

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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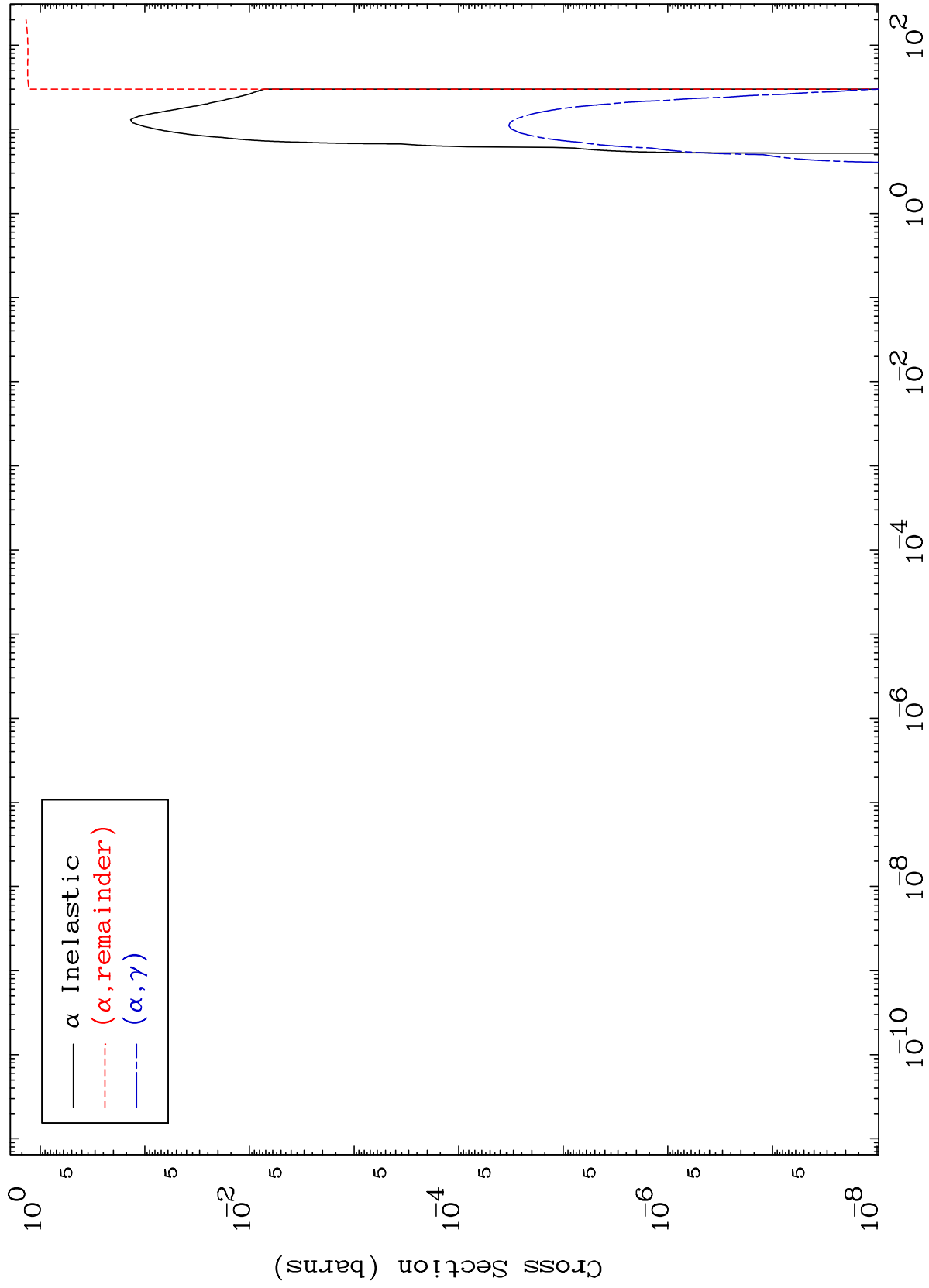
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 2119

0 Kelvin  $\alpha$  Major  
Cross Sections

21-Sc-43



1

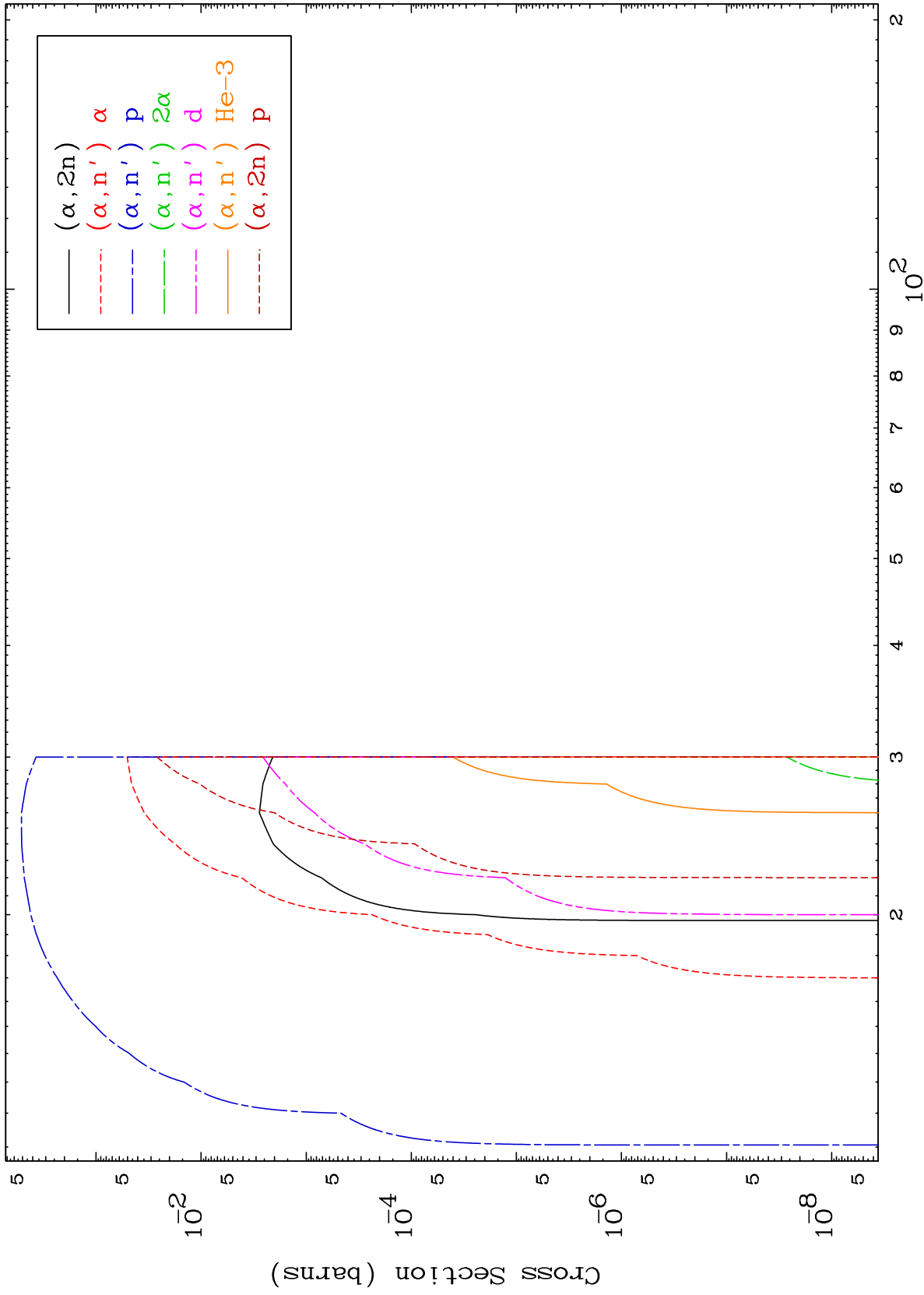
Incident Energy (MeV)

