

Program EVALPLOT  
(Version 2018-1)

by

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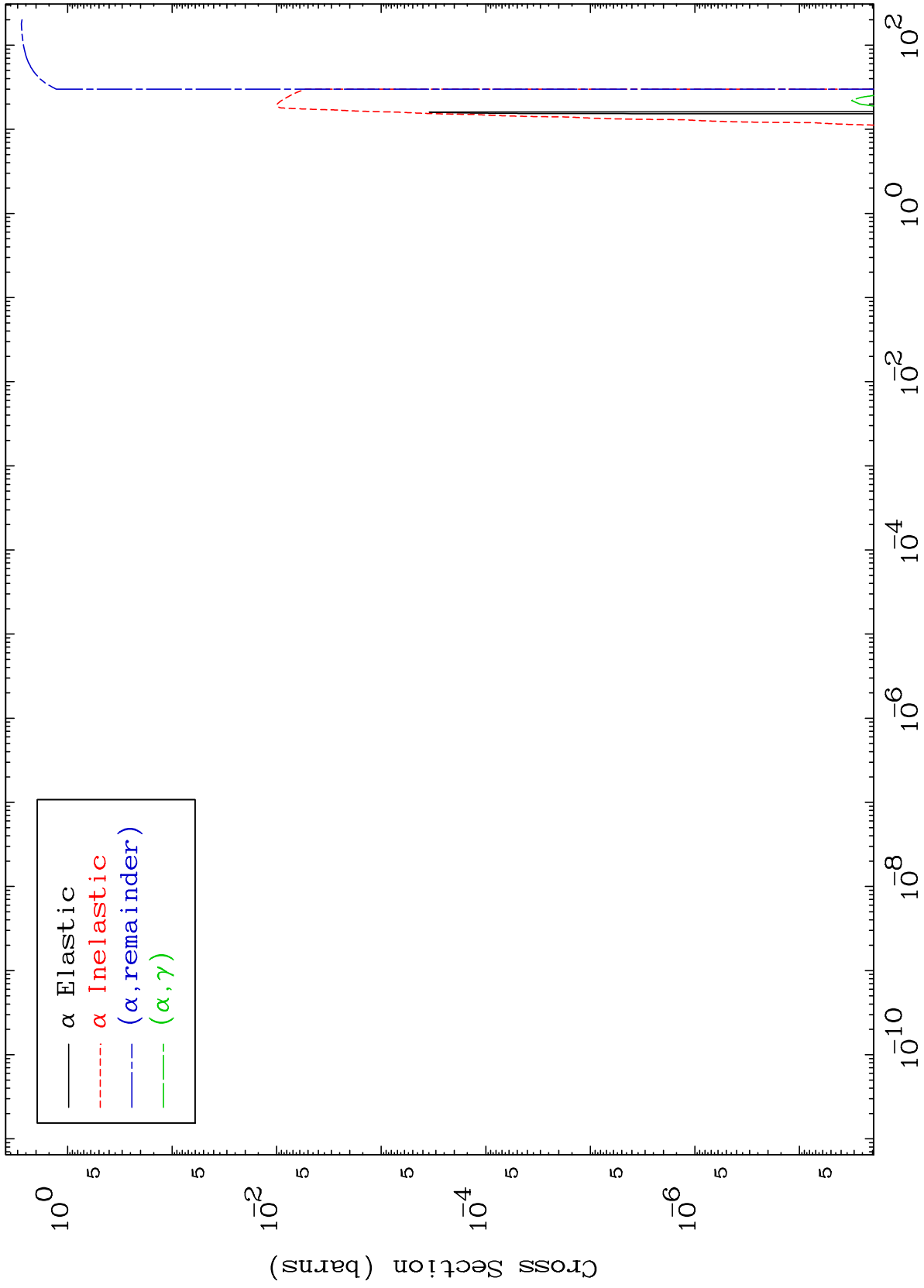
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7558

0 Kelvin  $\alpha$  Major  
Cross Sections

75-Re-196



1

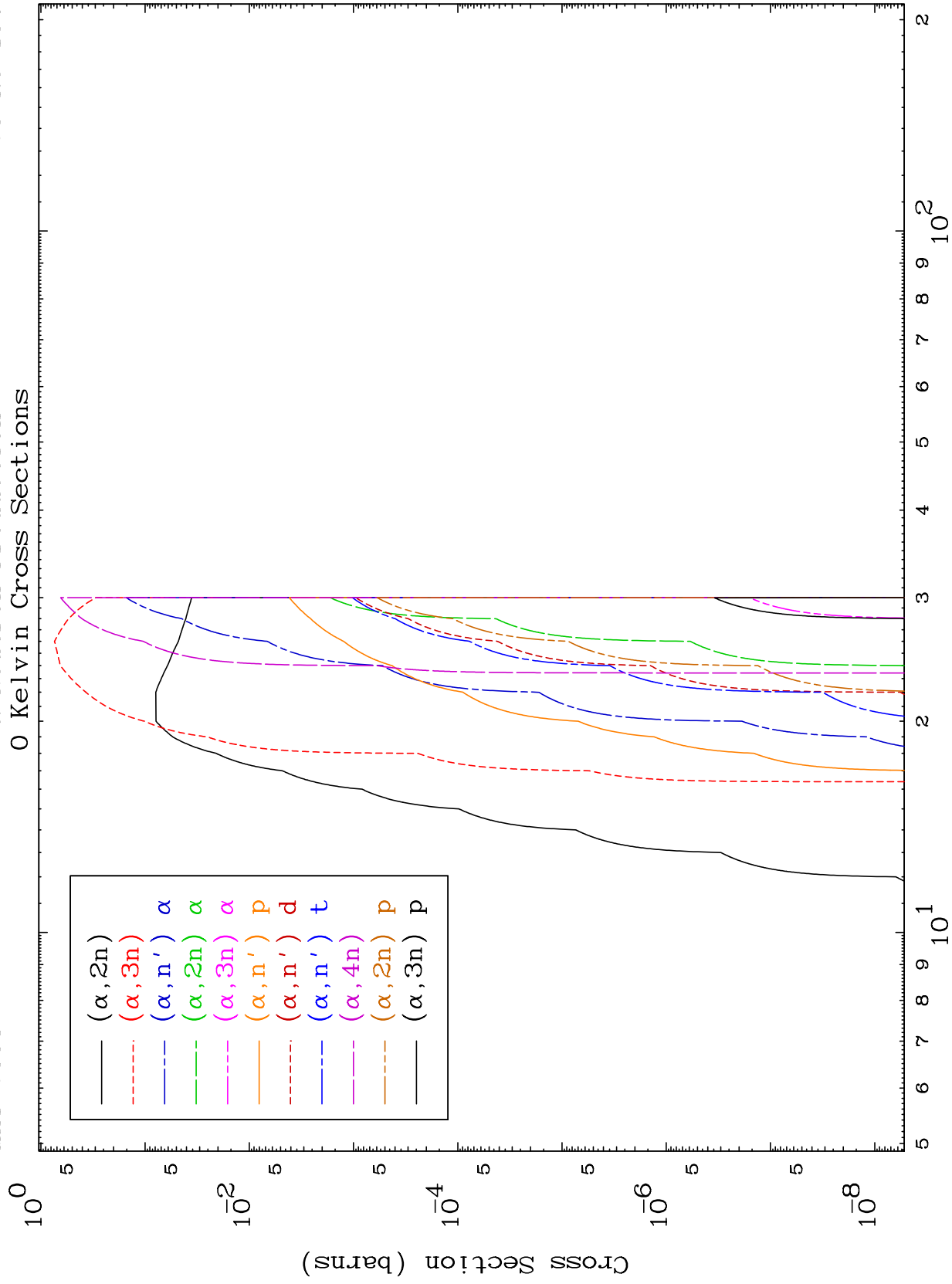
Incident Energy (MeV)

