

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

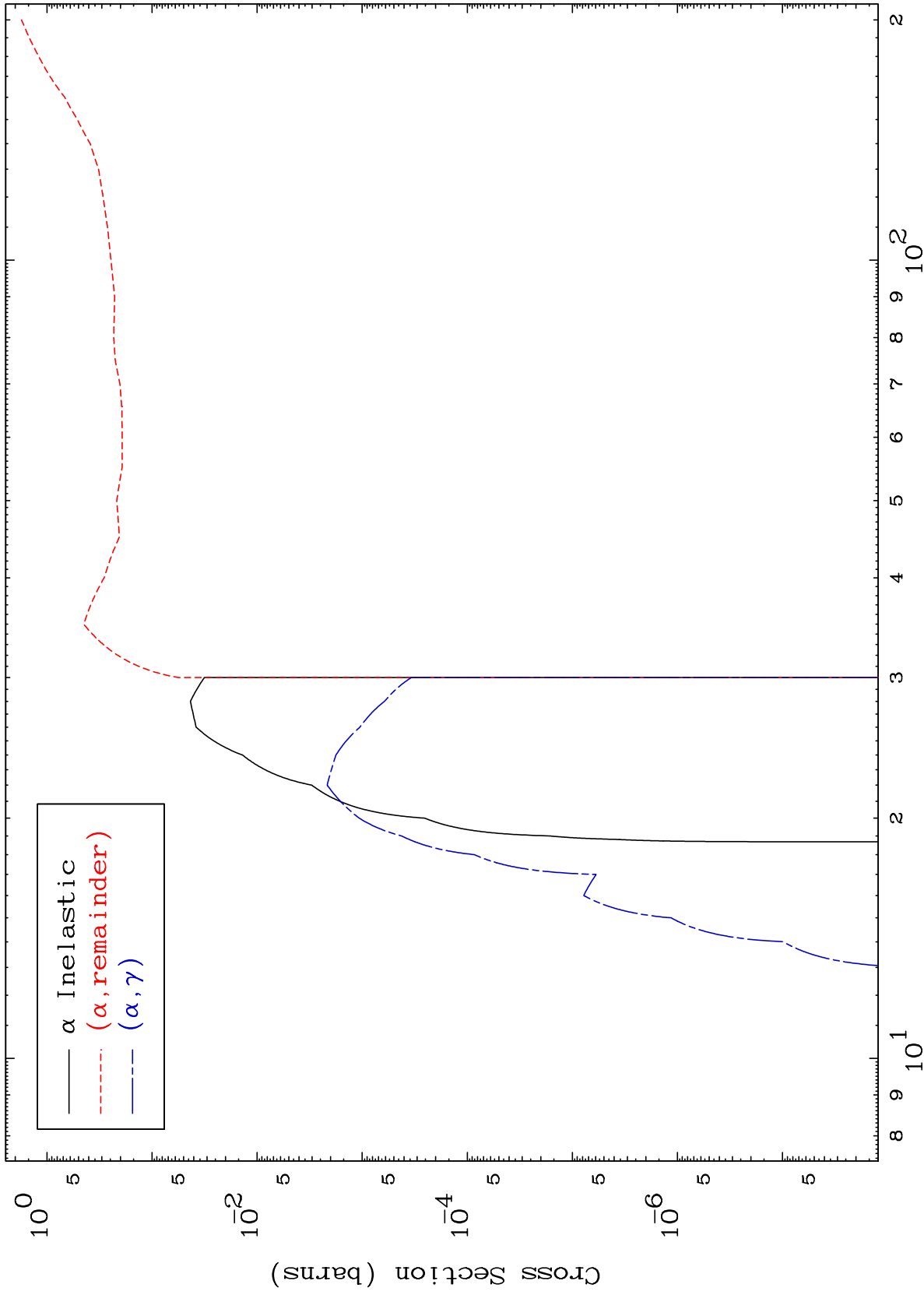
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8592

$\alpha$  Major  
0 Kelvin Cross Sections

86-Rn-200



1

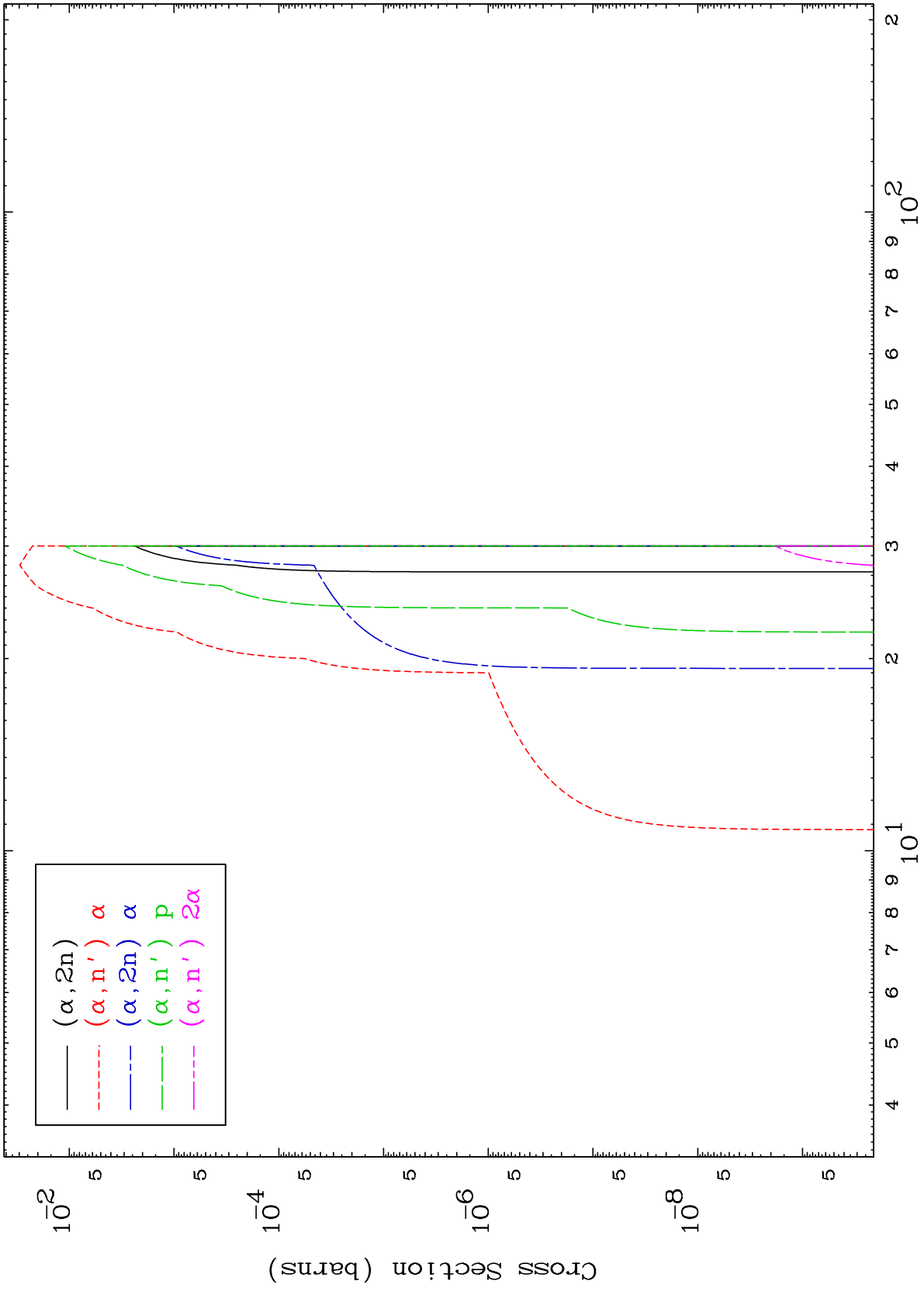
Incident Energy (MeV)

