

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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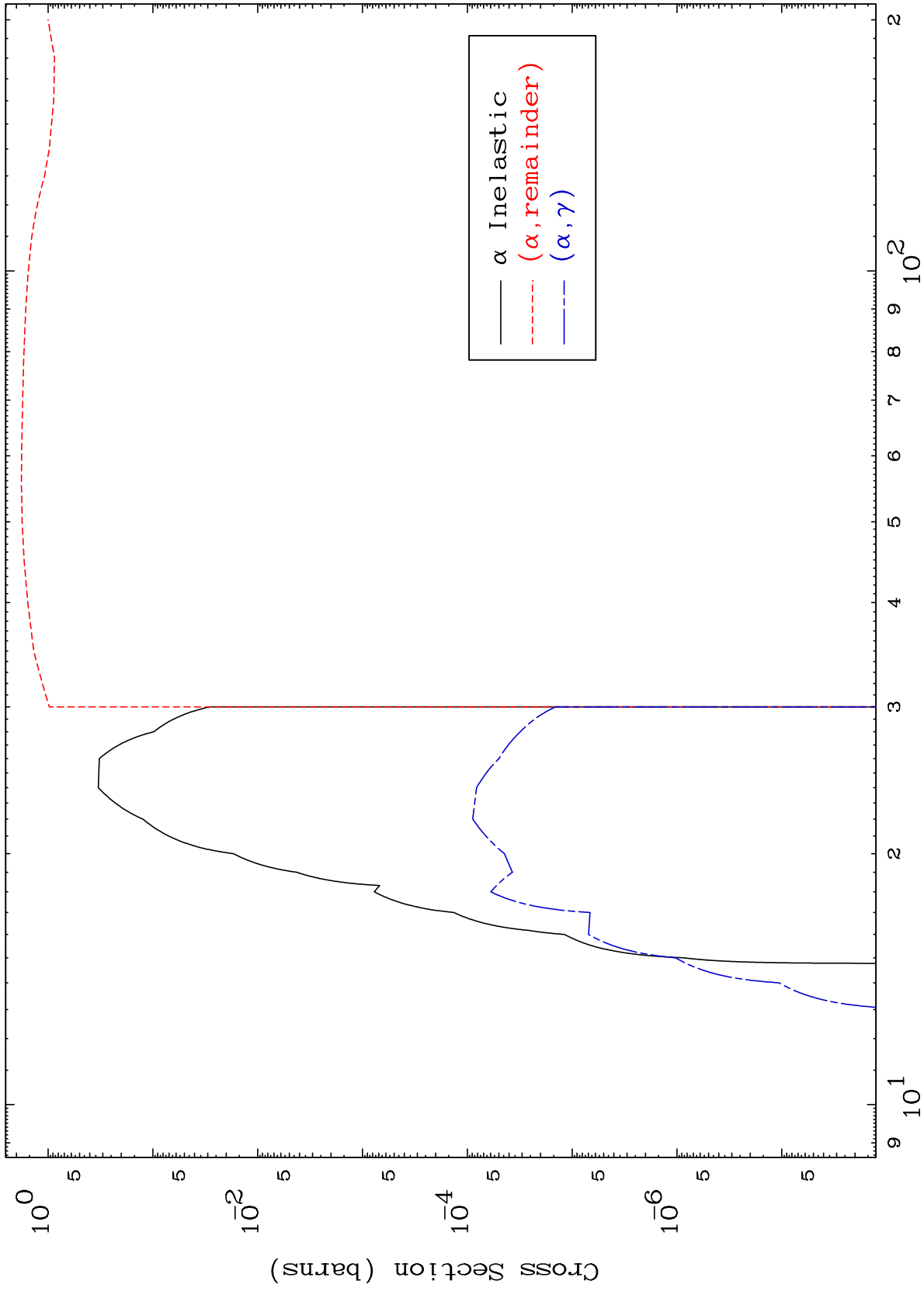
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8625

$\alpha$  Major  
0 Kelvin Cross Sections

86-Rn-211



1

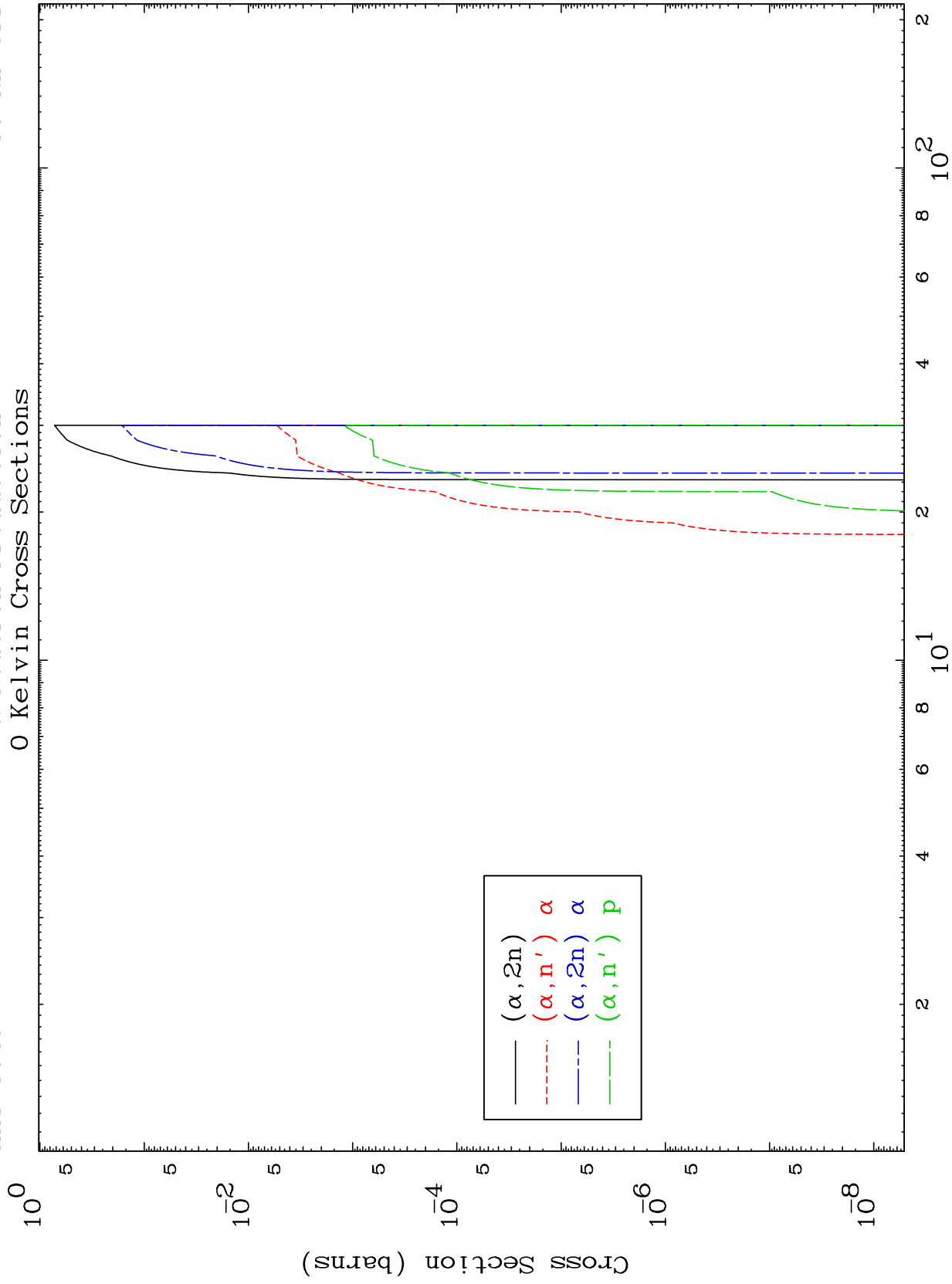
Incident Energy (MeV)

