

Program EVALPLOT  
(Version 2018-1)

by

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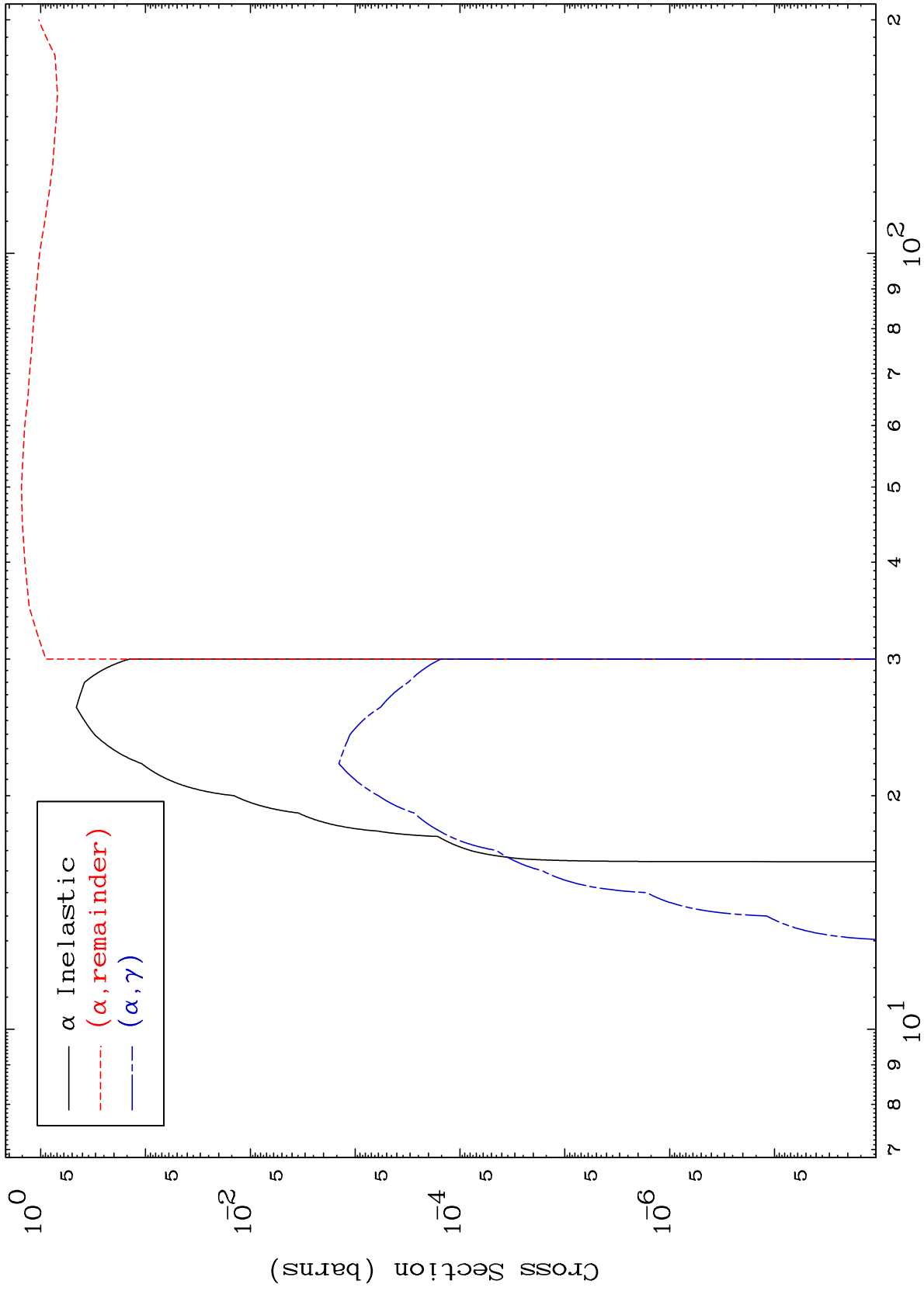
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8616