86-Rn-200

MAT 8592

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

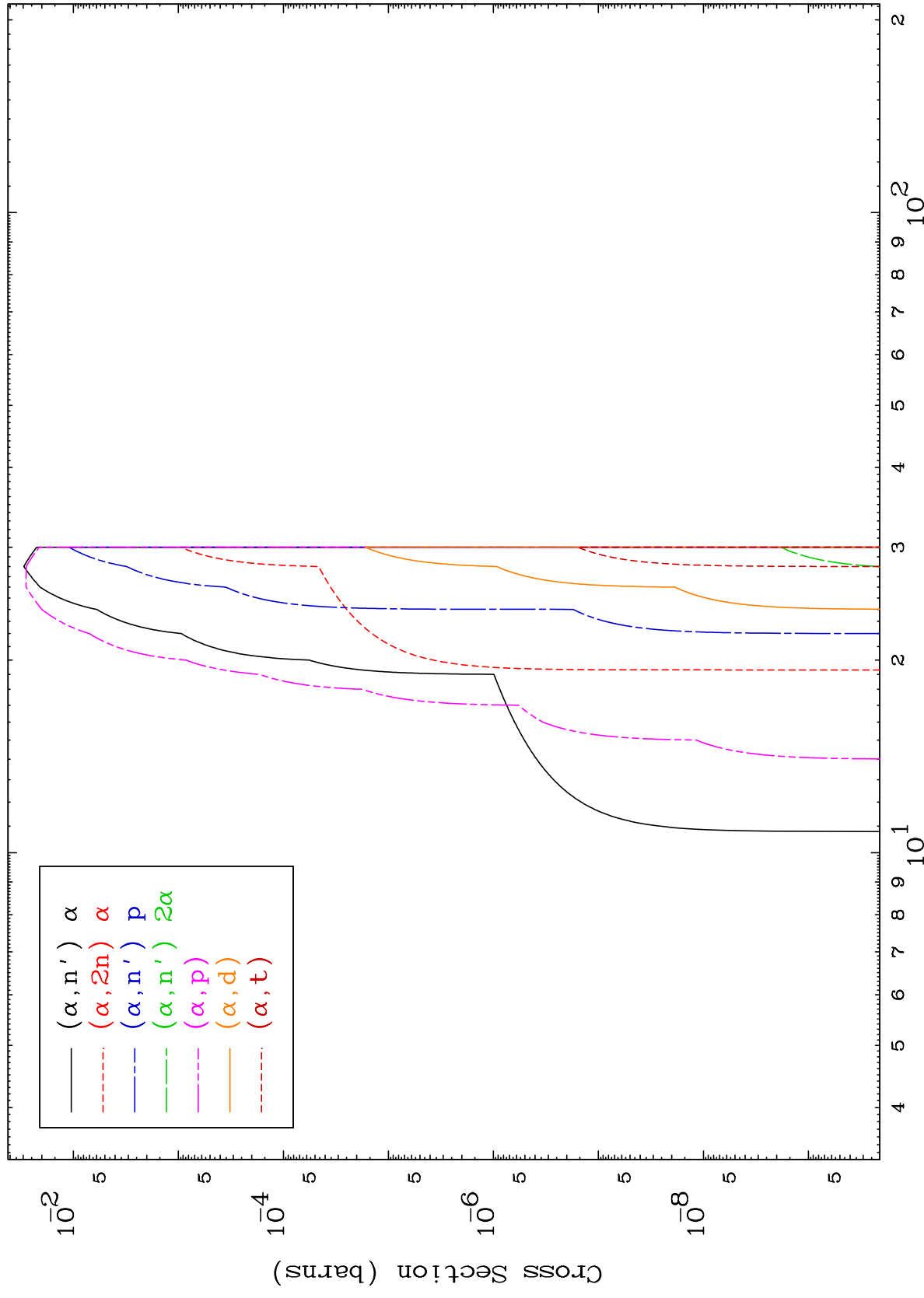
86-Rn-200



2

Incident Energy (MeV)