75-Re-196

MAT 7558

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

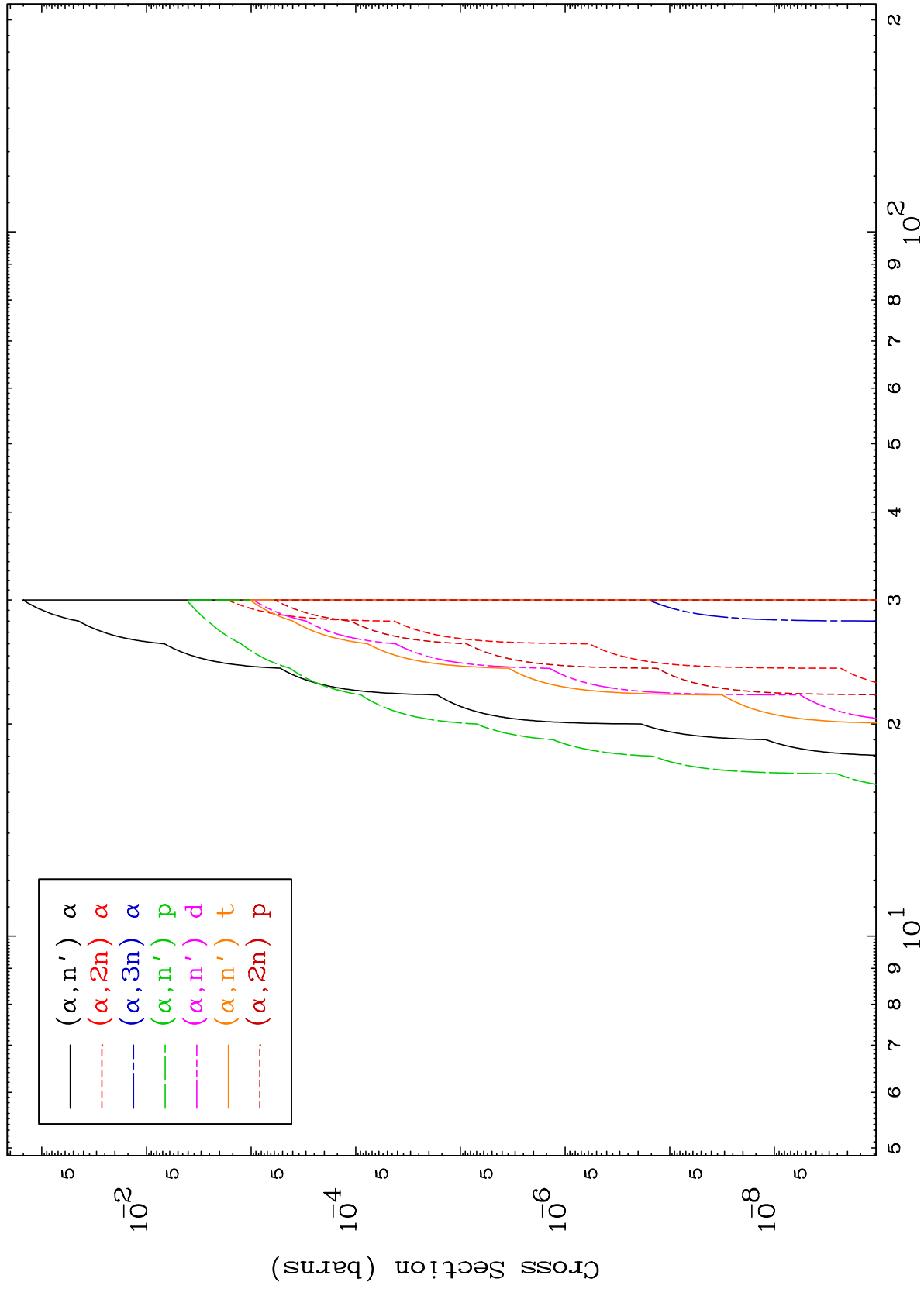
75-Re-196

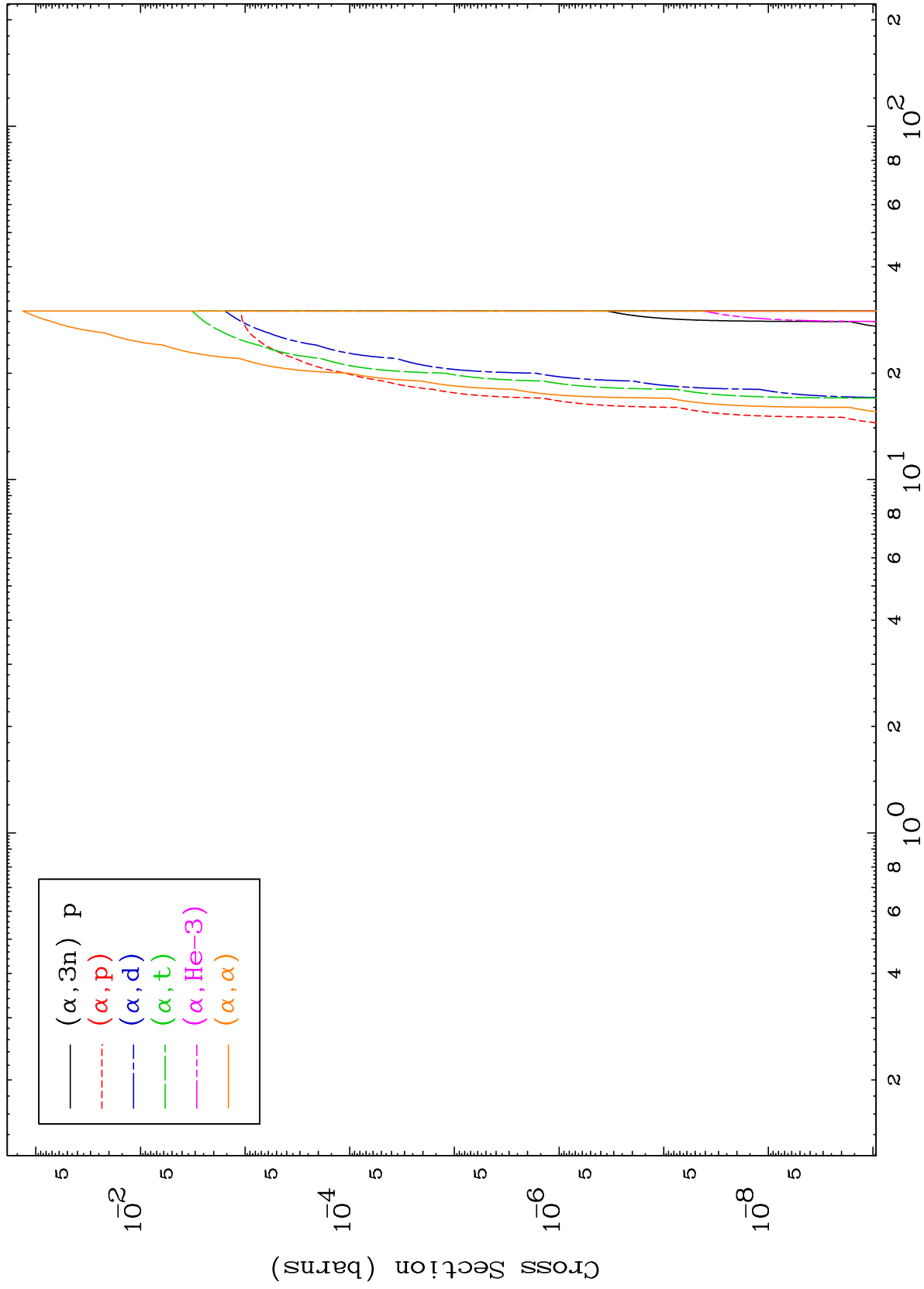


2

Incident Energy (MeV)