86-Rn-211

MAT 8625

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

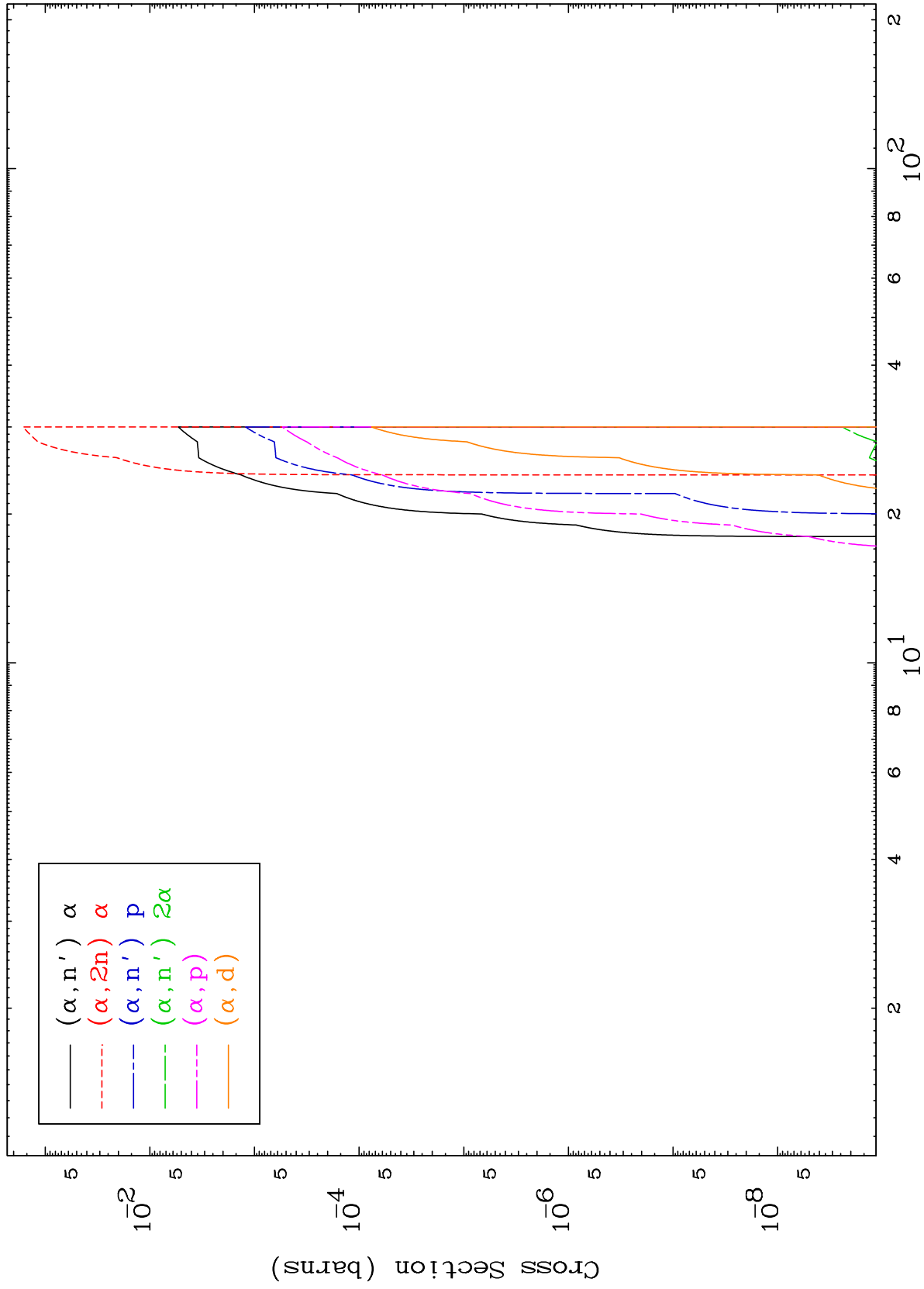
86-Rn-211



2

Incident Energy (MeV)

