

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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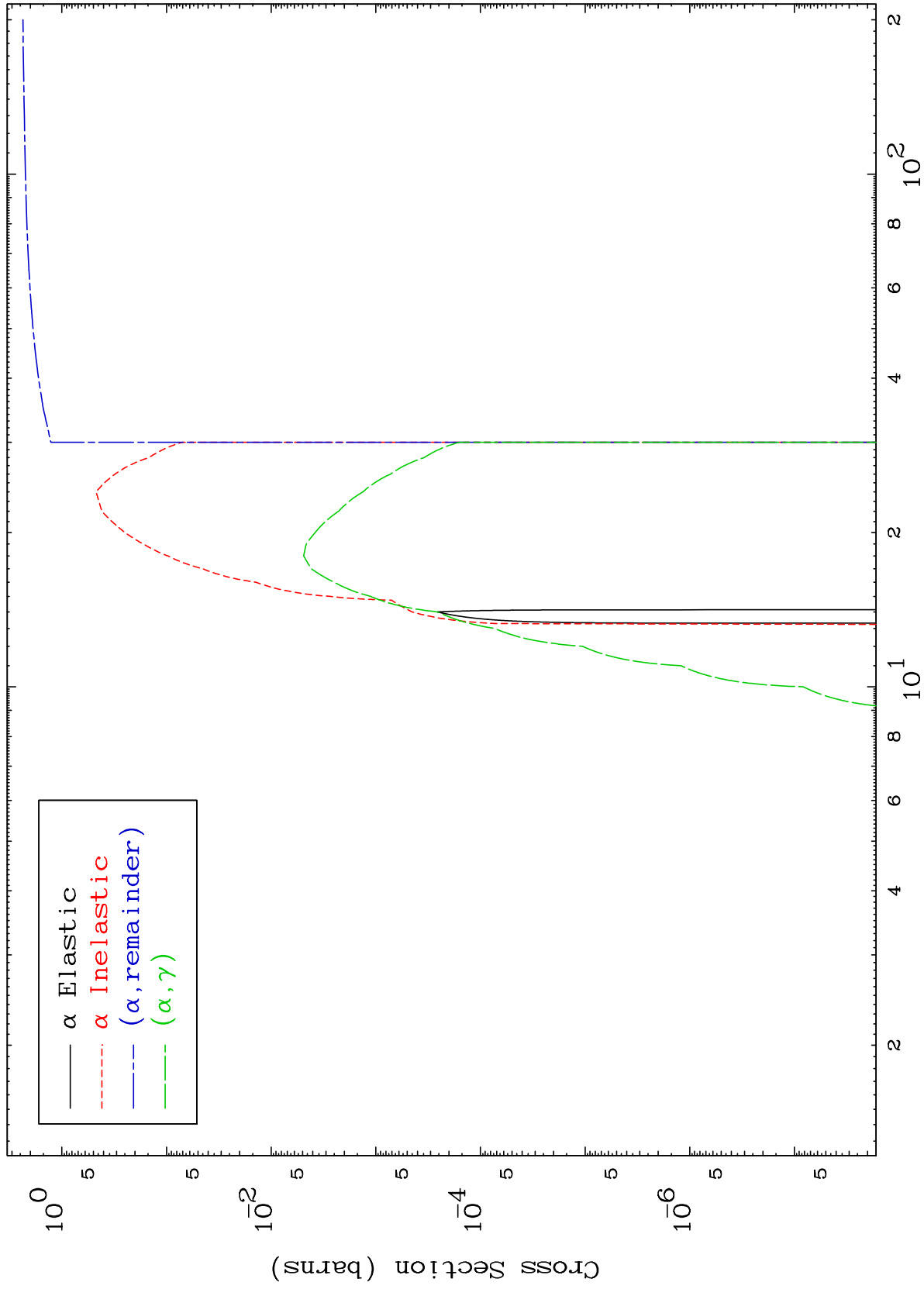
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 6213

