

Program EVALPLOT  
(Version 2018-1)

by

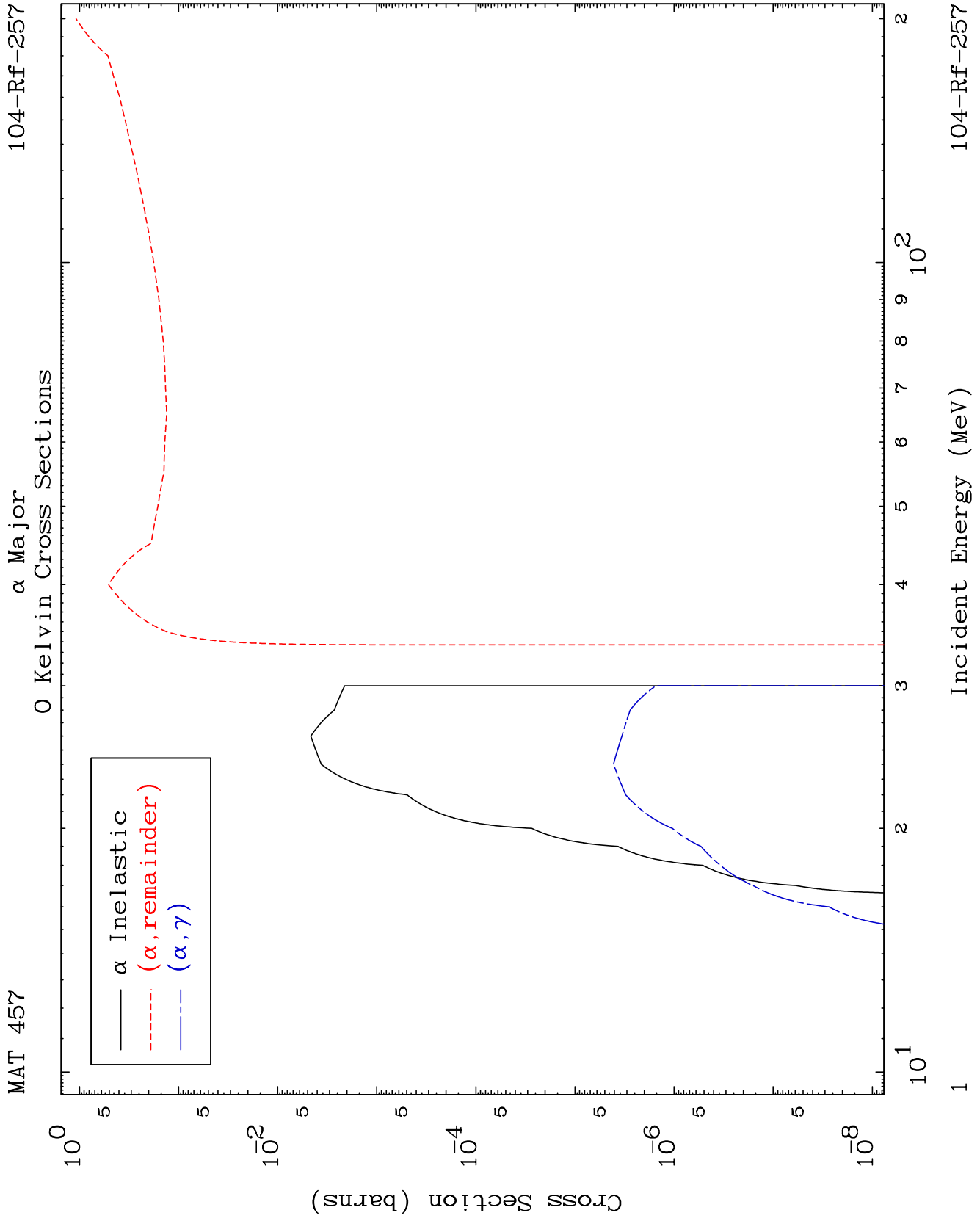
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E.Mail:redcullen1@comcast.net  
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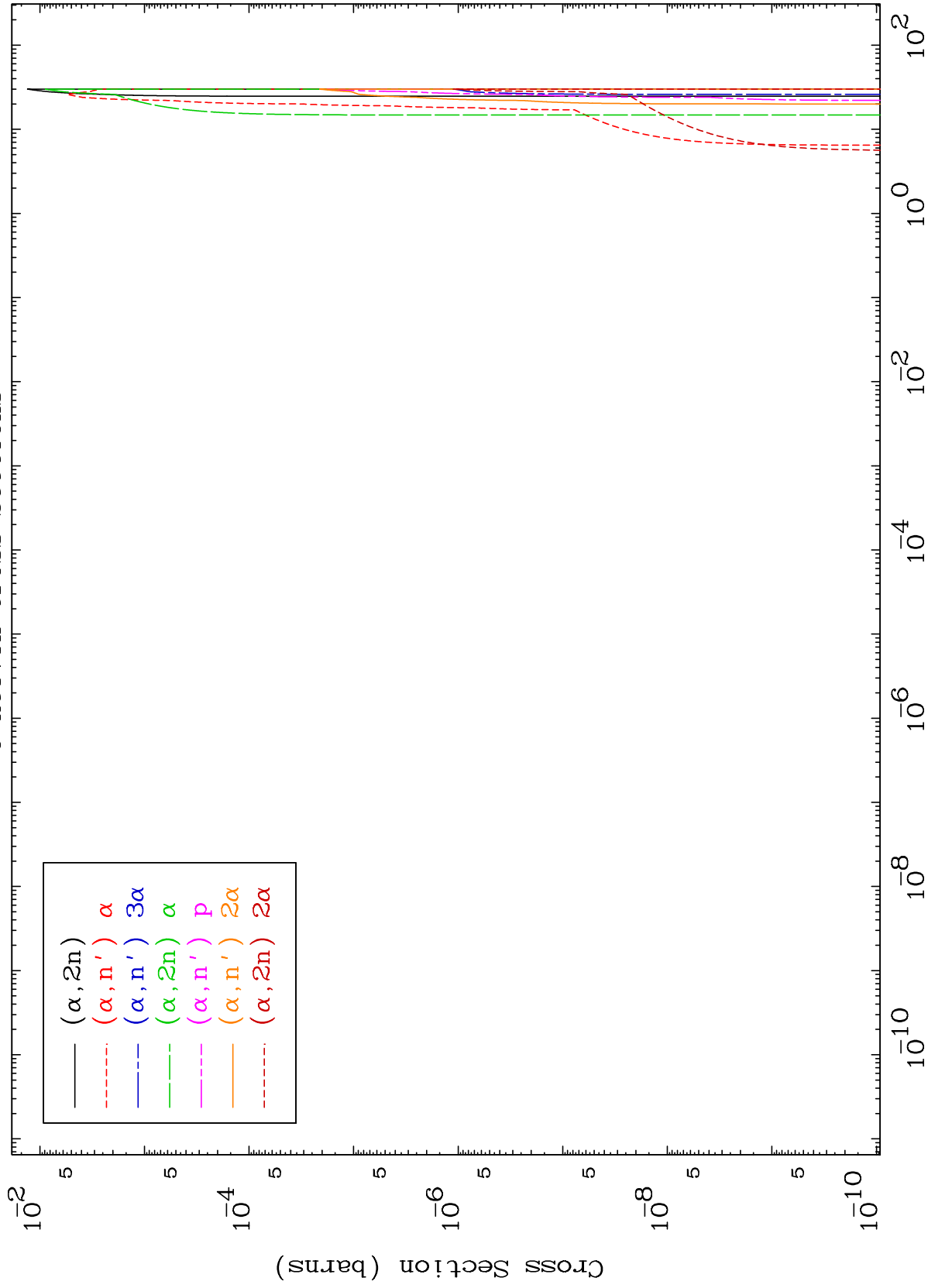
Press Mouse Button to Start



