

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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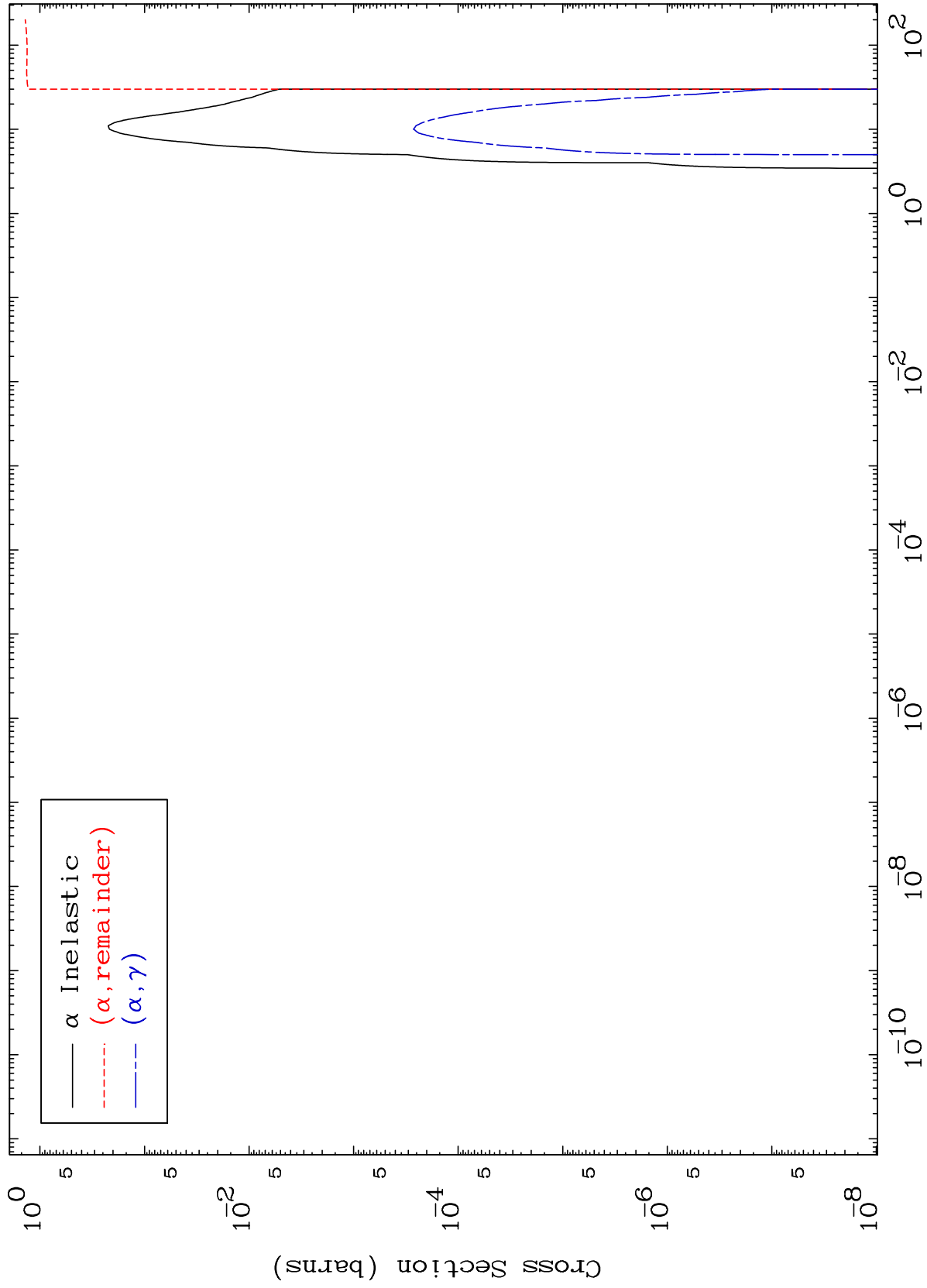
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 2123

0 Kelvin  $\alpha$  Major  
Cross Sections

21-Sc-44



1

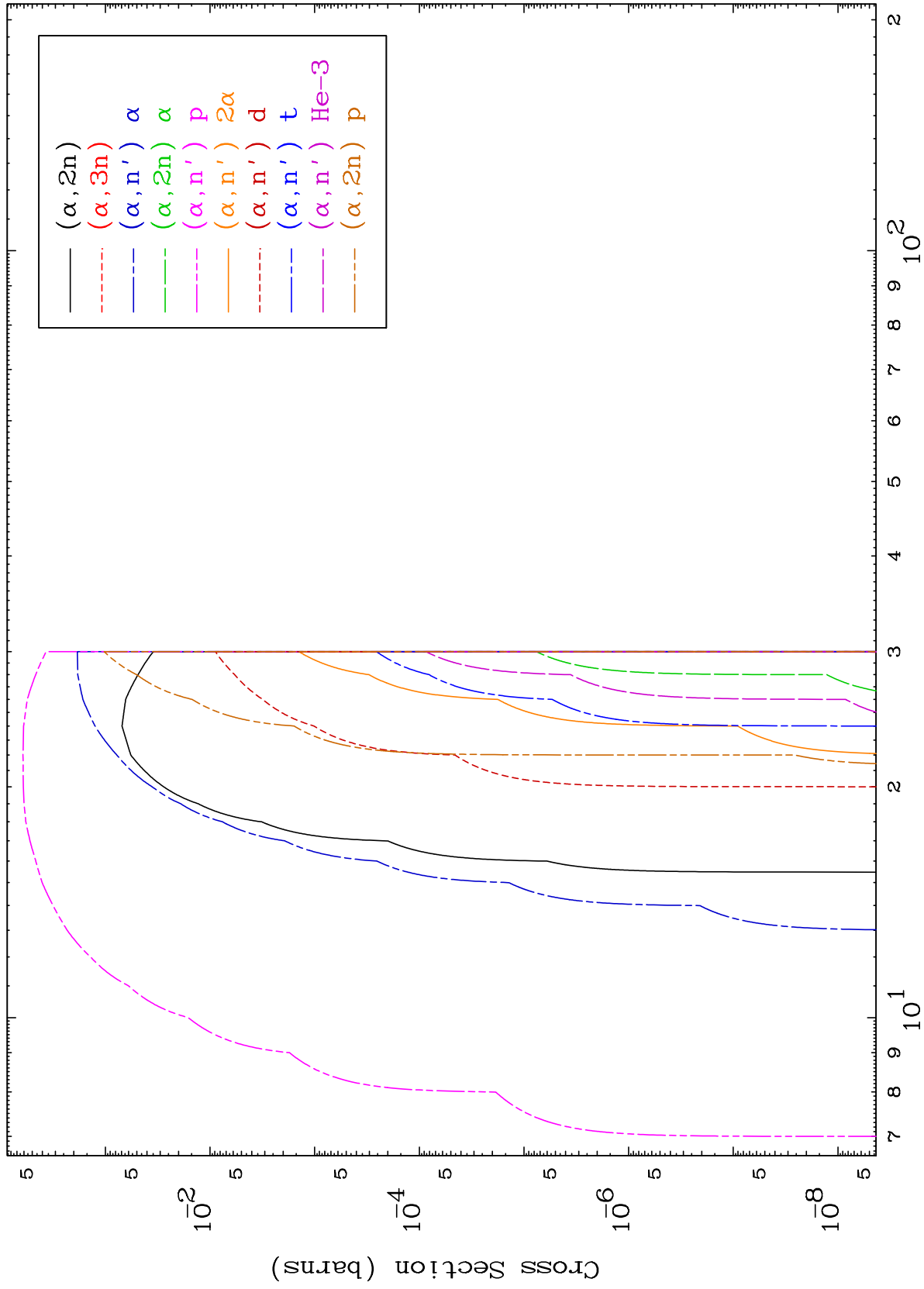
Incident Energy (MeV)

