

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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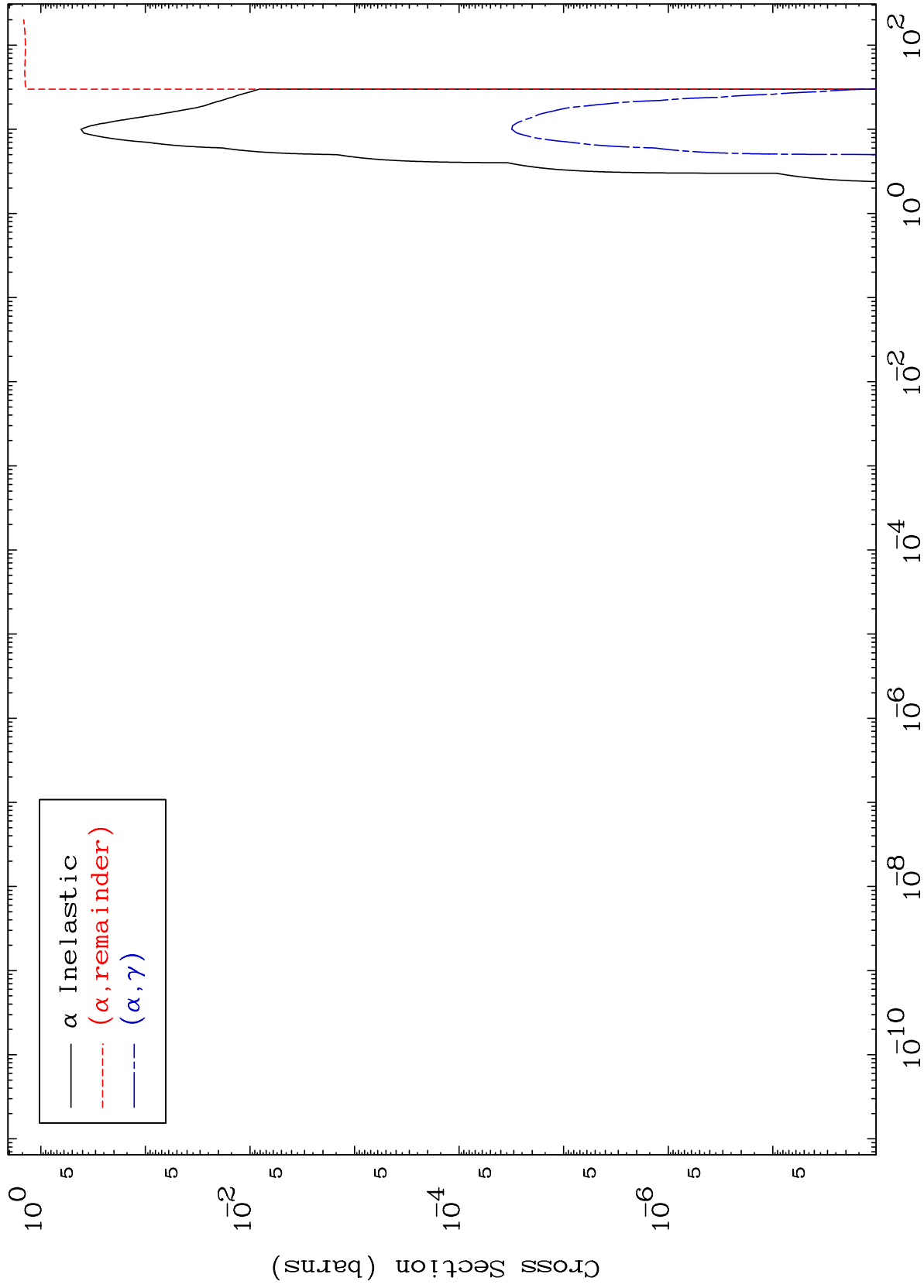
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 2137