21-Sc-43

MAT 2119

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

21-Sc-43



2

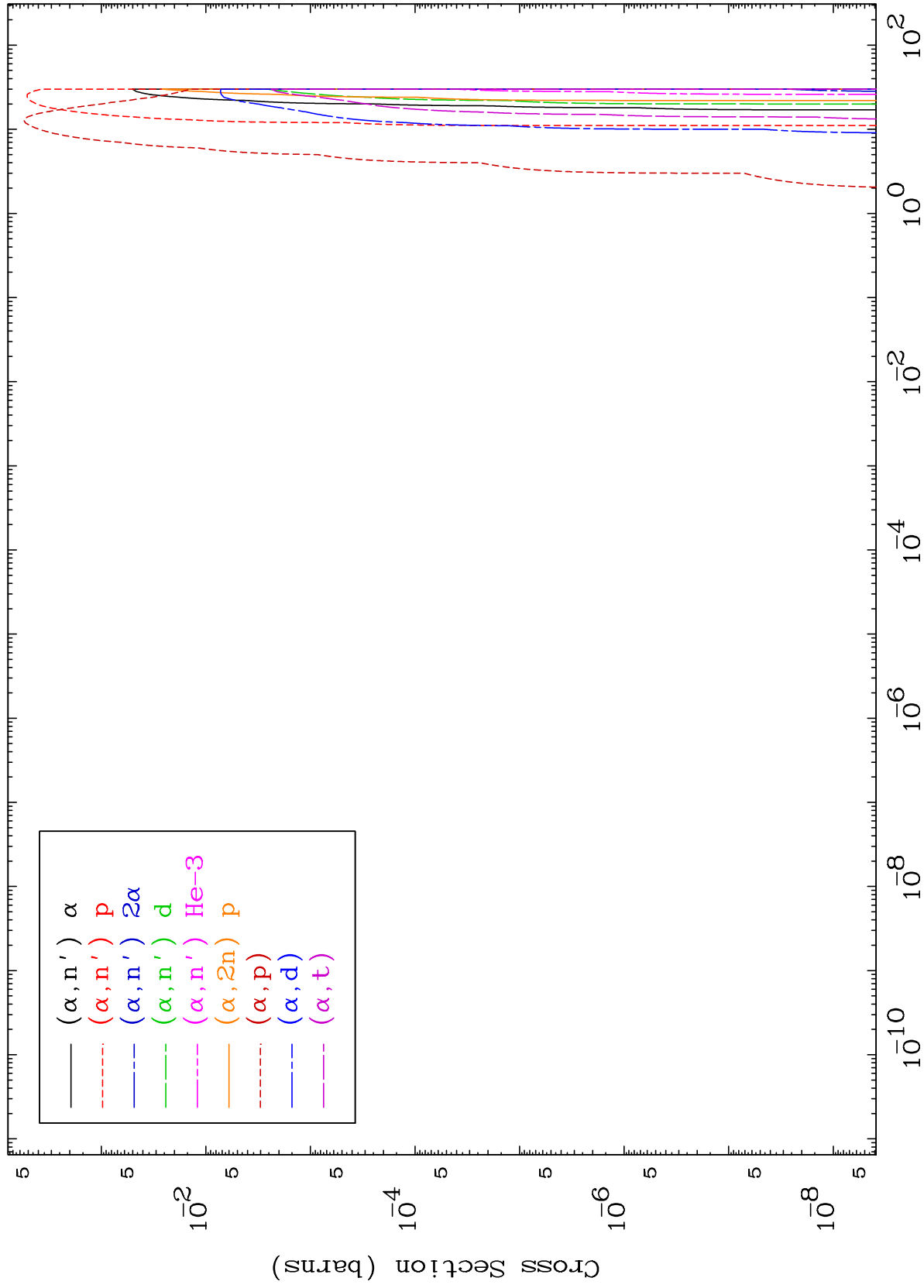
Incident Energy (MeV)

21-Sc-43

MAT 2119

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

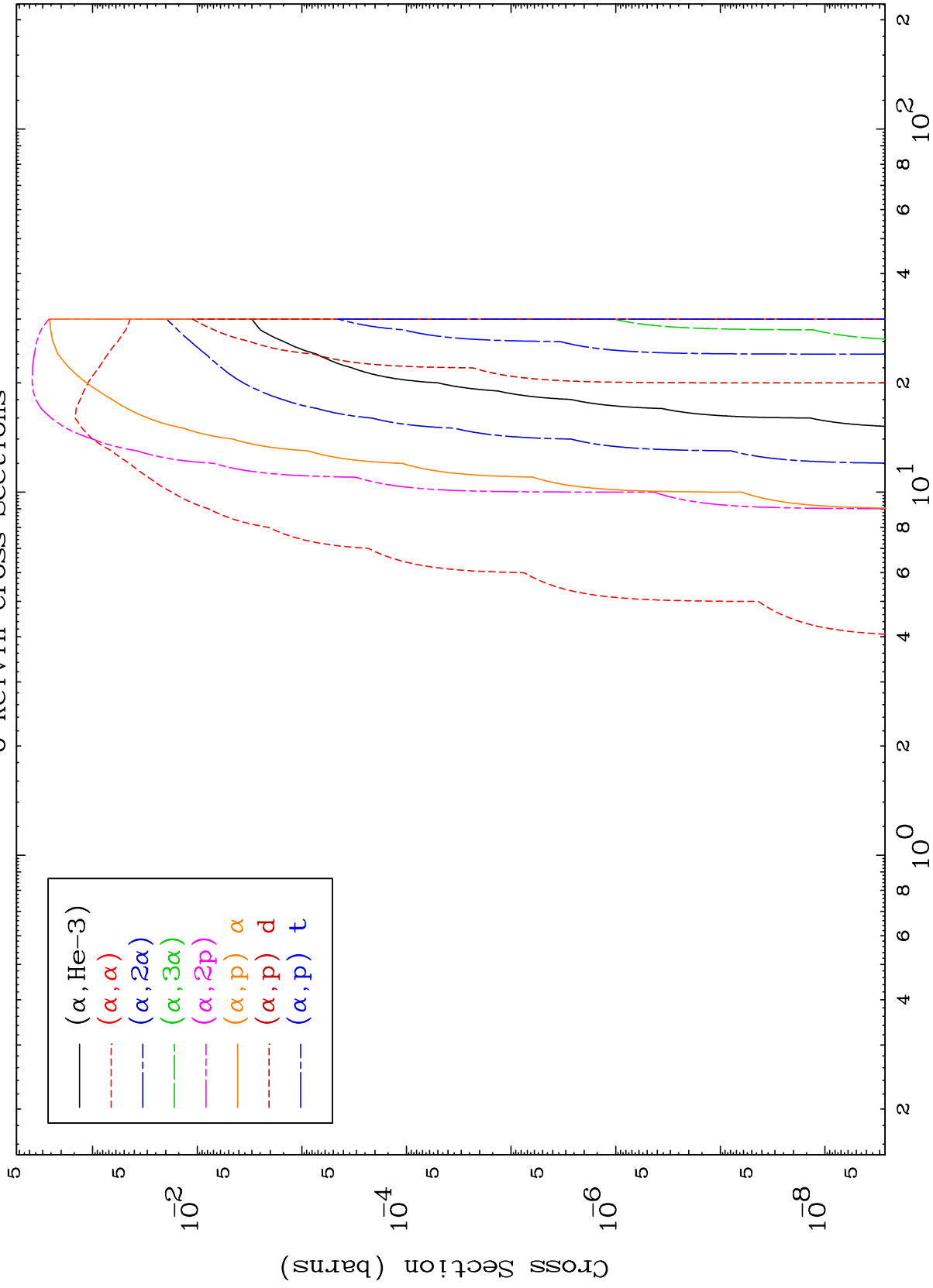
21-Sc-43



3

Incident Energy (MeV)