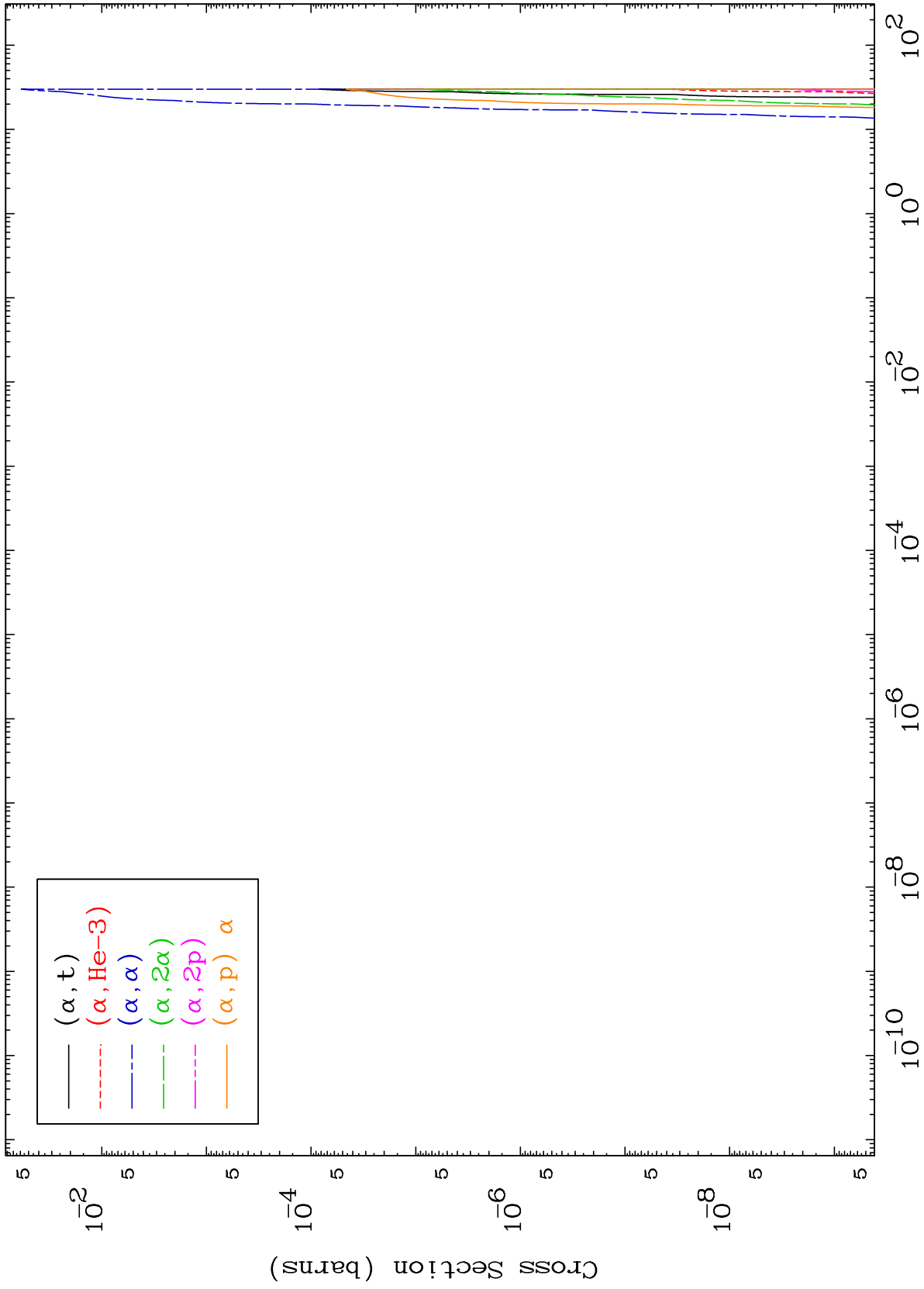
86-Rn-211



MAT 8625

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

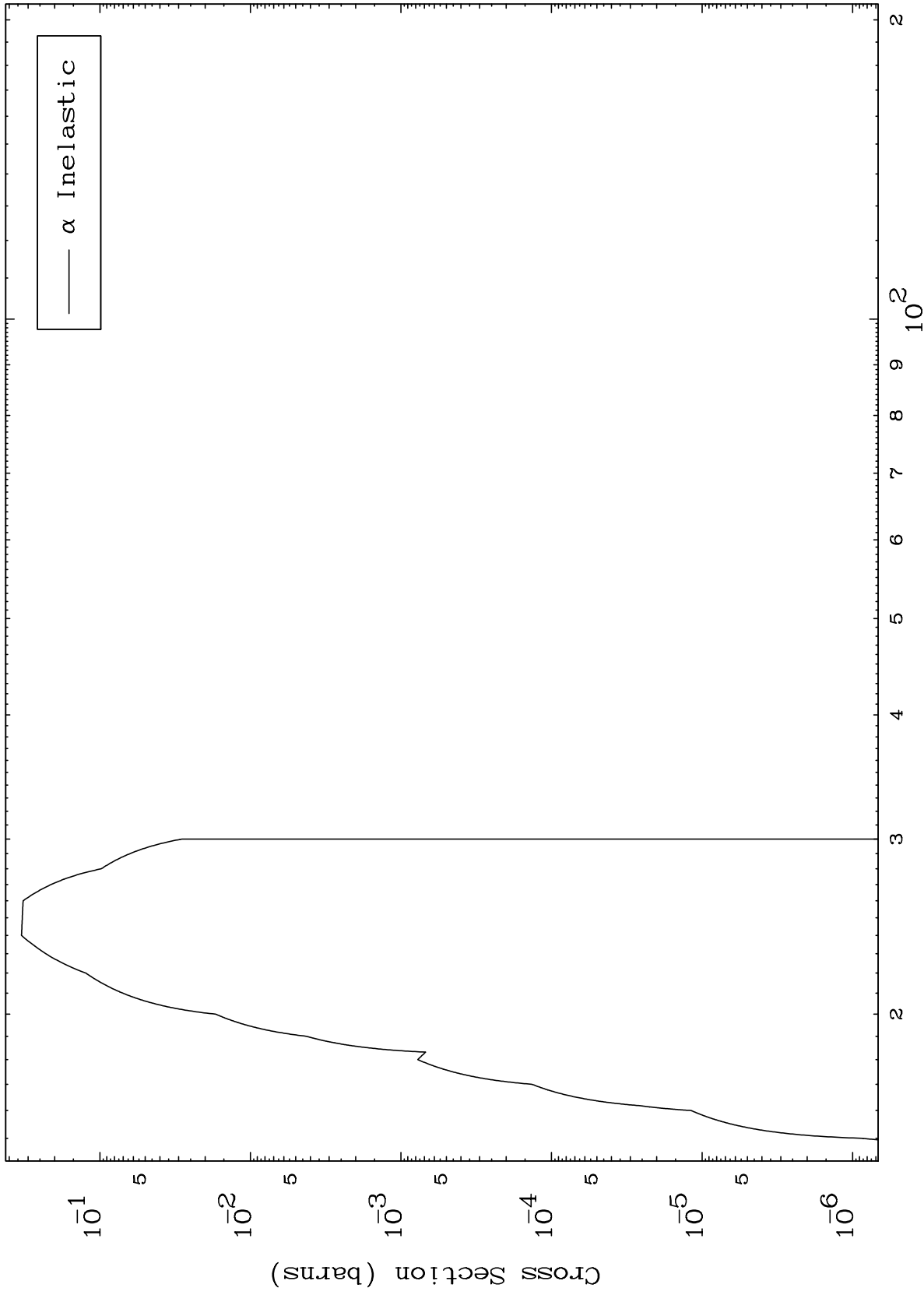