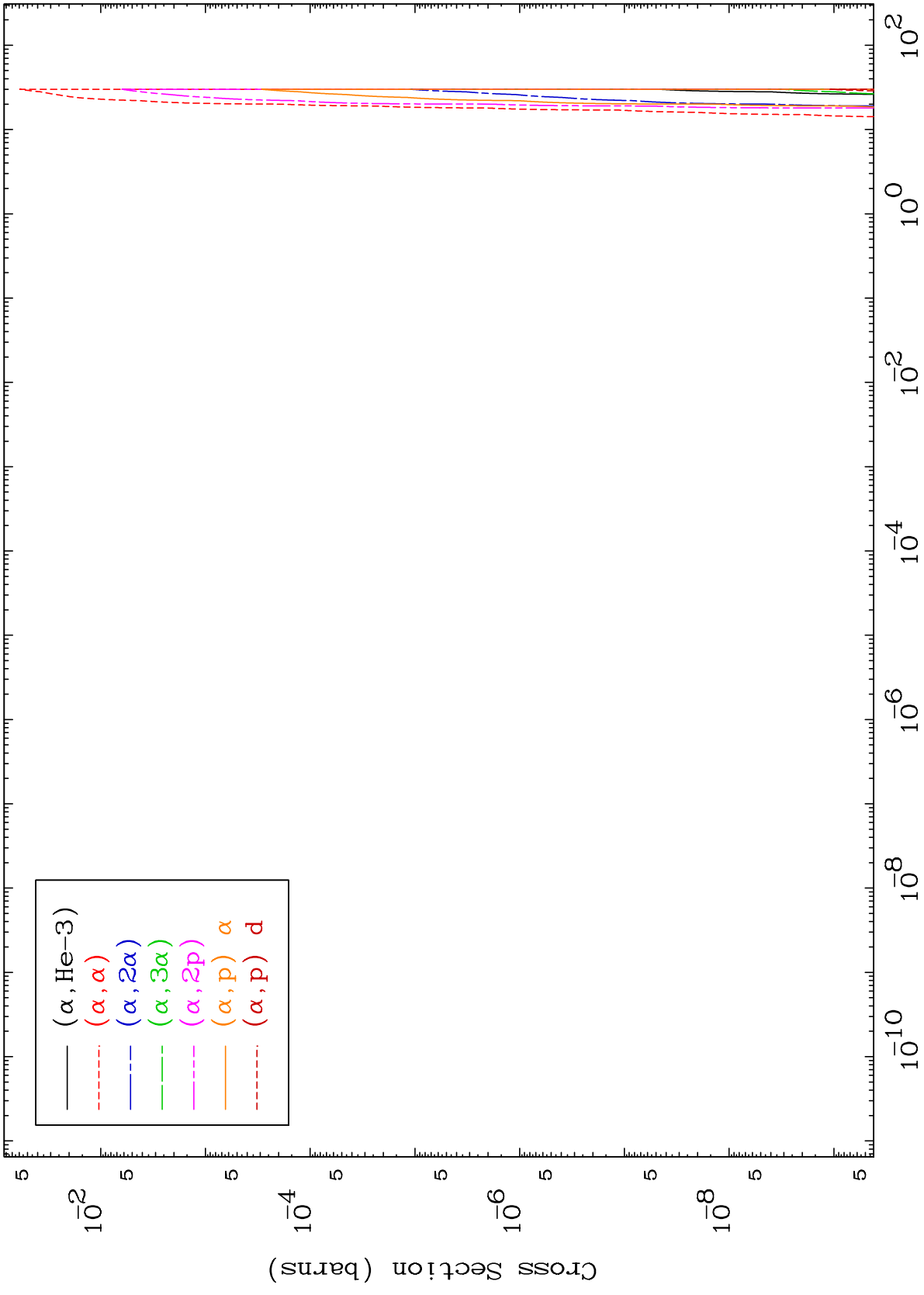
86-Rn-200



MAT 8592

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

86-Rn-200

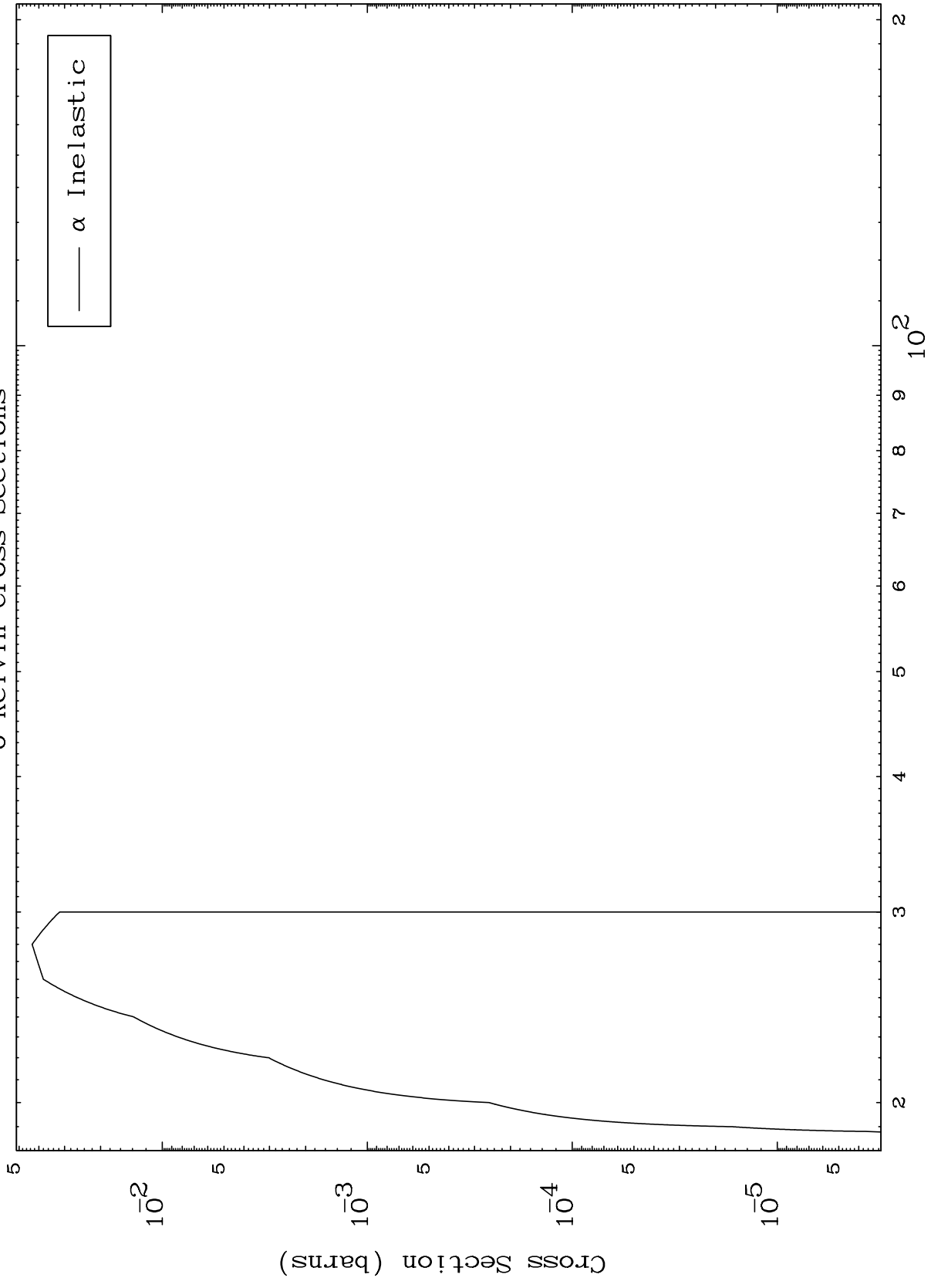


86-Rn-200

MAT 8592

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

86-Rn-200



5

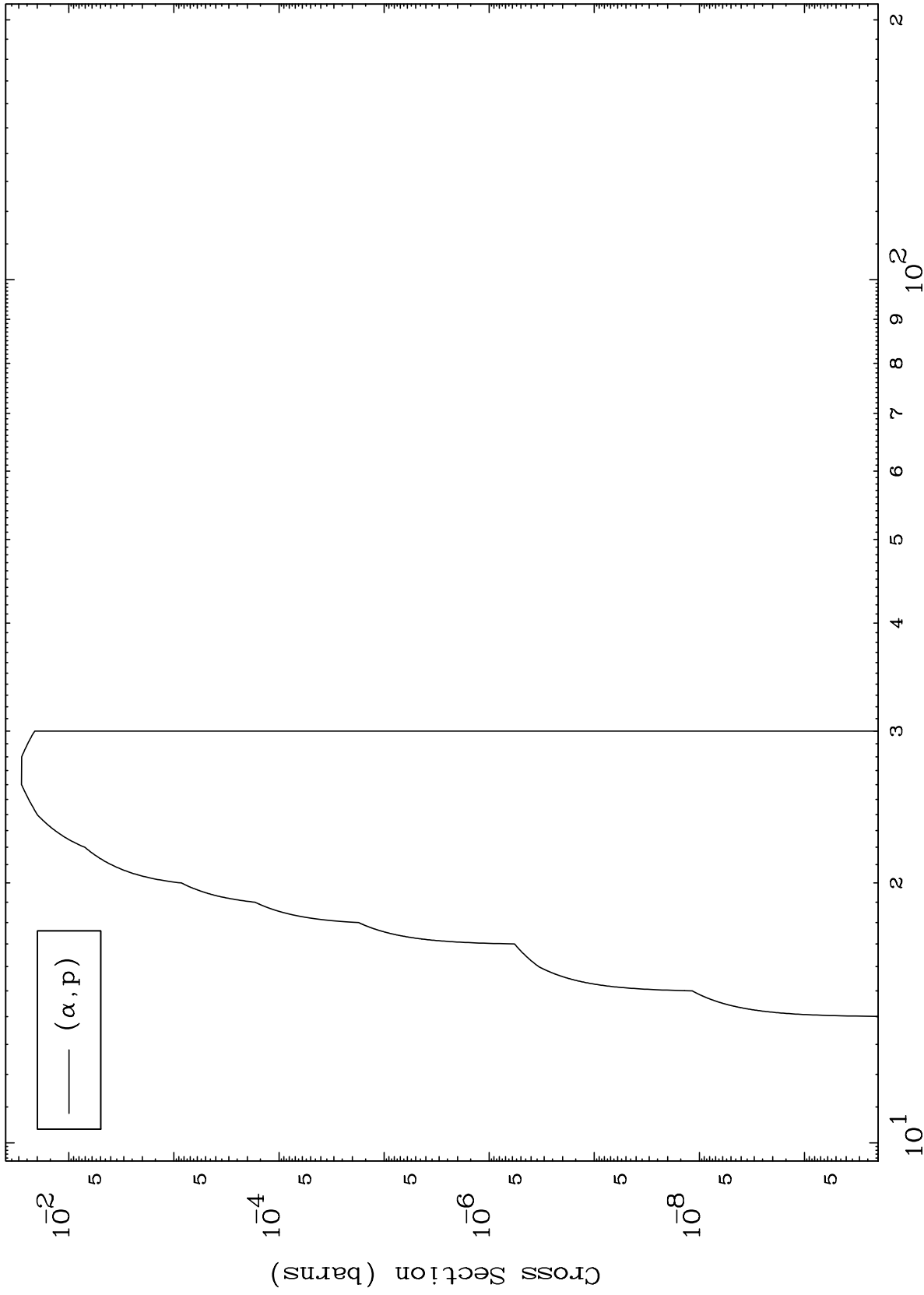
Incident Energy (MeV)

86-Rn-200

MAT 8592

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

86-Rn-200



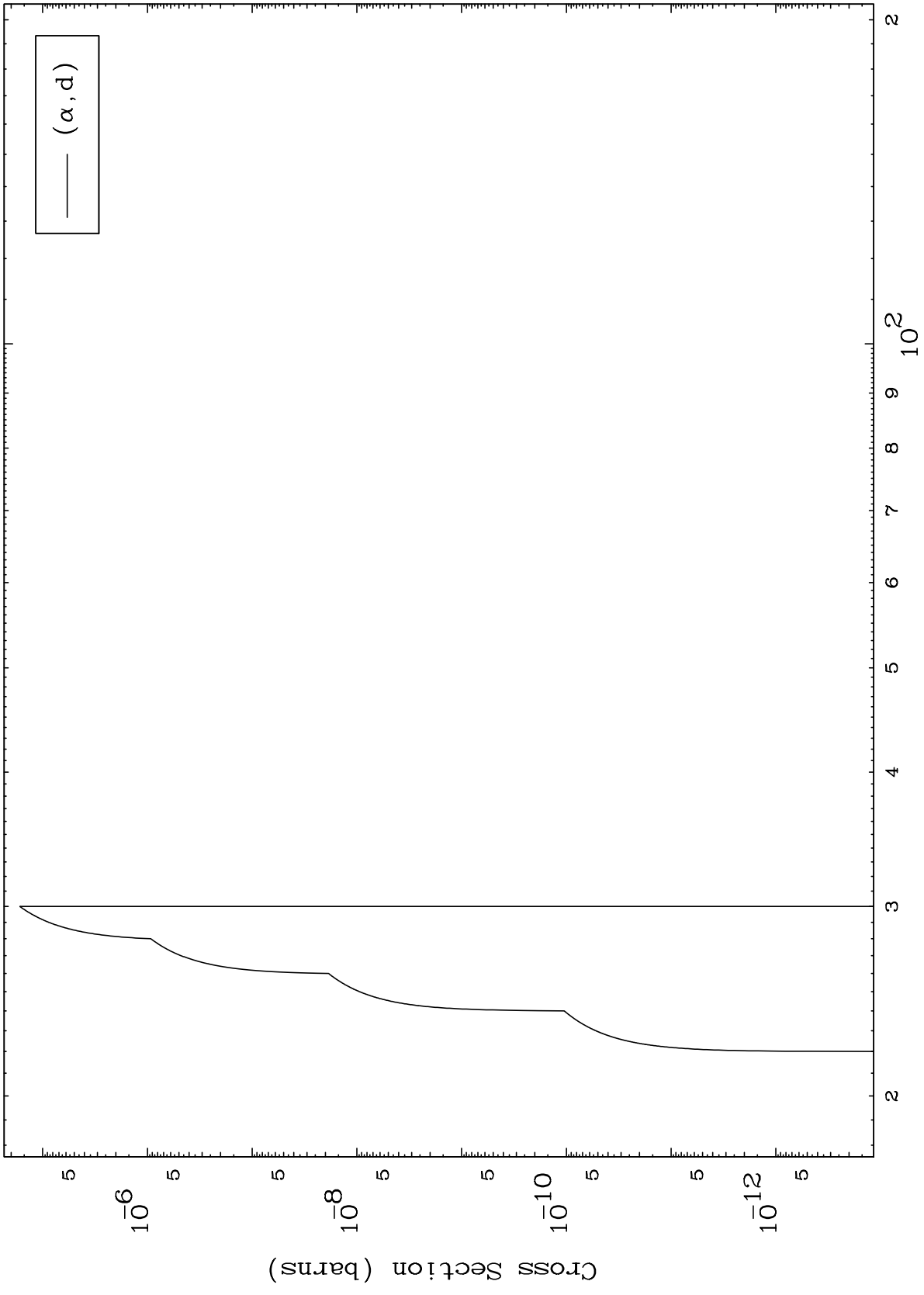
Incident Energy (MeV)