21-Sc-44

MAT 2123

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

21-Sc-44



2

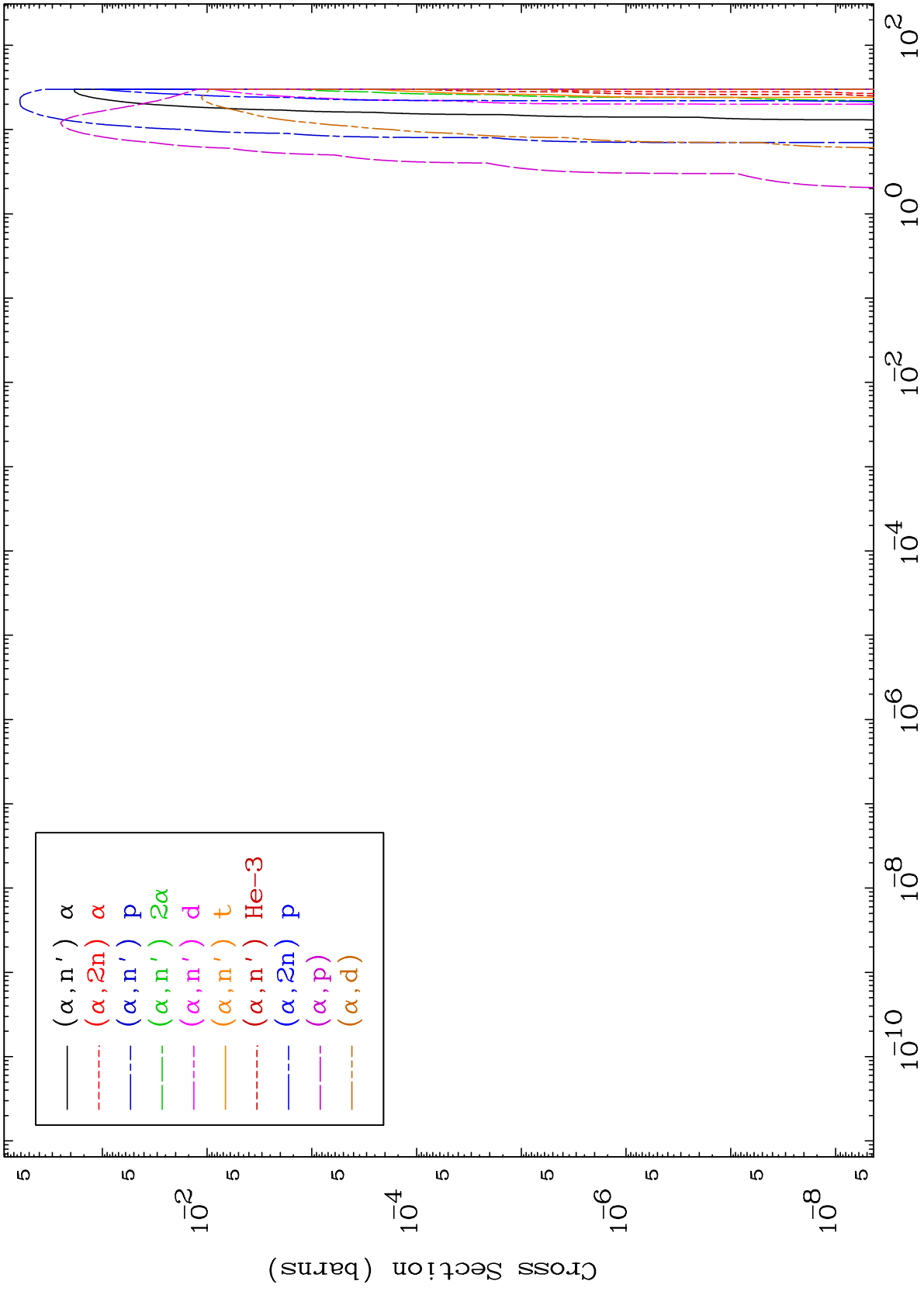
Incident Energy (MeV)