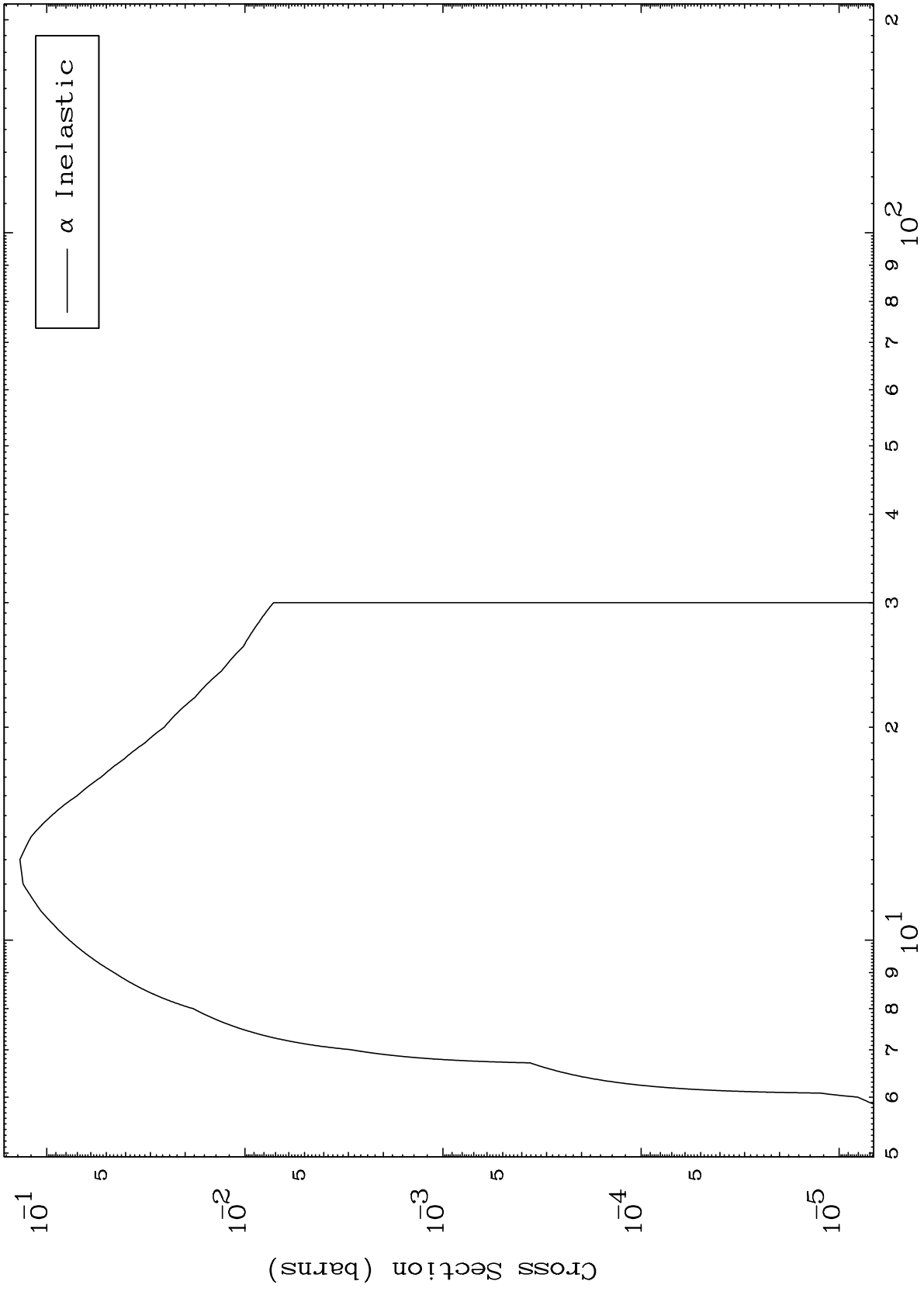
21-Sc-43



MAT 2119

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

21-Sc-43



5

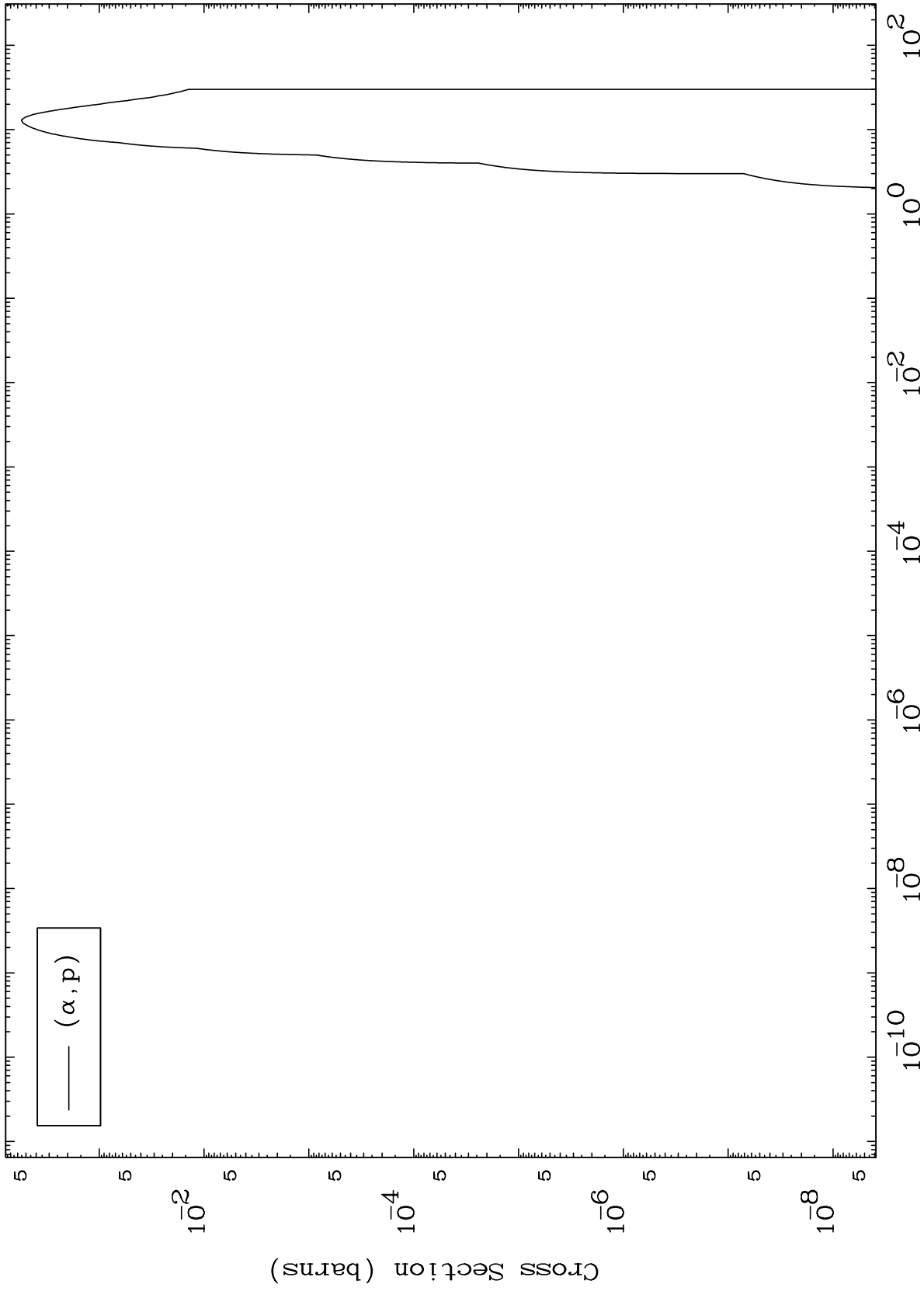
Incident Energy (MeV)

