Cross Section



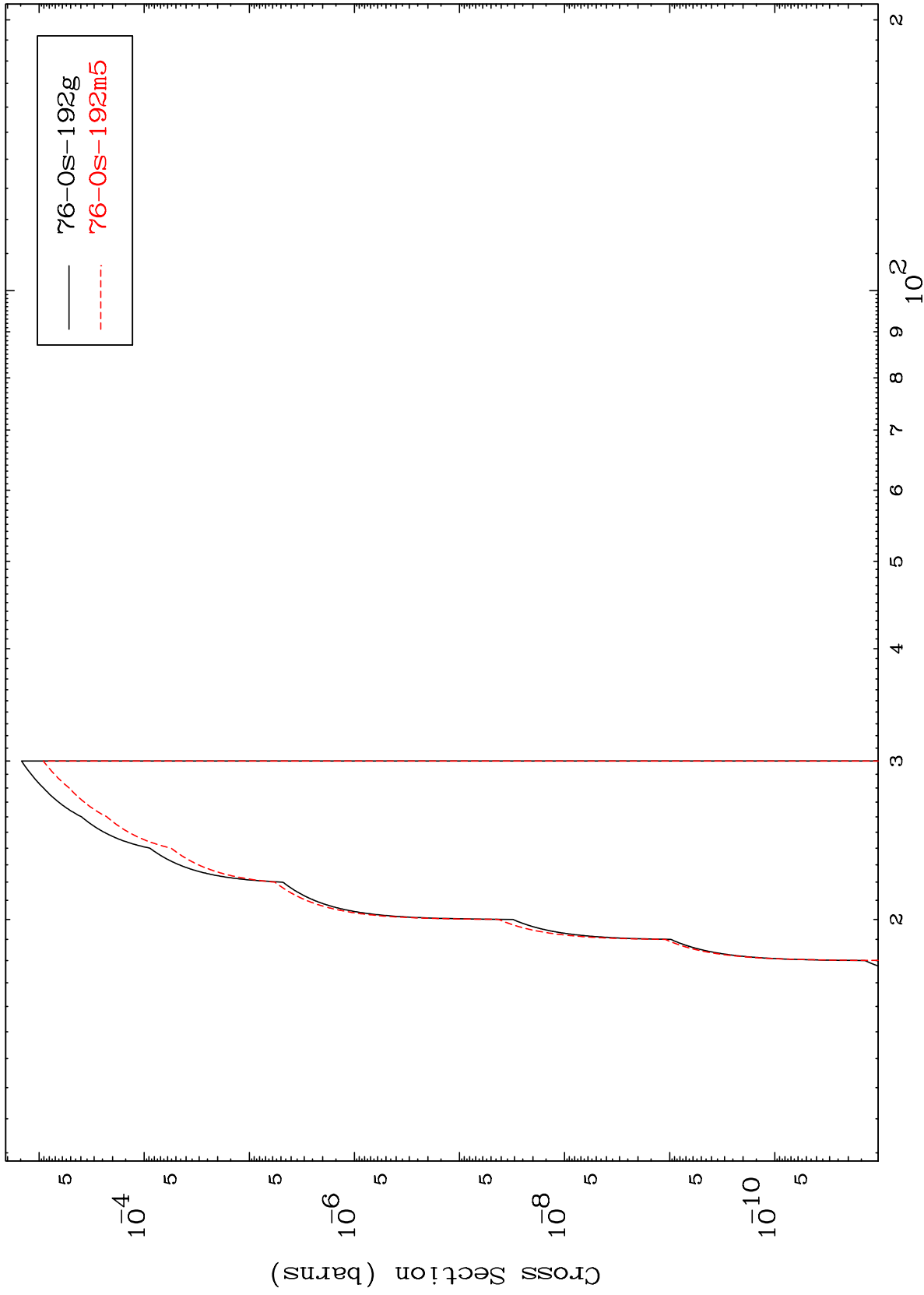
19

75-Re-191

MAT 7543

75-Re-191

(α, t)
Radionuclide Production Cross Section



20