0 Kelvin  $\alpha$  Major  
Cross Sections

86-Rn-208



1

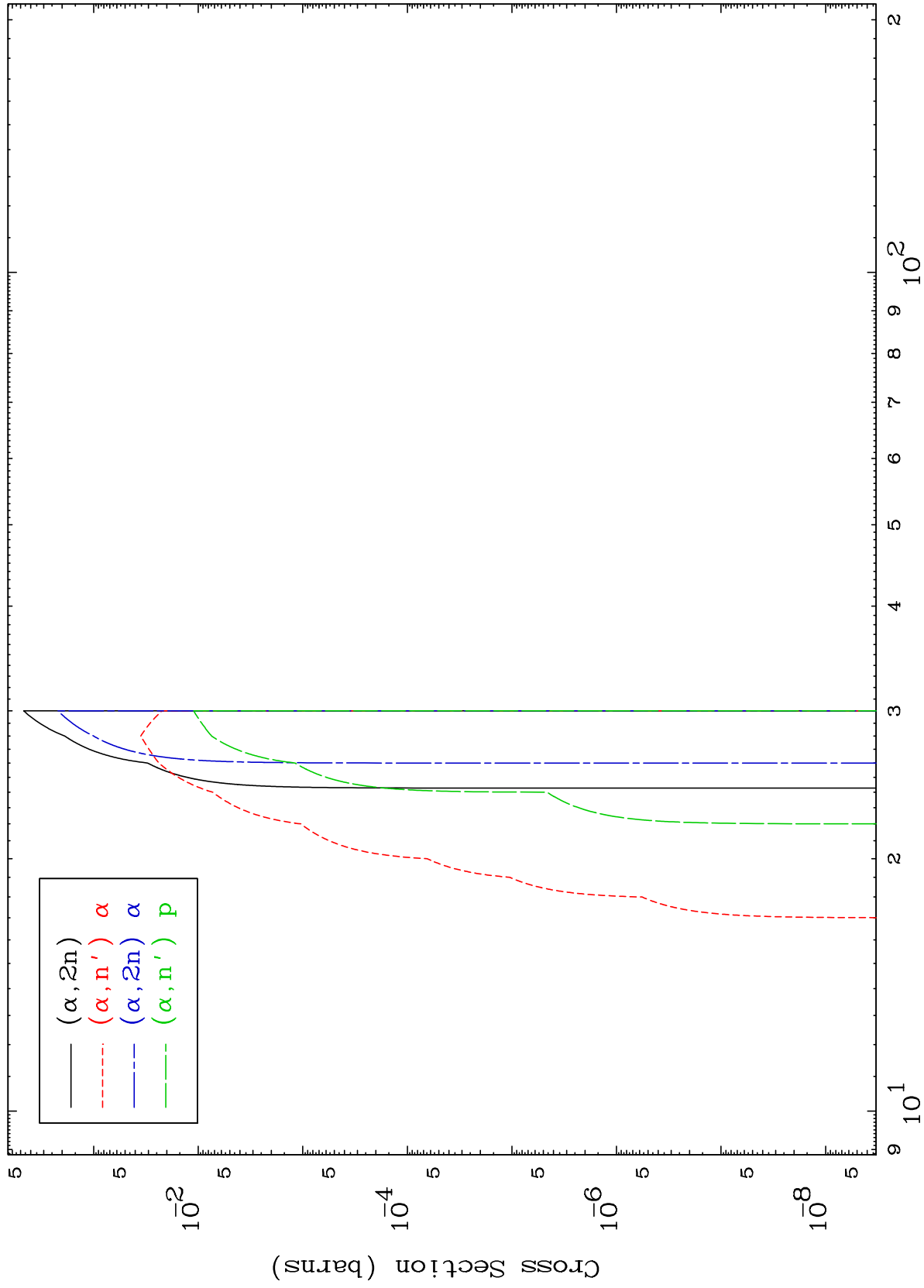
Incident Energy (MeV)