21-Sc-43

MAT 2119

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

21-Sc-43



6

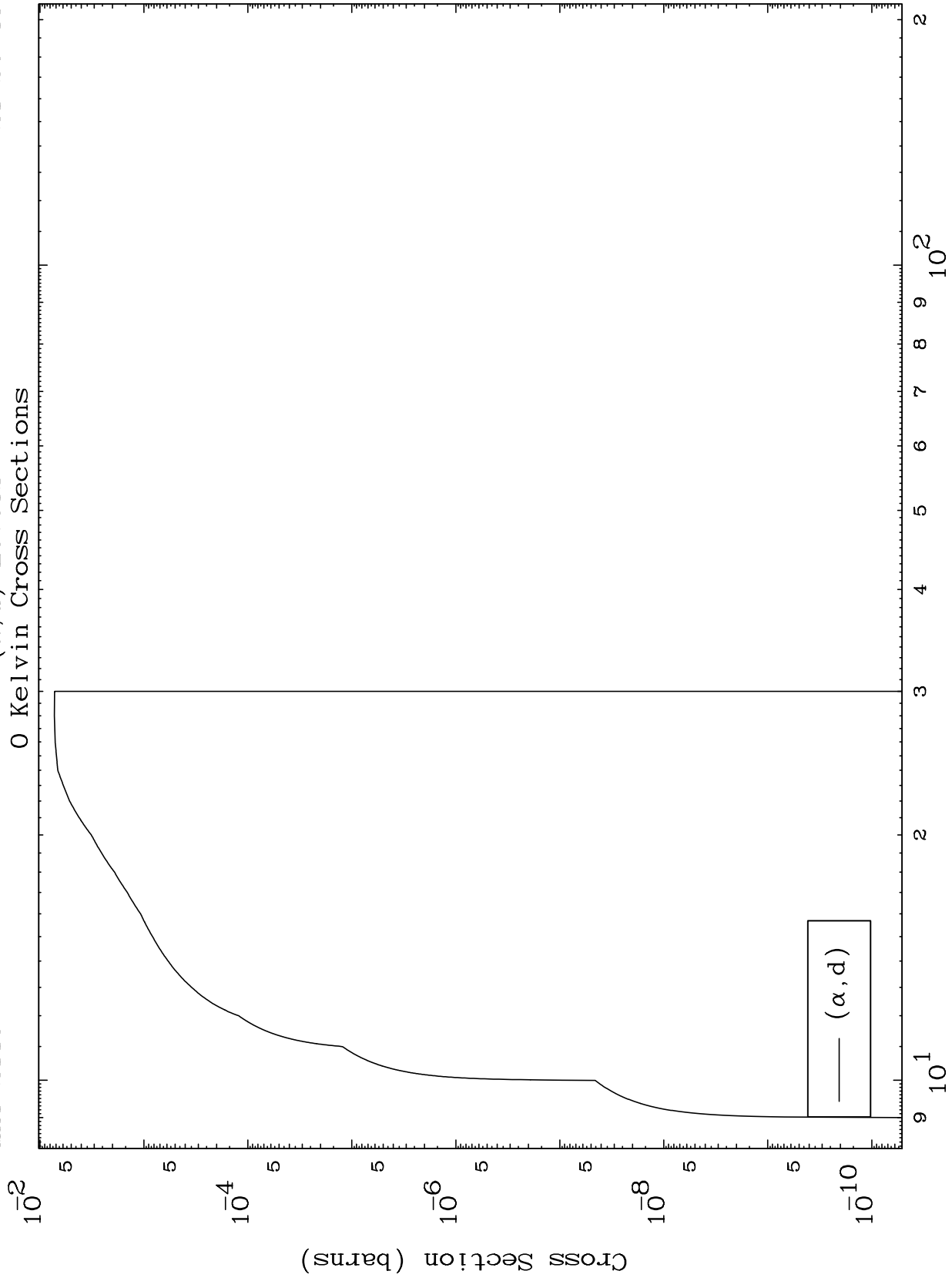
Incident Energy (MeV)