86-Rn-200

MAT 8592

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

86-Rn-200



7

Incident Energy (MeV)

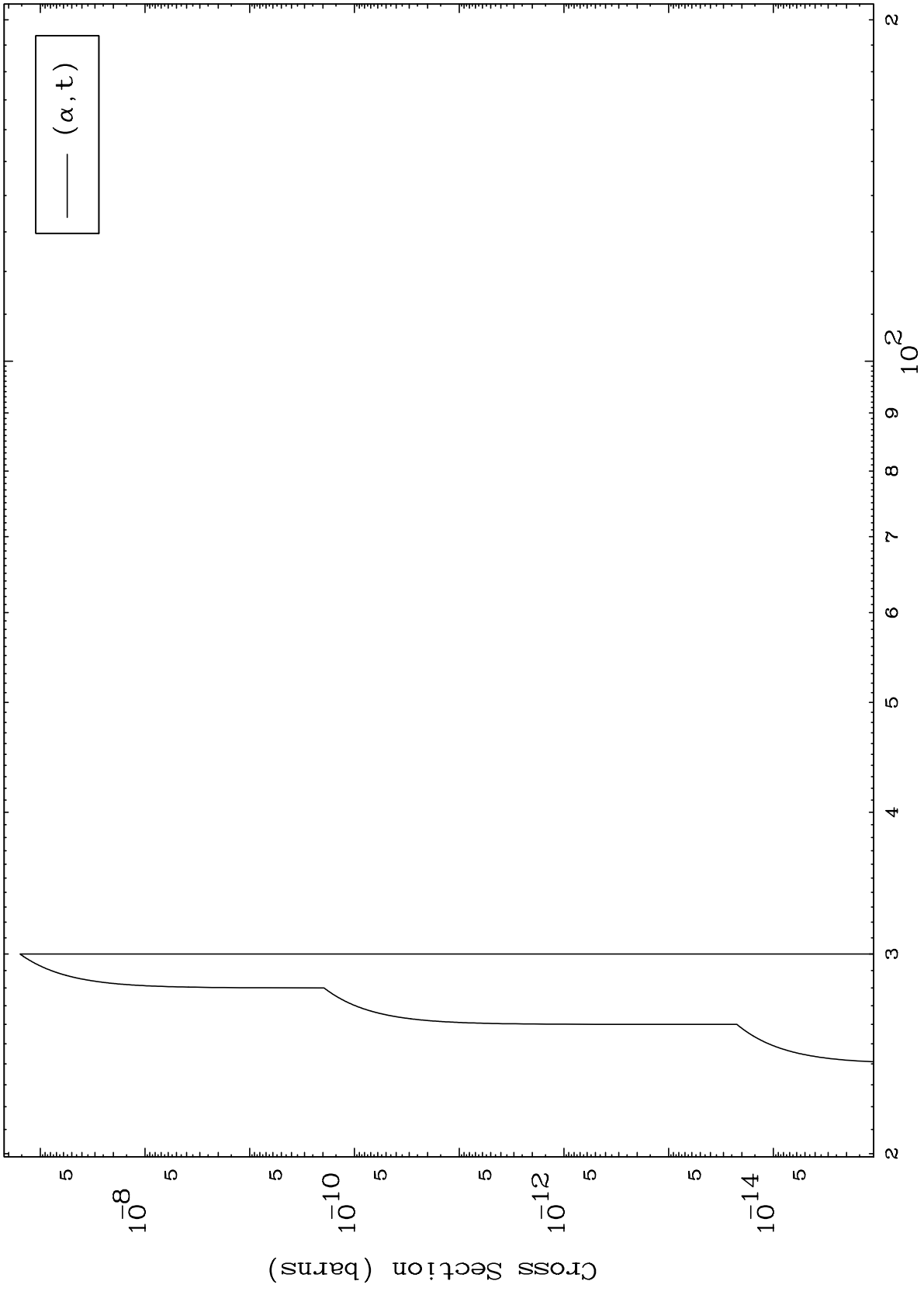
86-Rn-200



MAT 8592

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

86-Rn-200



8

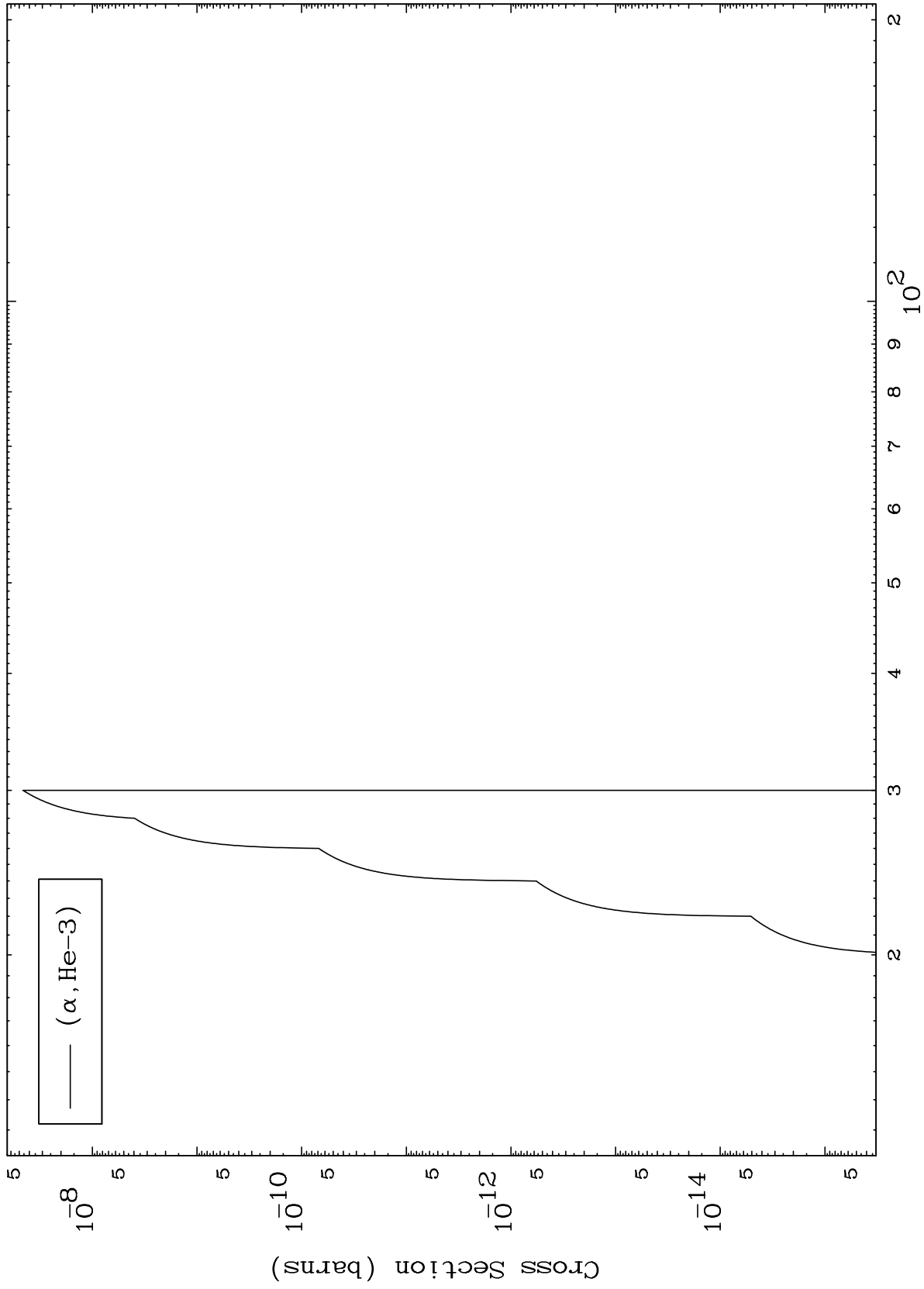
Incident Energy (MeV)