86-Rn-208

MAT 8616

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

86-Rn-208



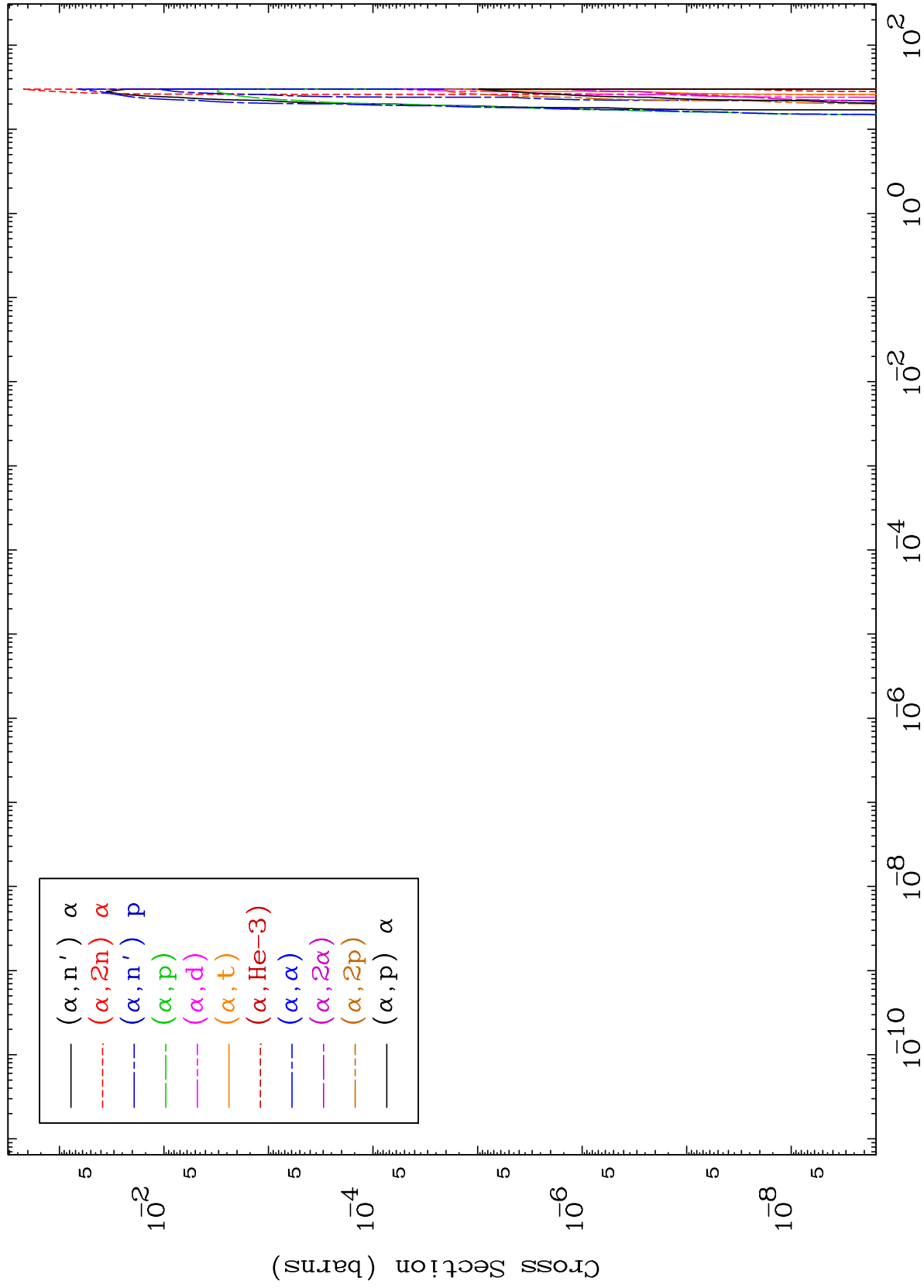
86-Rn-208

Incident Energy (MeV)

MAT 8616

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

86-Rn-208



3

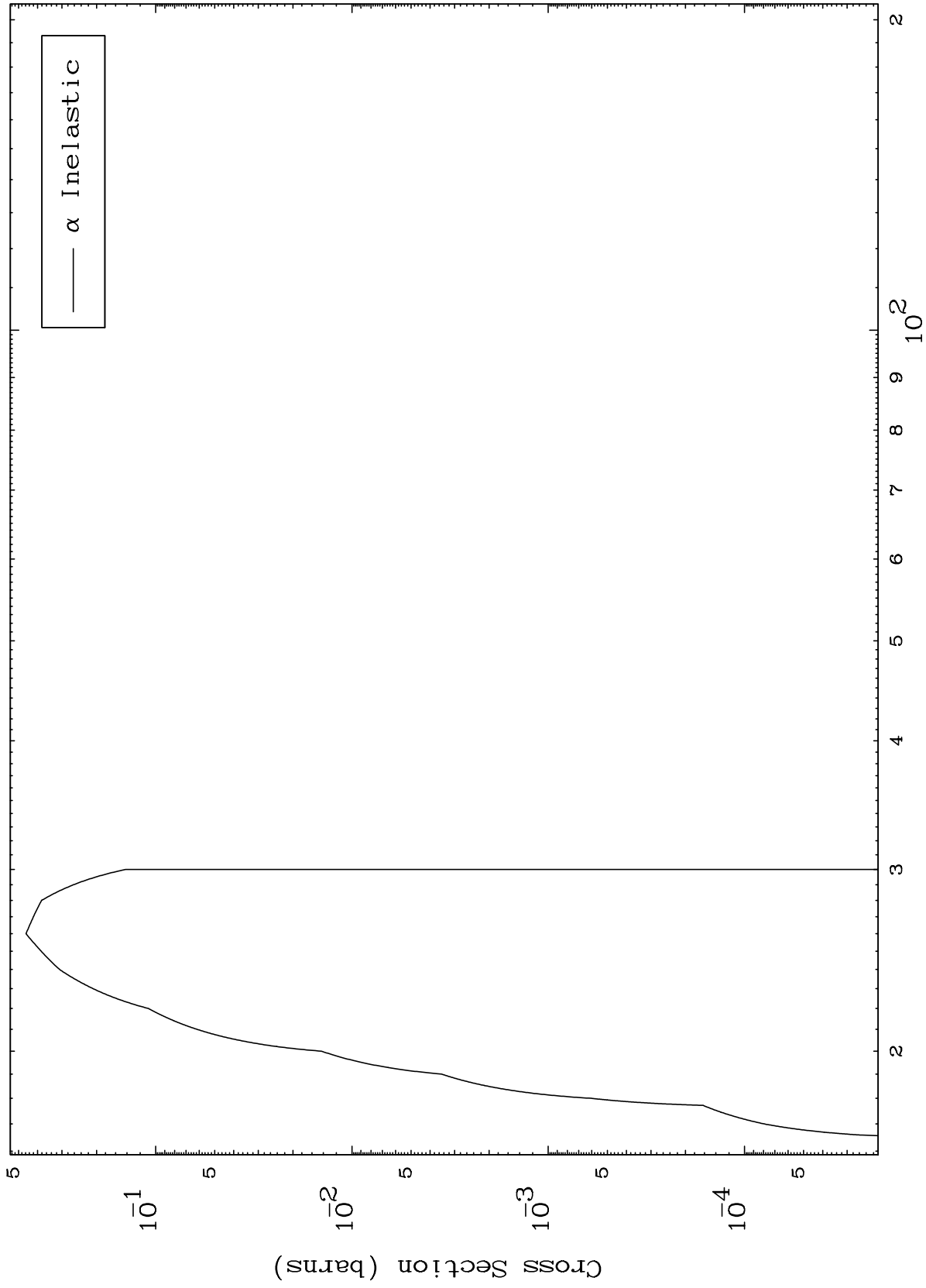
Incident Energy (MeV)