0 Kelvin  $\alpha$  Major

62-Sm-140



1

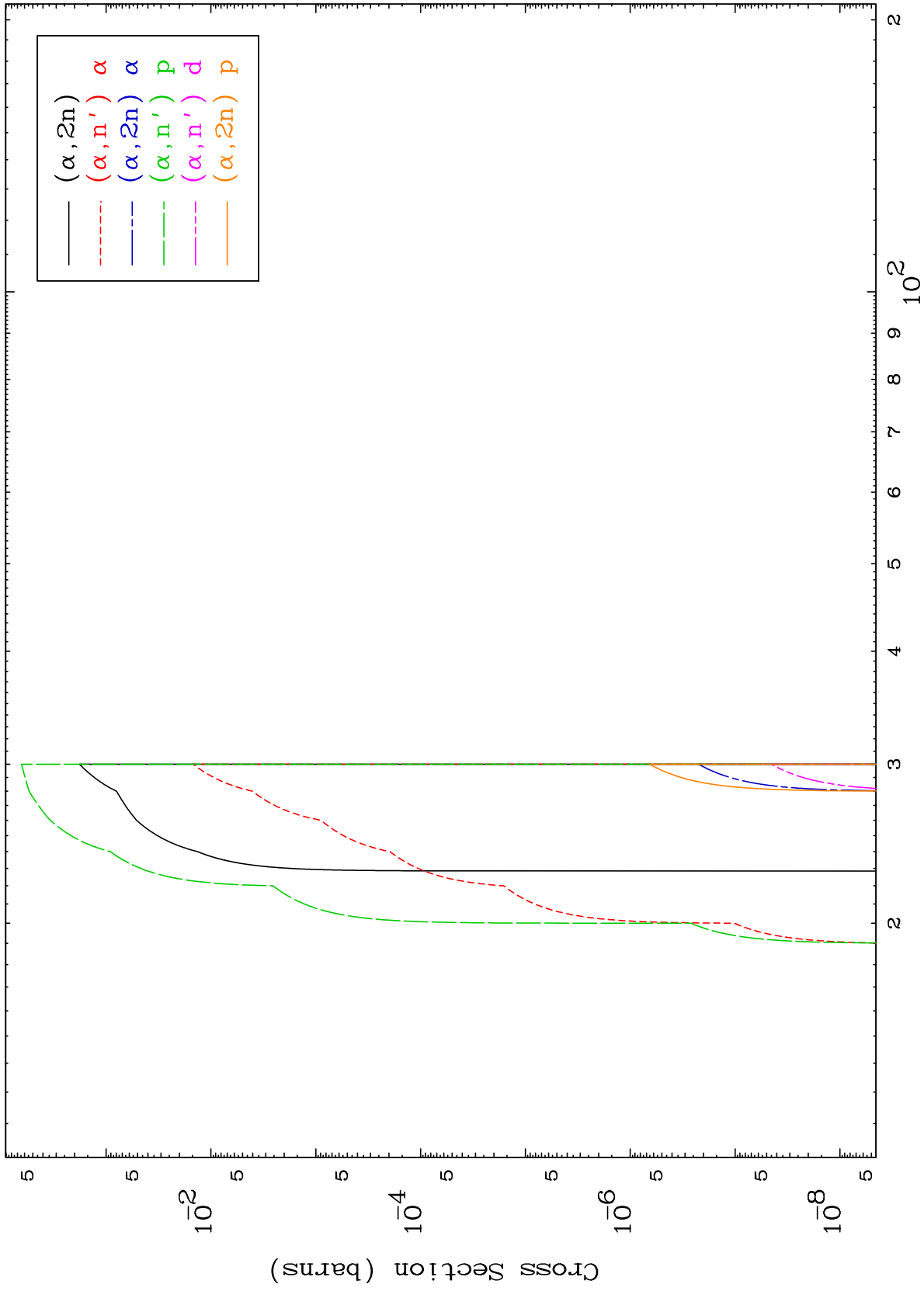
Incident Energy (MeV)