21-Sc-44

MAT 2123

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

21-Sc-44

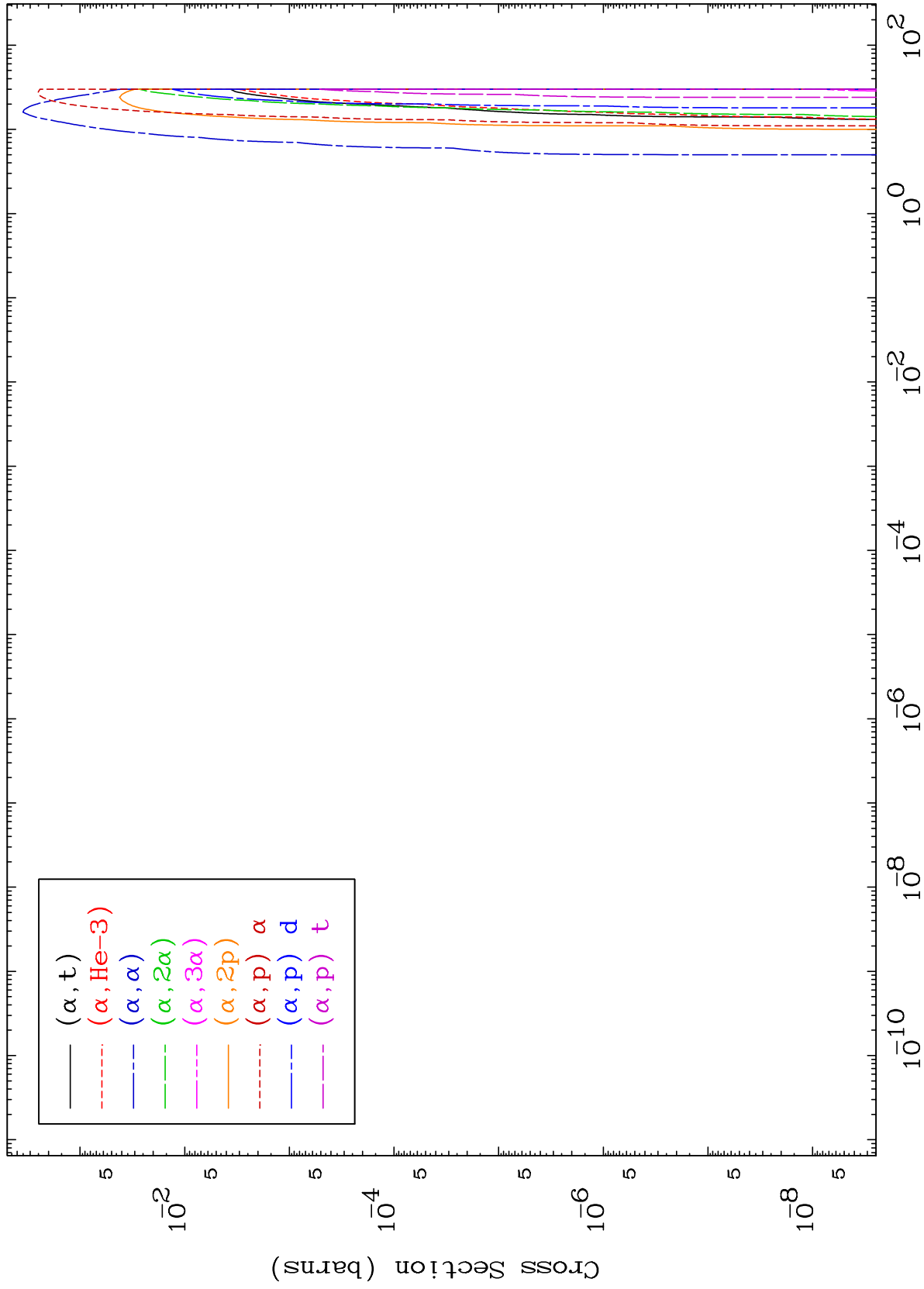


21-Sc-44

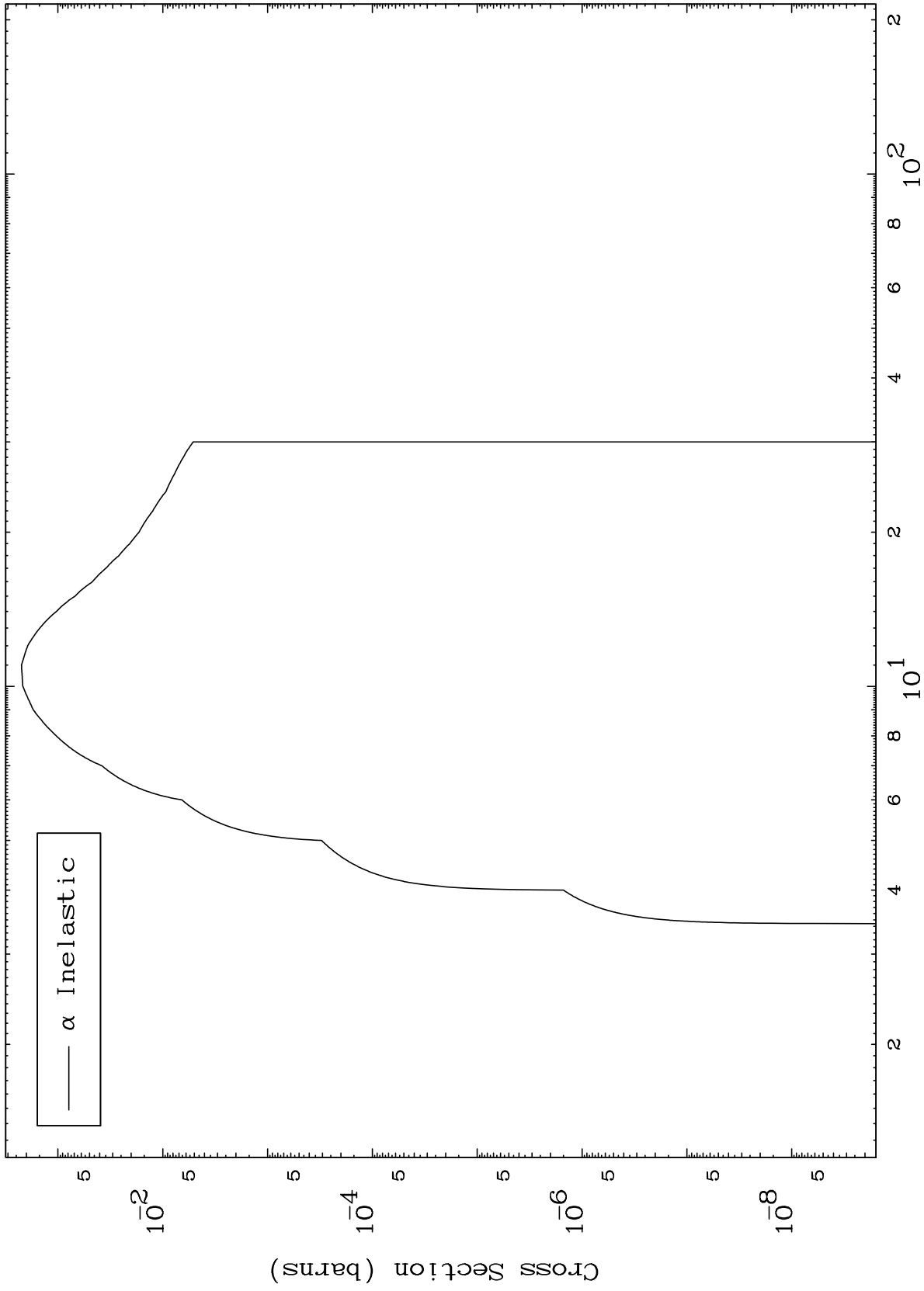
MAT 2123

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

21-Sc-44



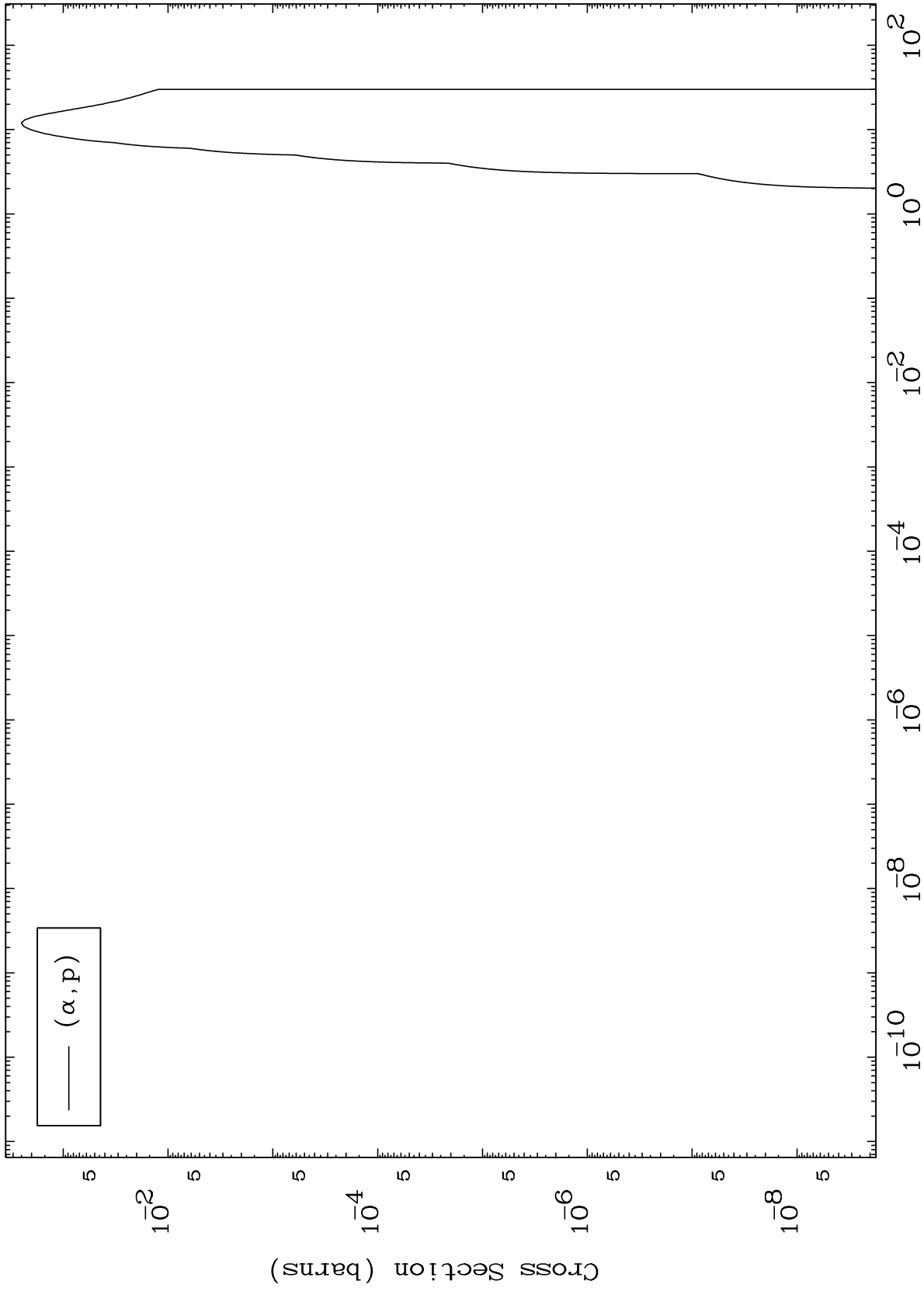
( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections



MAT 2123

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

21-Sc-44



6

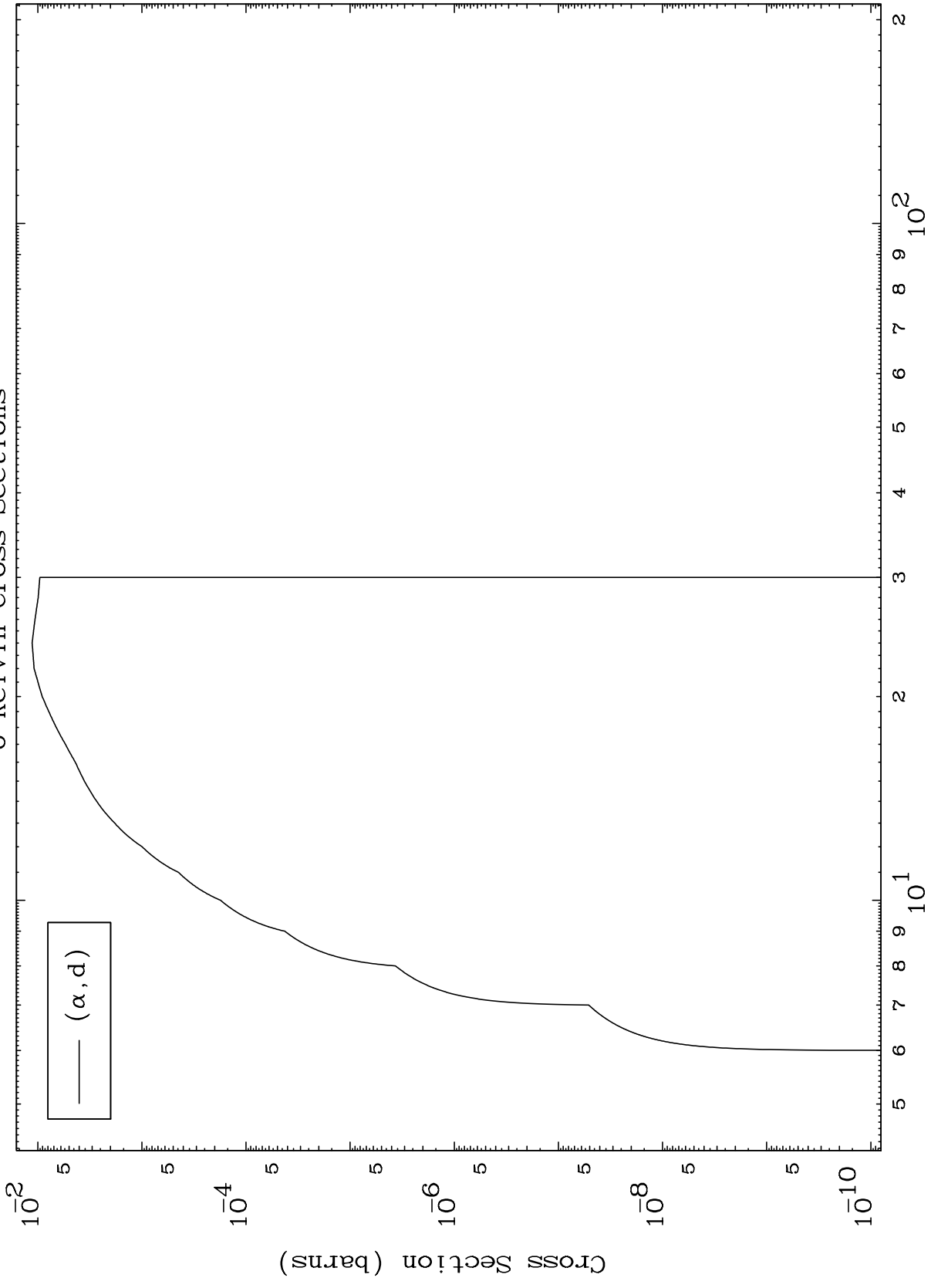
Incident Energy (MeV)