86-Rn-208

MAT 8616

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

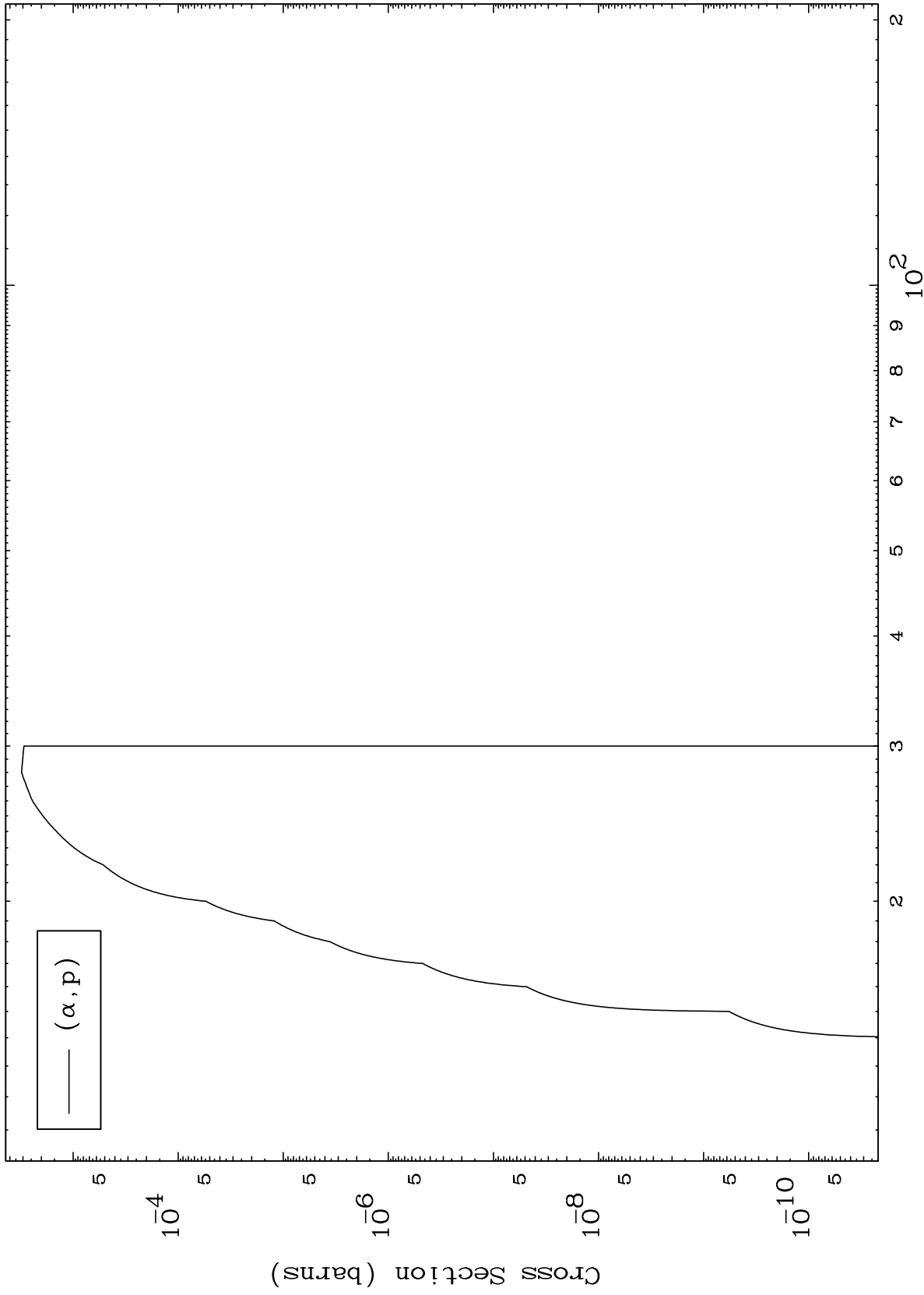
86-Rn-208



4

Incident Energy (MeV)