75-Re-196

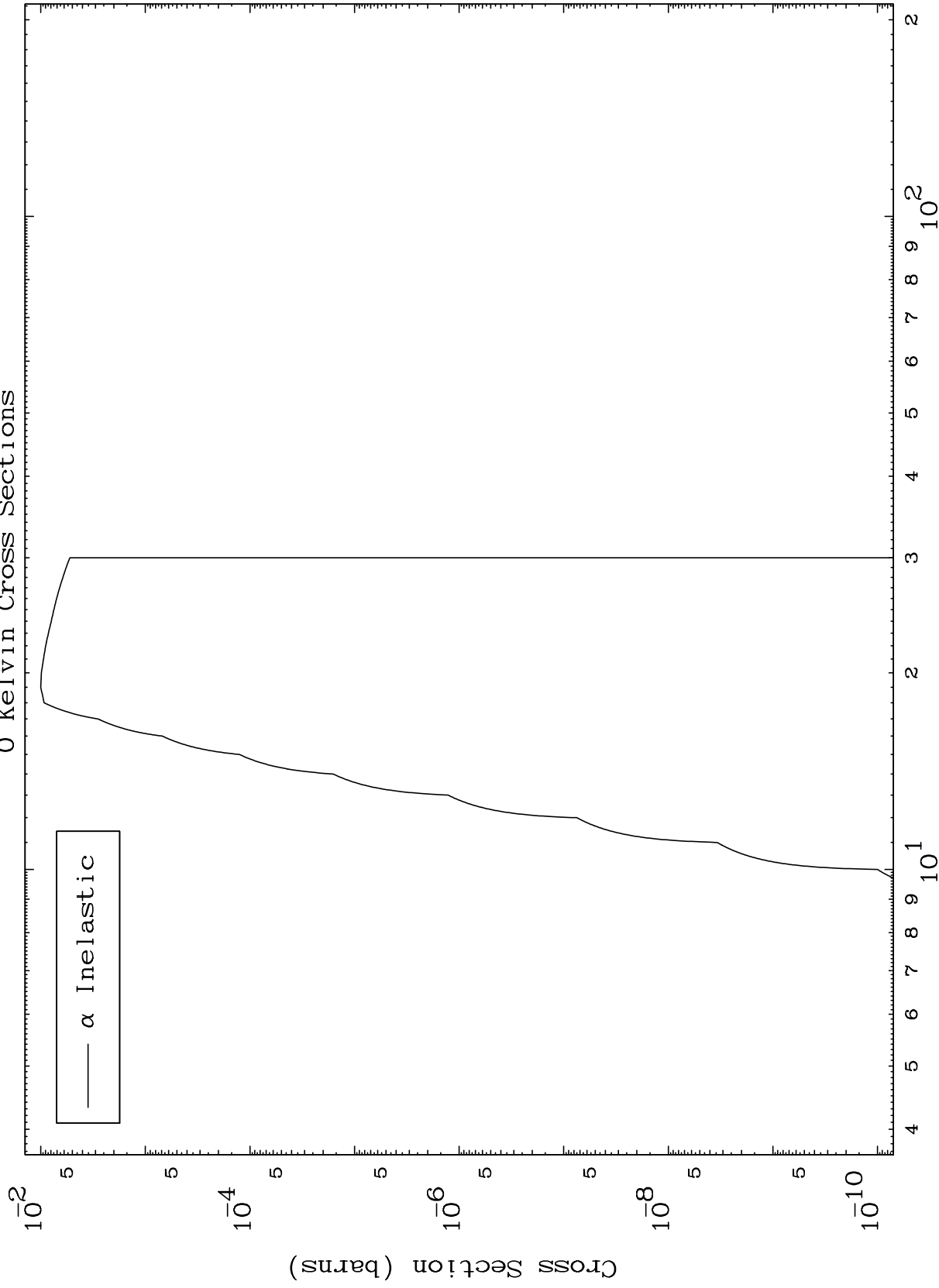




MAT 7558

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

75-Re-196



5

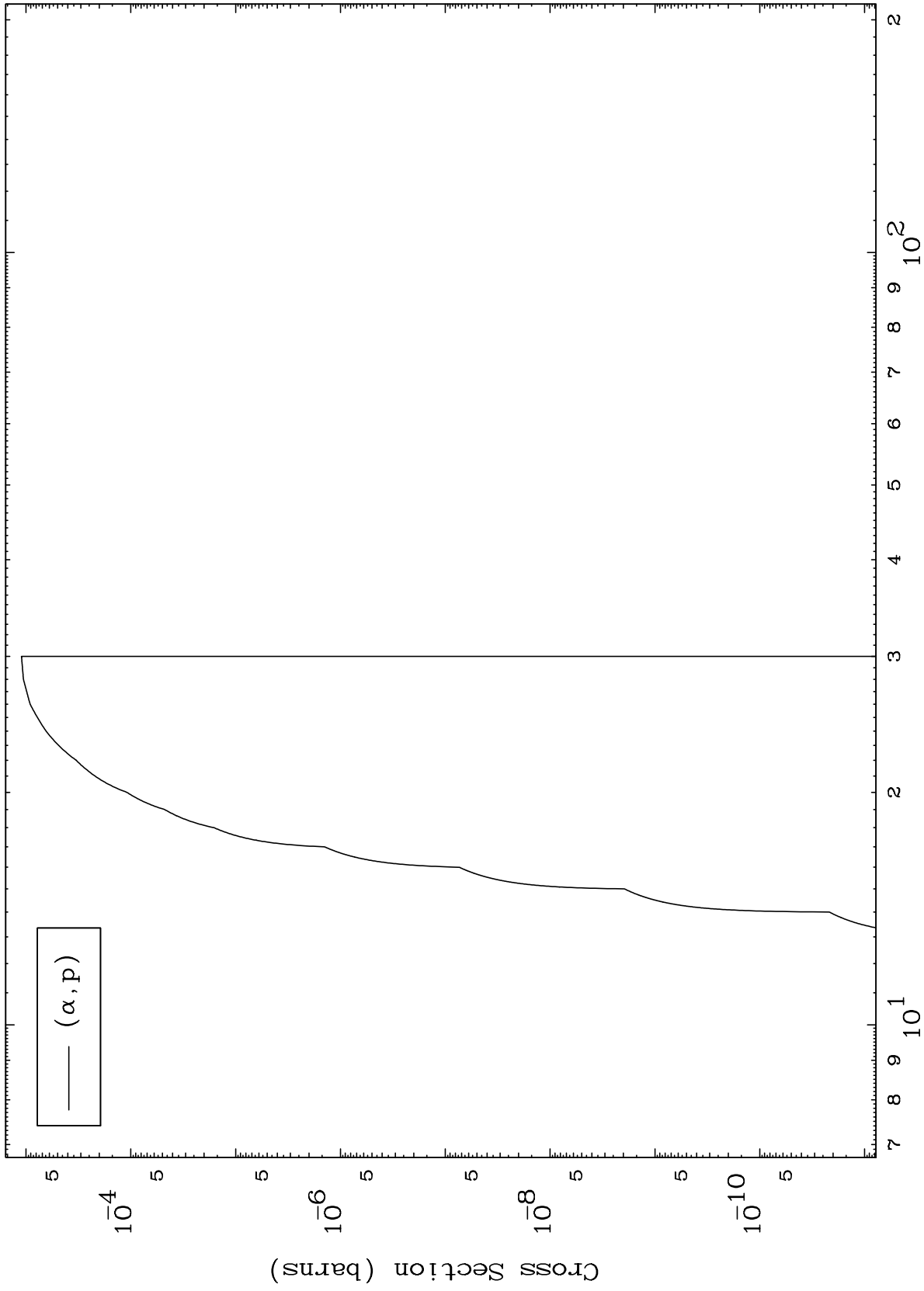