MAT 457

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

104-Rf-257



2

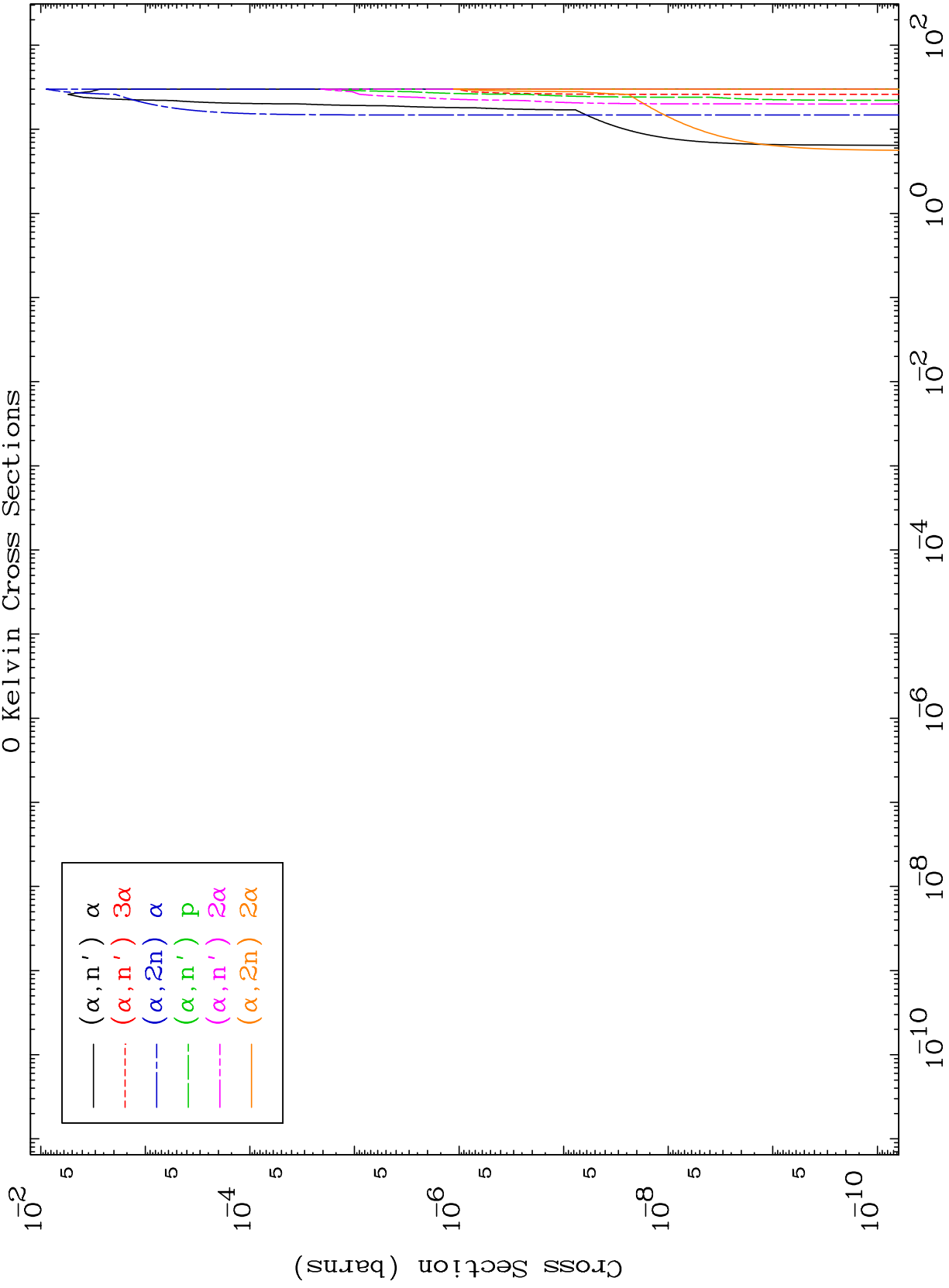
Incident Energy (MeV)

104-Rf-257

MAT 457

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

104-Rf-257



3

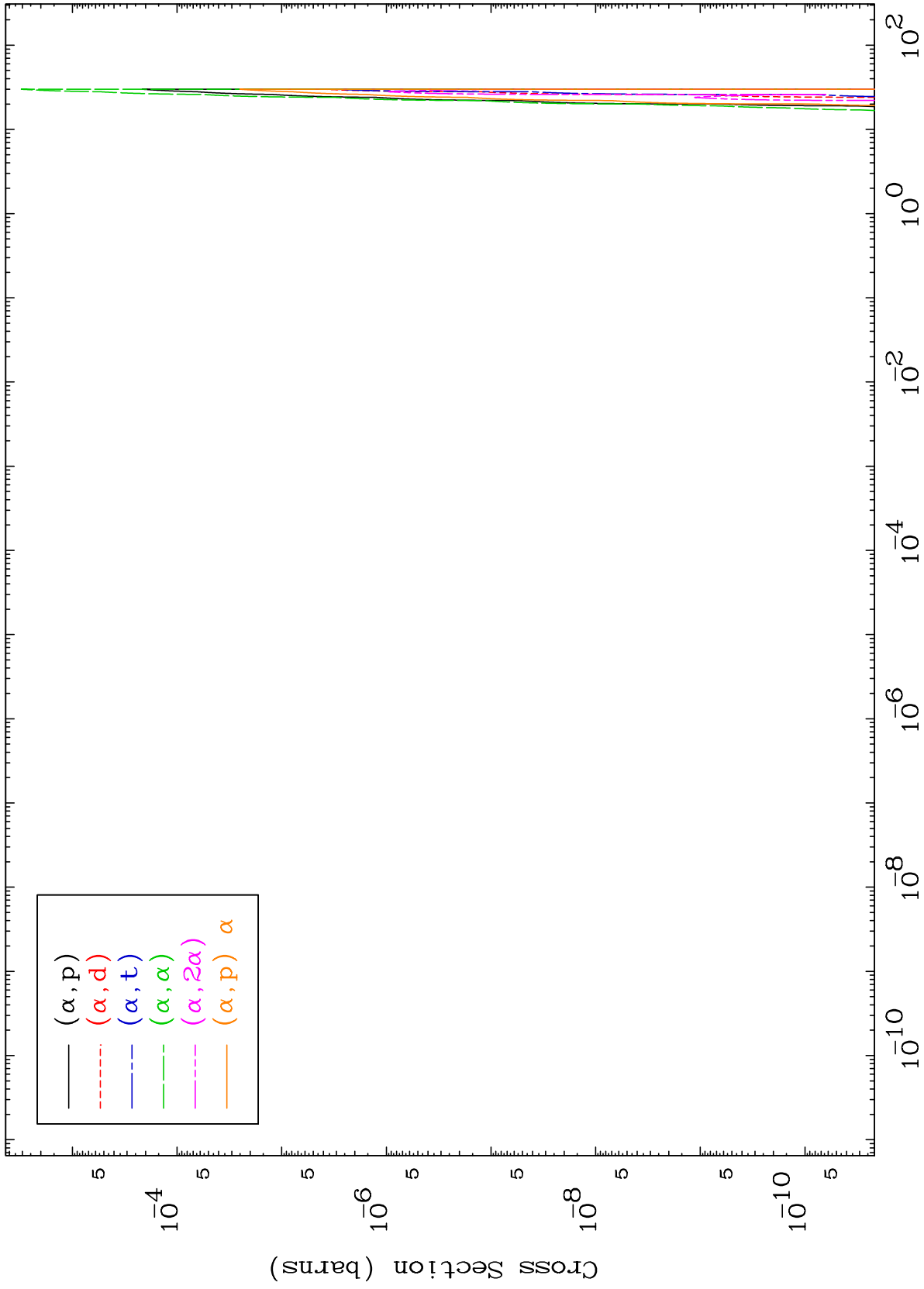
Incident Energy (MeV)