86-Rn-211



MAT 8625

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

86-Rn-211



5

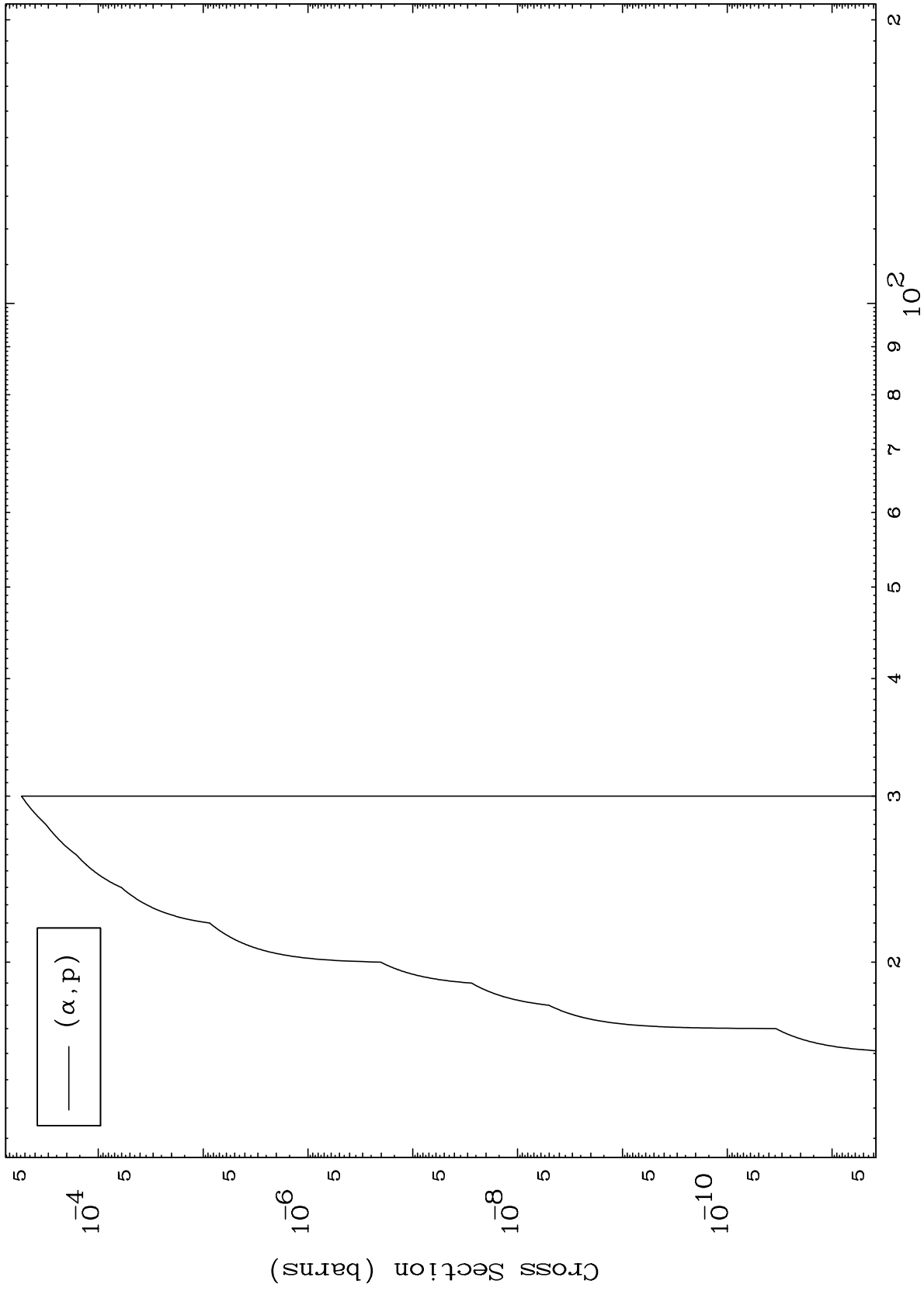
Incident Energy (MeV)