86-Rn-200

MAT 8592

( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections

86-Rn-200



9

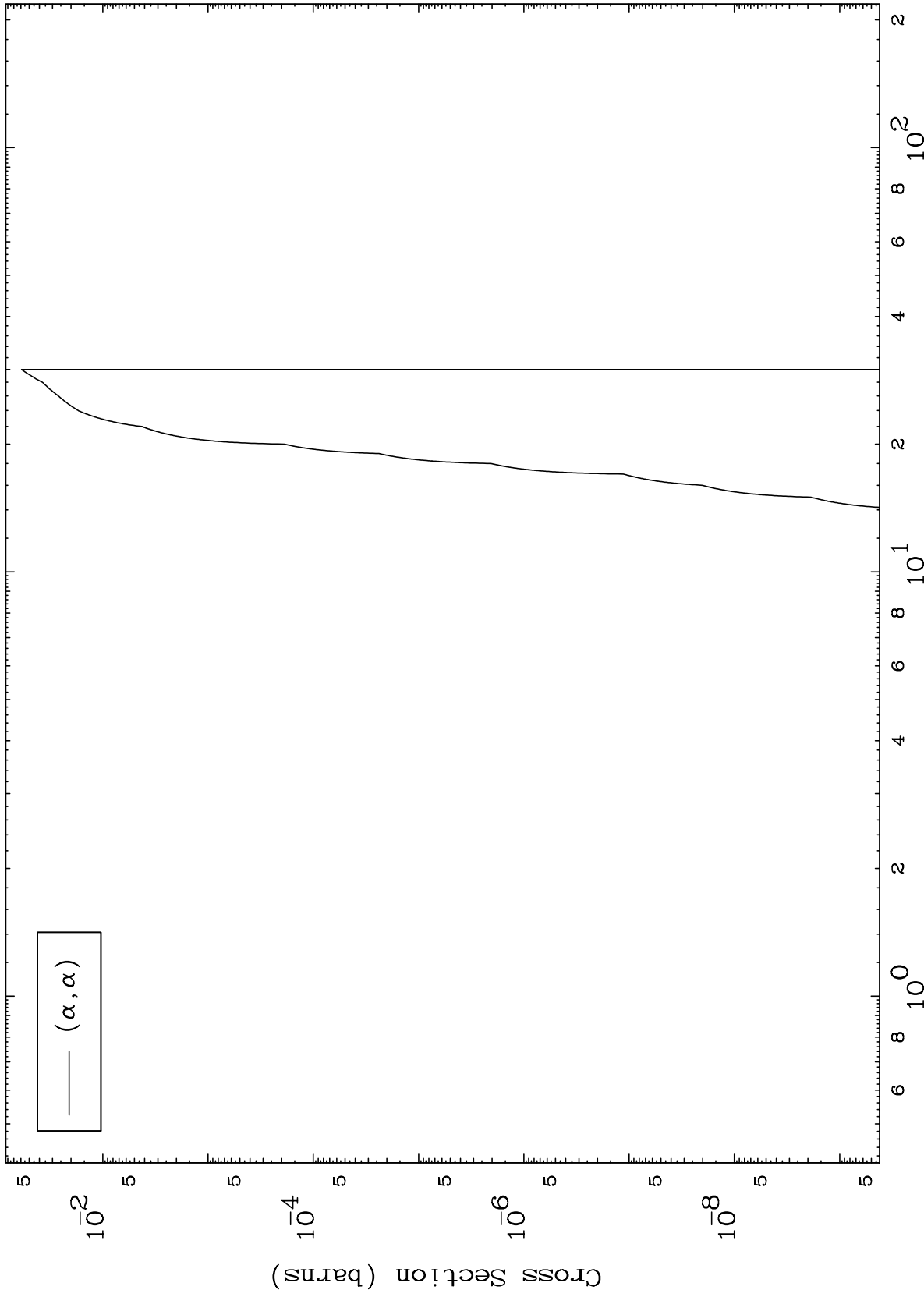
Incident Energy (MeV)

86-Rn-200

MAT 8592

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

86-Rn-200



10

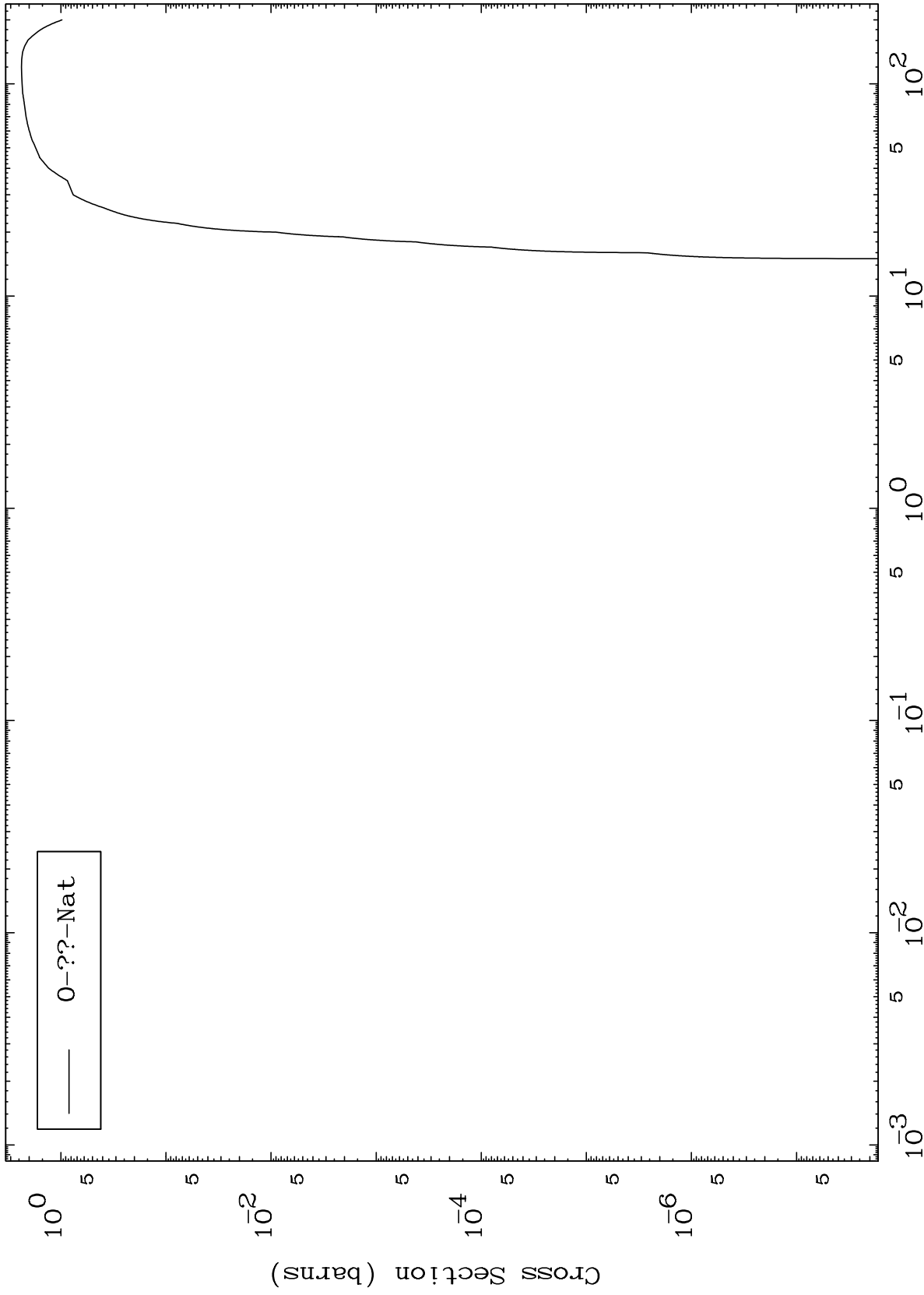
Incident Energy (MeV)

86-Rn-200

MAT 8592

$\alpha$  Fission  
Radionuclide Production Cross Section

86-Rn-200



11

Incident Energy (MeV)