0 Kelvin  $\alpha$  Major  
Cross Sections

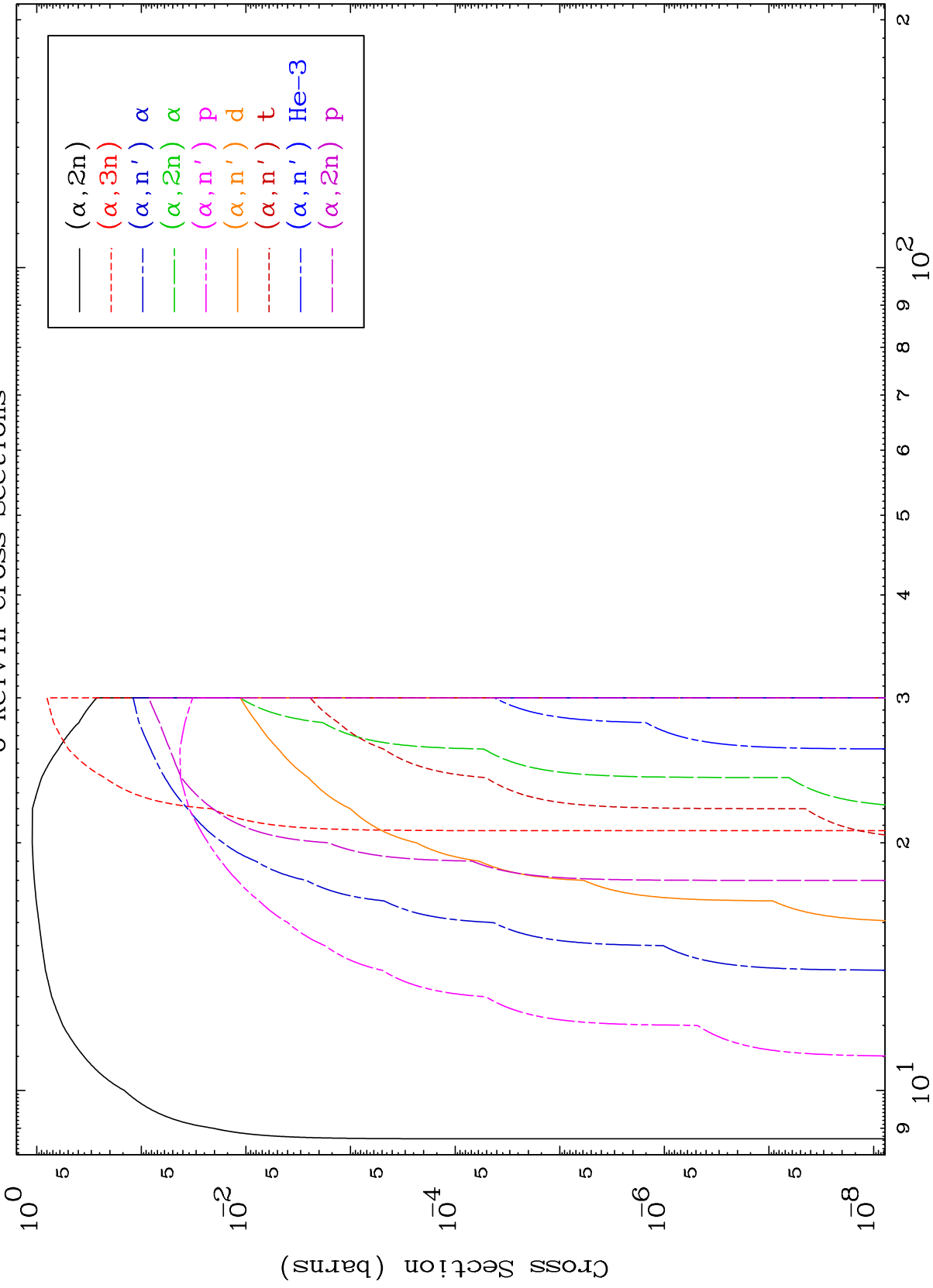
21-Sc-49



MAT 2137

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

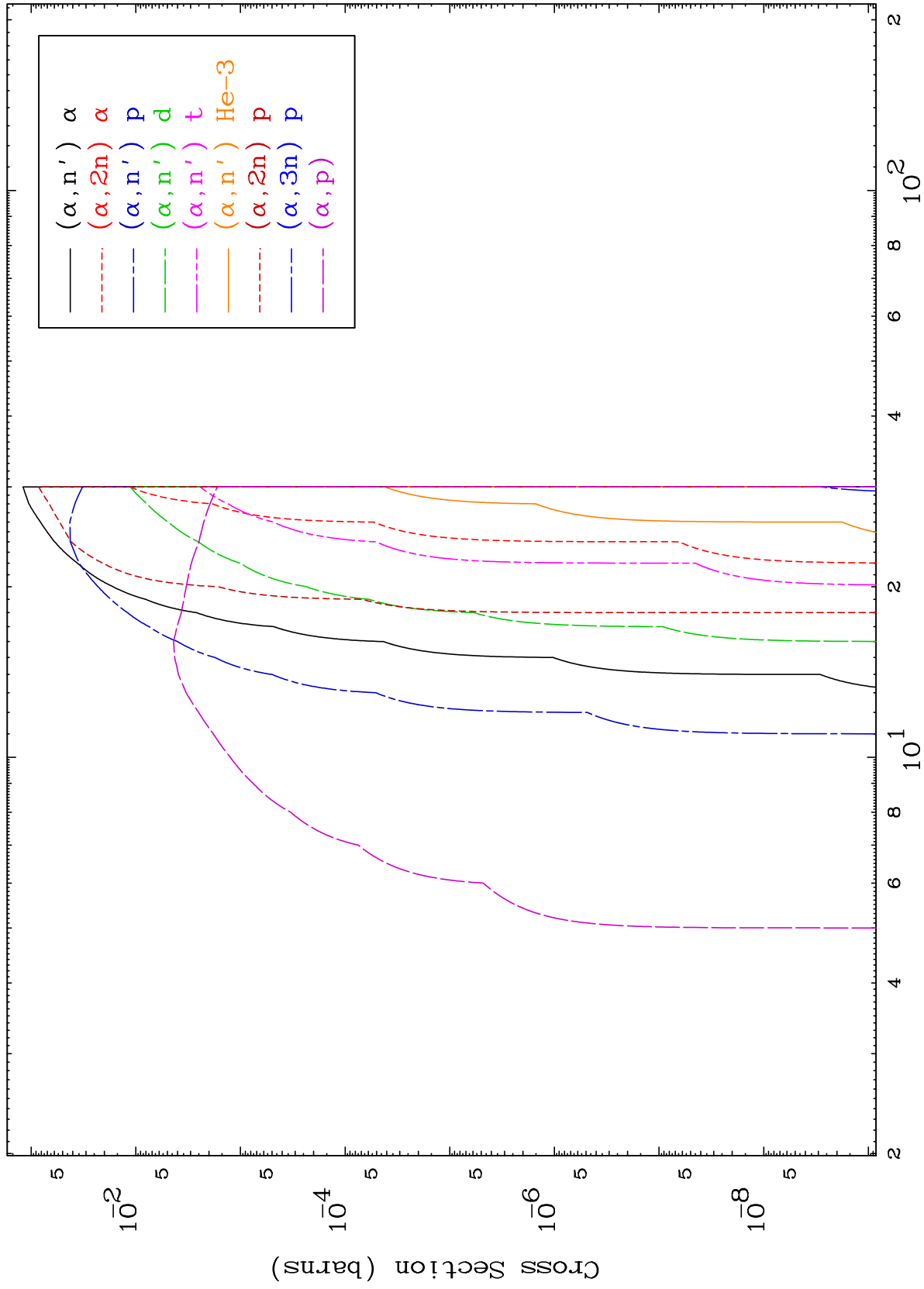
21-Sc-49

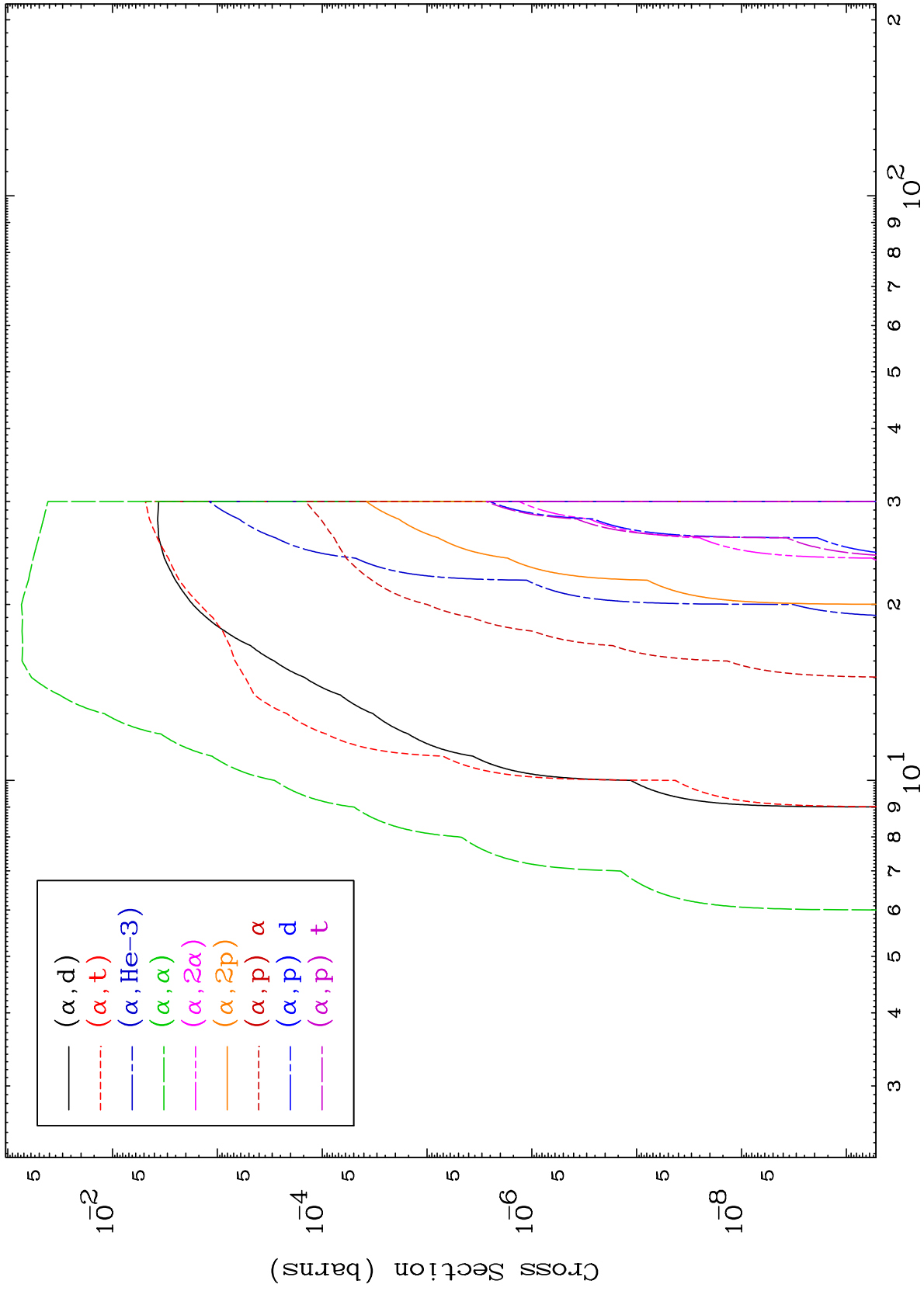


2

Incident Energy (MeV)