86-Rn-211

MAT 8625

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

86-Rn-211



6

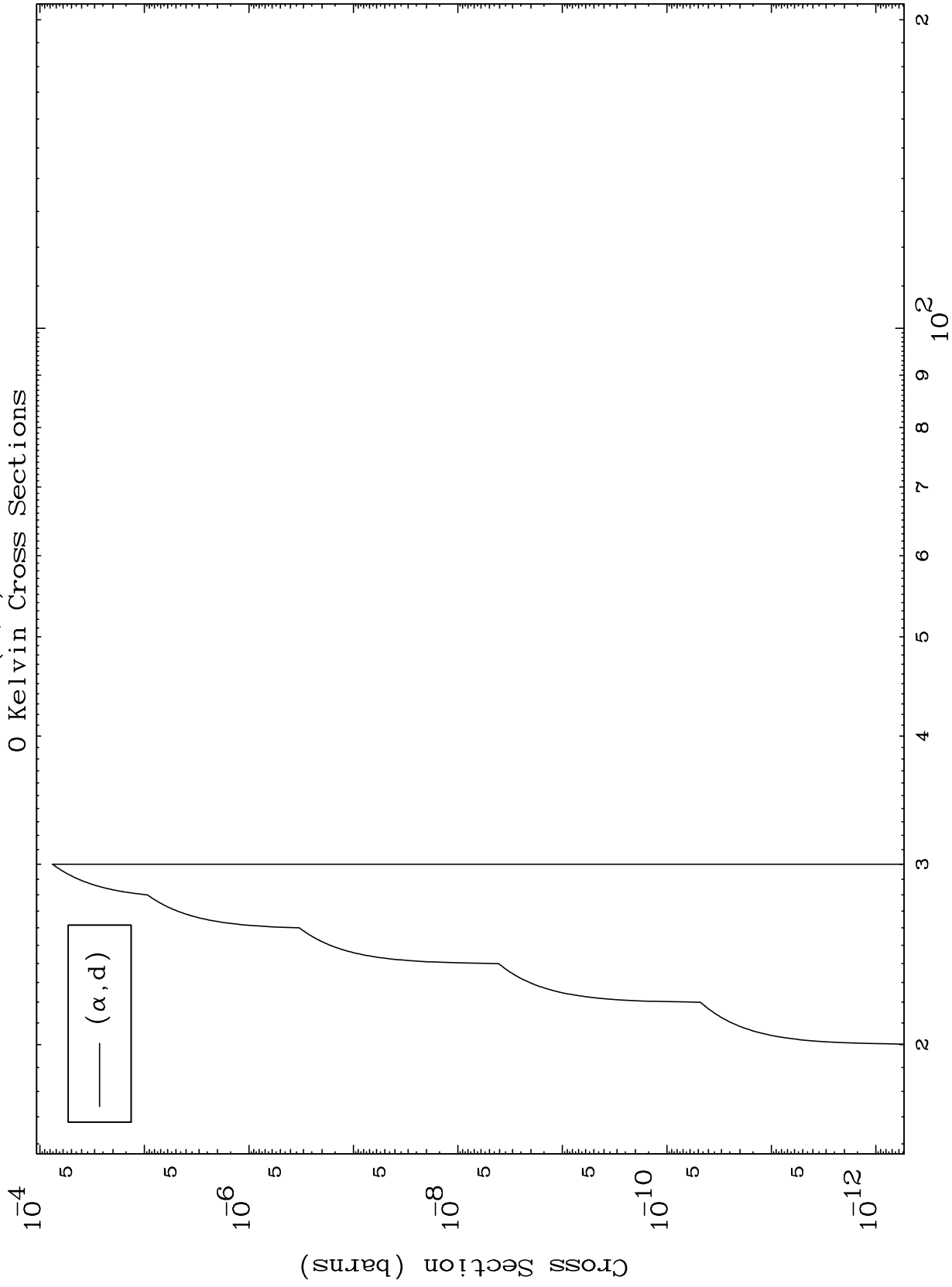
Incident Energy (MeV)

86-Rn-211

MAT 8625

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

86-Rn-211



7

Incident Energy (MeV)