21-Sc-43

MAT 2119

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

21-Sc-43

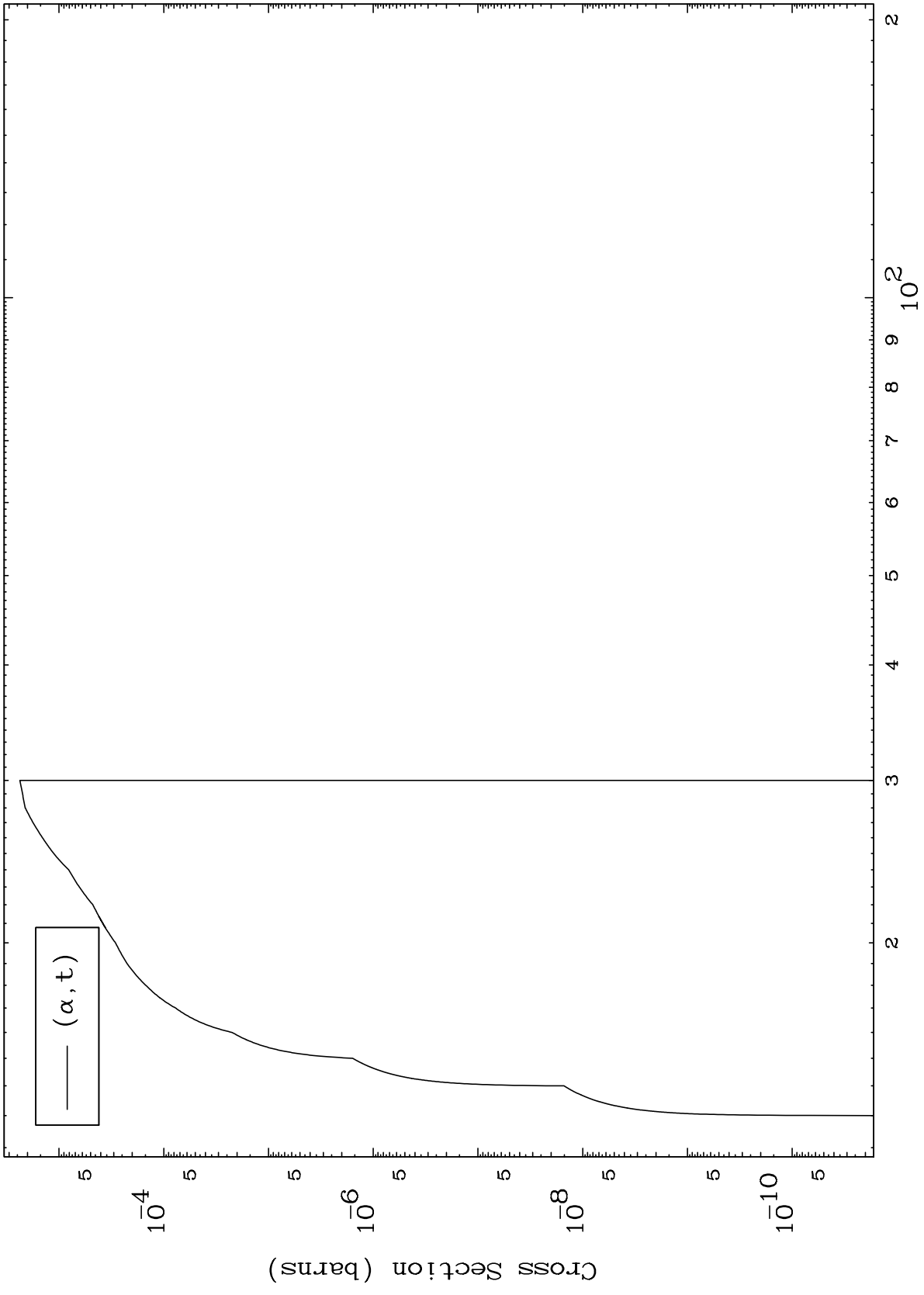


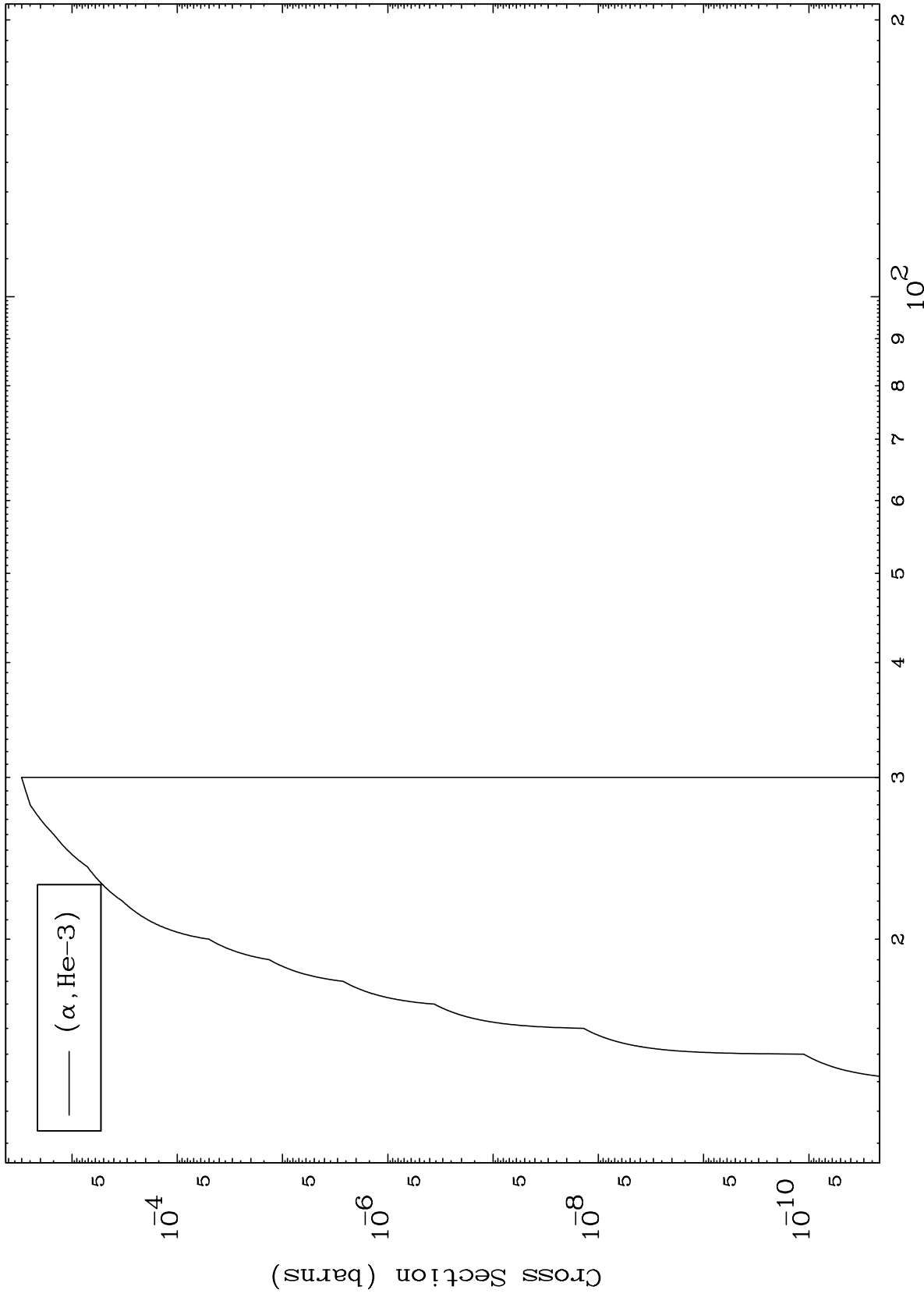
7

Incident Energy (MeV)

21-Sc-43



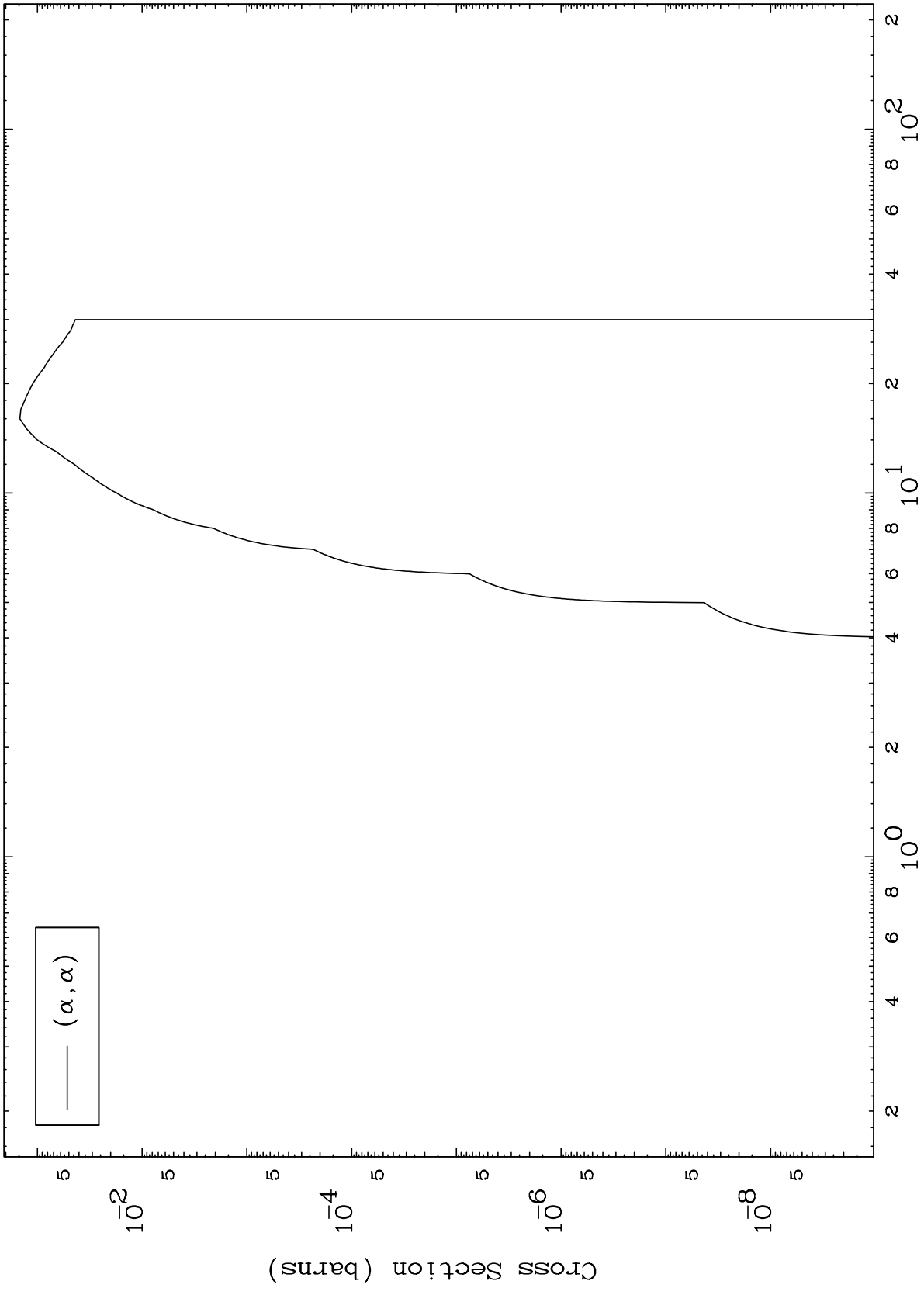




MAT 2119

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

21-Sc-43



10

Incident Energy (MeV)