104-Rf-257

MAT 457

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

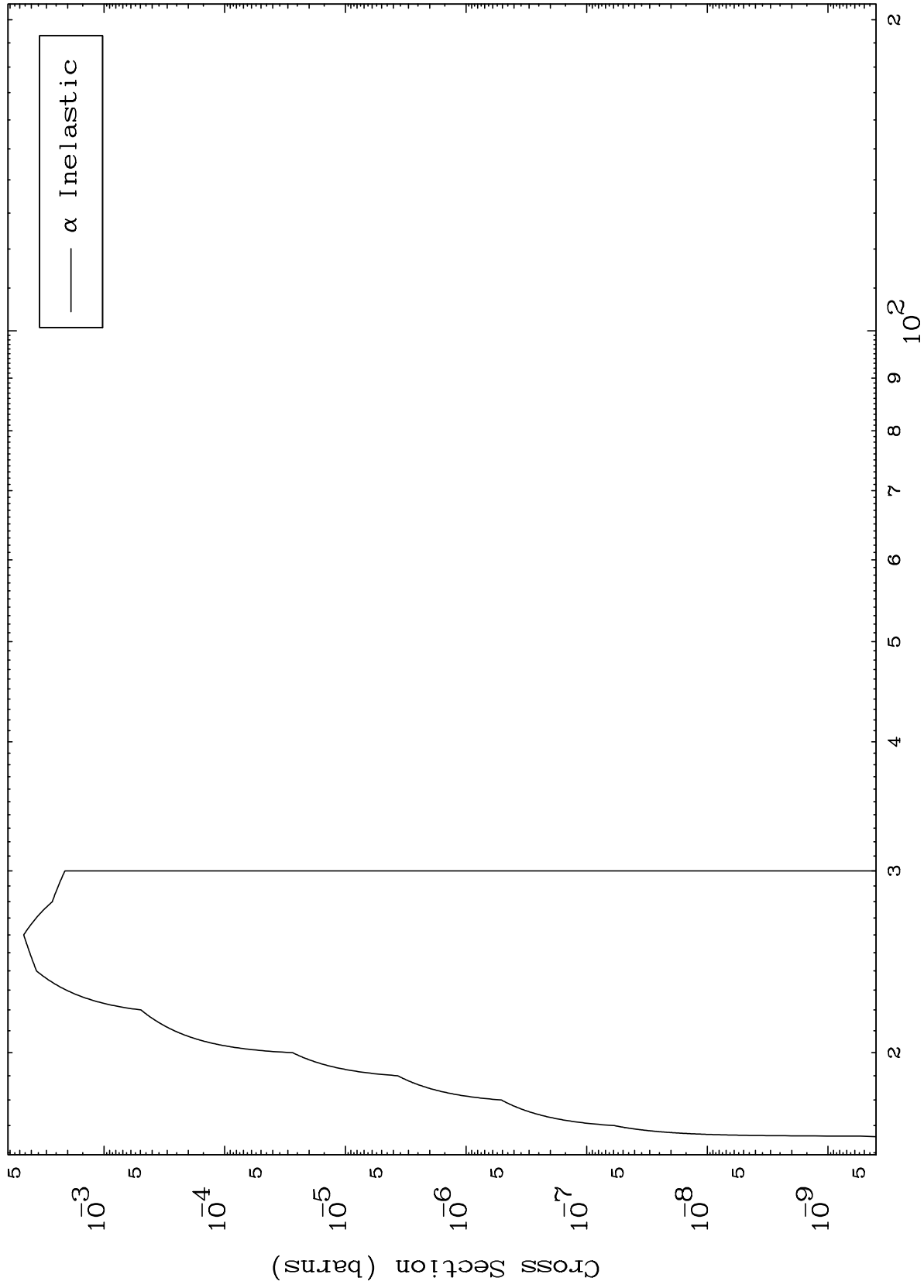
104-Rf-257



MAT 457

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

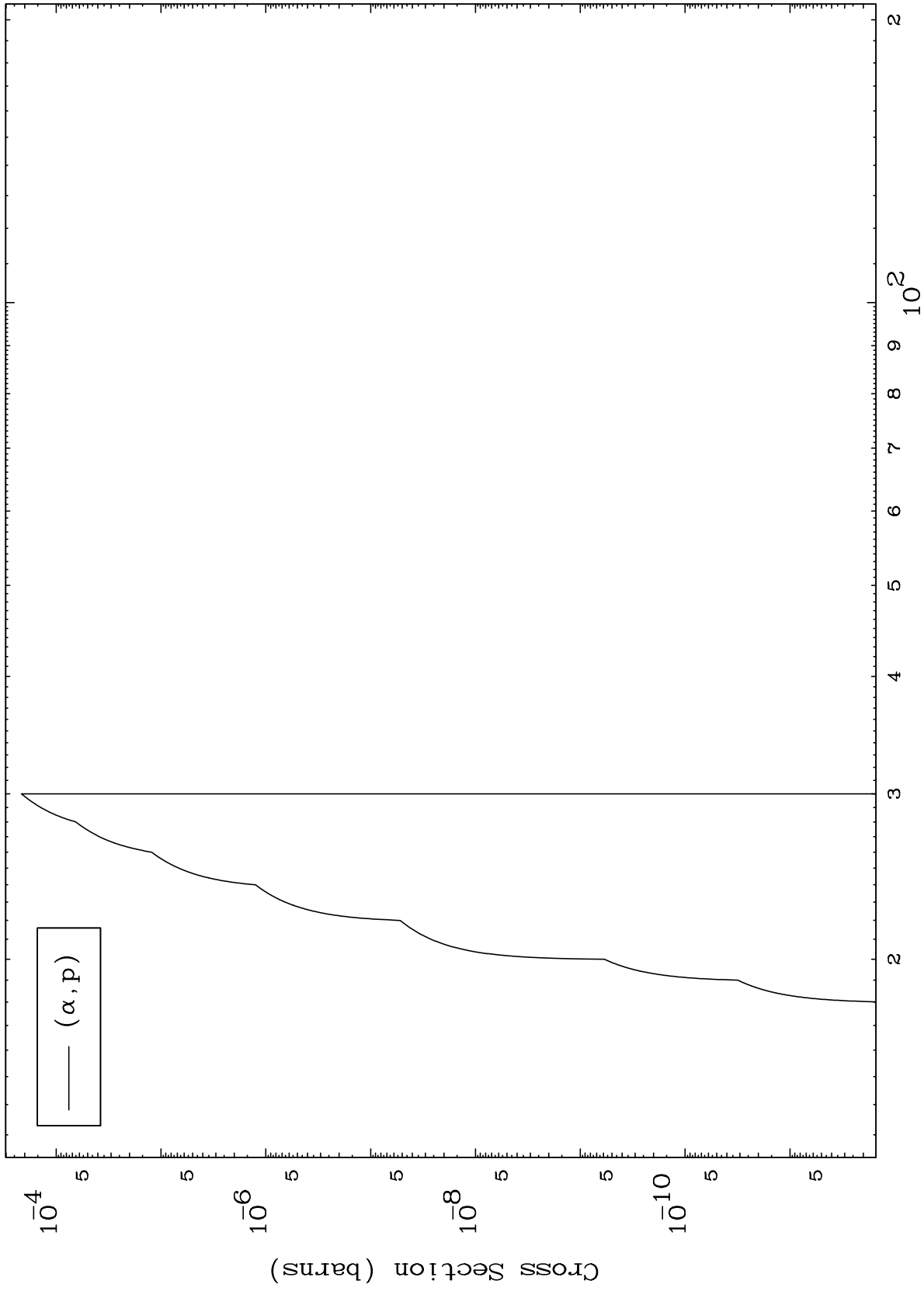
104-Rf-257



5

Incident Energy (MeV)

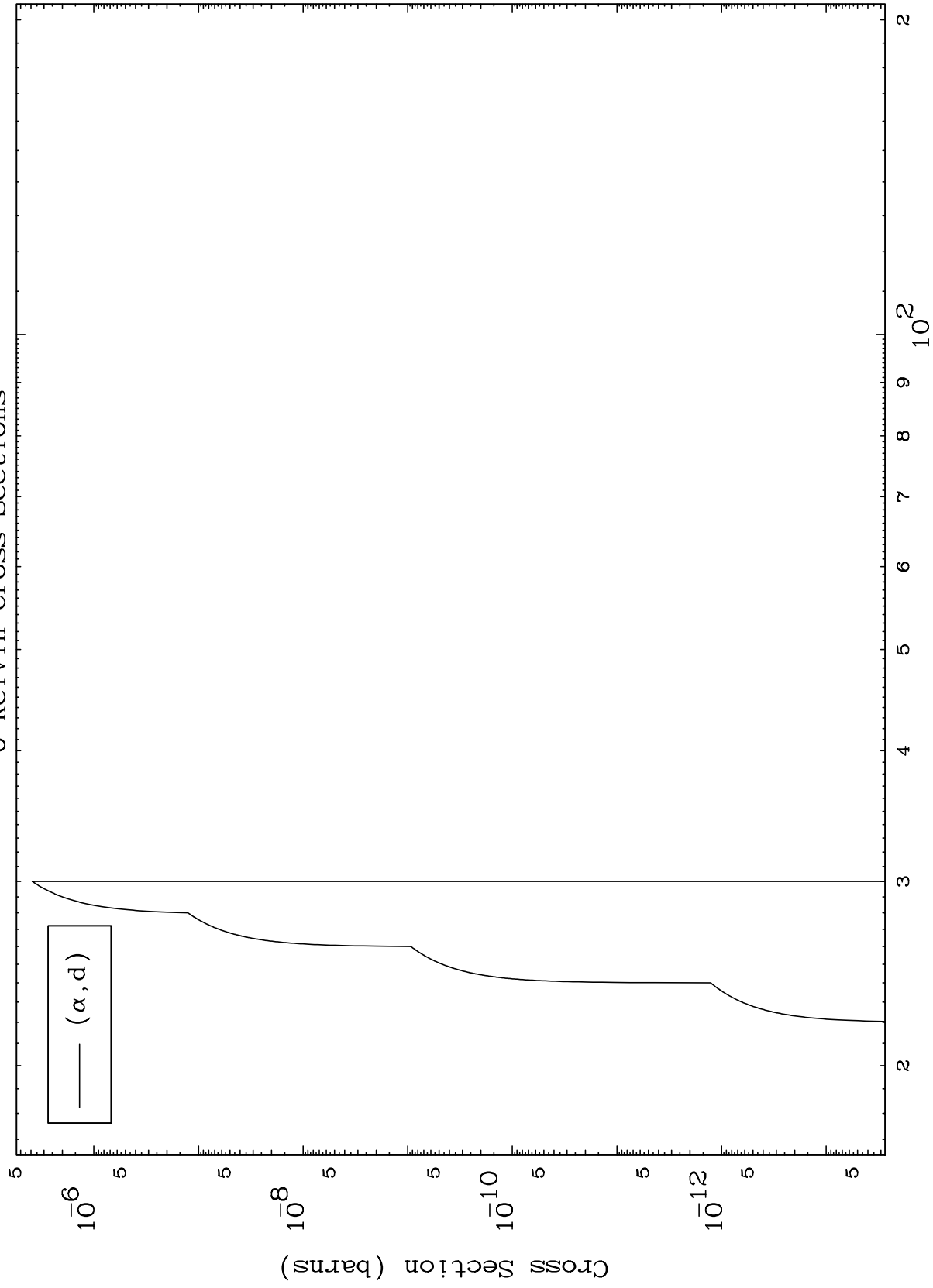
104-Rf-257



MAT 457

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

104-Rf-257



7

Incident Energy (MeV)