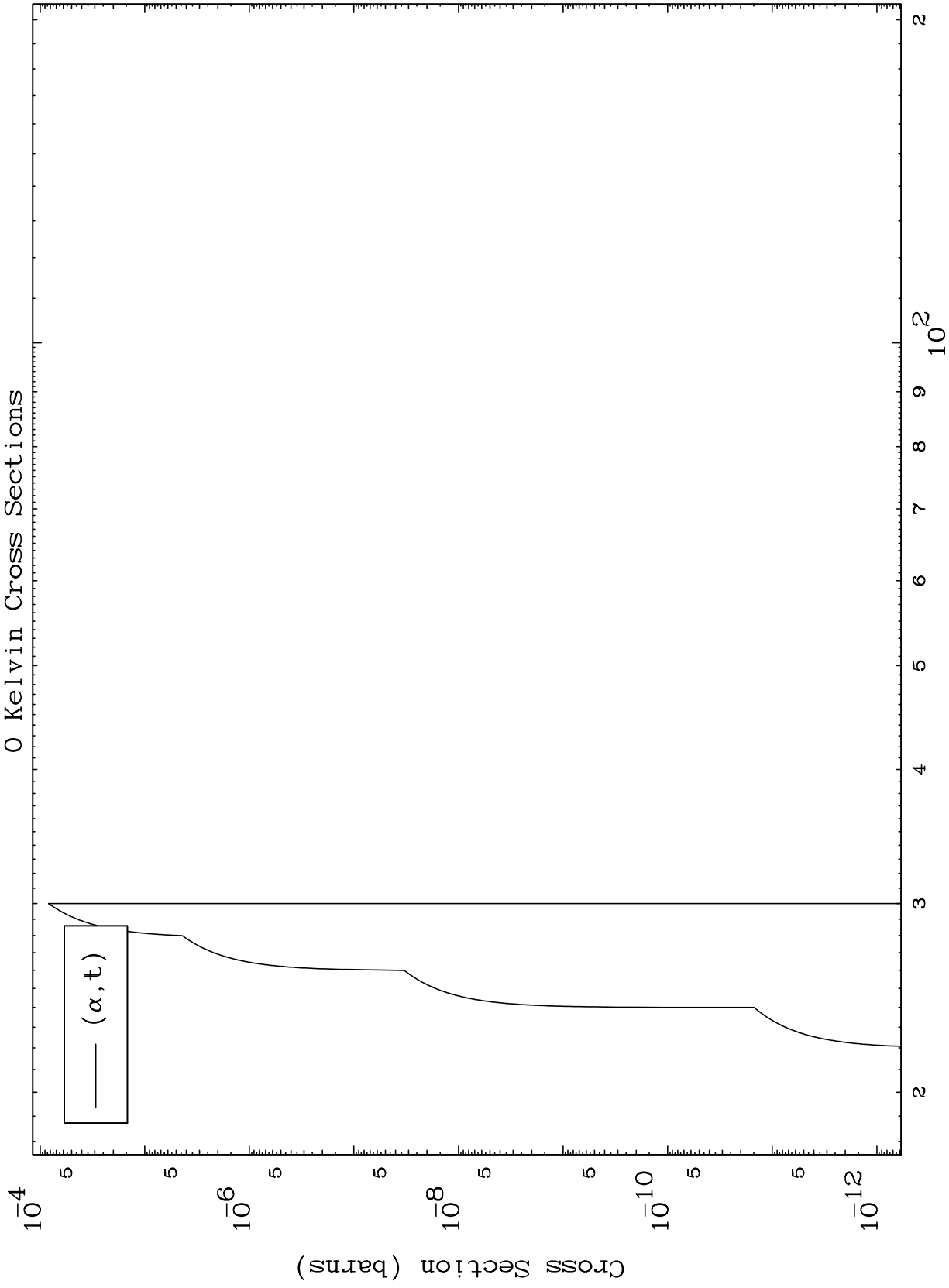
86-Rn-211



MAT 8625

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

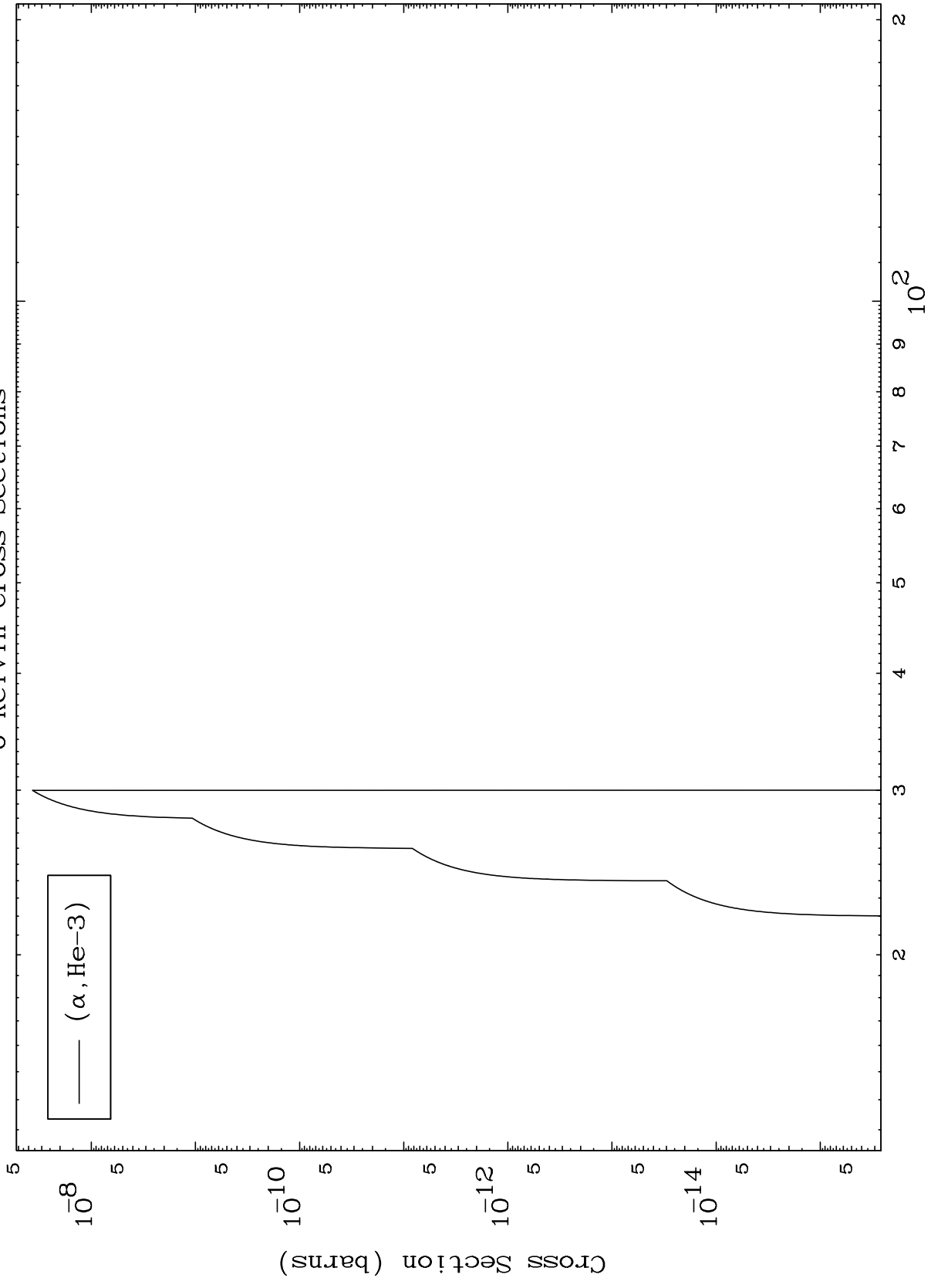
86-Rn-211



8

Incident Energy (MeV)