21-Sc-44

MAT 2123

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

21-Sc-44

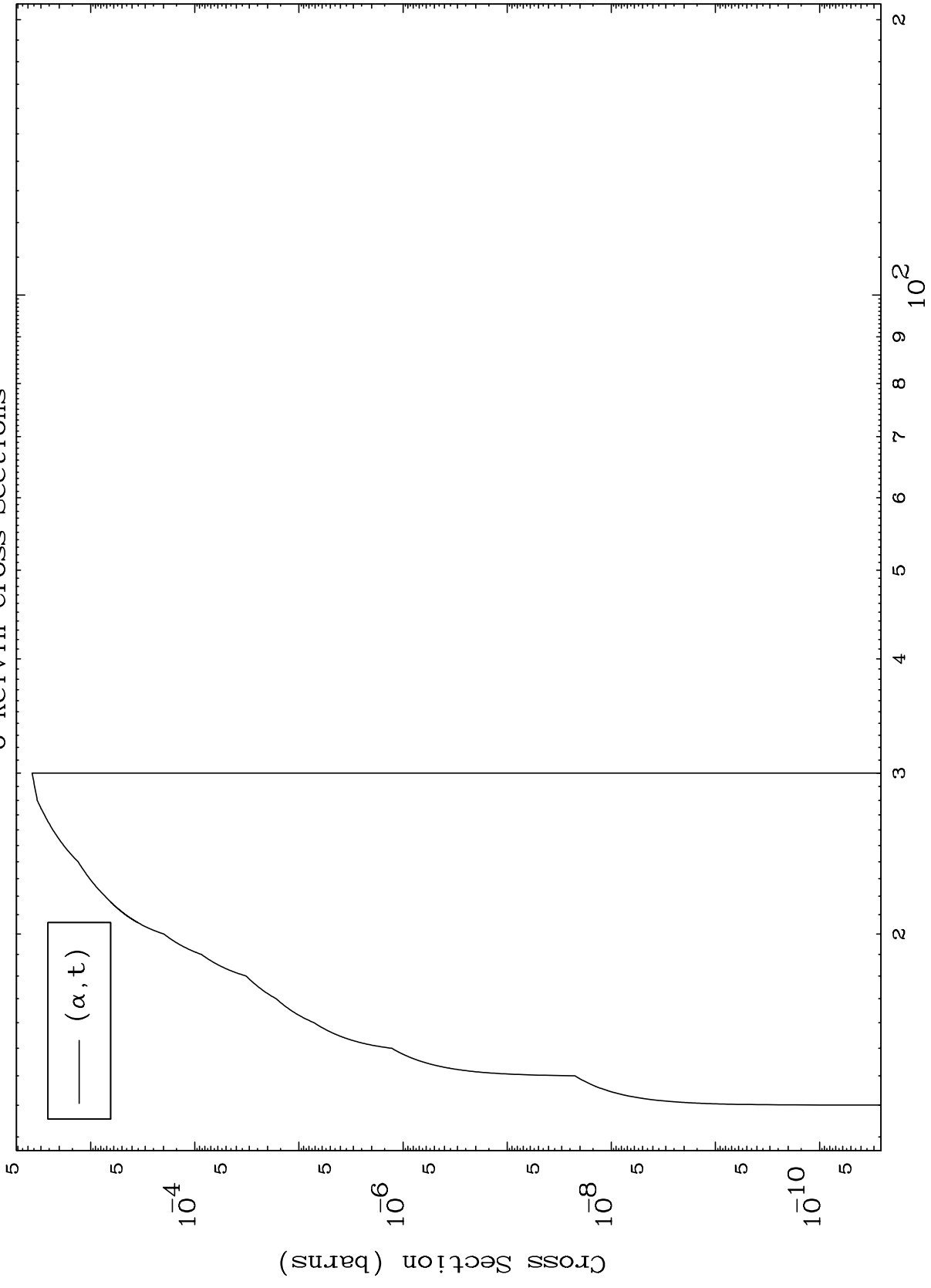


7

Incident Energy (MeV)