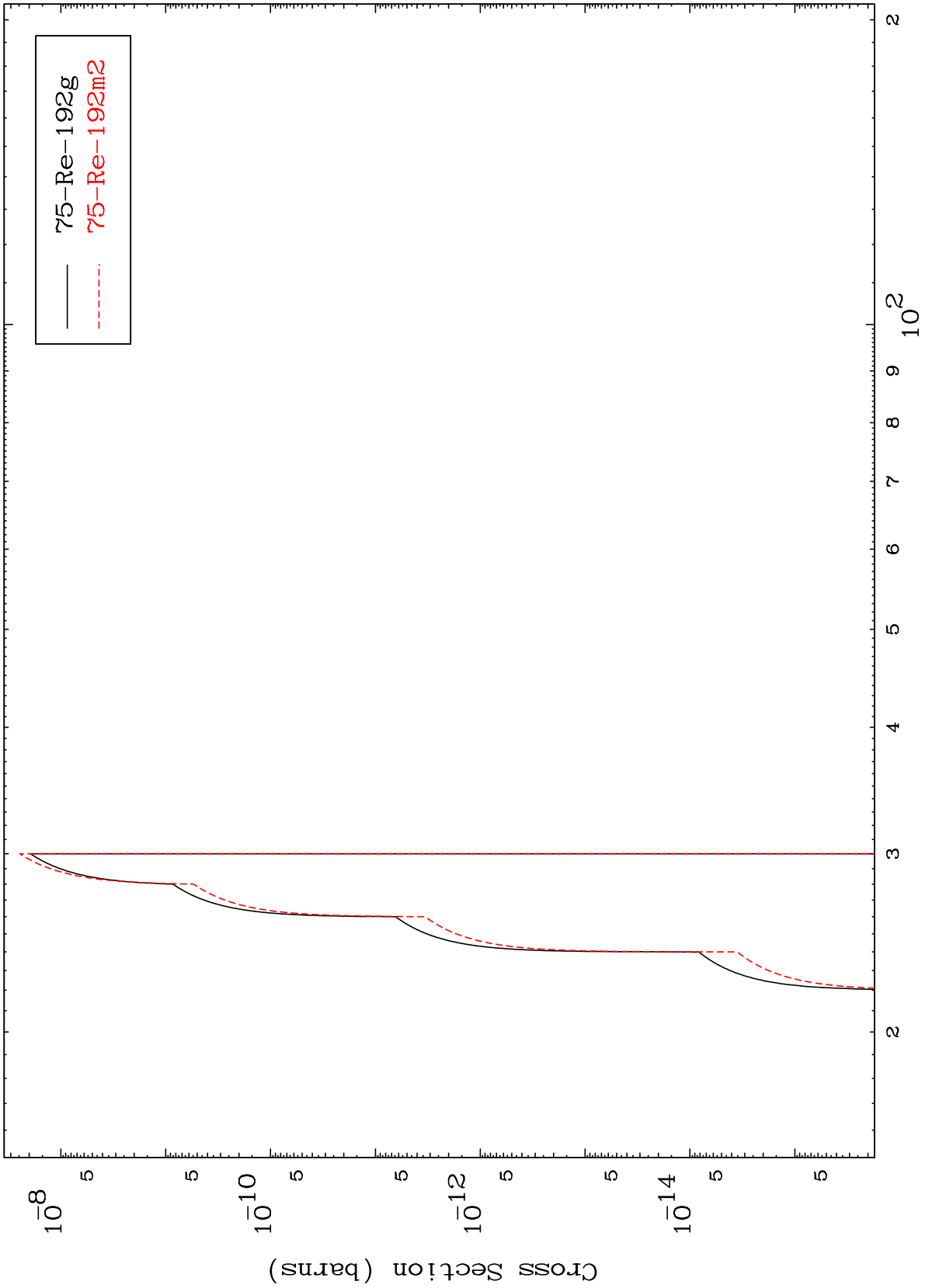
75-Re-191

MAT 7543

($\alpha, \text{He-3}$)

75-Re-191

Radionuclide Production Cross Section



21

75-Re-191