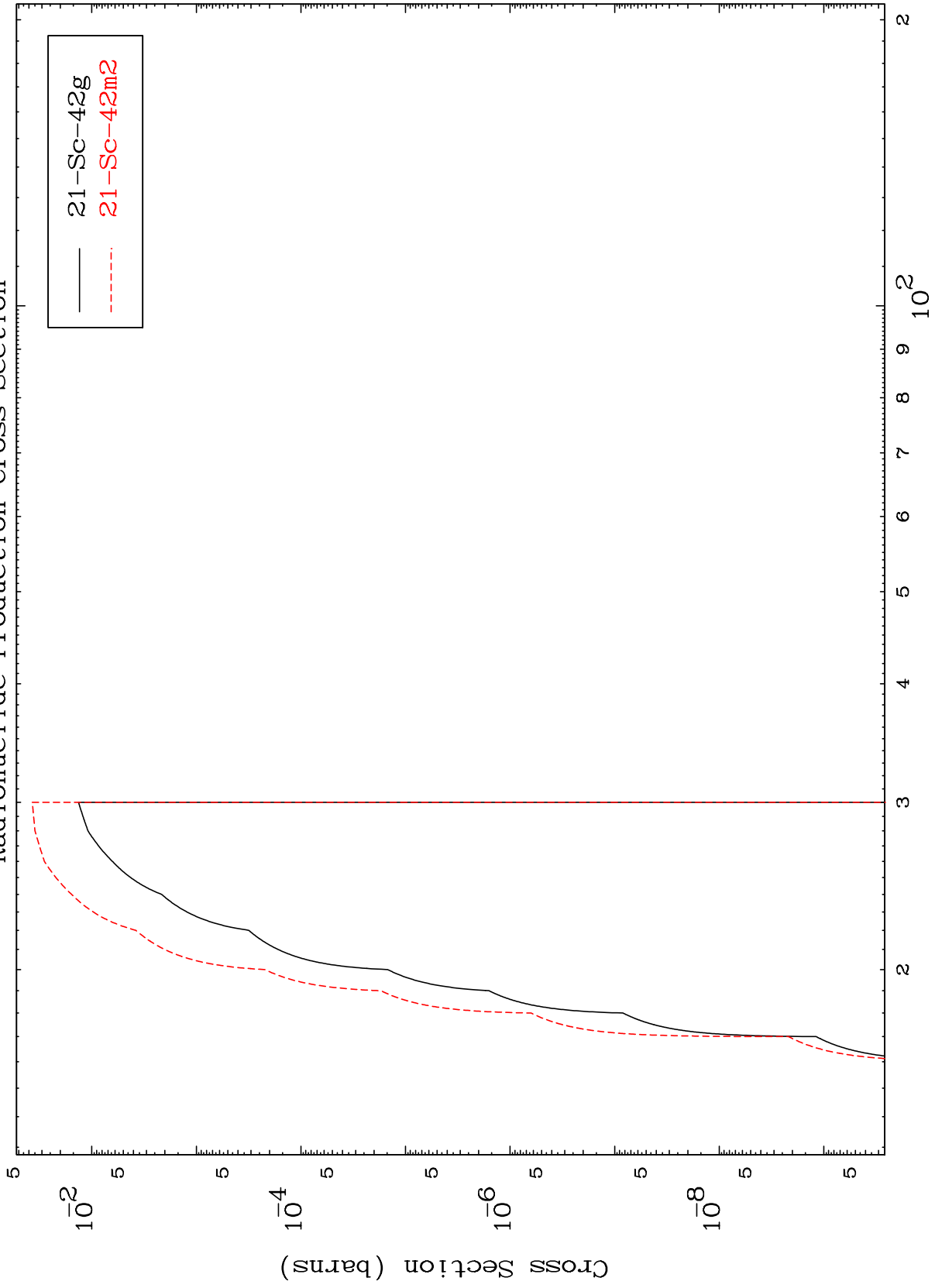
21-Sc-43

MAT 2119

$(\alpha, n')$   $\alpha$

21-Sc-43

Radionuclide Production Cross Section



11

Incident Energy (MeV)

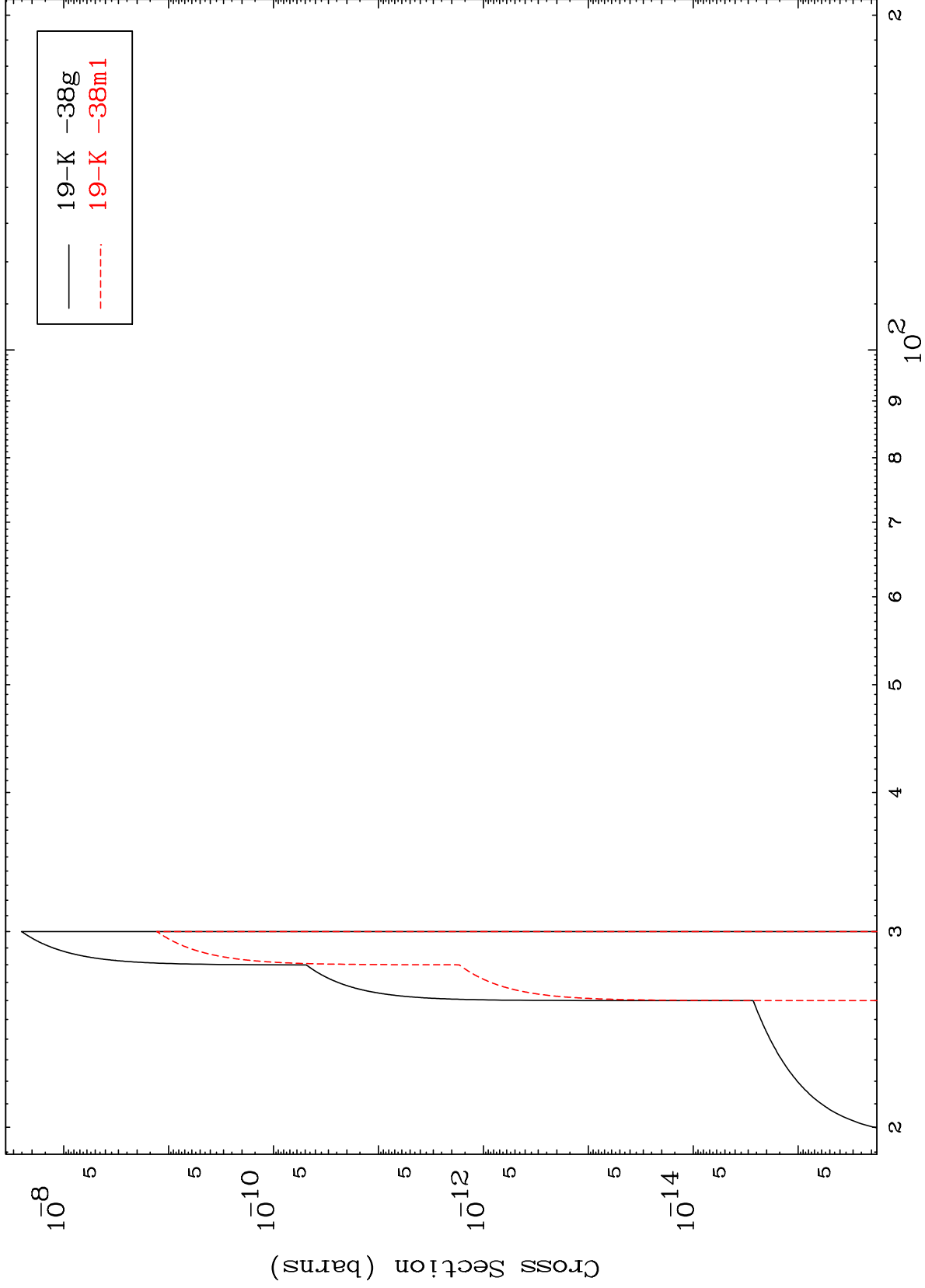
21-Sc-43

MAT 2119

( $\alpha, n'$ )  $2\alpha$

21-Sc-43

Radionuclide Production Cross Section



12

Incident Energy (MeV)