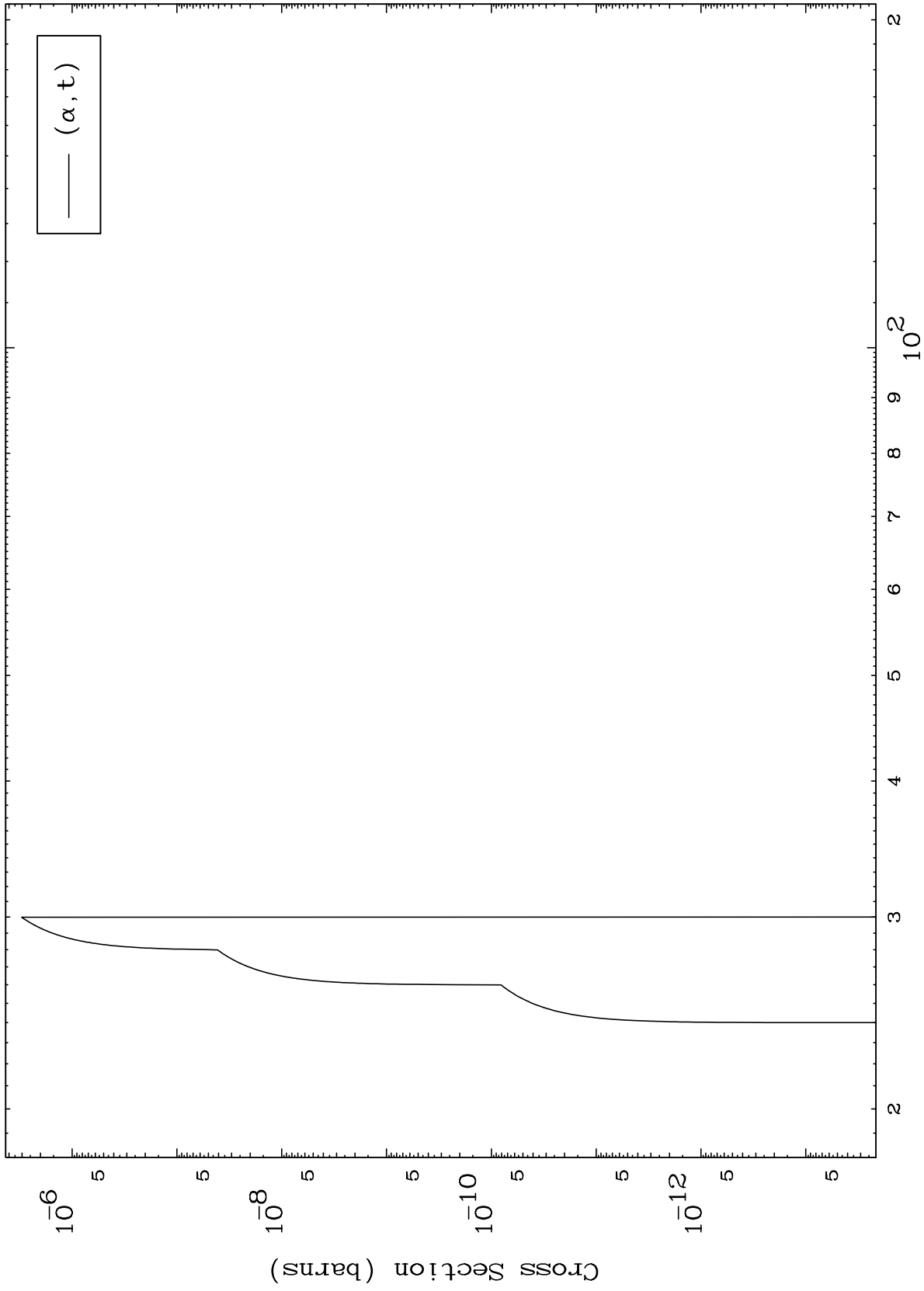
104-Rf-257



MAT 457

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

104-Rf-257



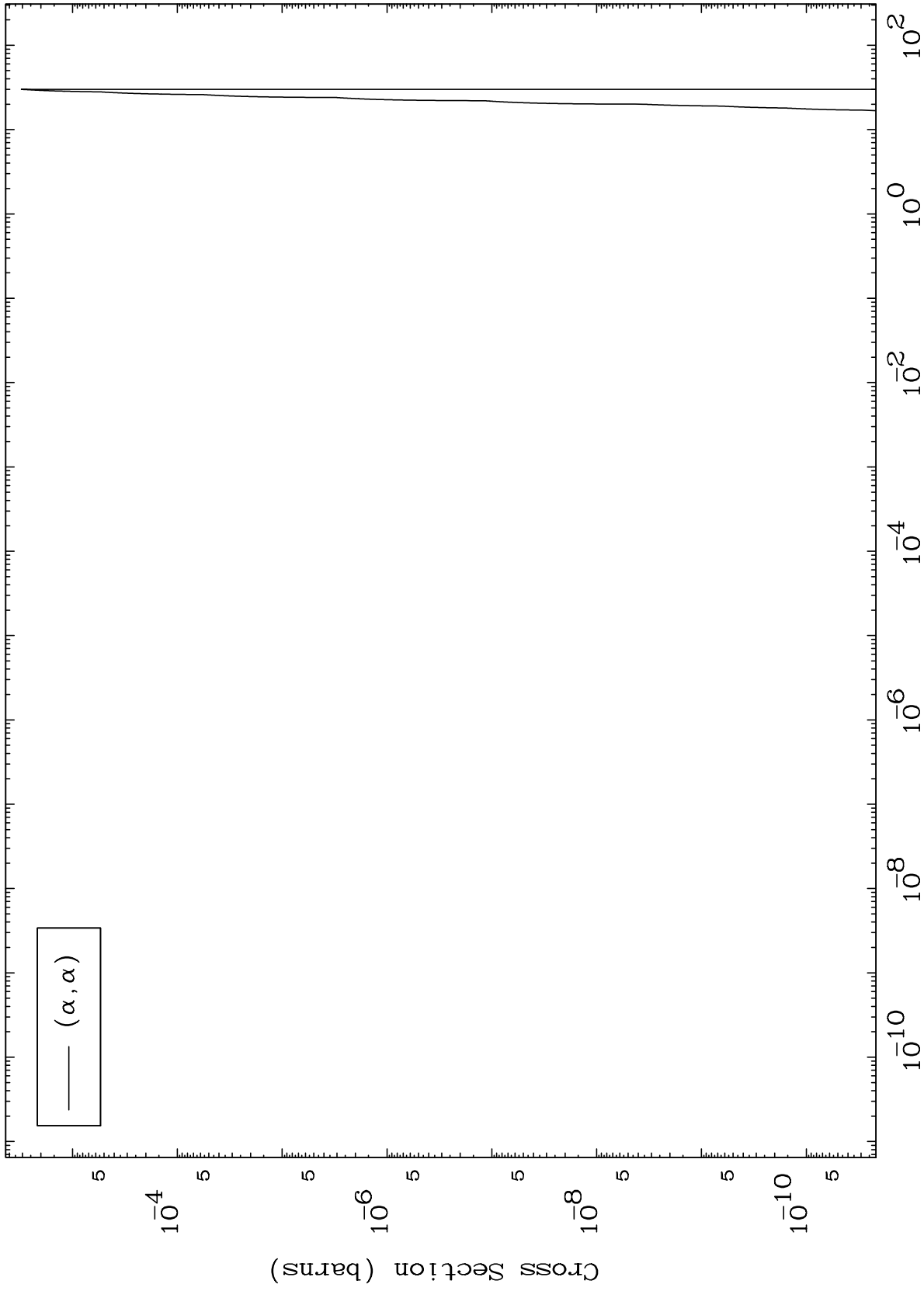
8

104-Rf-257

MAT 457