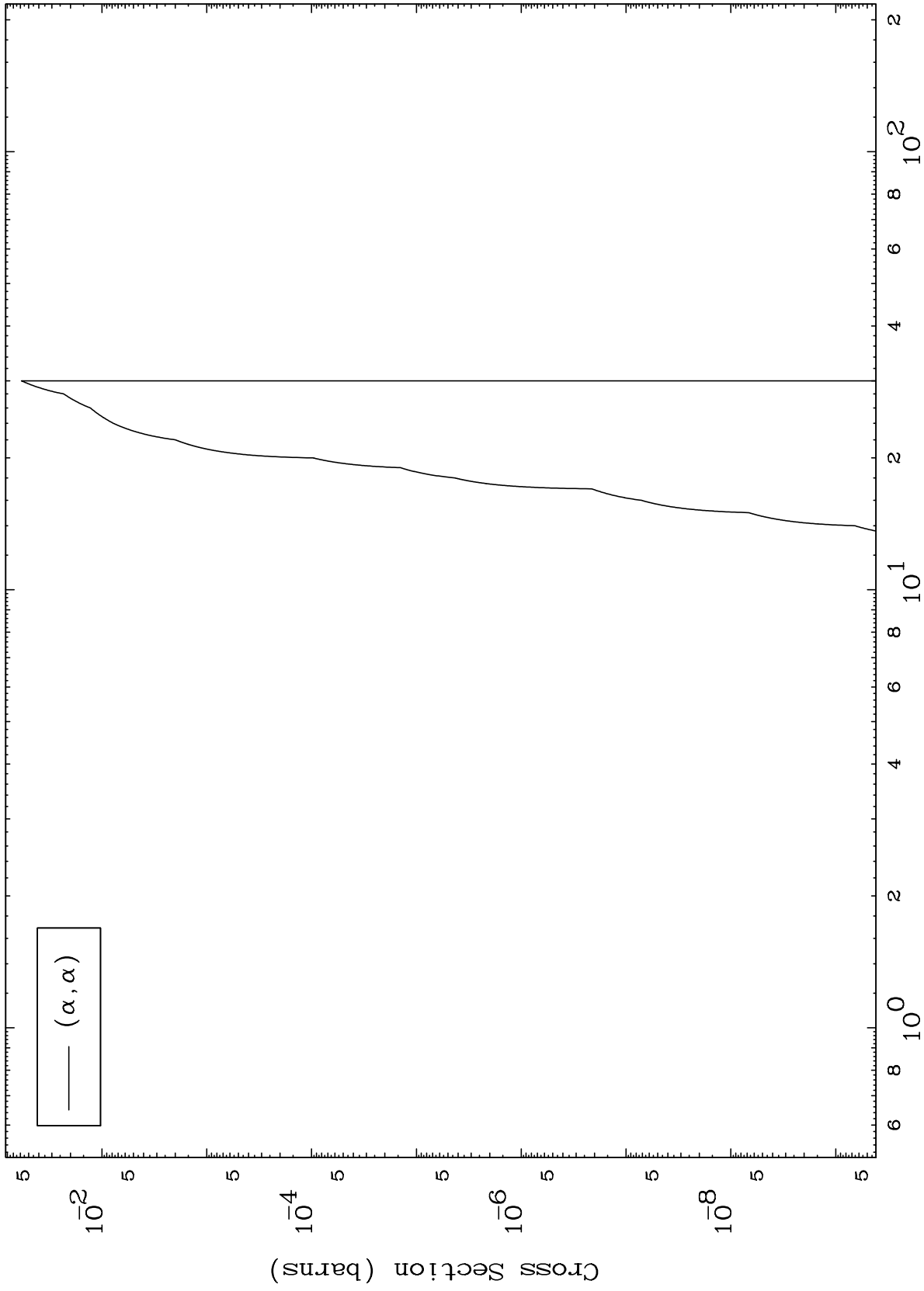
86-Rn-211



MAT 8625

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

86-Rn-211



10

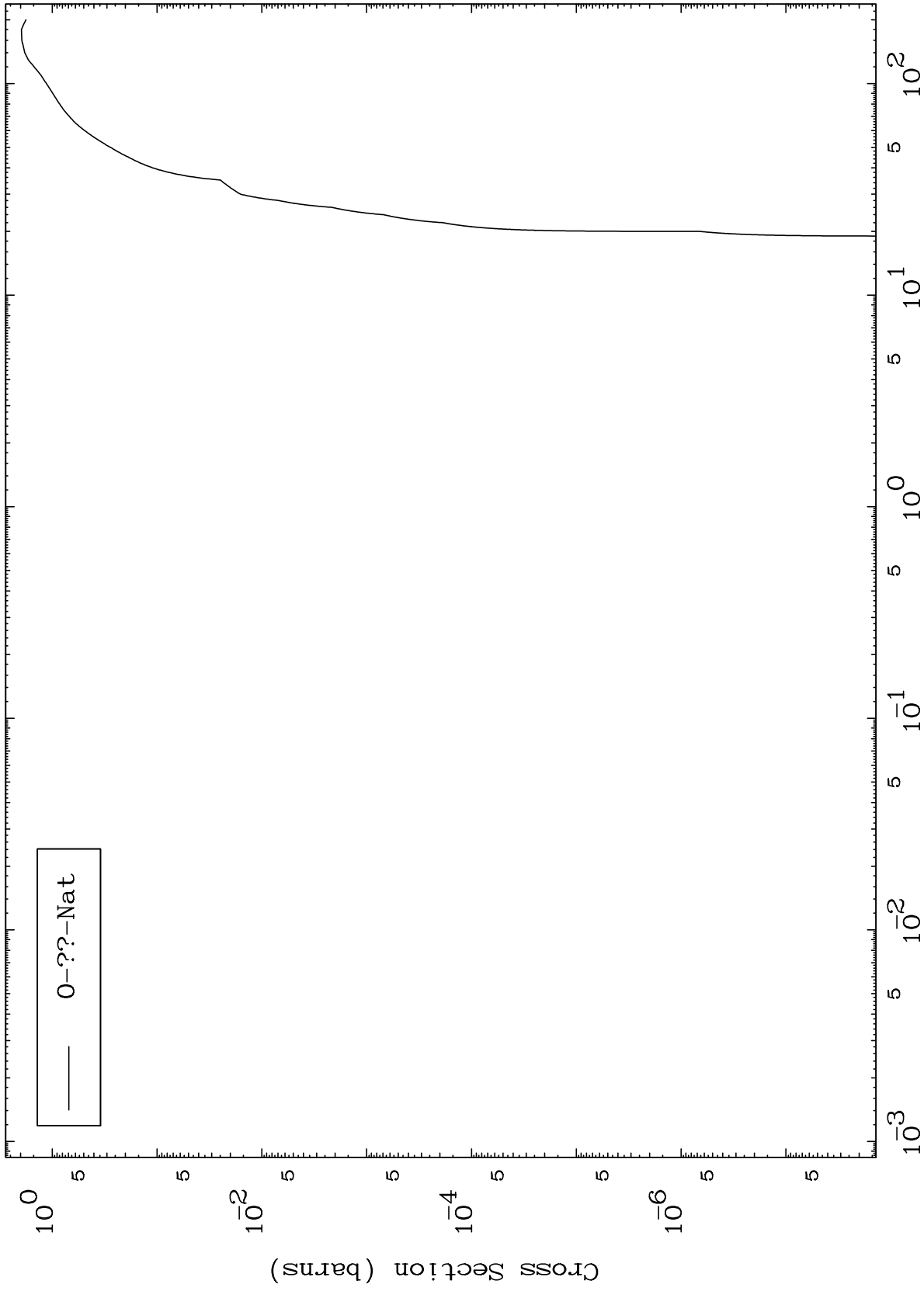
Incident Energy (MeV)