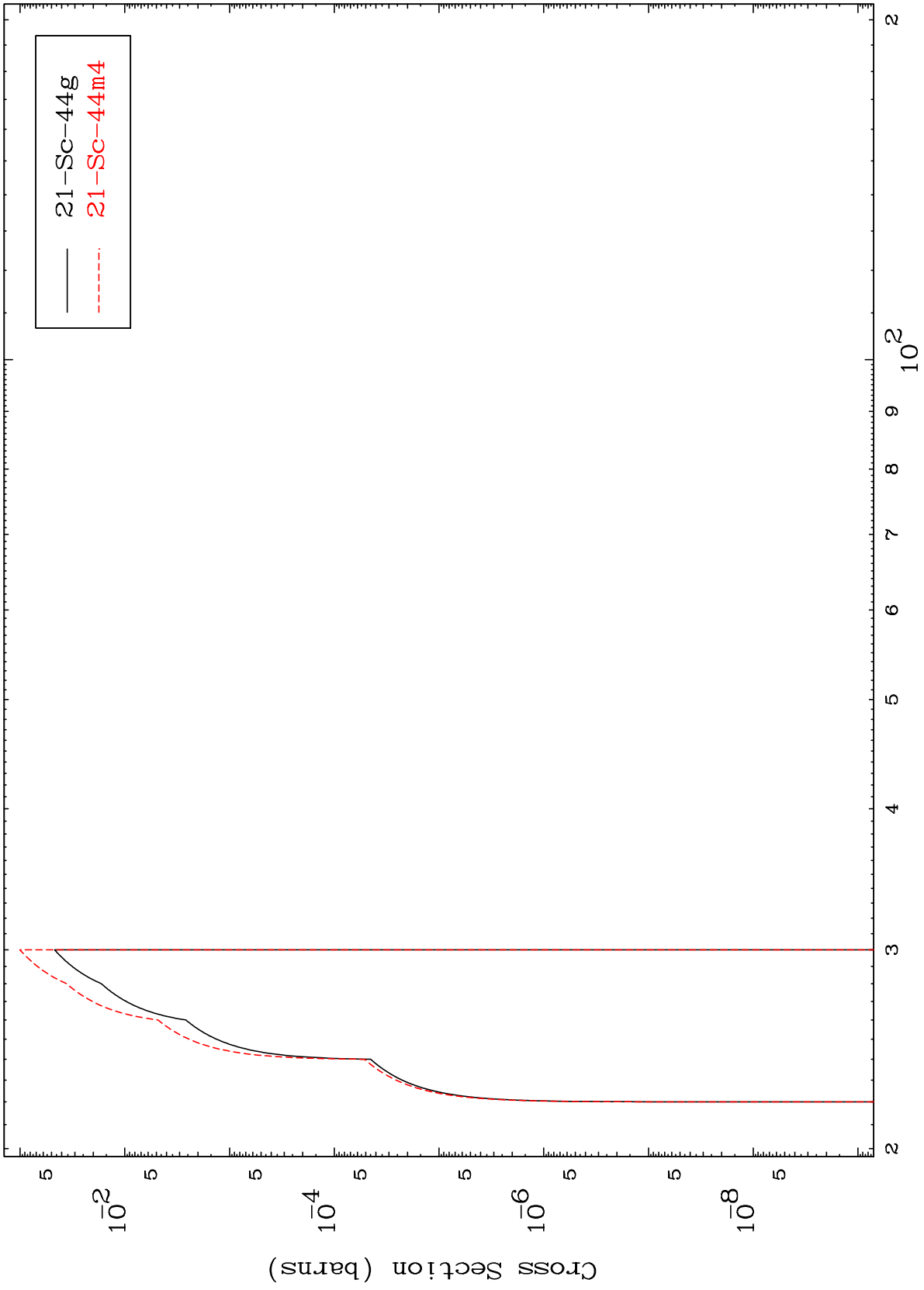
21-Sc-43

MAT 2119

( $\alpha, 2n$ ) p

21-Sc-43

Radionuclide Production Cross Section



13

Incident Energy (MeV)

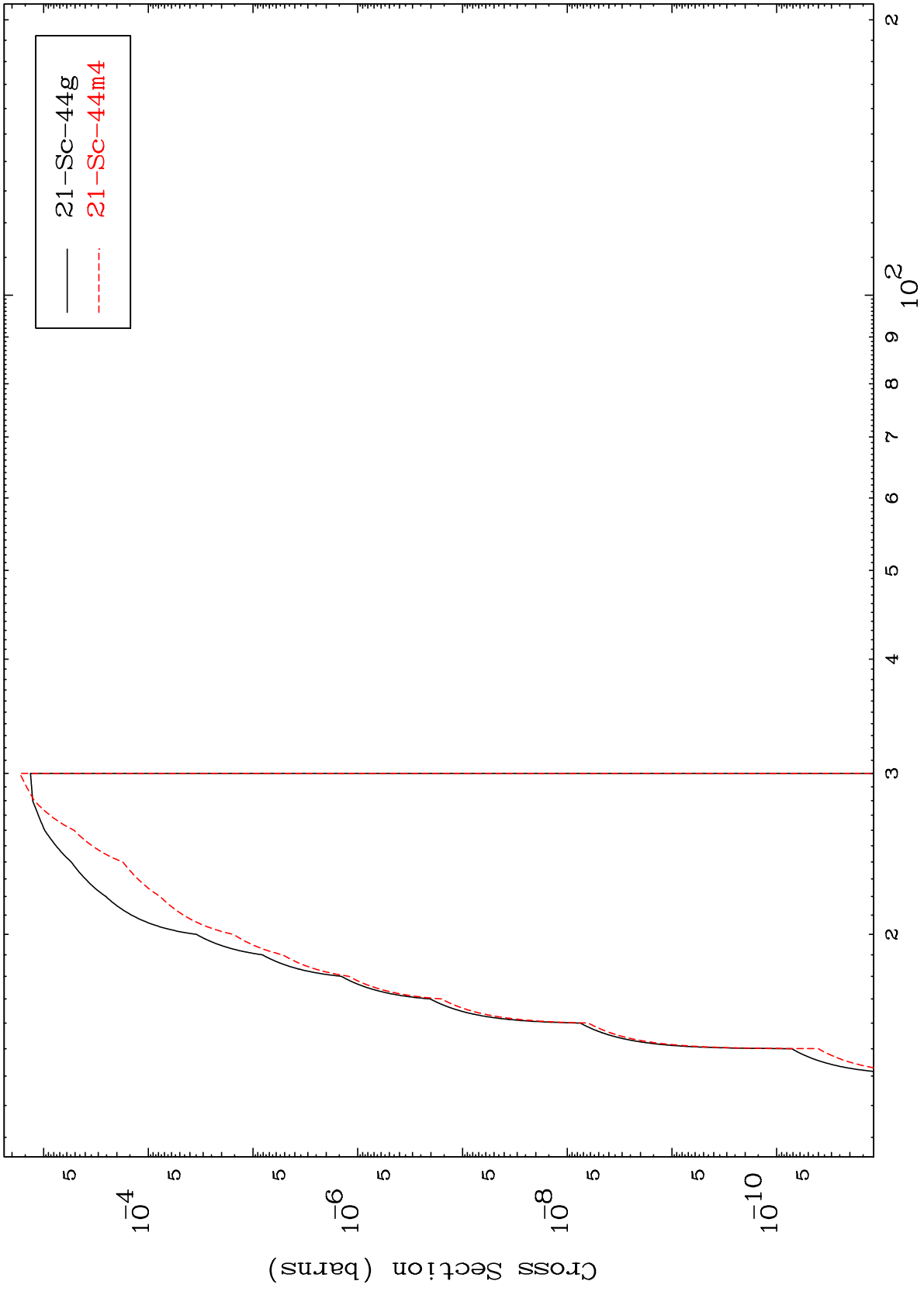
21-Sc-43

MAT 2119

( $\alpha, \text{He-3}$ )