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

104-Rf-257



9

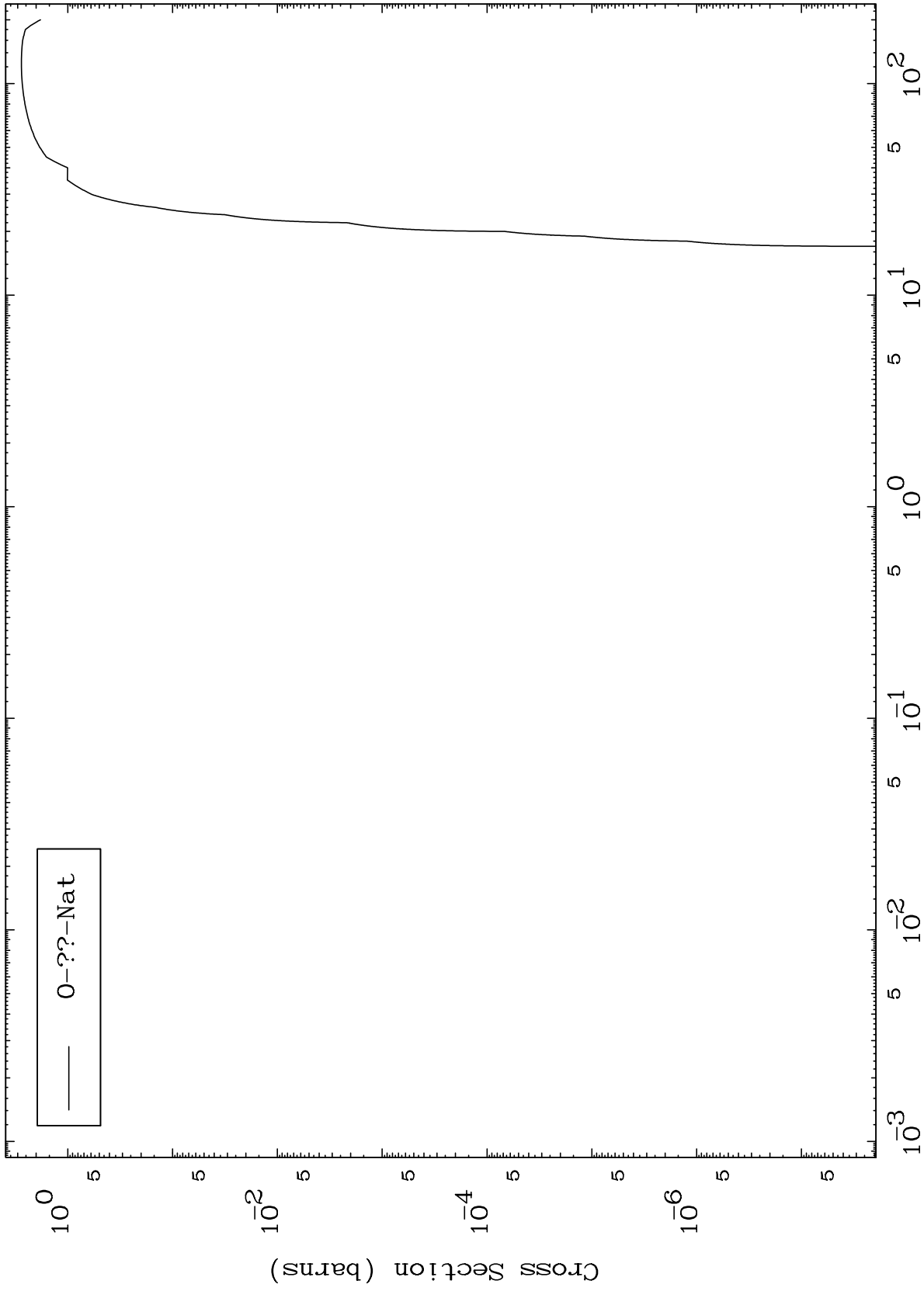
Incident Energy (MeV)

104-Rf-257

MAT 457

$\alpha$  Fission  
Radionuclide Production Cross Section

104-Rf-257



Incident Energy (MeV)