21-Sc-49

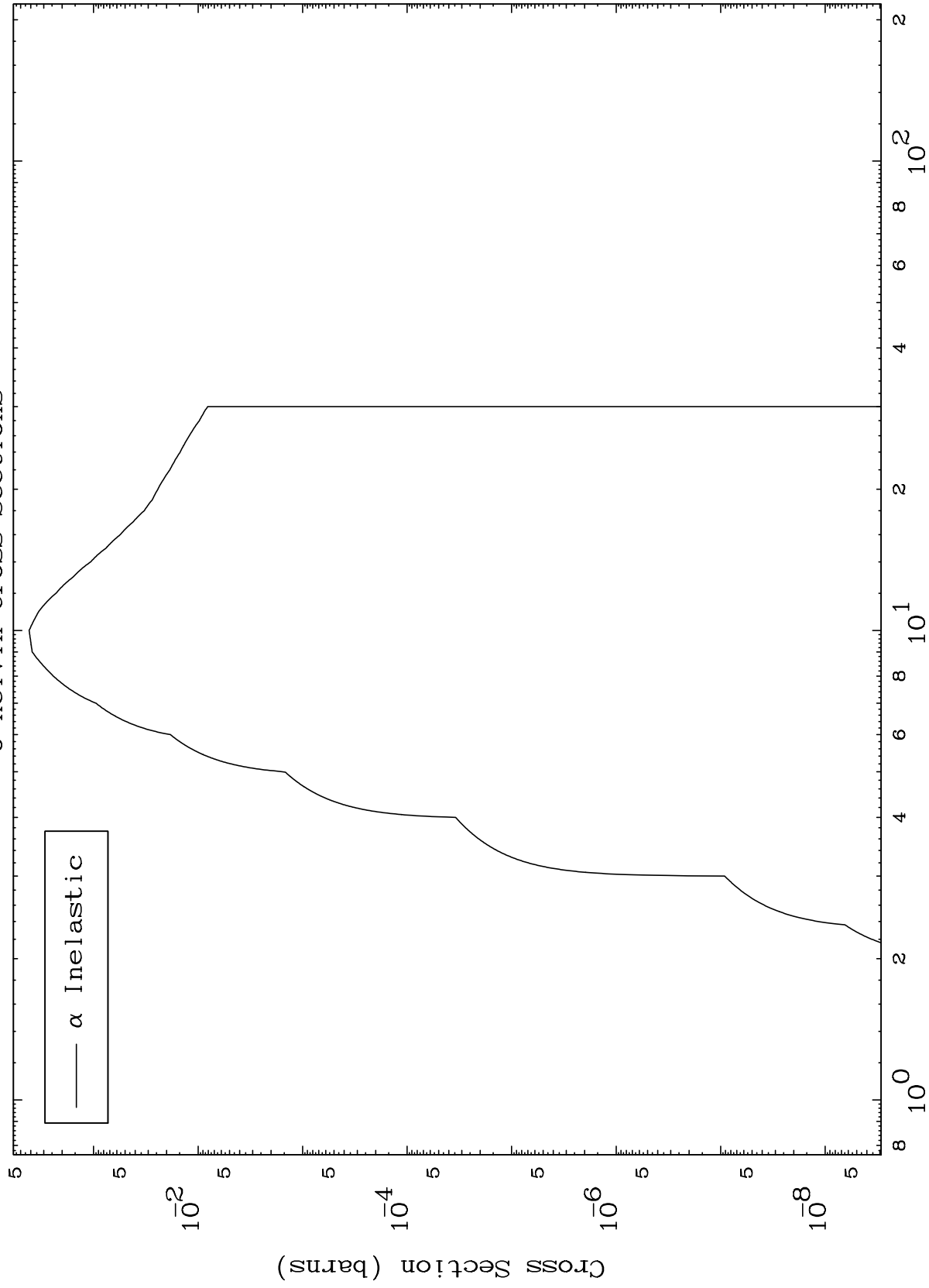




MAT 2137

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

21-Sc-49



$\alpha$  Inelastic

5

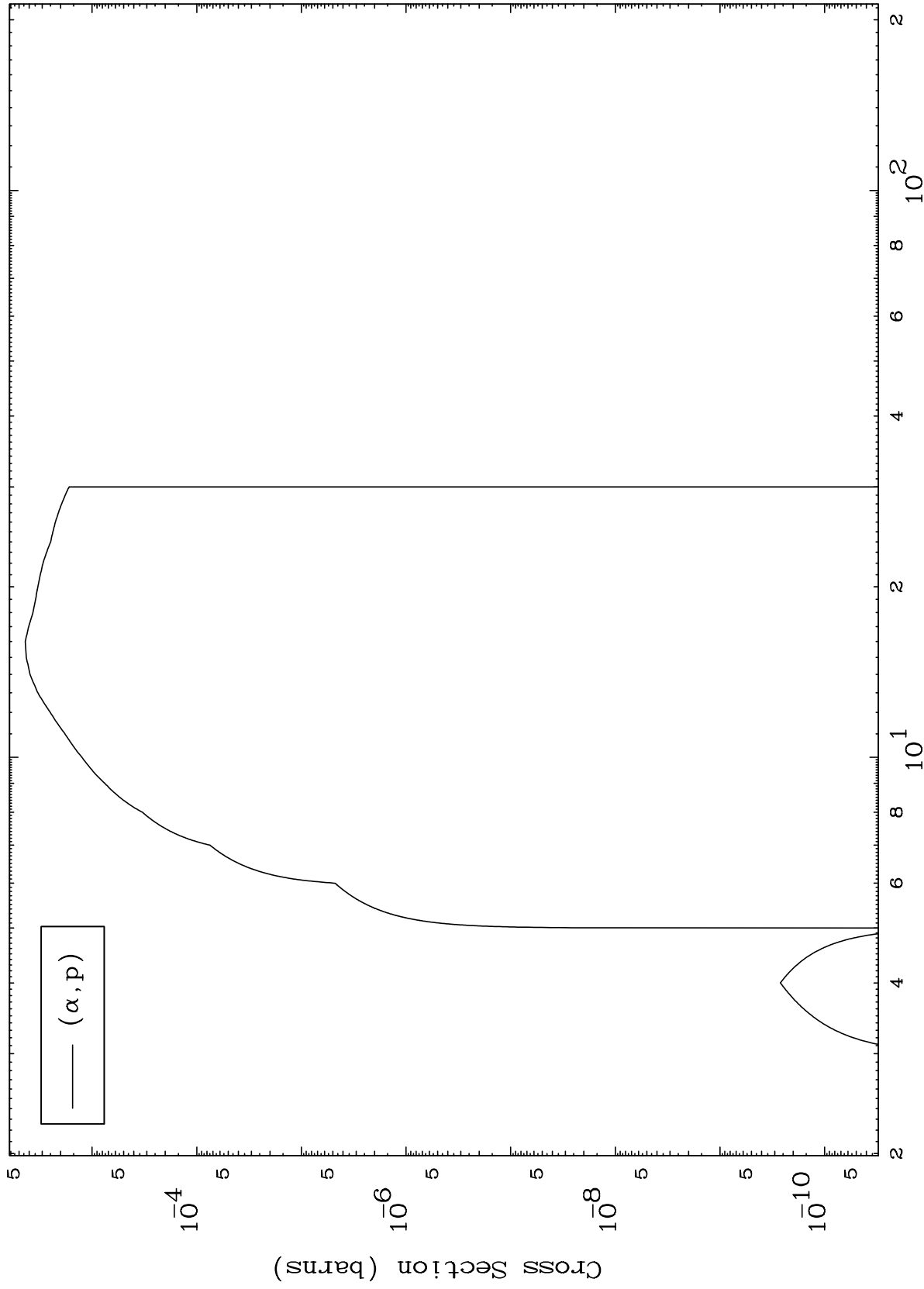