Incident Energy (MeV)

75-Re-196

MAT 7558

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

75-Re-196



6

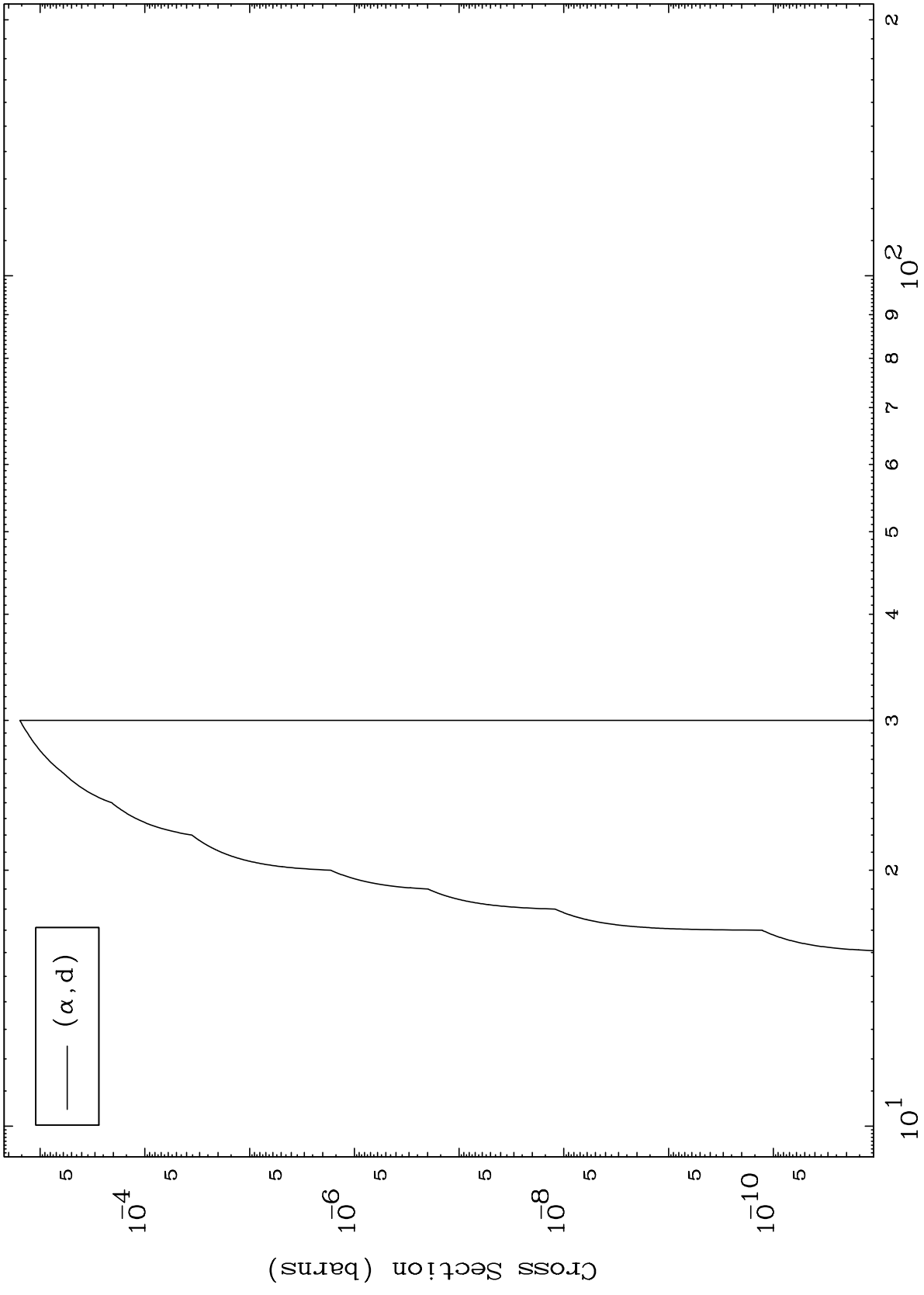
Incident Energy (MeV)