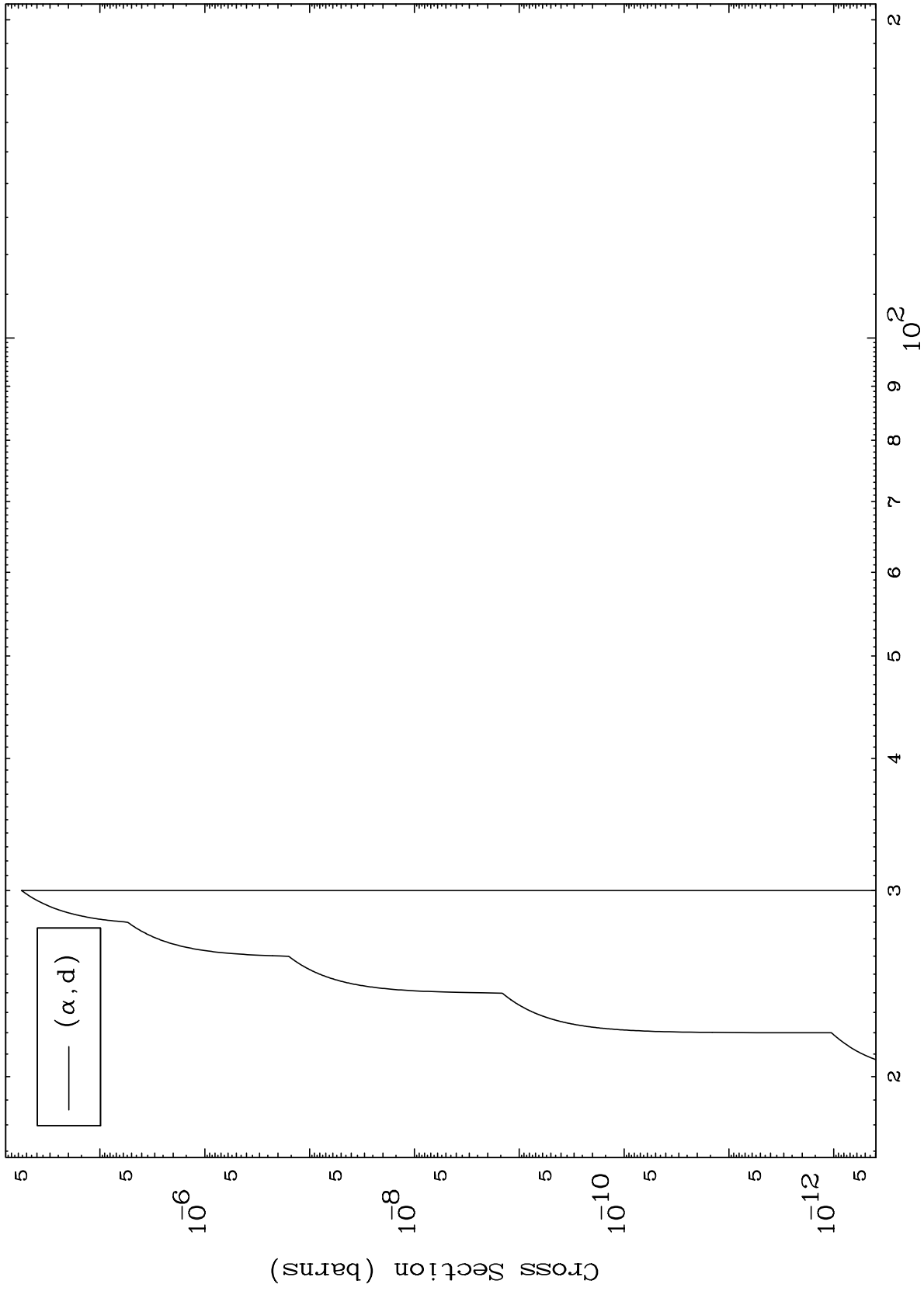
86-Rn-208



MAT 8616

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

86-Rn-208



6

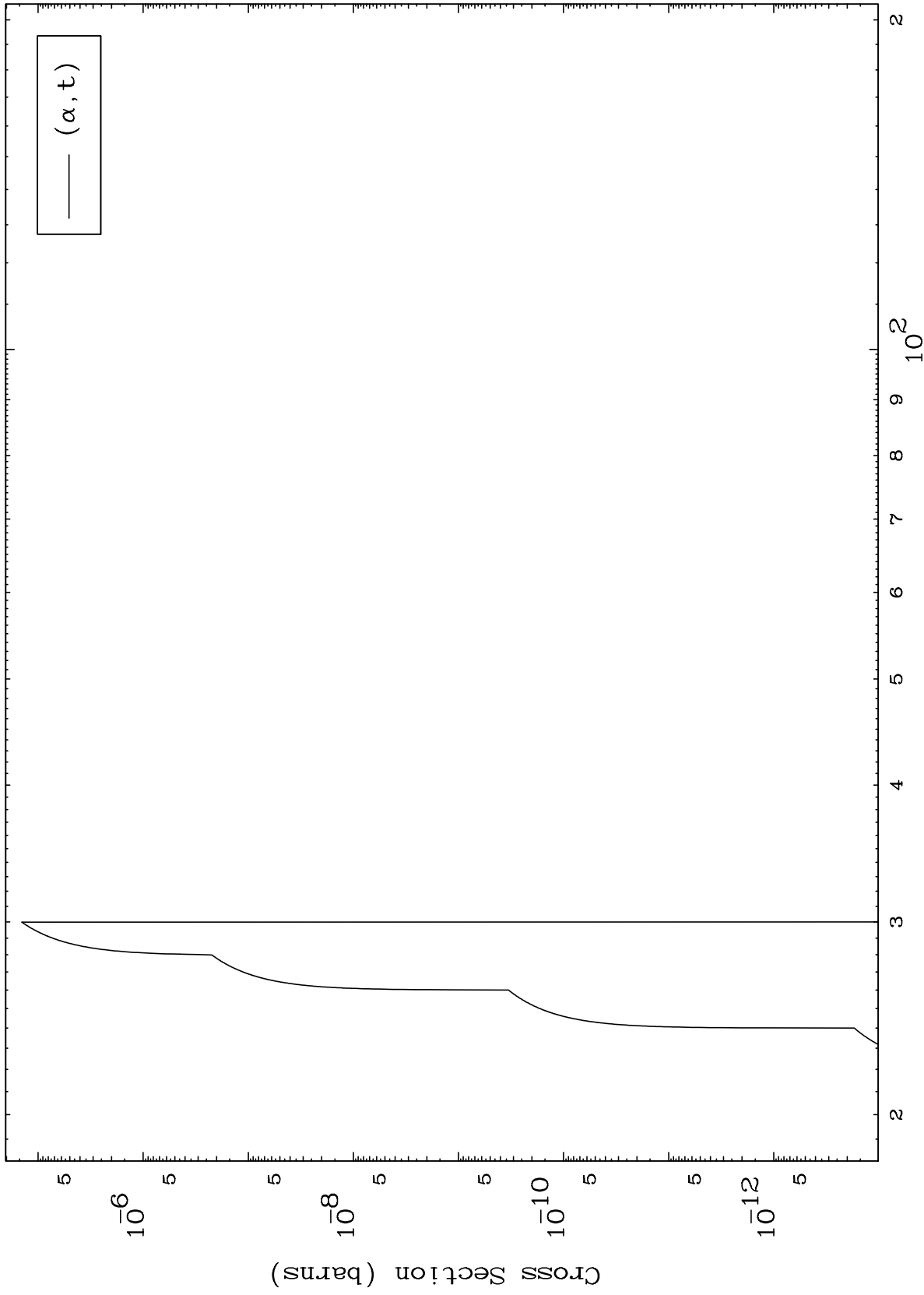
Incident Energy (MeV)

86-Rn-208

MAT 8616

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

86-Rn-208



7

Incident Energy (MeV)