21-Sc-44

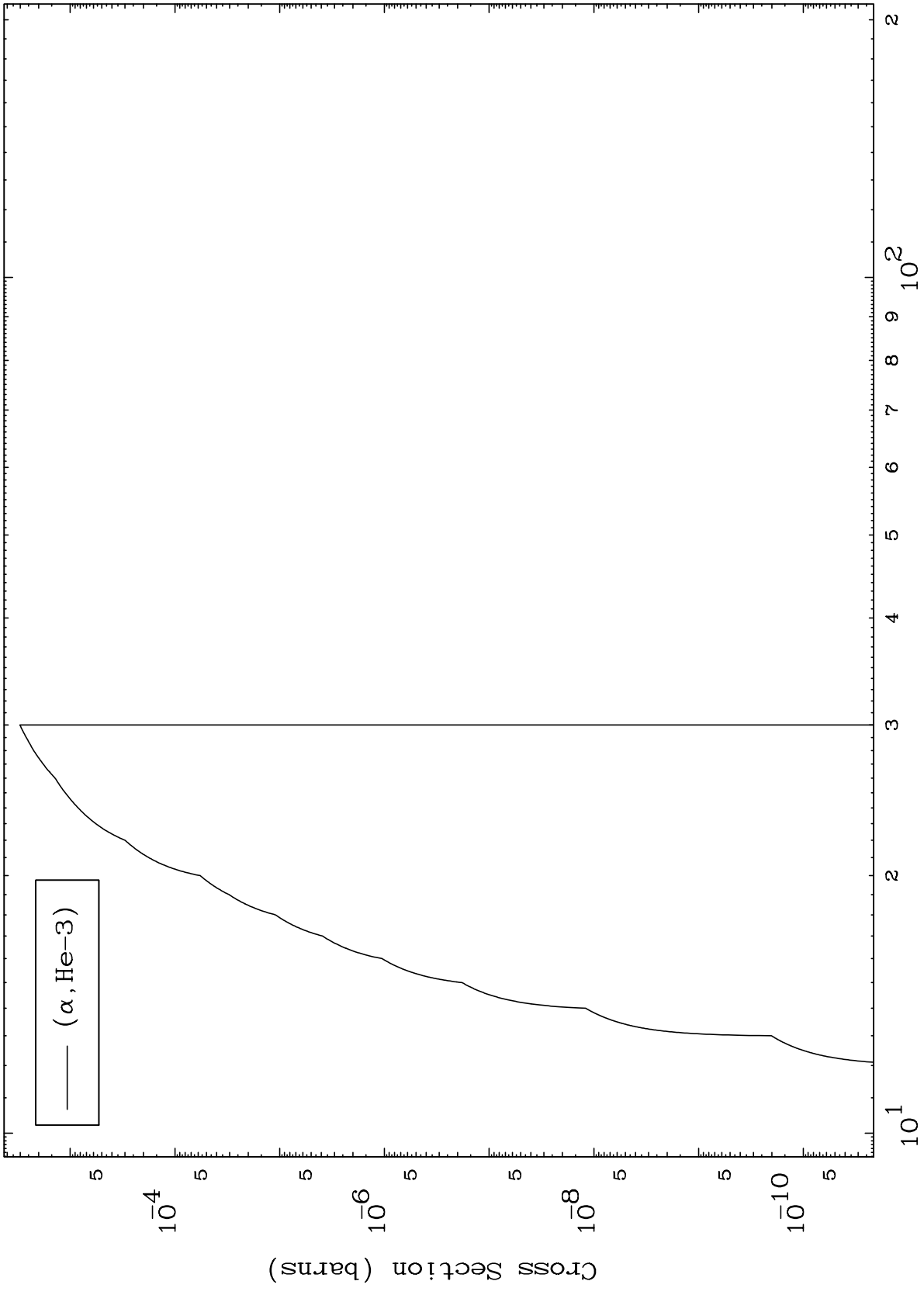




MAT 2123

( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections

21-Sc-44



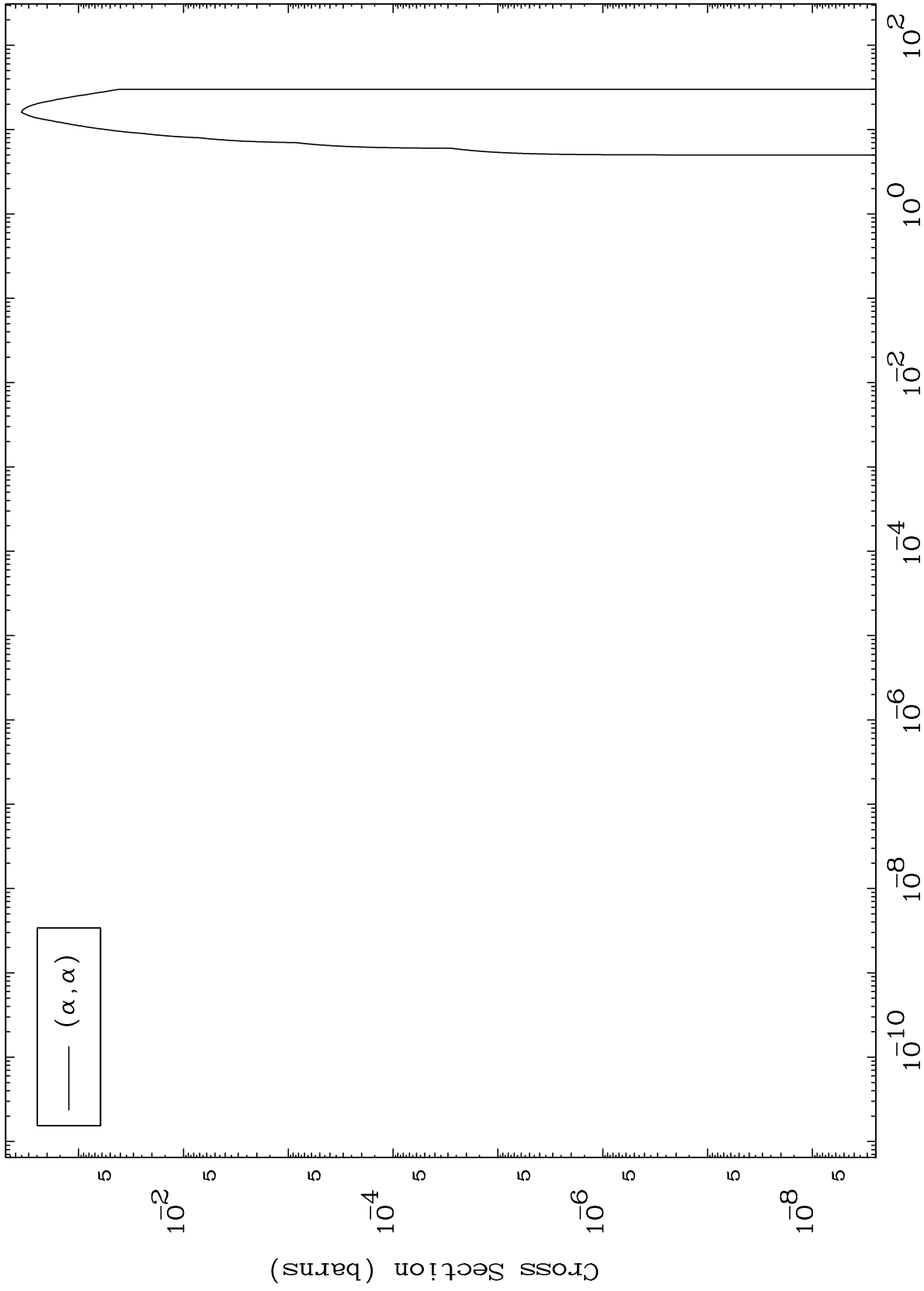
Incident Energy (MeV)

21-Sc-44

MAT 2123

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

21-Sc-44



10

Incident Energy (MeV)

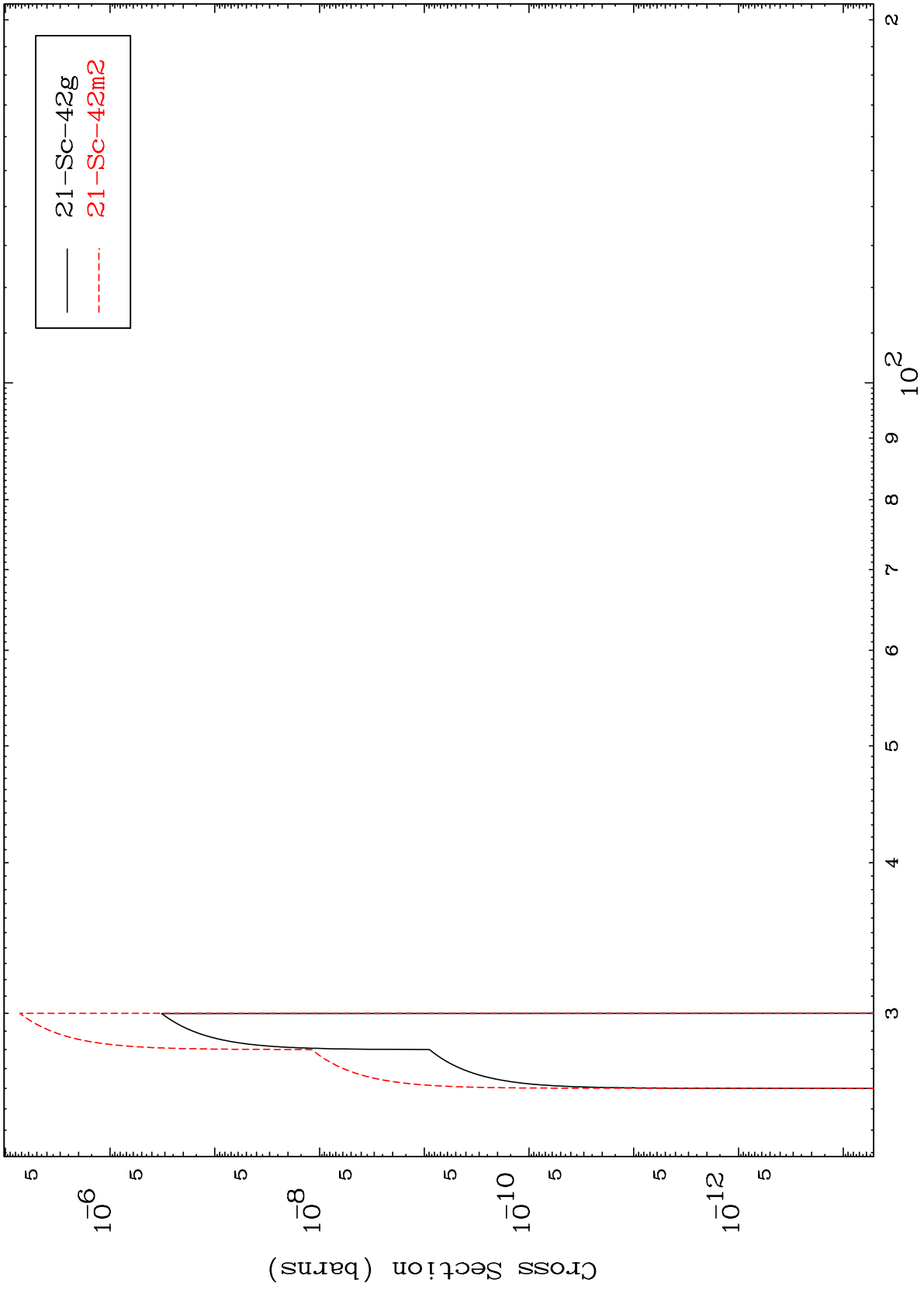
21-Sc-44

MAT 2123

( $\alpha, 2n$ )  $\alpha$

21-Sc-44

Radionuclide Production Cross Section



11

Incident Energy (MeV)