21-Sc-43

Radionuclide Production Cross Section



14

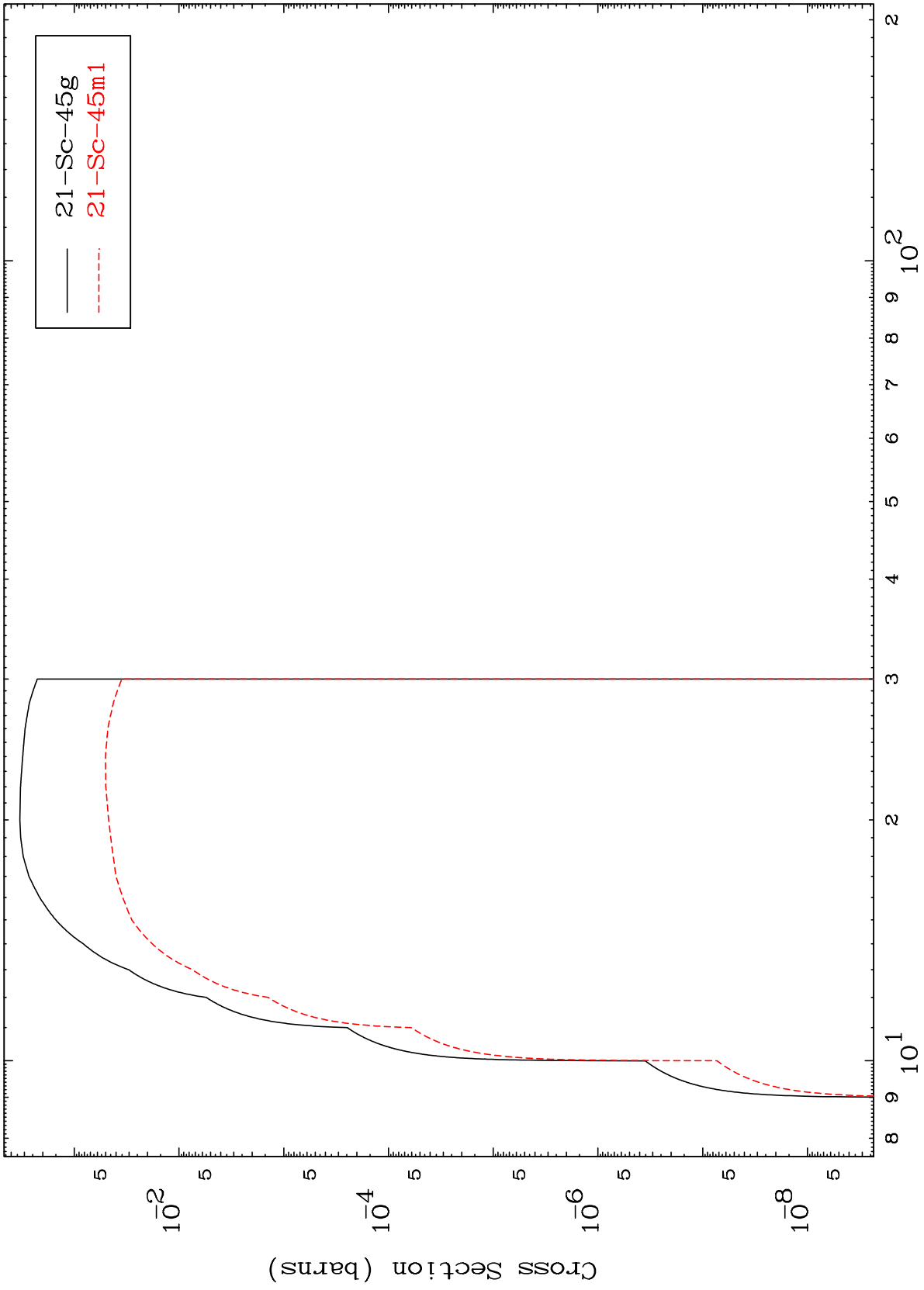
Incident Energy (MeV)

21-Sc-43

MAT 2119

21-Sc-43

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



15

Incident Energy (MeV)

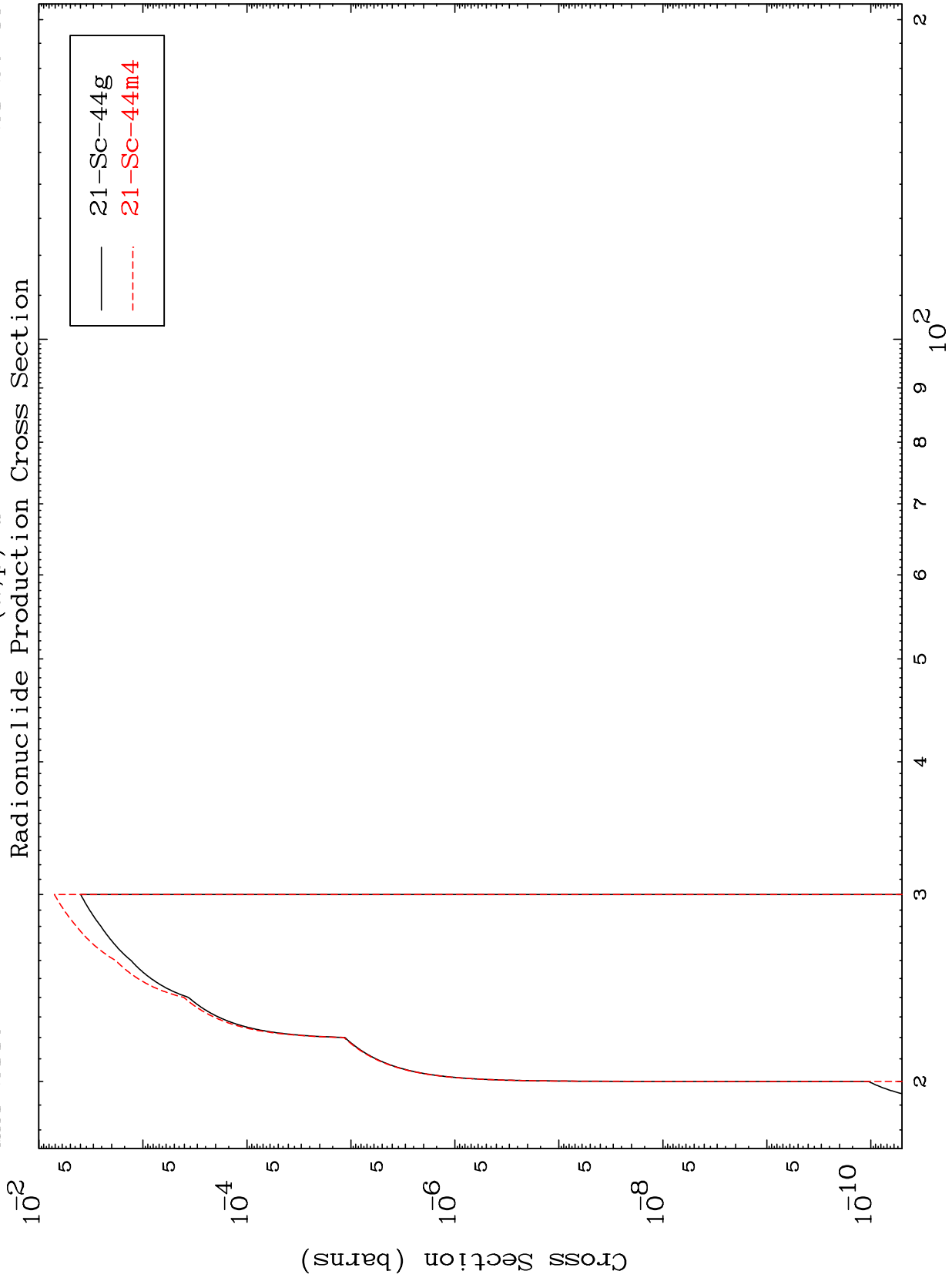
21-Sc-43



MAT 2119

( $\alpha, p$ ) d

21-Sc-43



16

Incident Energy (MeV)

21-Sc-43