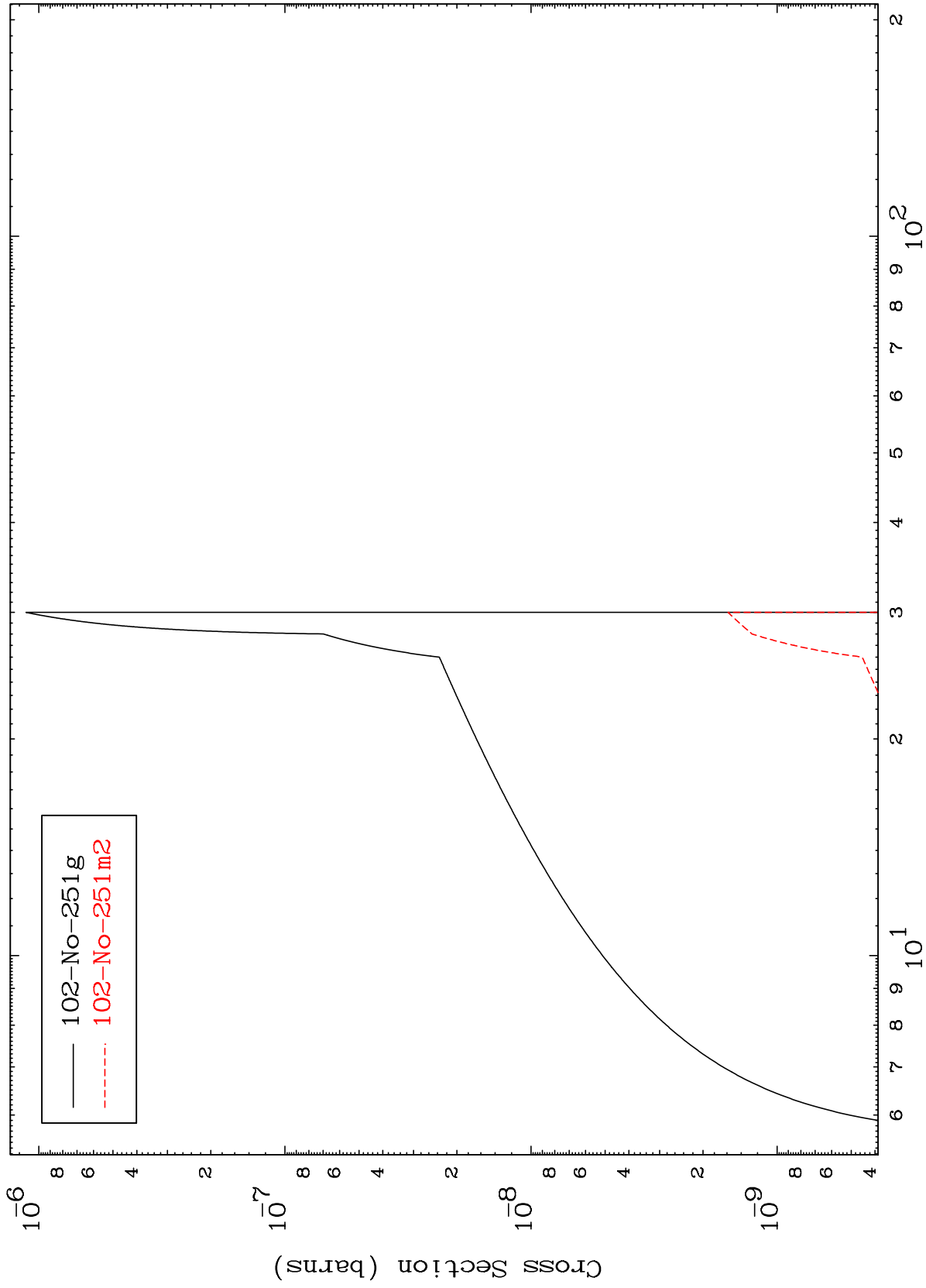
104-Rf-257

MAT 457

$(\alpha, 2n) 2\alpha$

104-Rf-257

Radionuclide Production Cross Section



102-No-251g  
102-No-251m2

11

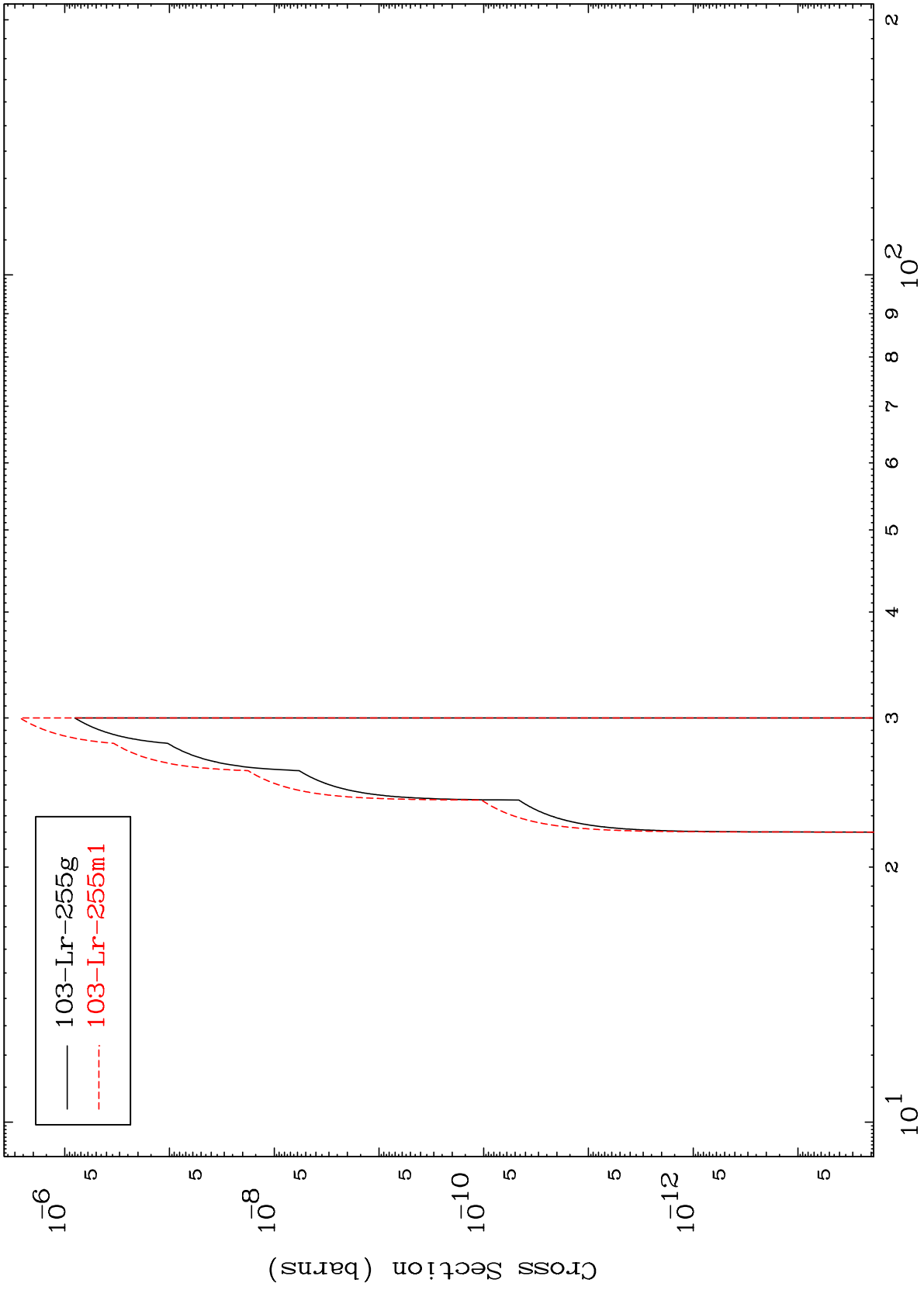
104-Rf-257

MAT 457

( $\alpha, n'$ ) p  $\alpha$