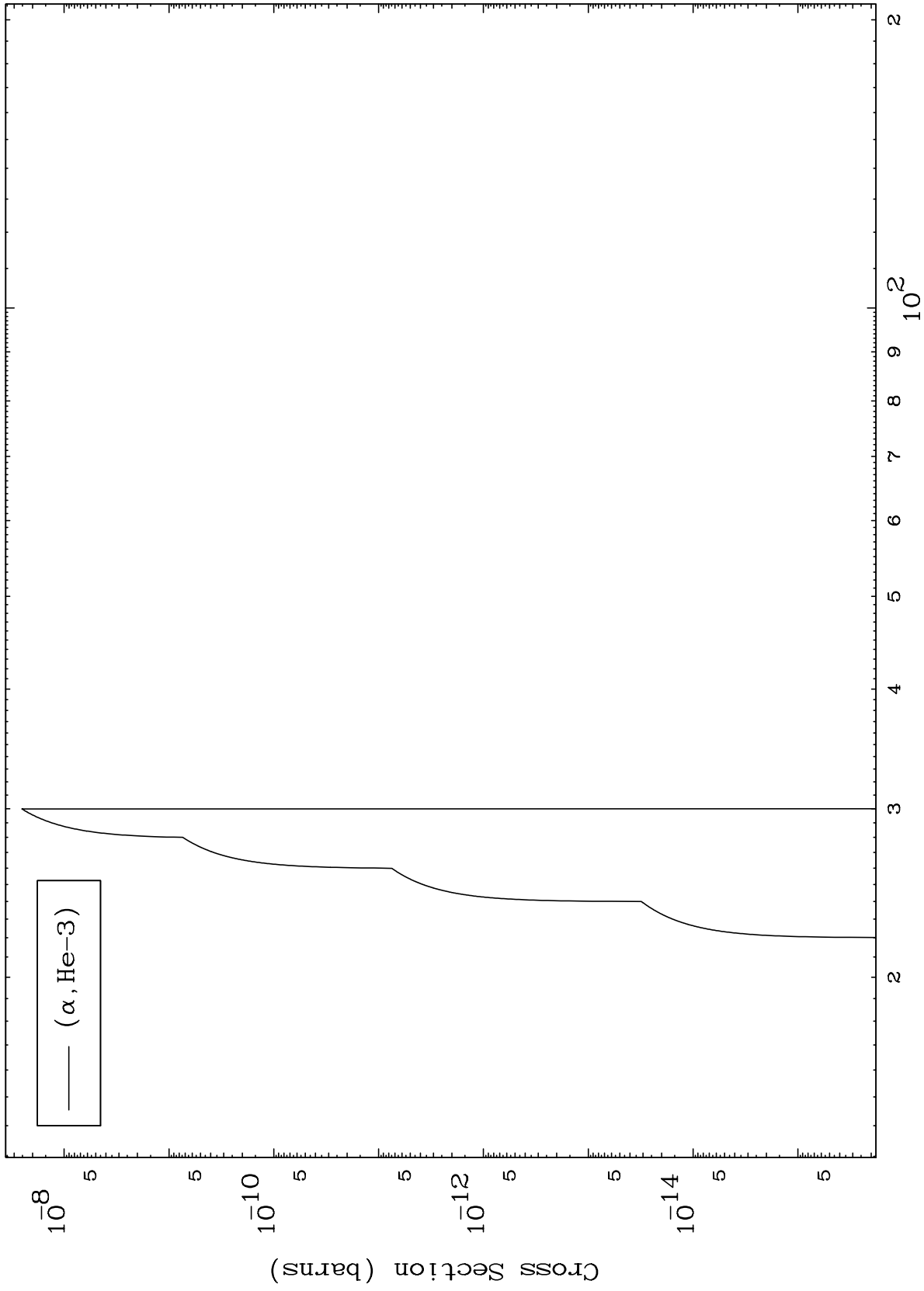
86-Rn-208



MAT 8616

( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections

86-Rn-208



8

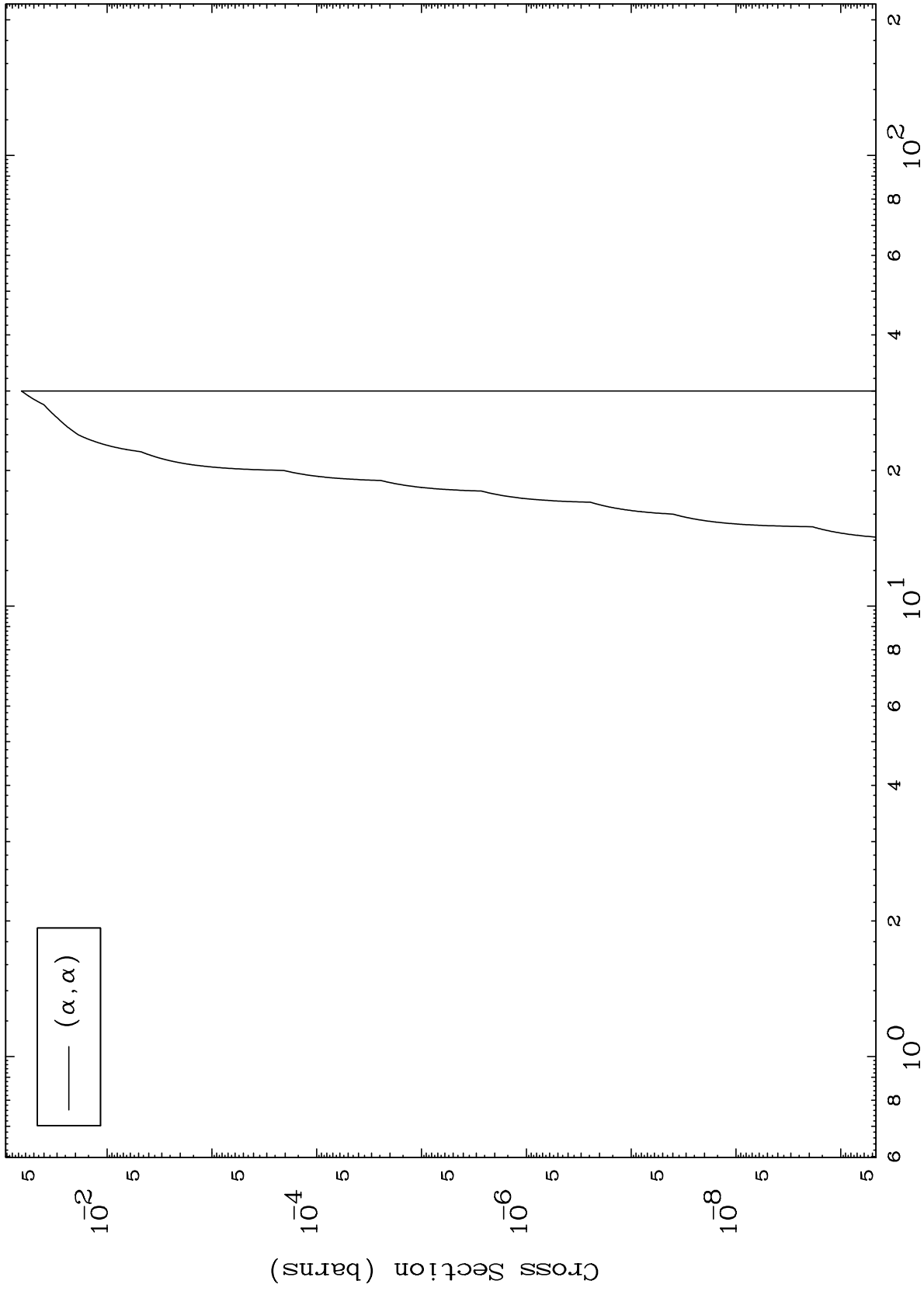
Incident Energy (MeV)