75-Re-196

MAT 7558

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

75-Re-196



Incident Energy (MeV)

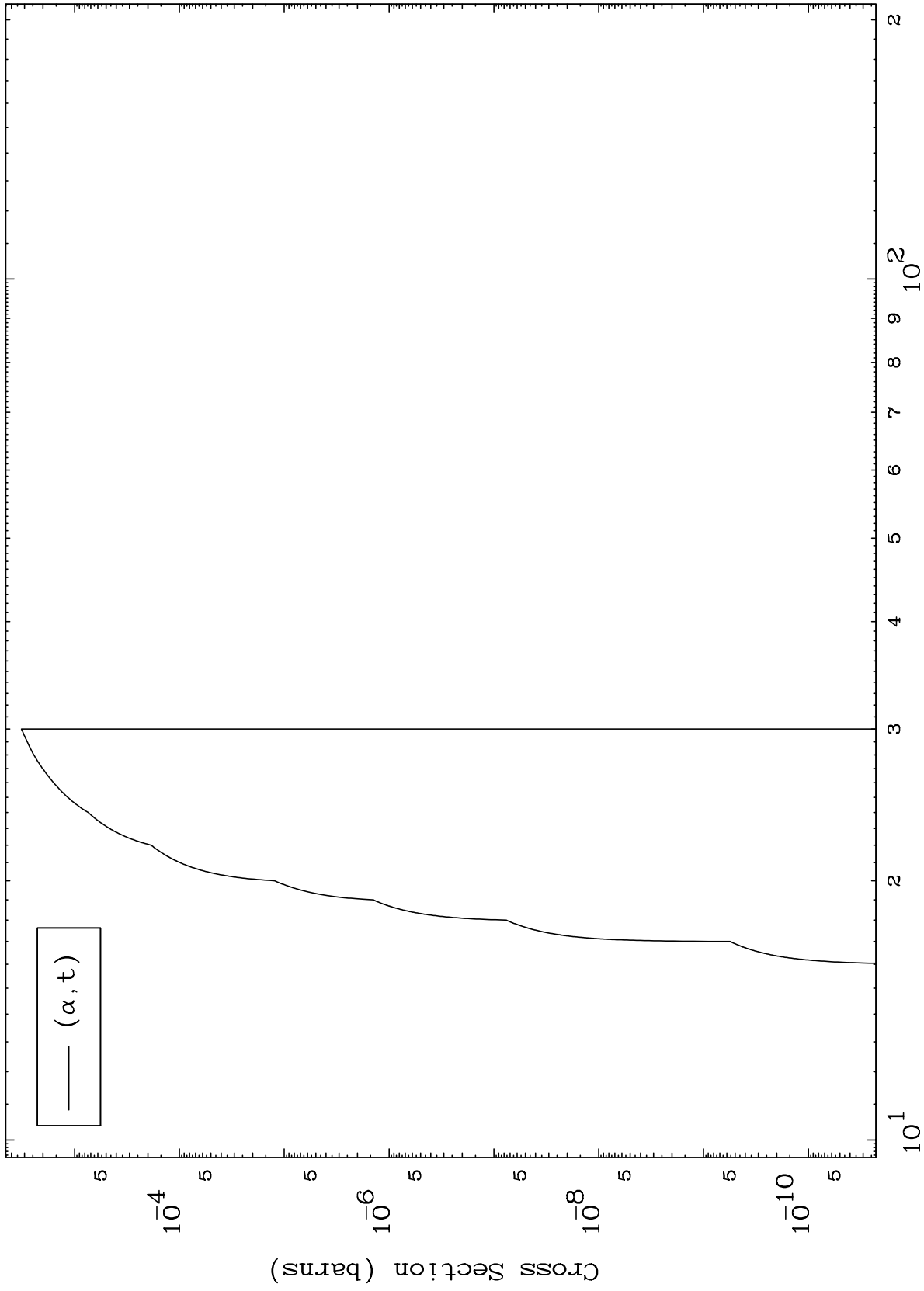
75-Re-196



MAT 7558

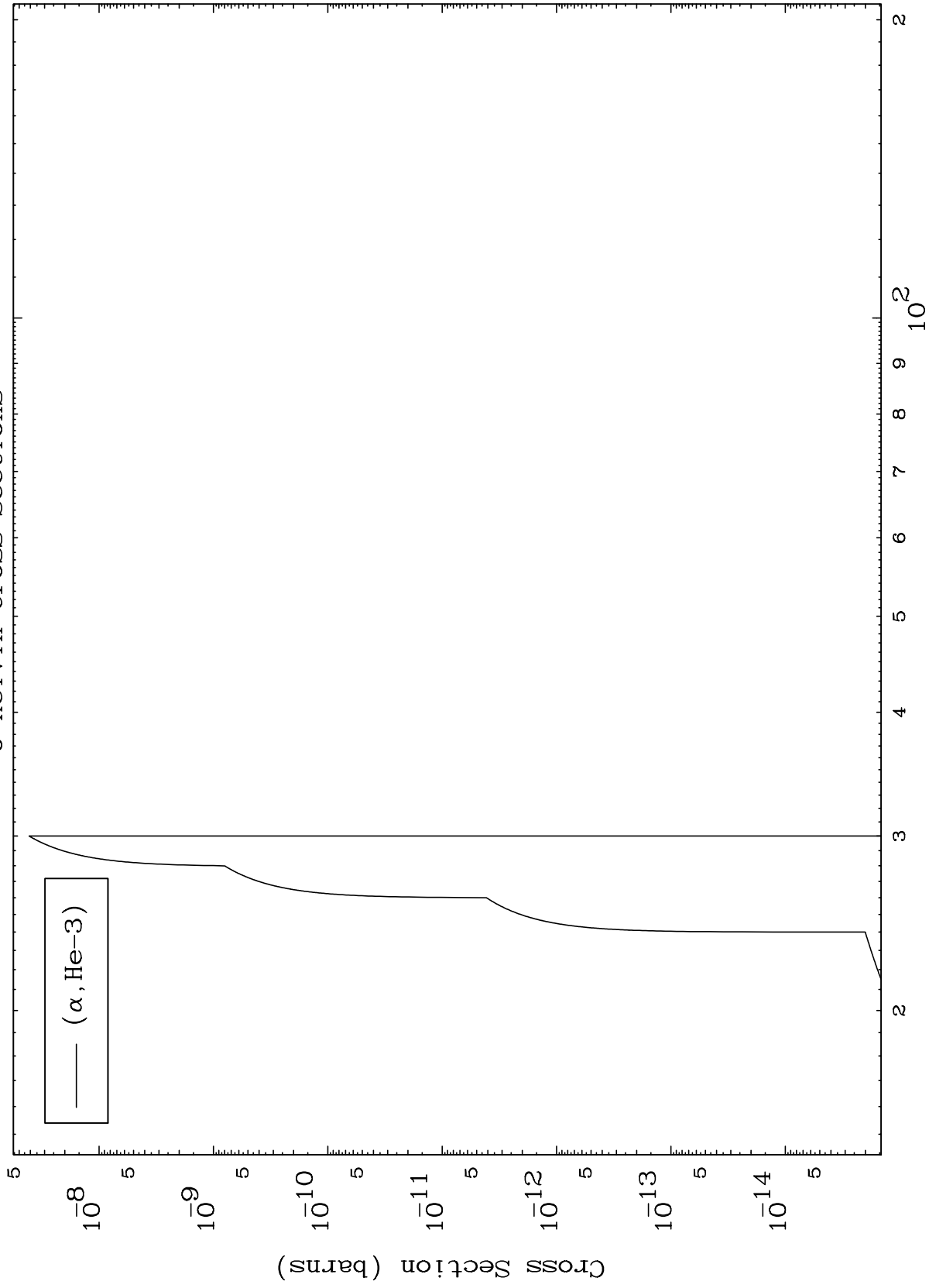
( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

75-Re-196



Incident Energy (MeV)