86-Rn-211

MAT 8625

$\alpha$  Fission  
Radionuclide Production Cross Section

86-Rn-211



11

Incident Energy (MeV)

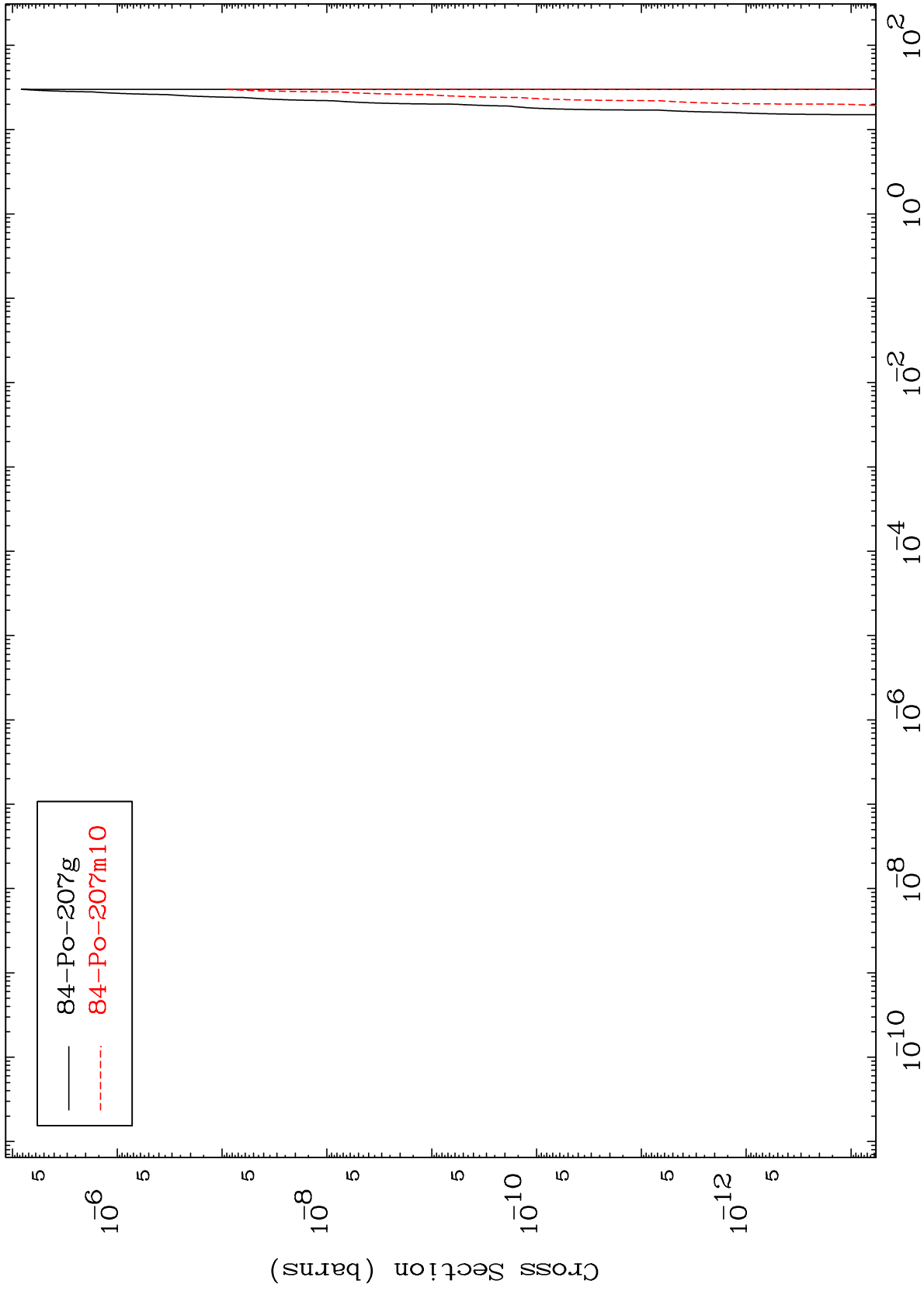
86-Rn-211

MAT 8625

( $\alpha, 2\alpha$ )

86-Rn-211

Radionuclide Production Cross Section



12

Incident Energy (MeV)

86-Rn-211