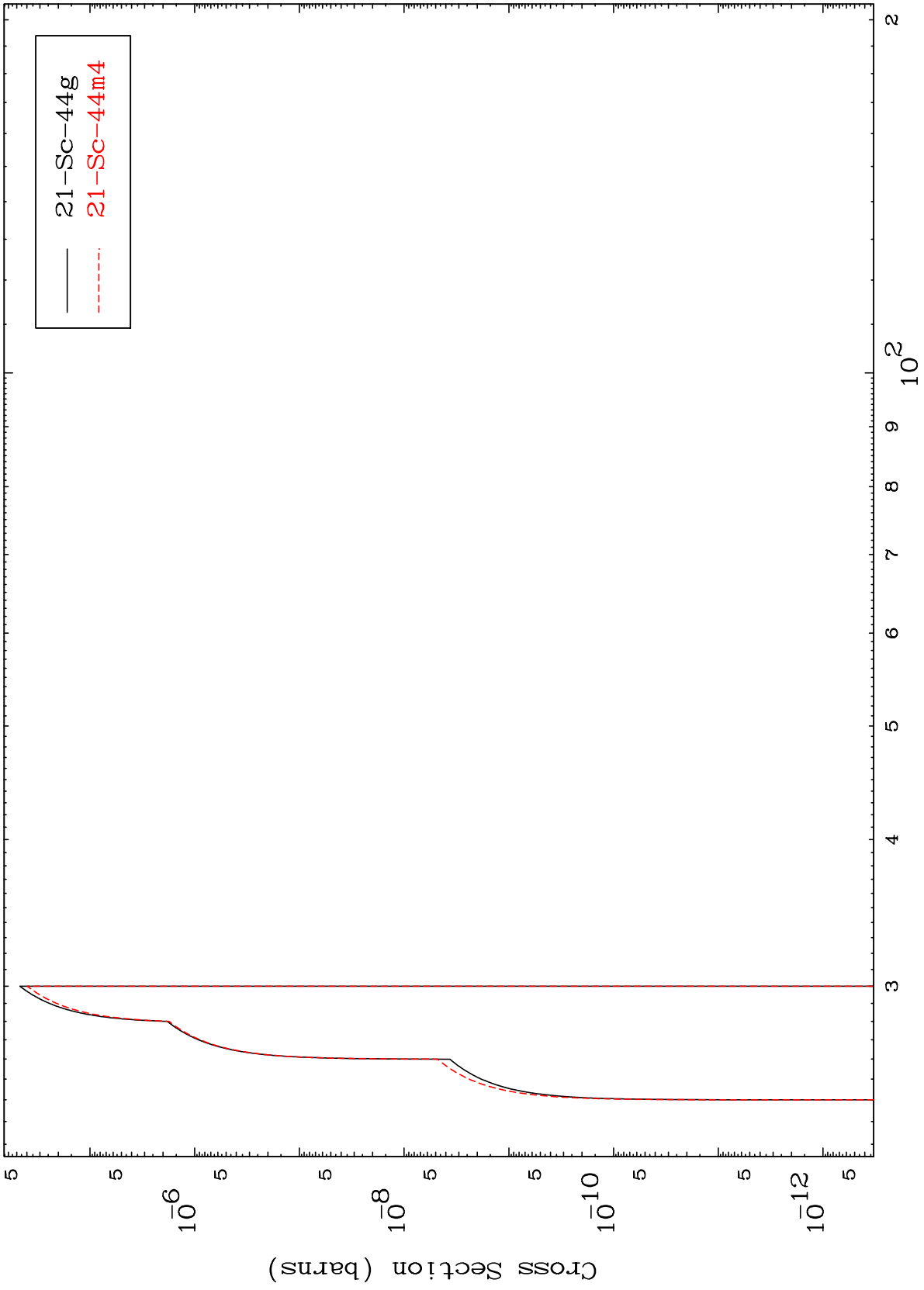
21-Sc-44

MAT 2123

( $\alpha, n'$ ) He-3

21-Sc-44

Radionuclide Production Cross Section



12

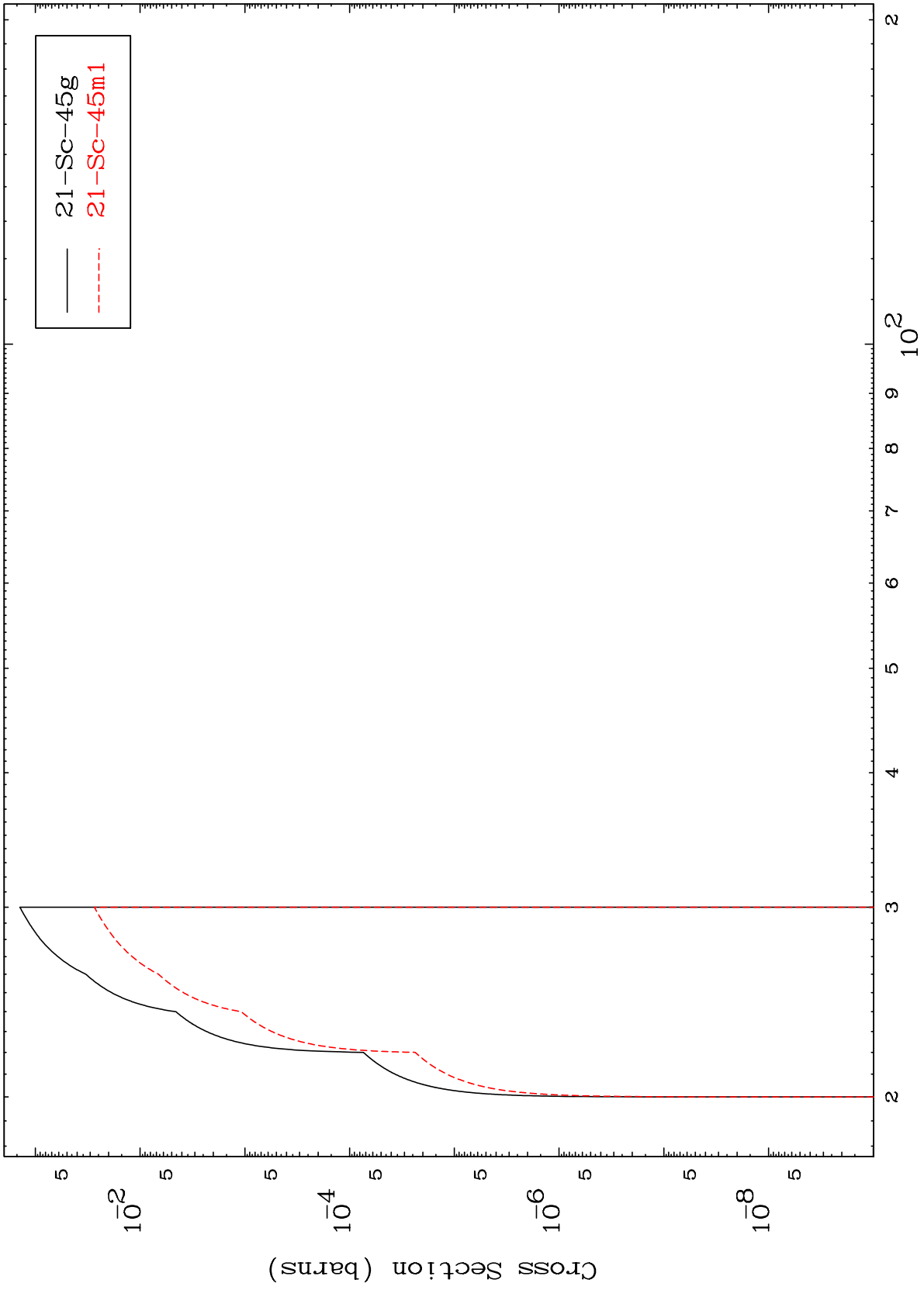
Incident Energy (MeV)

21-Sc-44

MAT 2123

21-Sc-44

( $\alpha, 2n$ ) p  
Radionuclide Production Cross Section



13

21-Sc-44

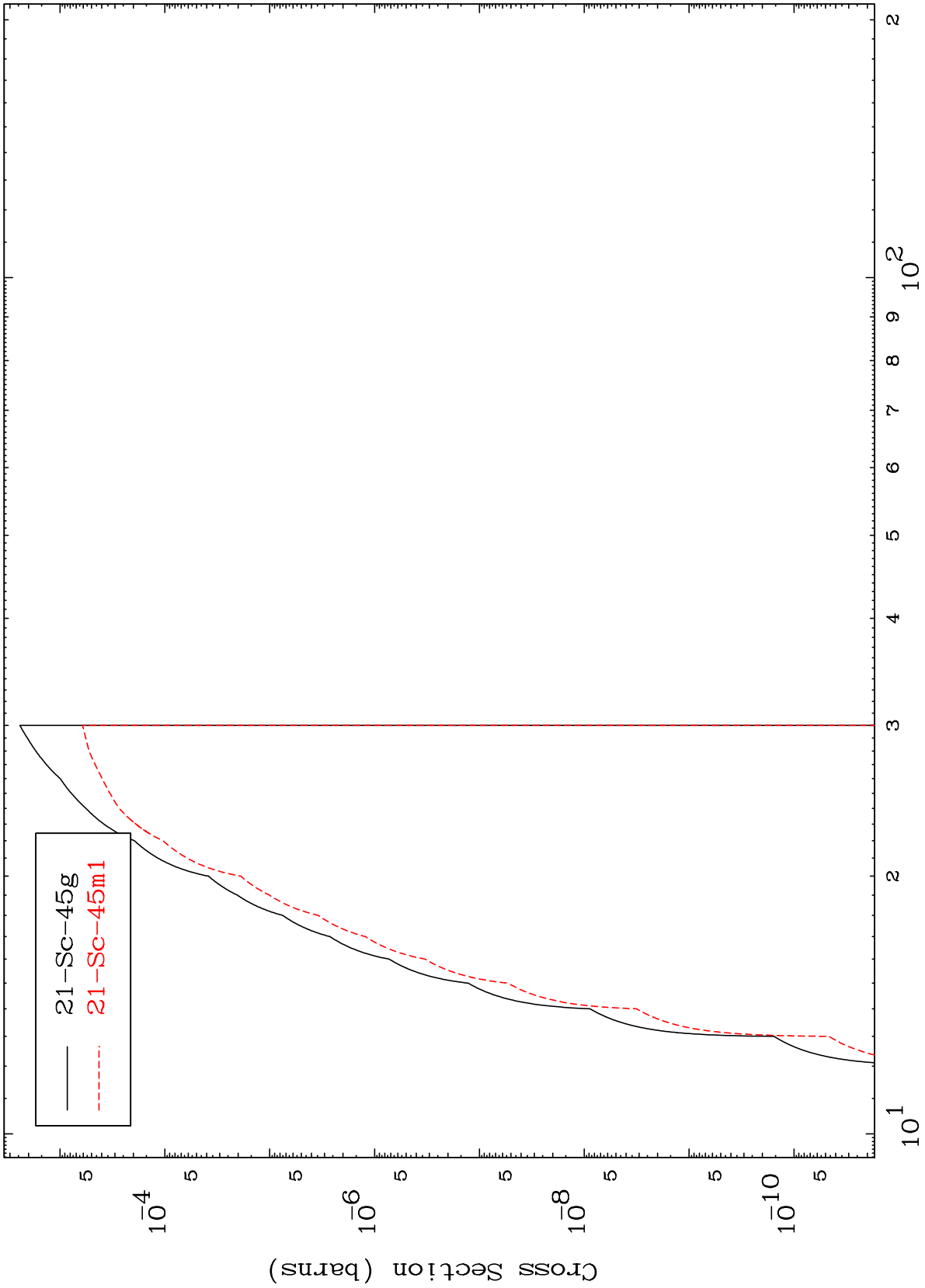
Incident Energy (MeV)