Incident Energy (MeV)

21-Sc-49

MAT 2137

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

21-Sc-49



6

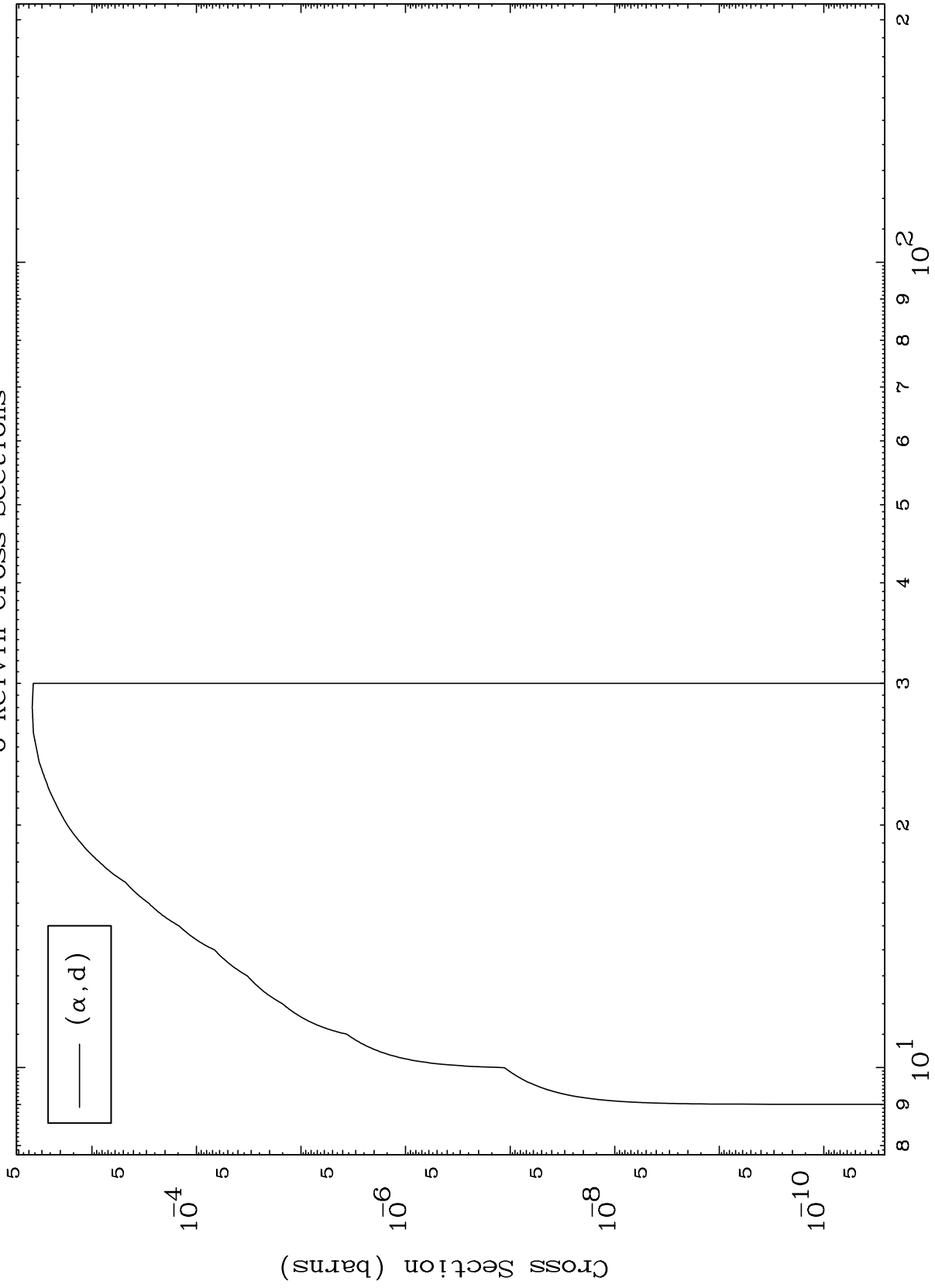
Incident Energy (MeV)