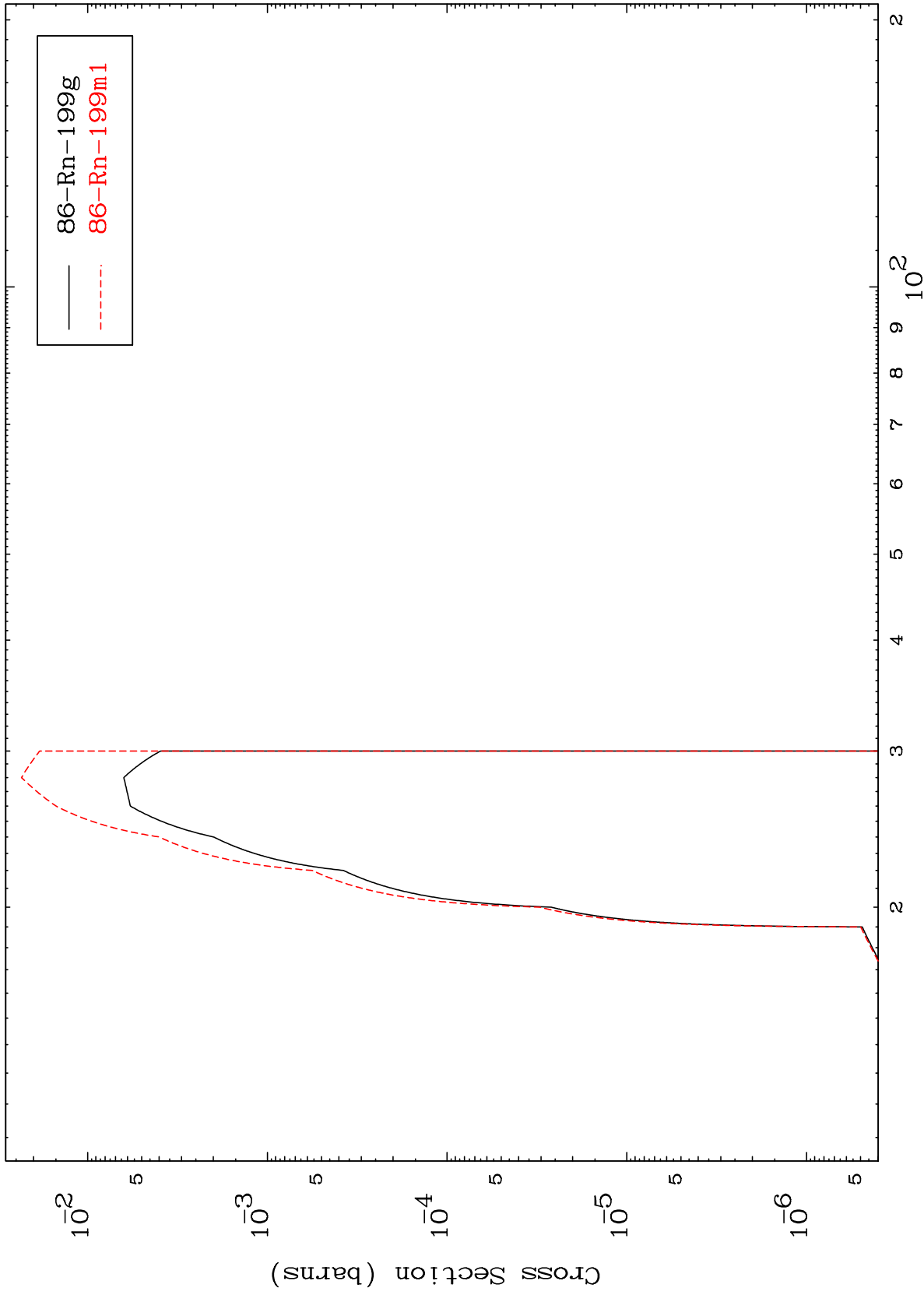
86-Rn-200

MAT 8592

$(\alpha, n')$   $\alpha$

86-Rn-200

Radionuclide Production Cross Section



12

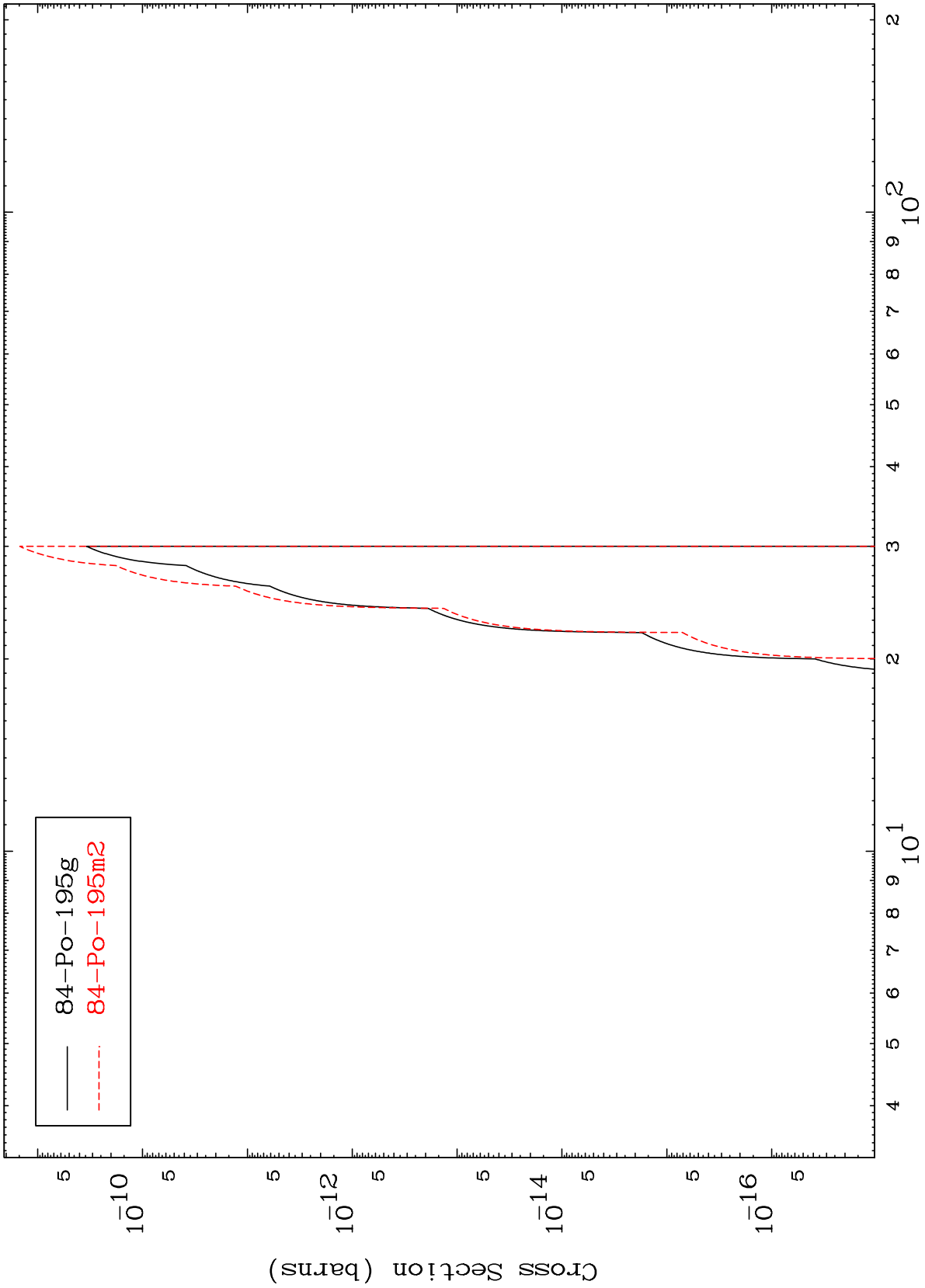
Incident Energy (MeV)

86-Rn-200

MAT 8592

86-Rn-200

$(\alpha, n')$   $2\alpha$   
Radionuclide Production Cross Section



13

86-Rn-200

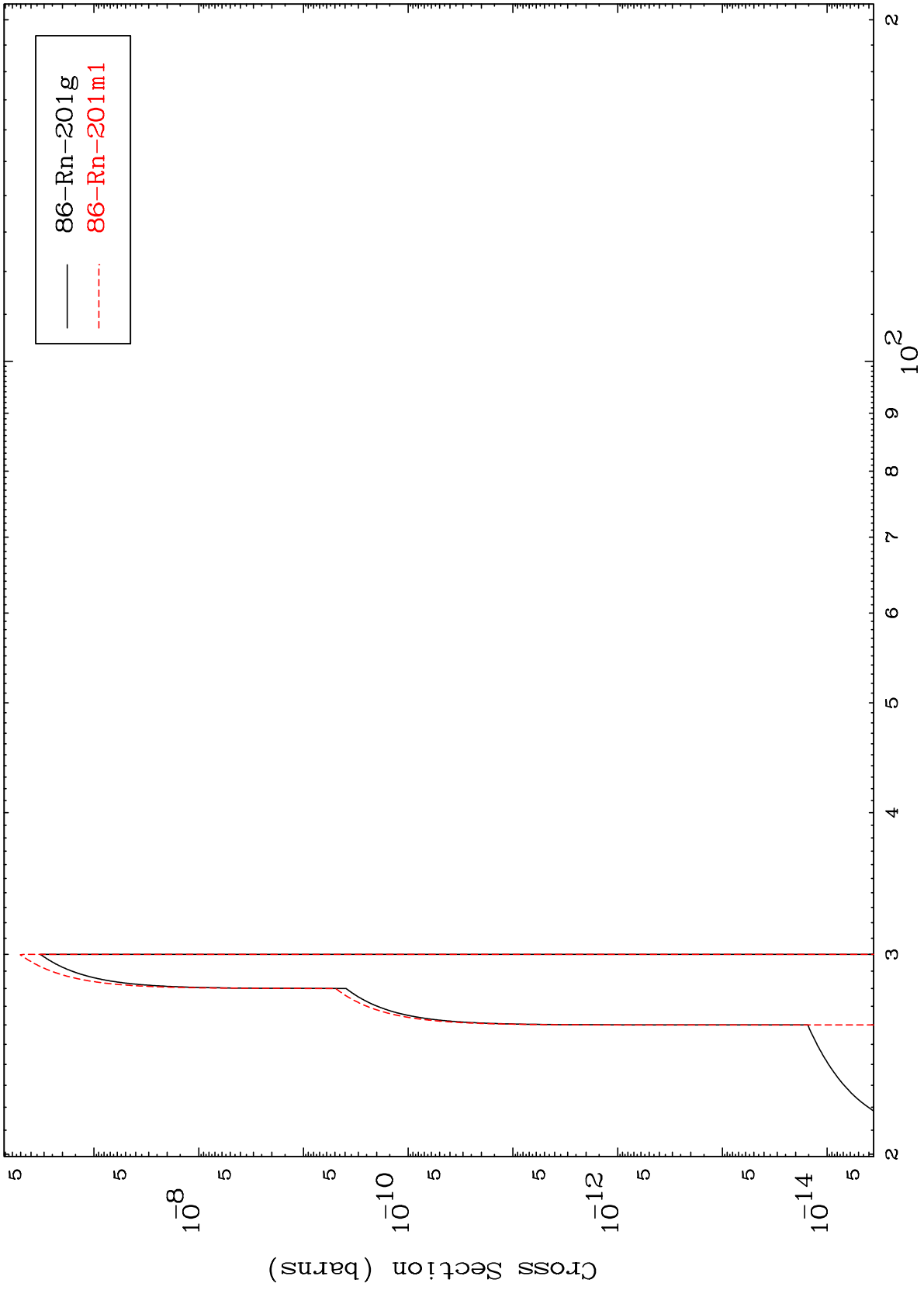
Incident Energy (MeV)