62-Sm-140

MAT 6213

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

62-Sm-140



2

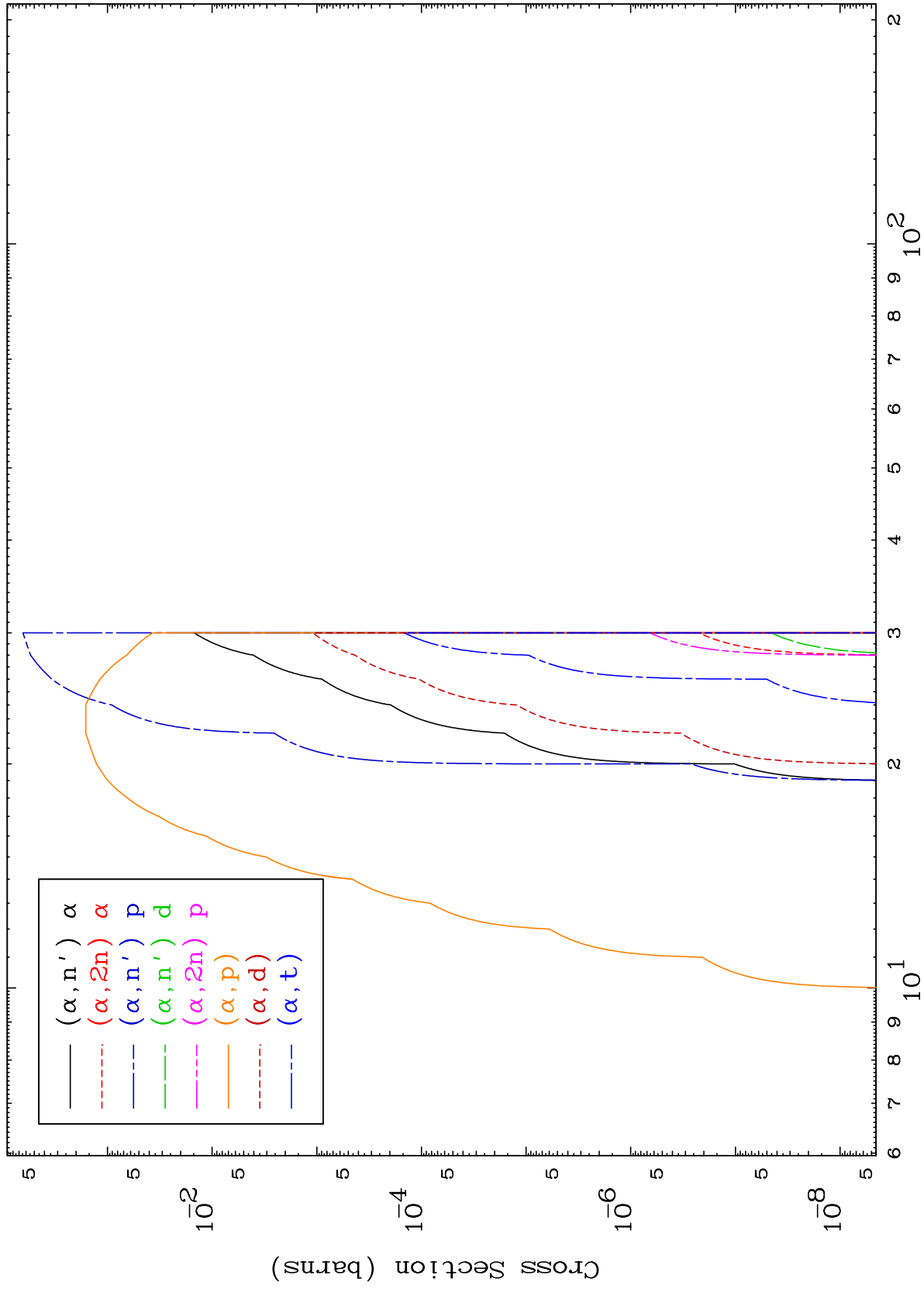
Incident Energy (MeV)