21-Sc-49

MAT 2137

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

21-Sc-49



7

Incident Energy (MeV)

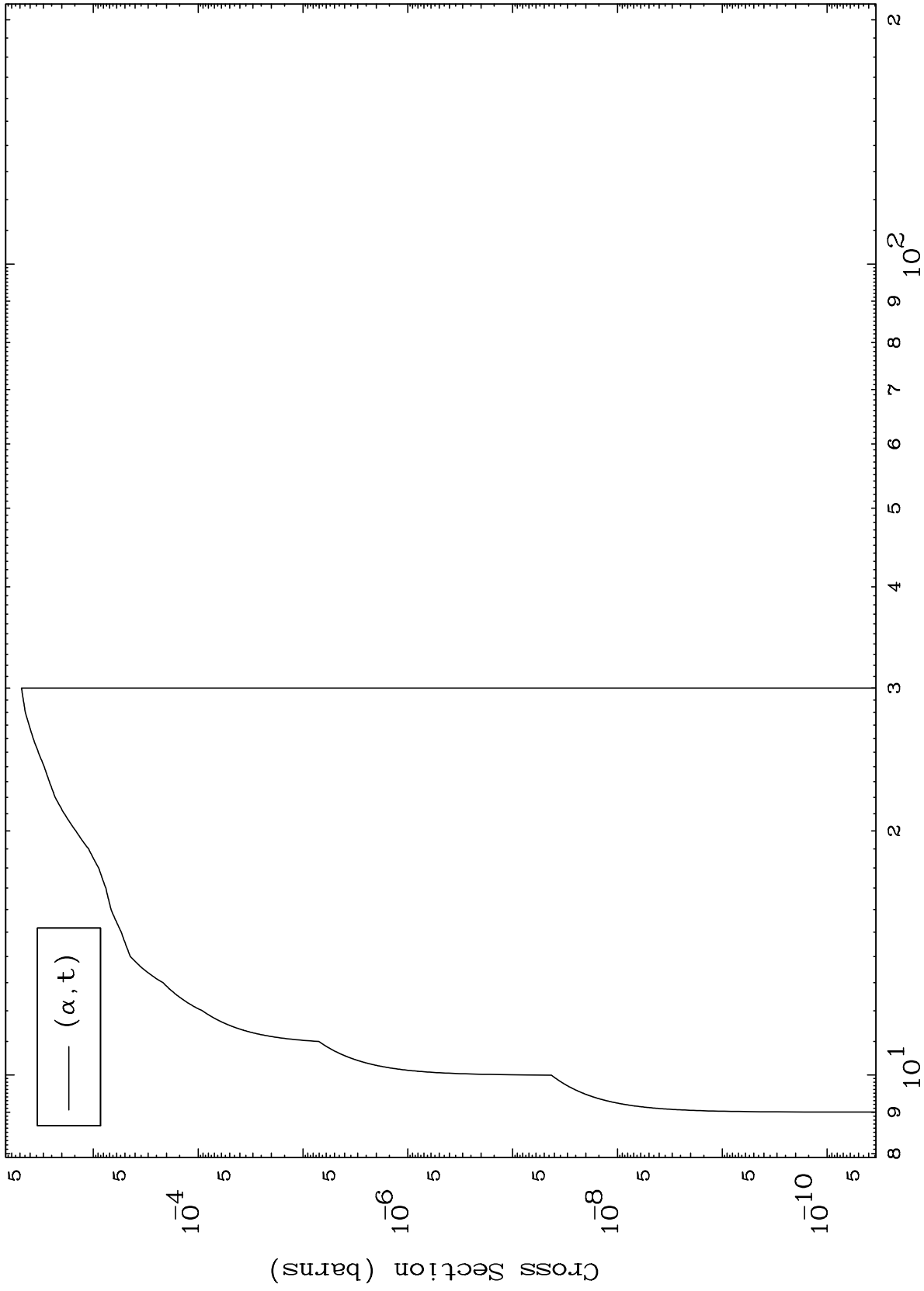
21-Sc-49



MAT 2137

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