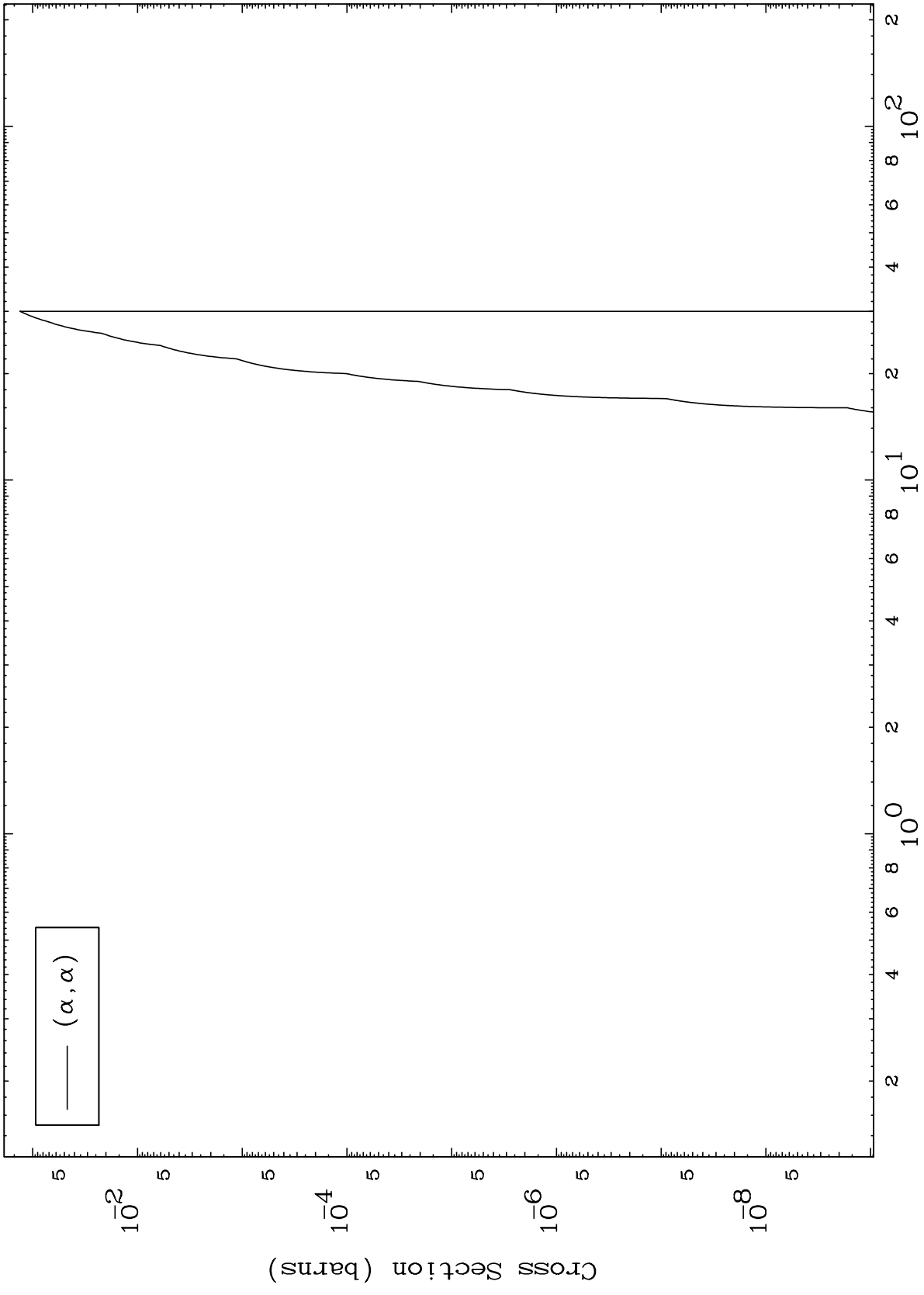
75-Re-196



MAT 7558

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

75-Re-196

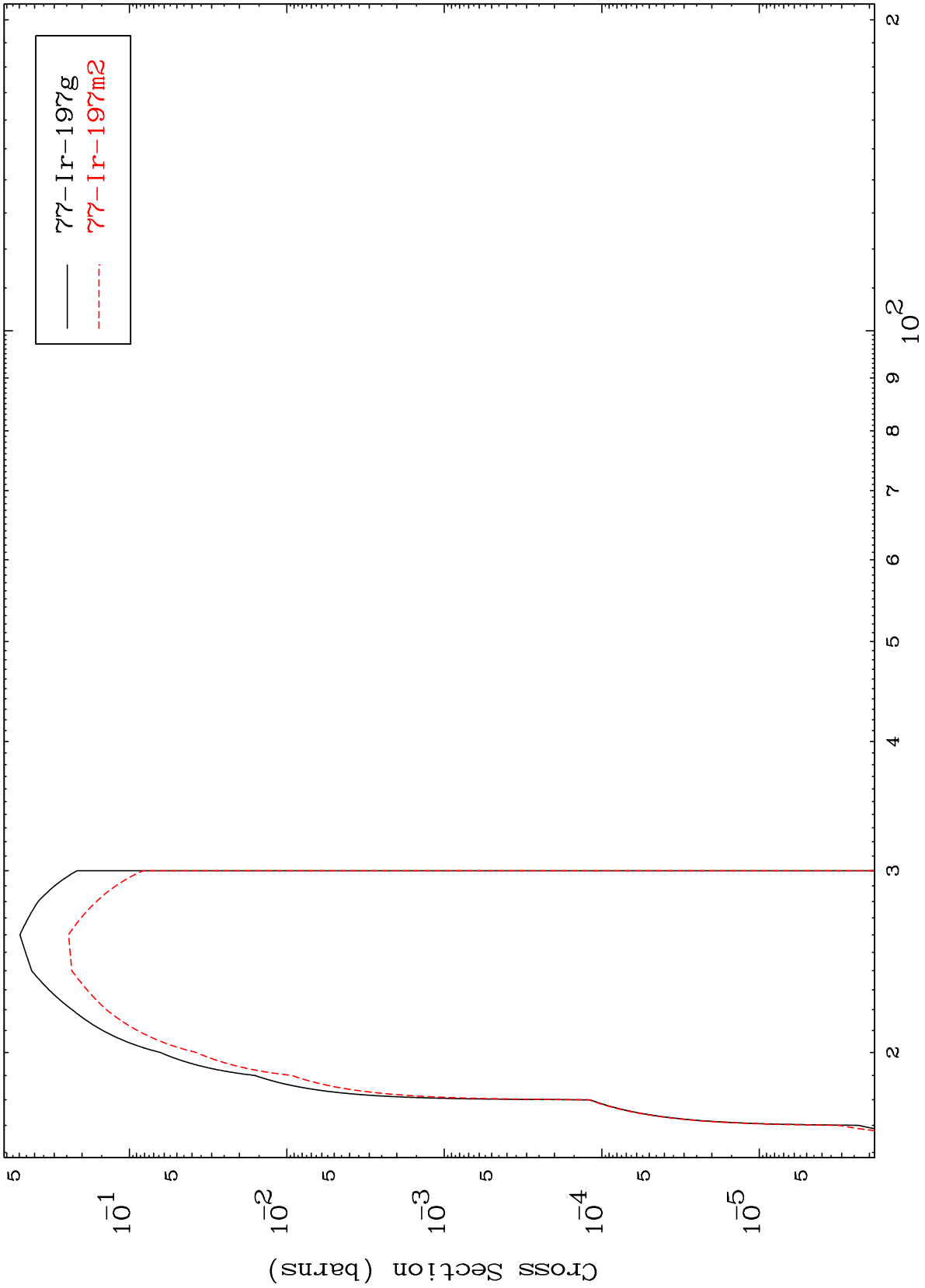


10

Incident Energy (MeV)