86-Rn-208

MAT 8616

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

86-Rn-208



9

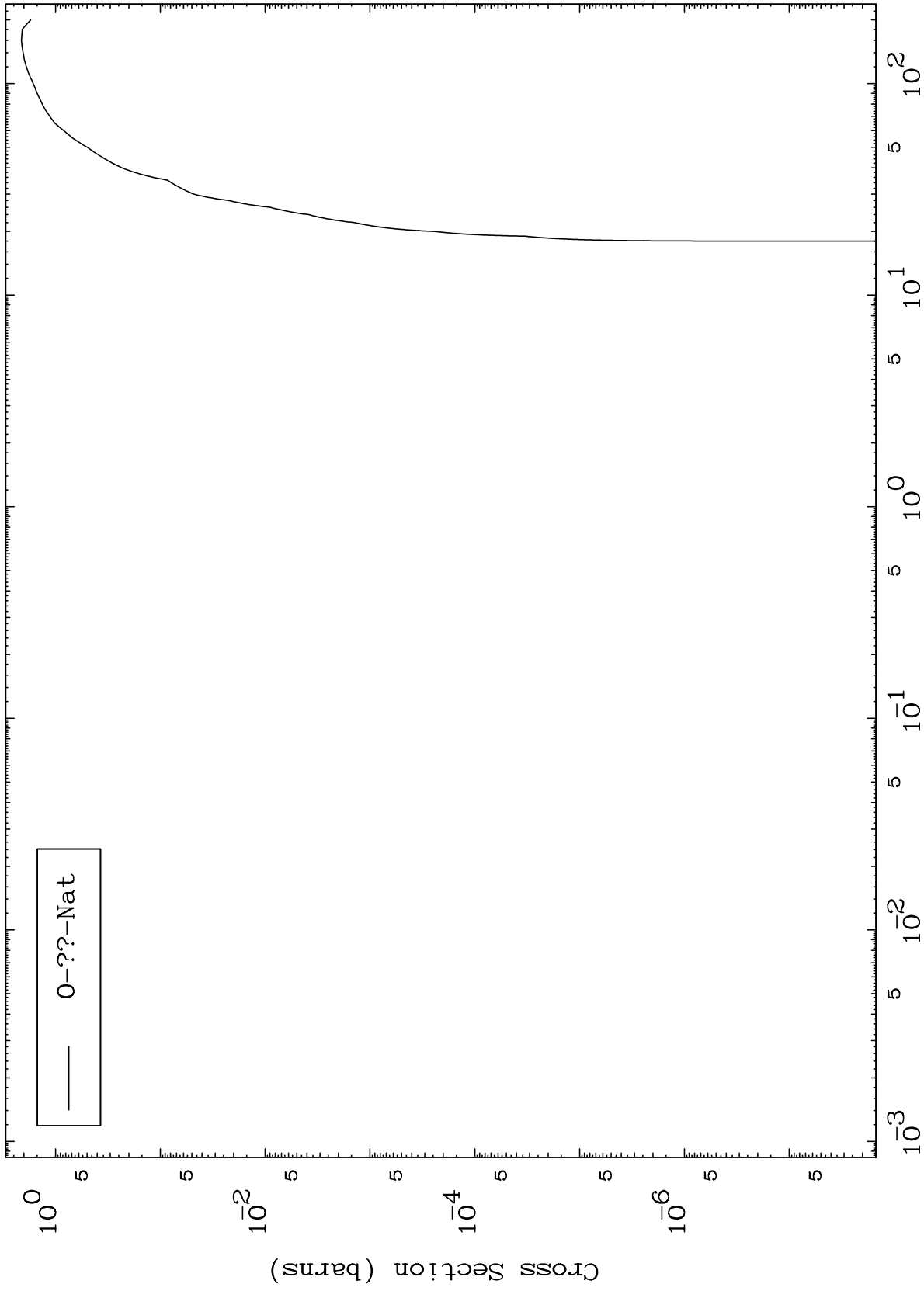
Incident Energy (MeV)

86-Rn-208

MAT 8616

$\alpha$  Fission  
Radionuclide Production Cross Section

86-Rn-208



Incident Energy (MeV)

86-Rn-208