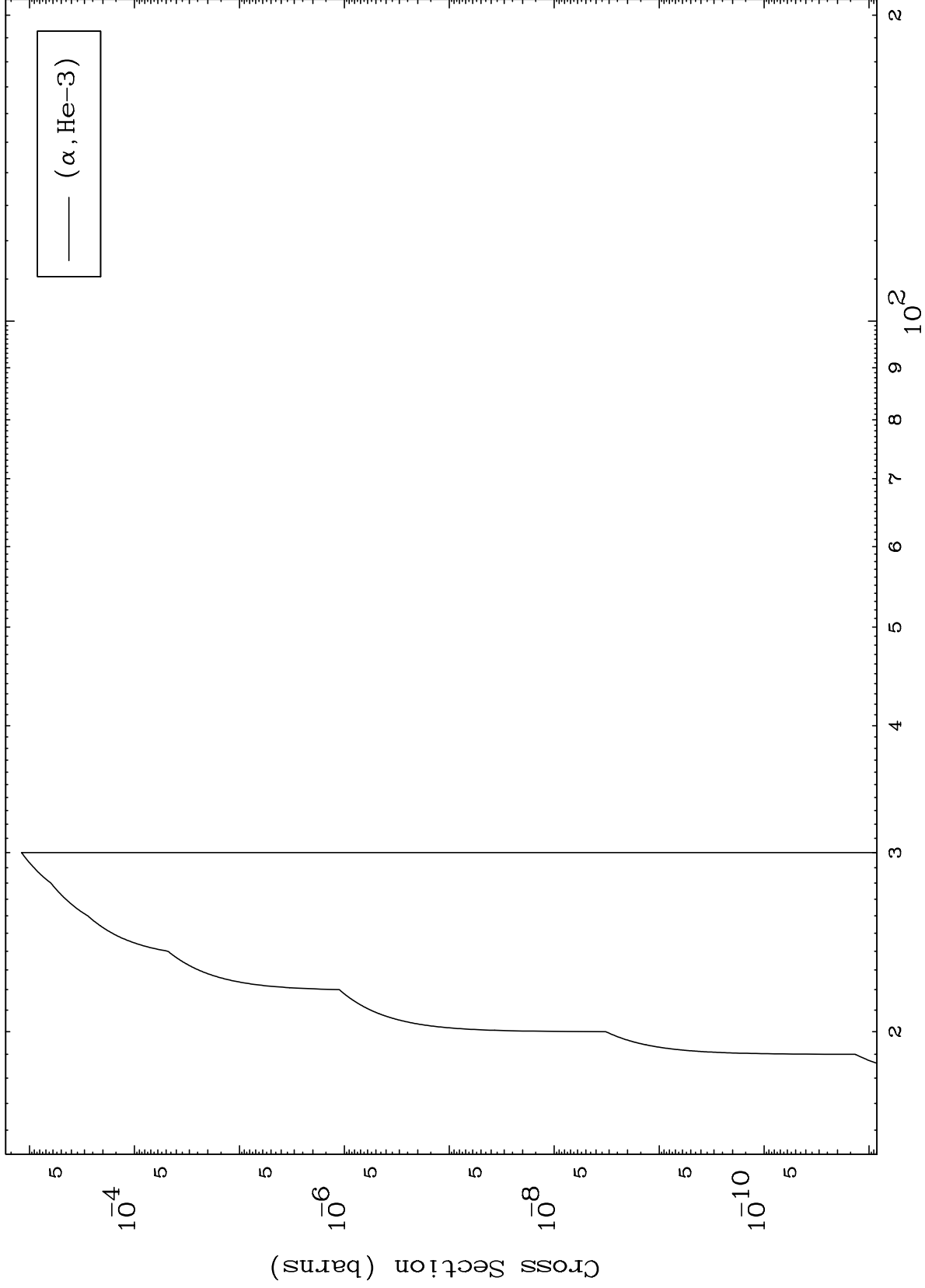
21-Sc-49



8

Incident Energy (MeV)

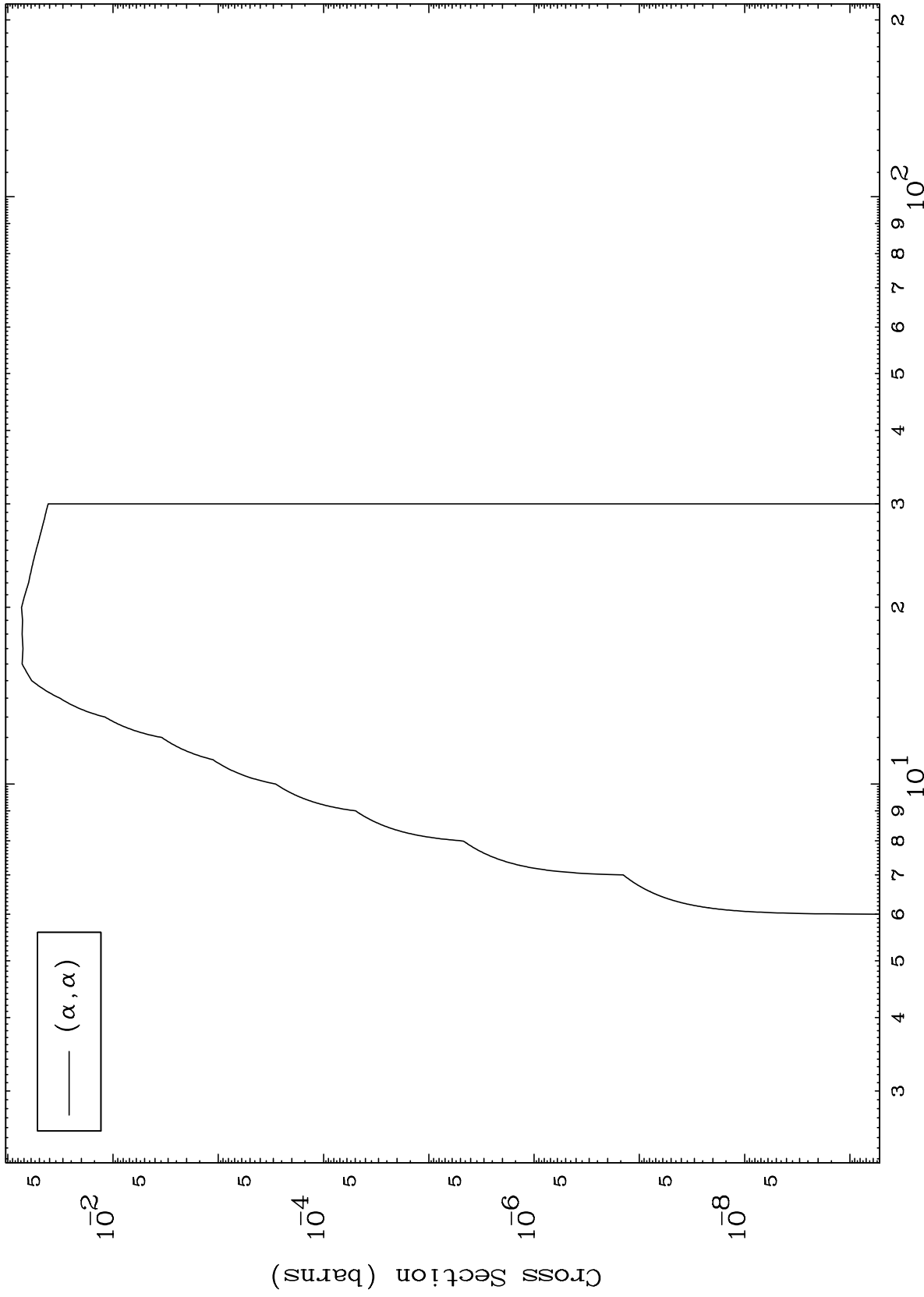
21-Sc-49



MAT 2137

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

21-Sc-49



10

Incident Energy (MeV)