75-Re-196

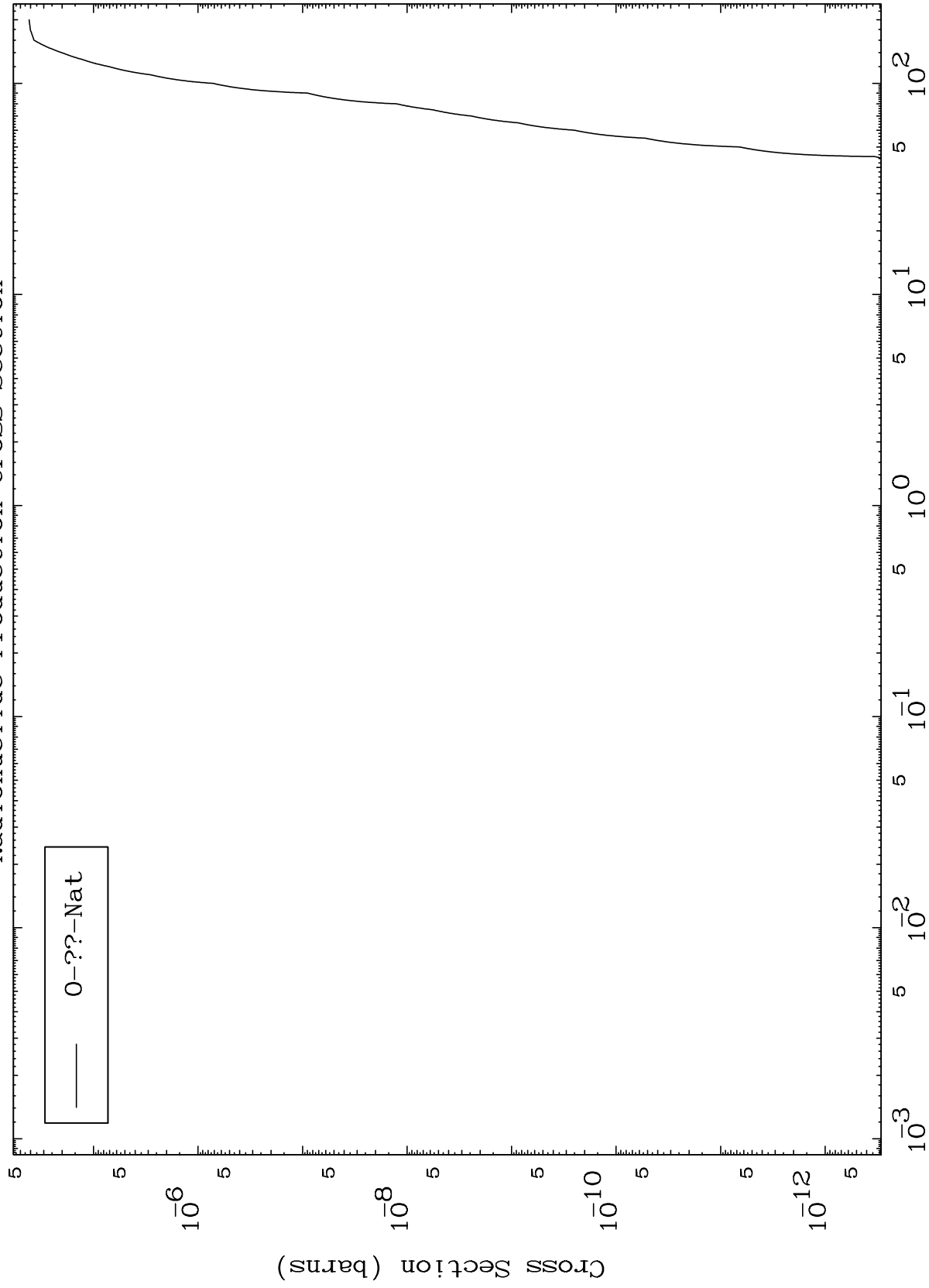
Radionuclide Production Cross Section



MAT 7558

$\alpha$  Fission  
Radionuclide Production Cross Section

75-Re-196



12