104-Rf-257

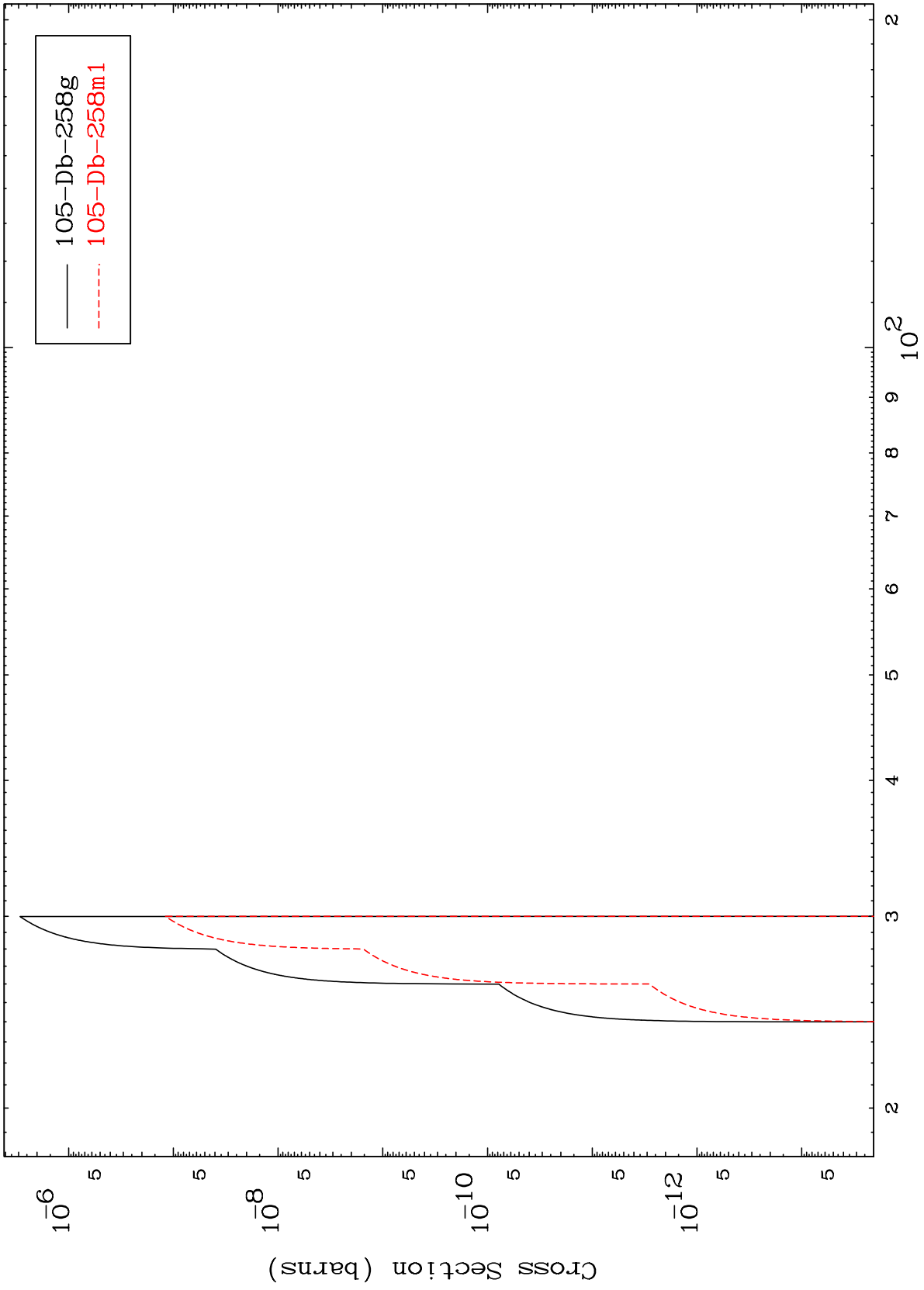
Radionuclide Production Cross Section



MAT 457

104-Rf-257

( $\alpha, t$ )  
Radionuclide Production Cross Section



13

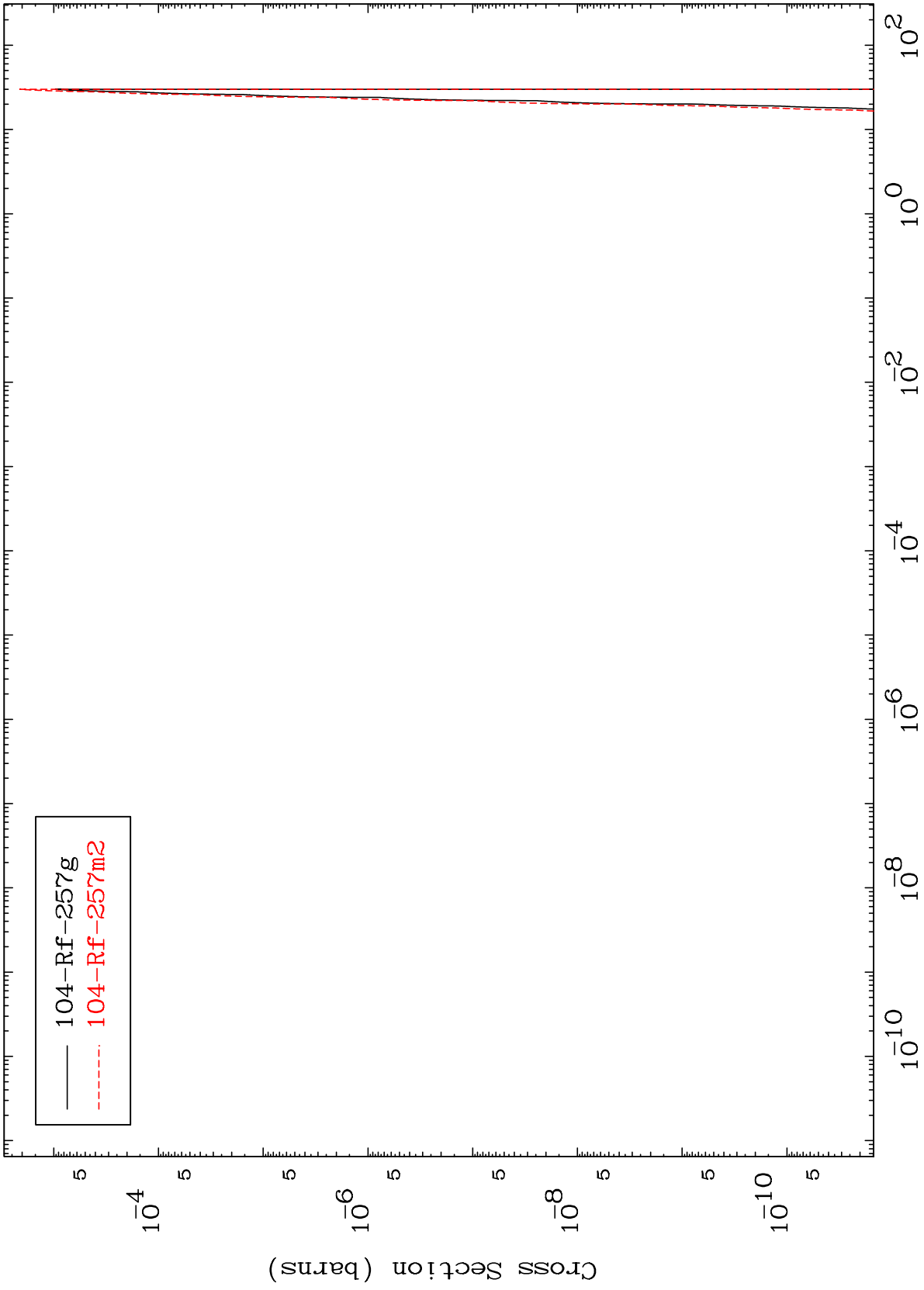
104-Rf-257

MAT 457

( $\alpha, \alpha$ )