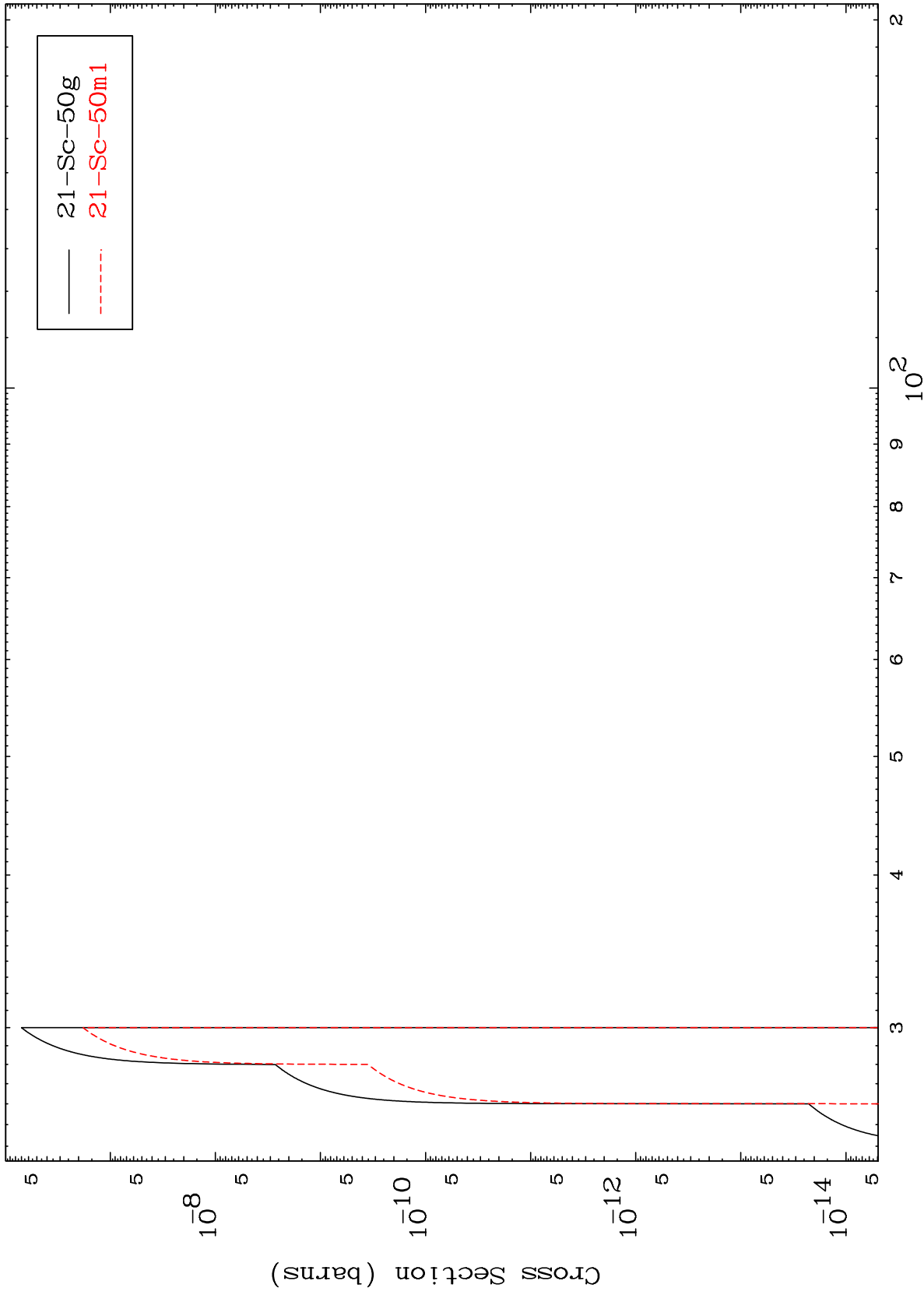
21-Sc-49

MAT 2137

( $\alpha, 2n$ ) p

21-Sc-49

Radionuclide Production Cross Section



11

Incident Energy (MeV)