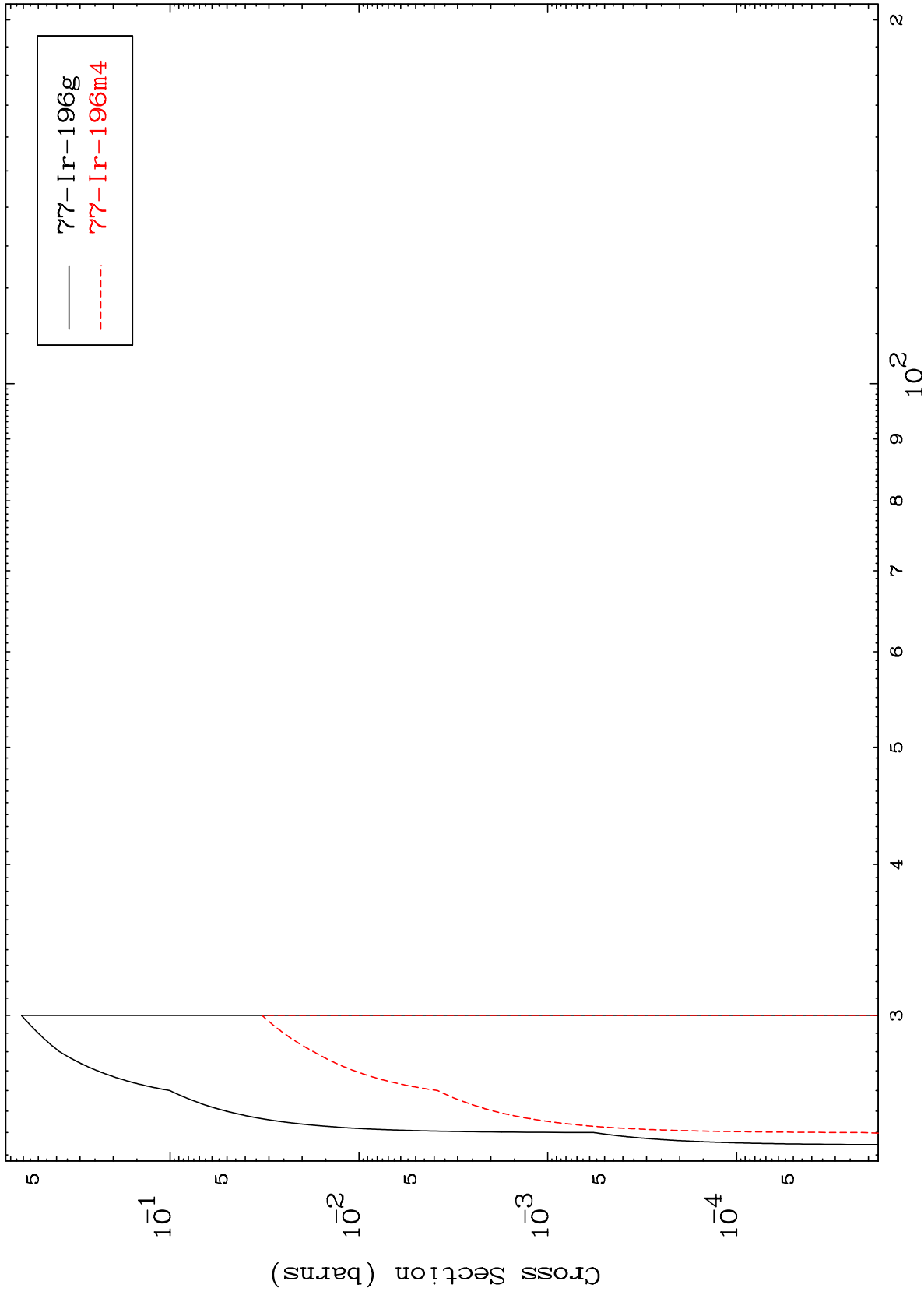
Incident Energy (MeV)

75-Re-196

MAT 7558

75-Re-196

( $\alpha, 4n$ )  
Radionuclide Production Cross Section



13

Incident Energy (MeV)

75-Re-196