MAT 2123

( $\alpha$ , He-3)

21-Sc-44

Radionuclide Production Cross Section



14

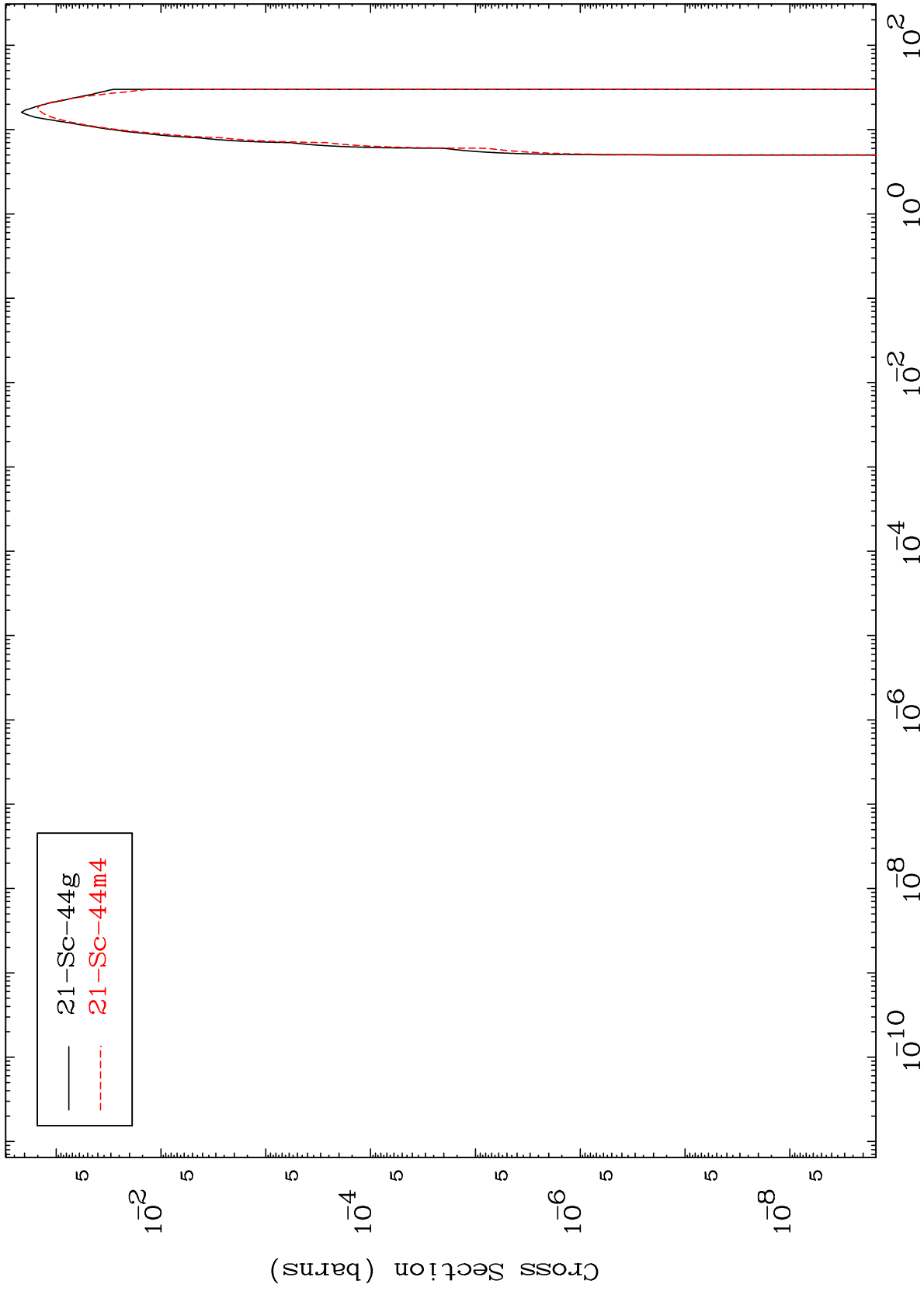
Incident Energy (MeV)