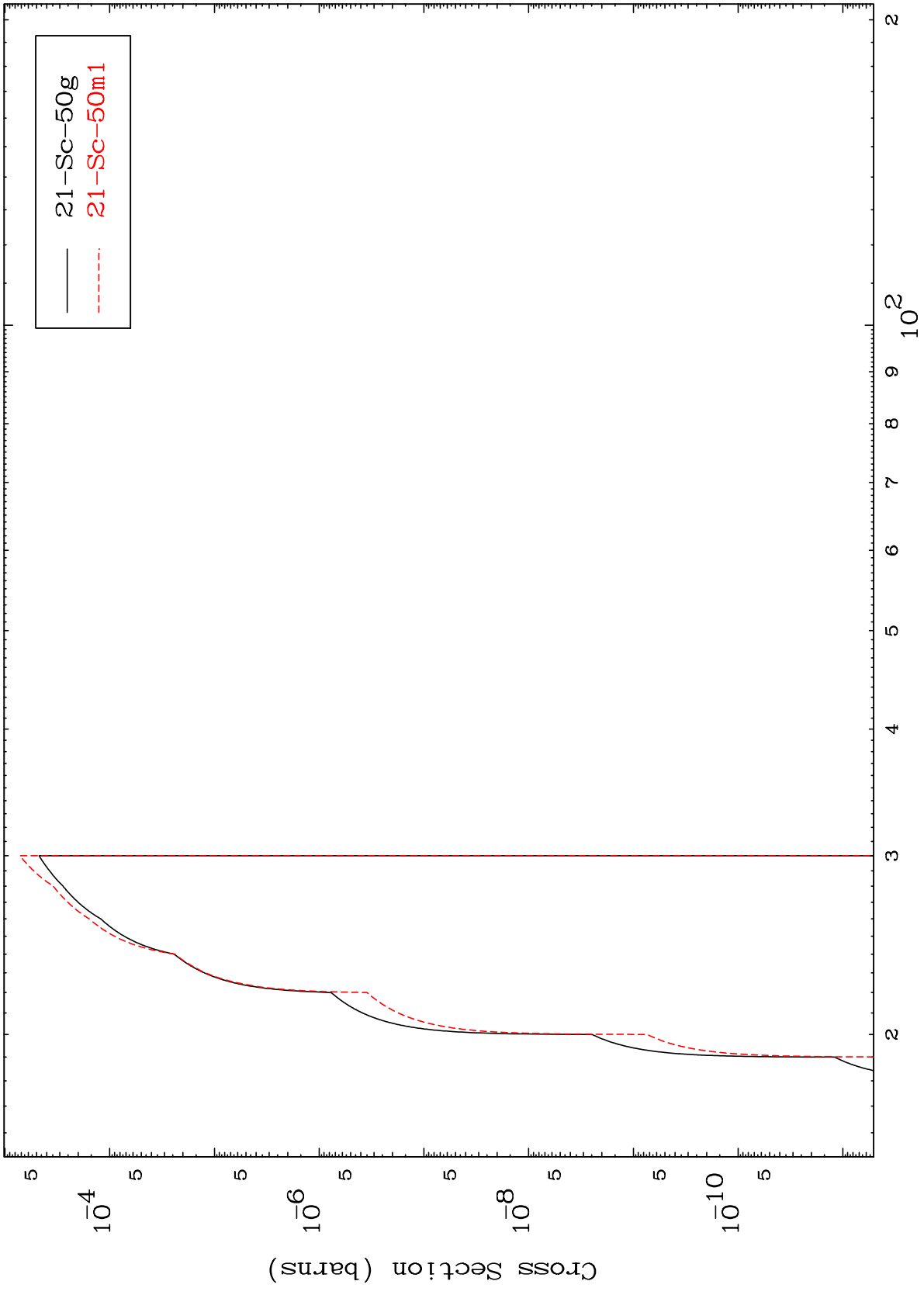
21-Sc-49

MAT 2137

( $\alpha, \text{He-3}$ )

21-Sc-49

Radionuclide Production Cross Section



12

Incident Energy (MeV)

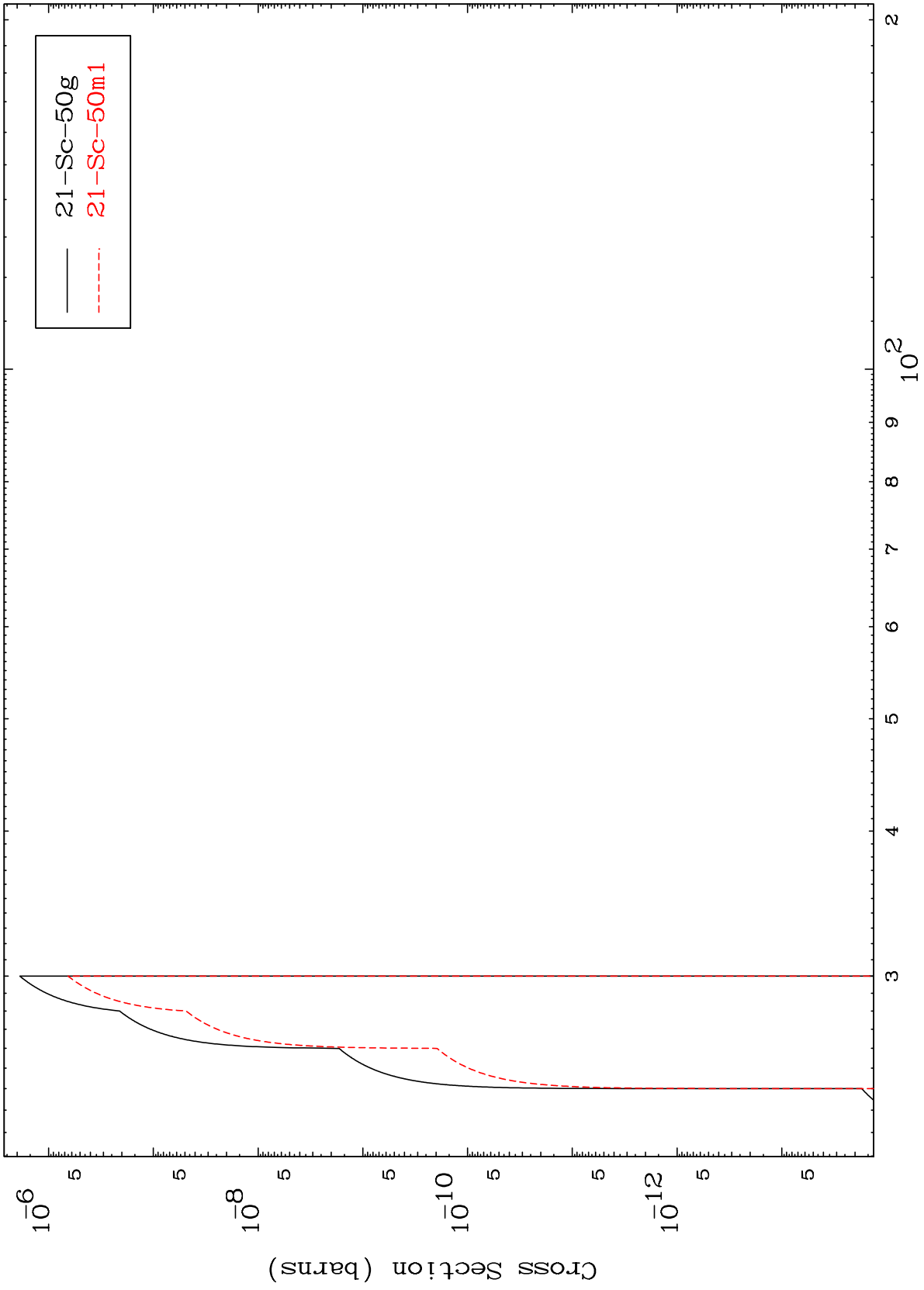
21-Sc-49

MAT 2137

( $\alpha, p$ ) d

21-Sc-49

Radionuclide Production Cross Section



13

Incident Energy (MeV)

21-Sc-49