MAT 8592

( $\alpha, 2n$ ) p

86-Rn-200

Radionuclide Production Cross Section



14

Incident Energy (MeV)

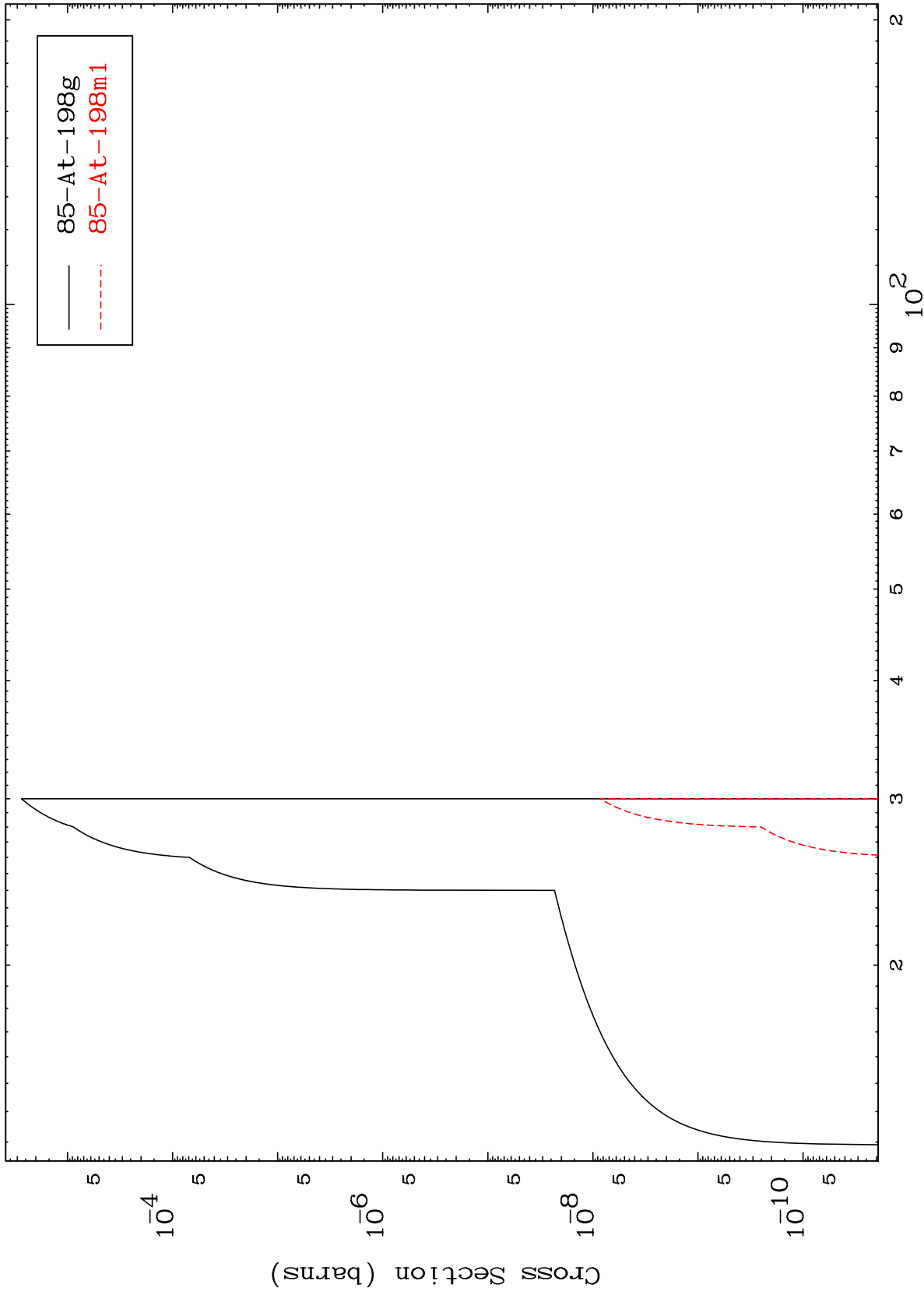
86-Rn-200

MAT 8592

$(\alpha, n')$  p  $\alpha$

86-Rn-200

Radionuclide Production Cross Section



15

Incident Energy (MeV)

86-Rn-200

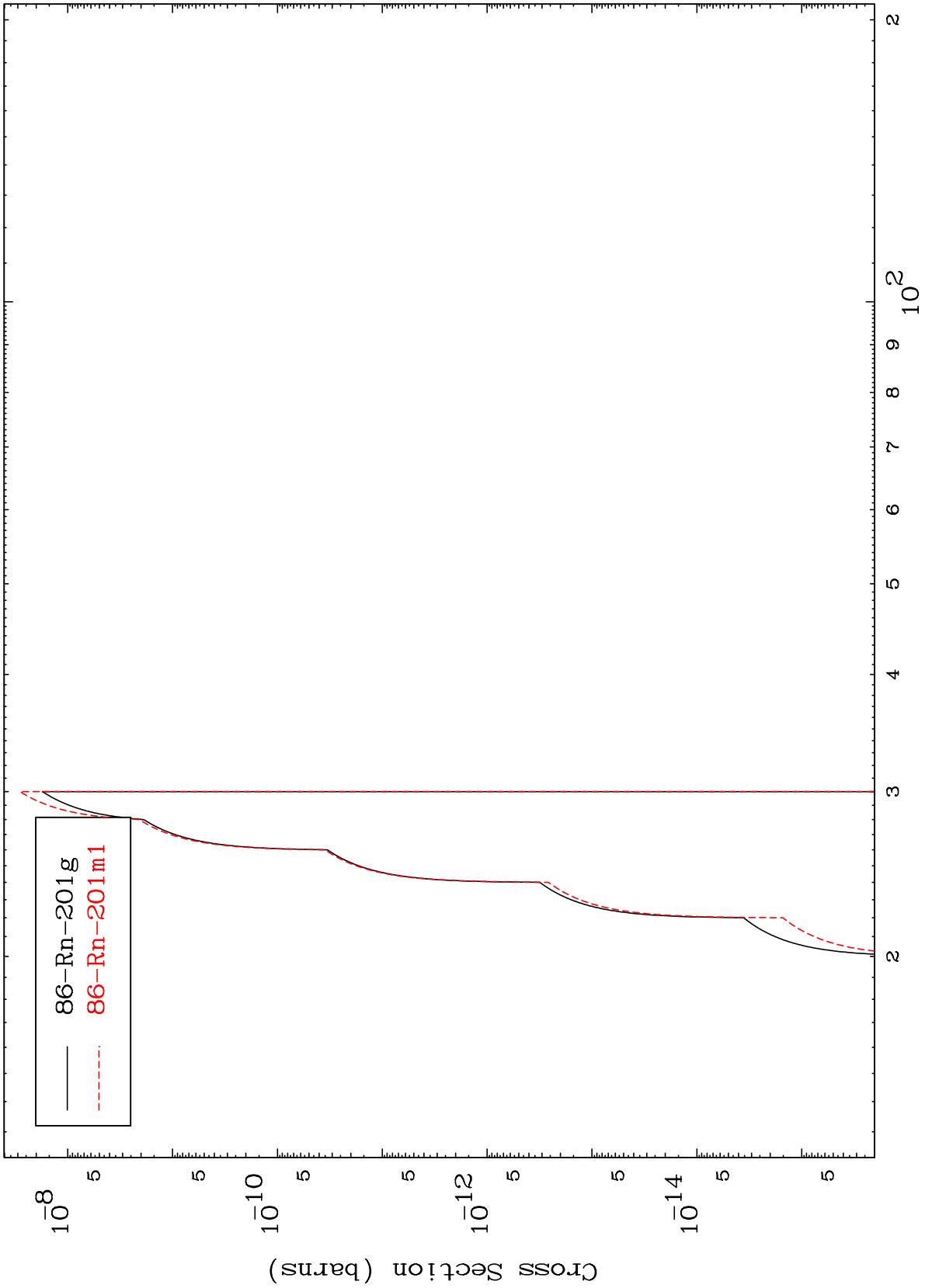


MAT 8592

( $\alpha, \text{He-3}$ )