21-Sc-44

MAT 2123

21-Sc-44

( $\alpha, \alpha$ )  
Radionuclide Production Cross Section



15

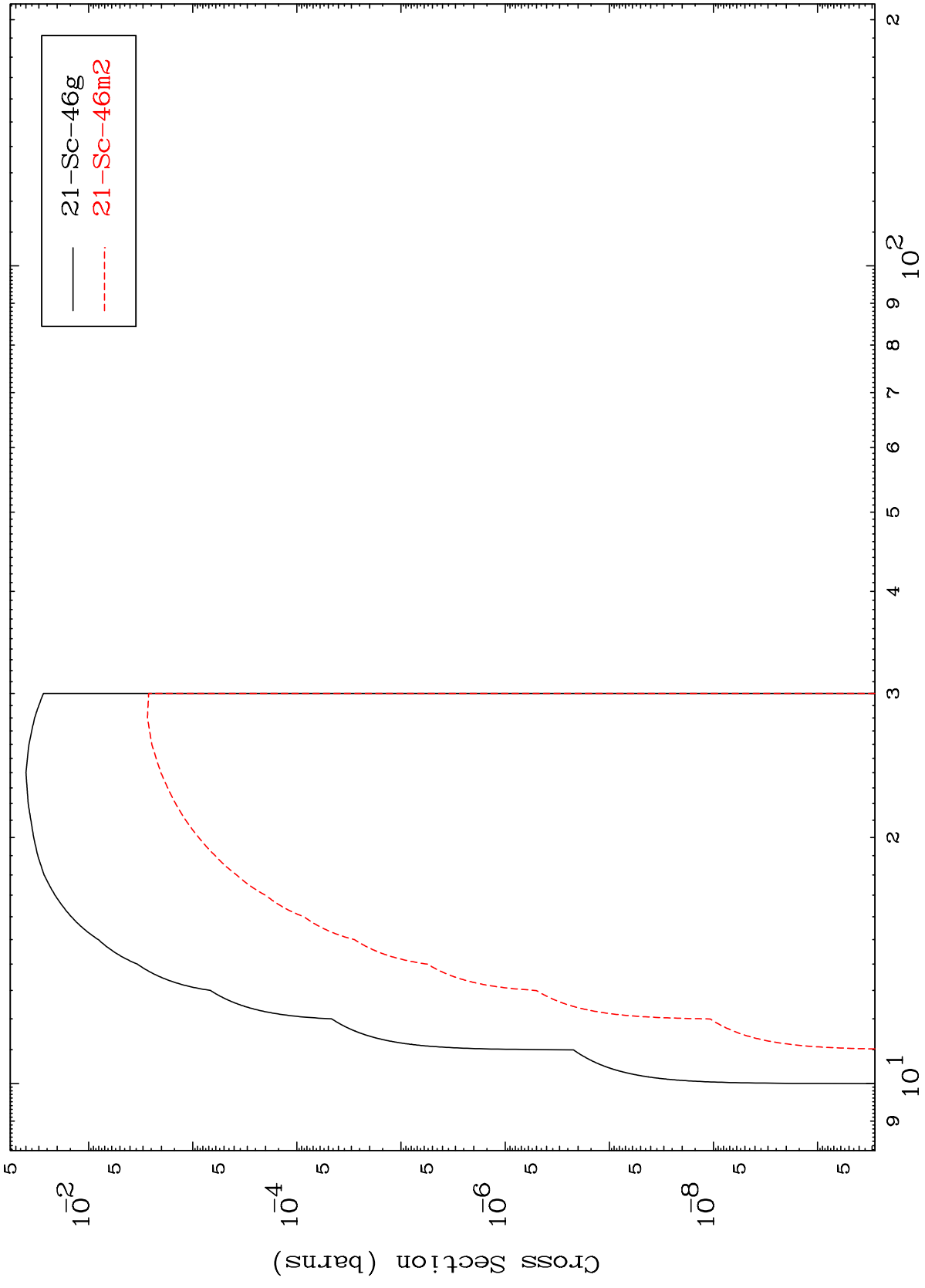
21-Sc-44



MAT 2123

21-Sc-44

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



21-Sc-44

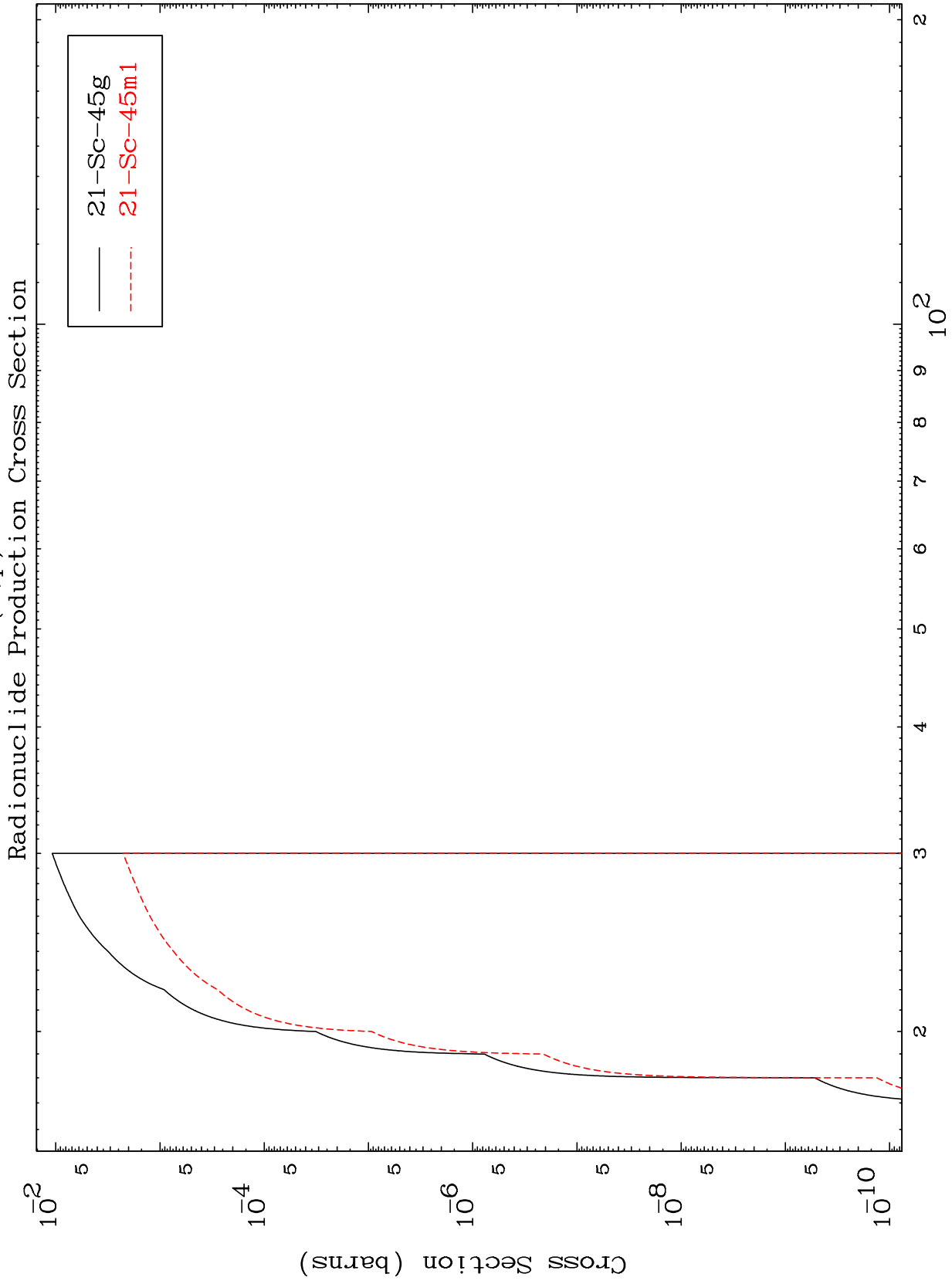
Incident Energy (MeV)

16

MAT 2123

( $\alpha, p$ ) d

21-Sc-44



17

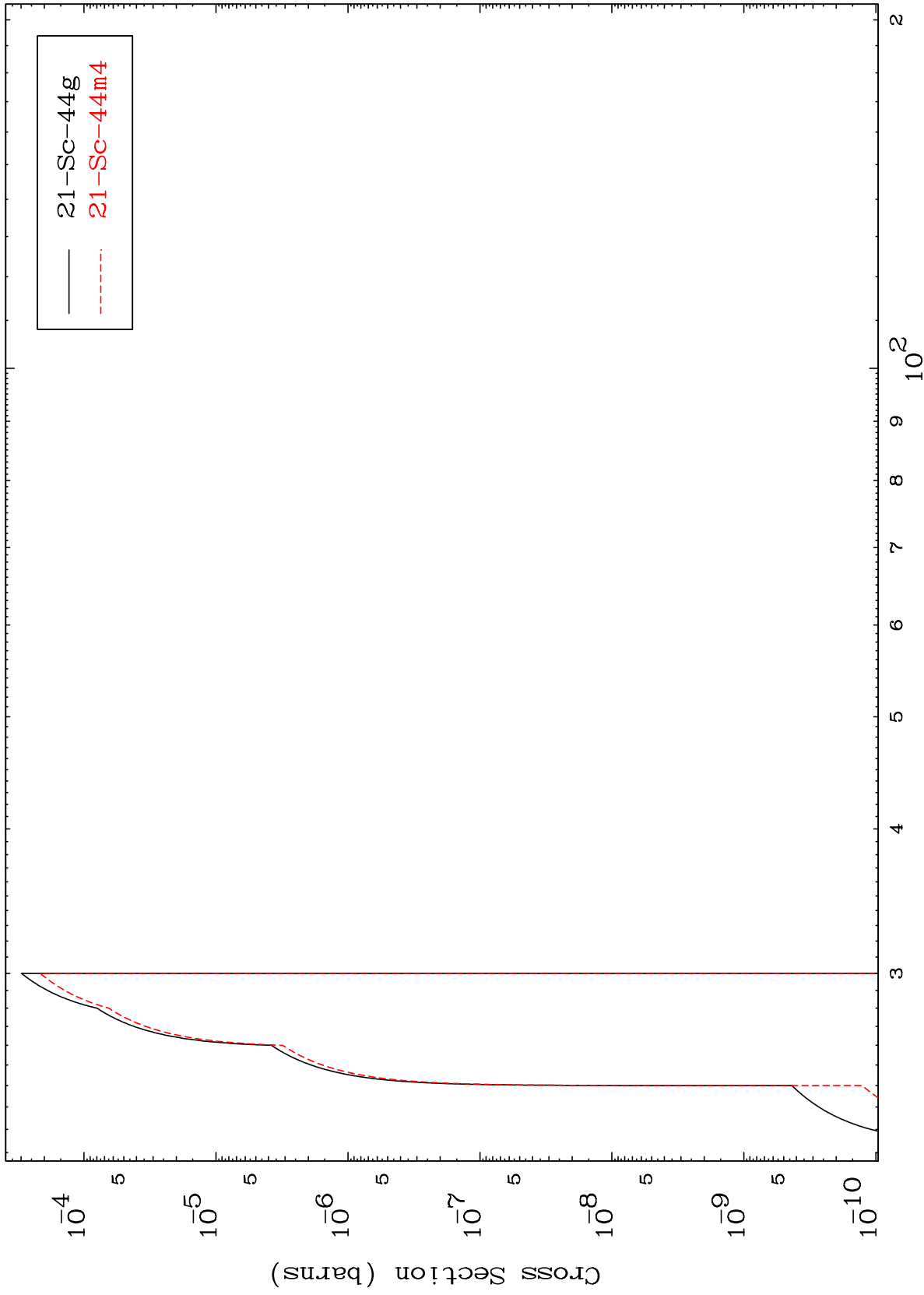
Incident Energy (MeV)

21-Sc-44

MAT 2123

21-Sc-44

( $\alpha, p$ ) t  
Radionuclide Production Cross Section



18

21-Sc-44