104-Rf-257

Radionuclide Production Cross Section



14

Incident Energy (MeV)

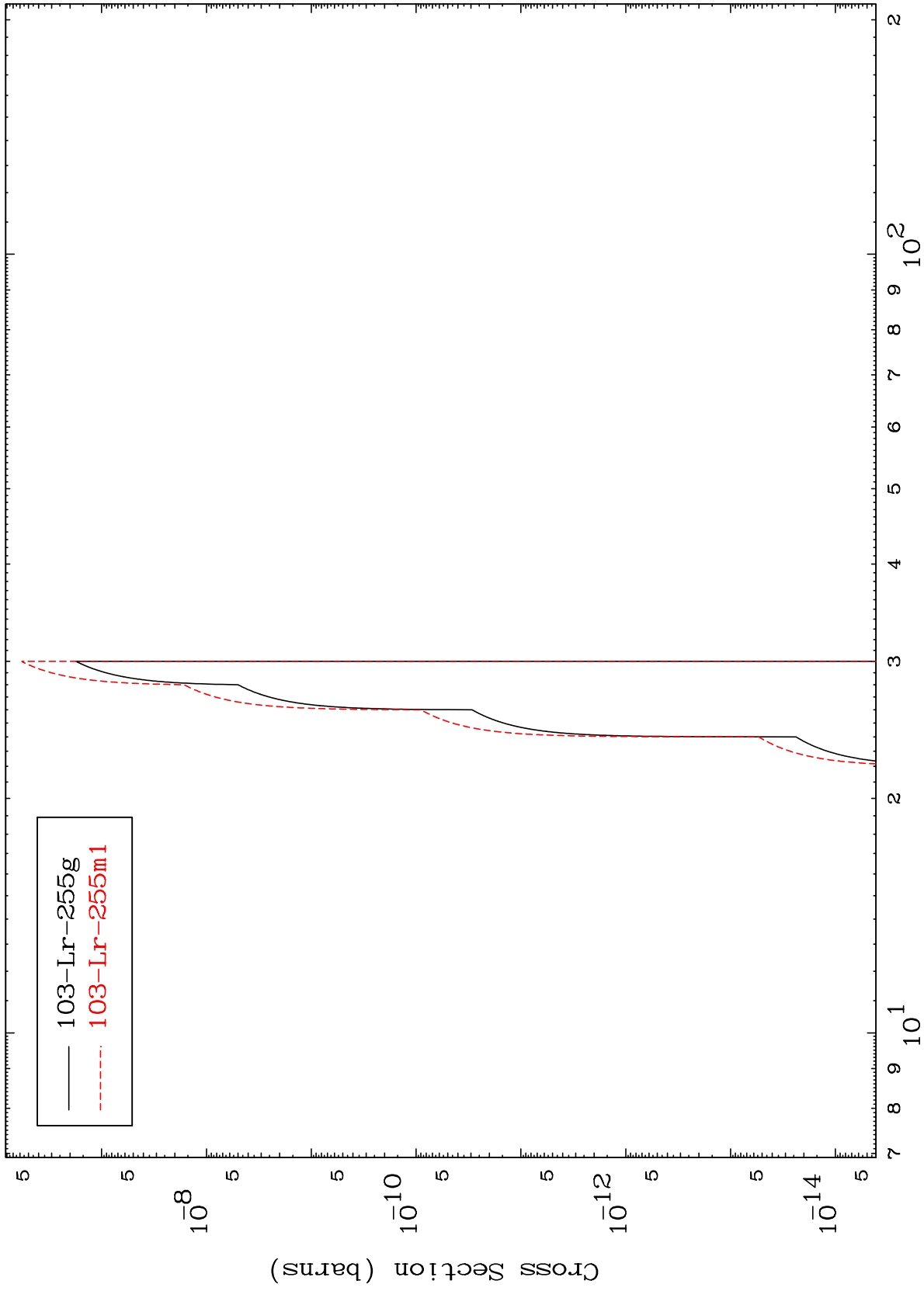
104-Rf-257

MAT 457

( $\alpha, d$ )  $\alpha$

104-Rf-257

Radionuclide Production Cross Section



15

Incident Energy (MeV)

104-Rf-257