86-Rn-200

Radionuclide Production Cross Section



16

Incident Energy (MeV)

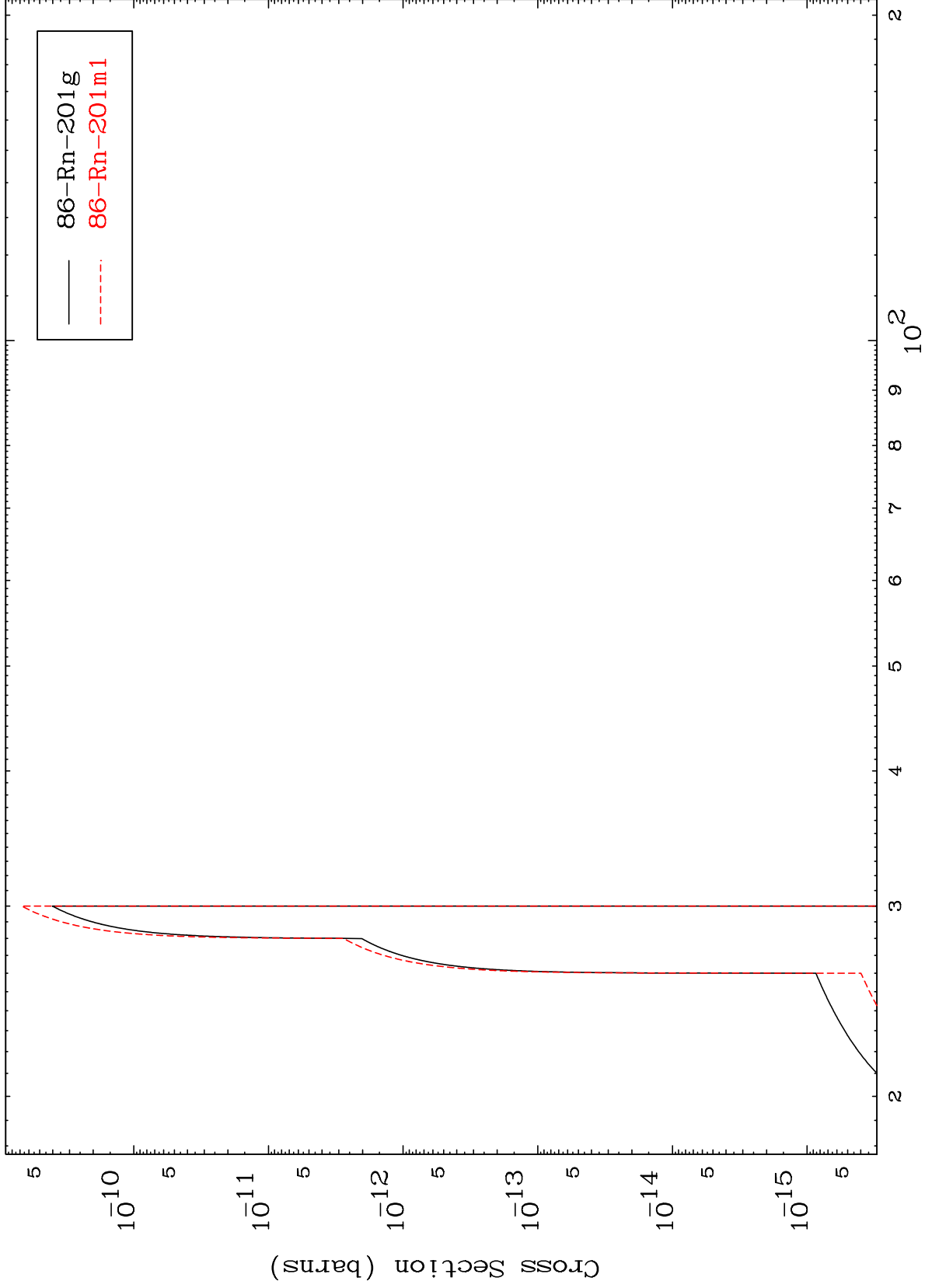
86-Rn-200

MAT 8592

( $\alpha, p$ ) d

86-Rn-200

Radionuclide Production Cross Section



17

Incident Energy (MeV)

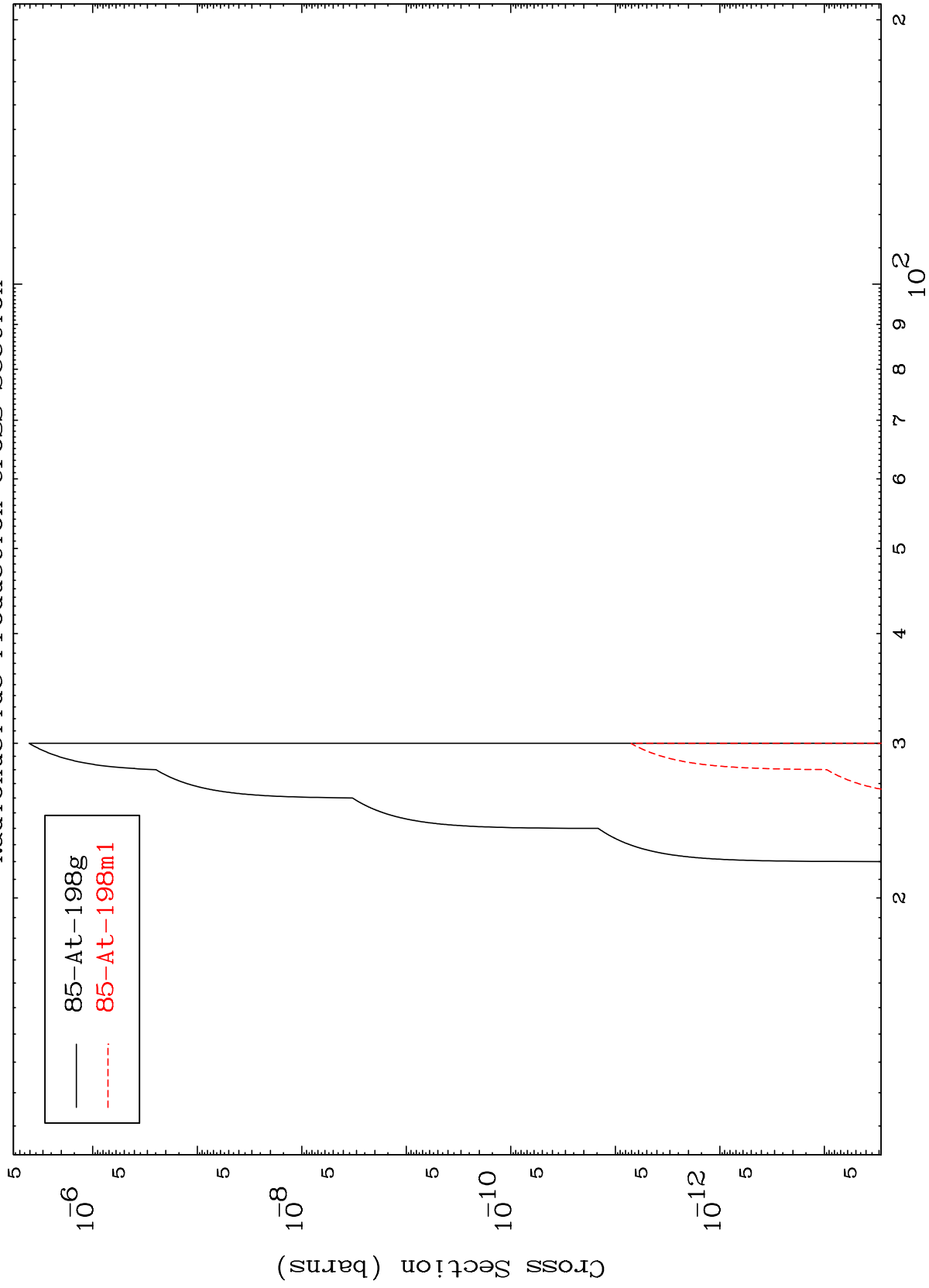
86-Rn-200

MAT 8592

( $\alpha, d$ )  $\alpha$

86-Rn-200

Radionuclide Production Cross Section



18

Incident Energy (MeV)

86-Rn-200