62-Sm-140

MAT 6213

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

62-Sm-140



3

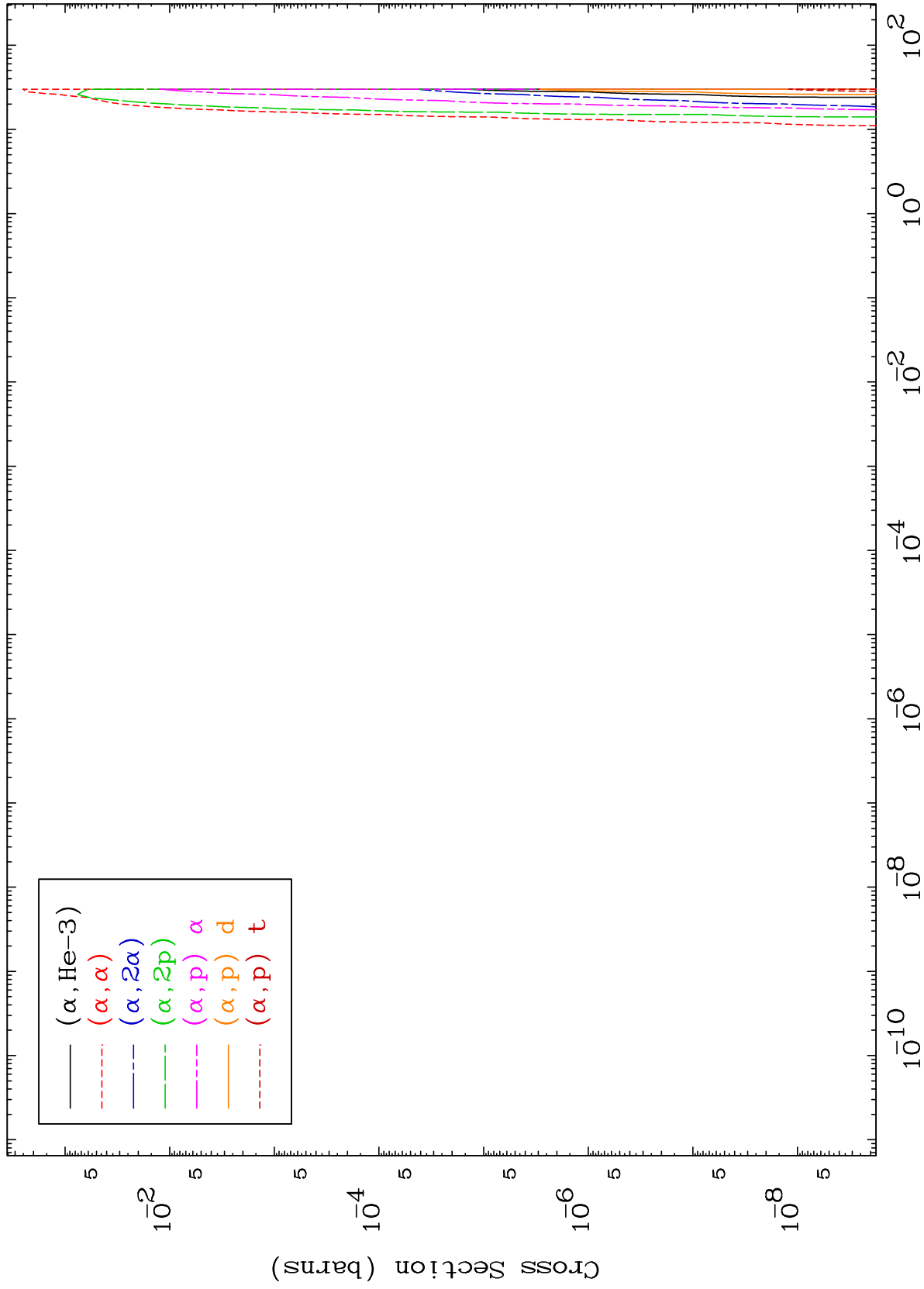
Incident Energy (MeV)

62-Sm-140

MAT 6213

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

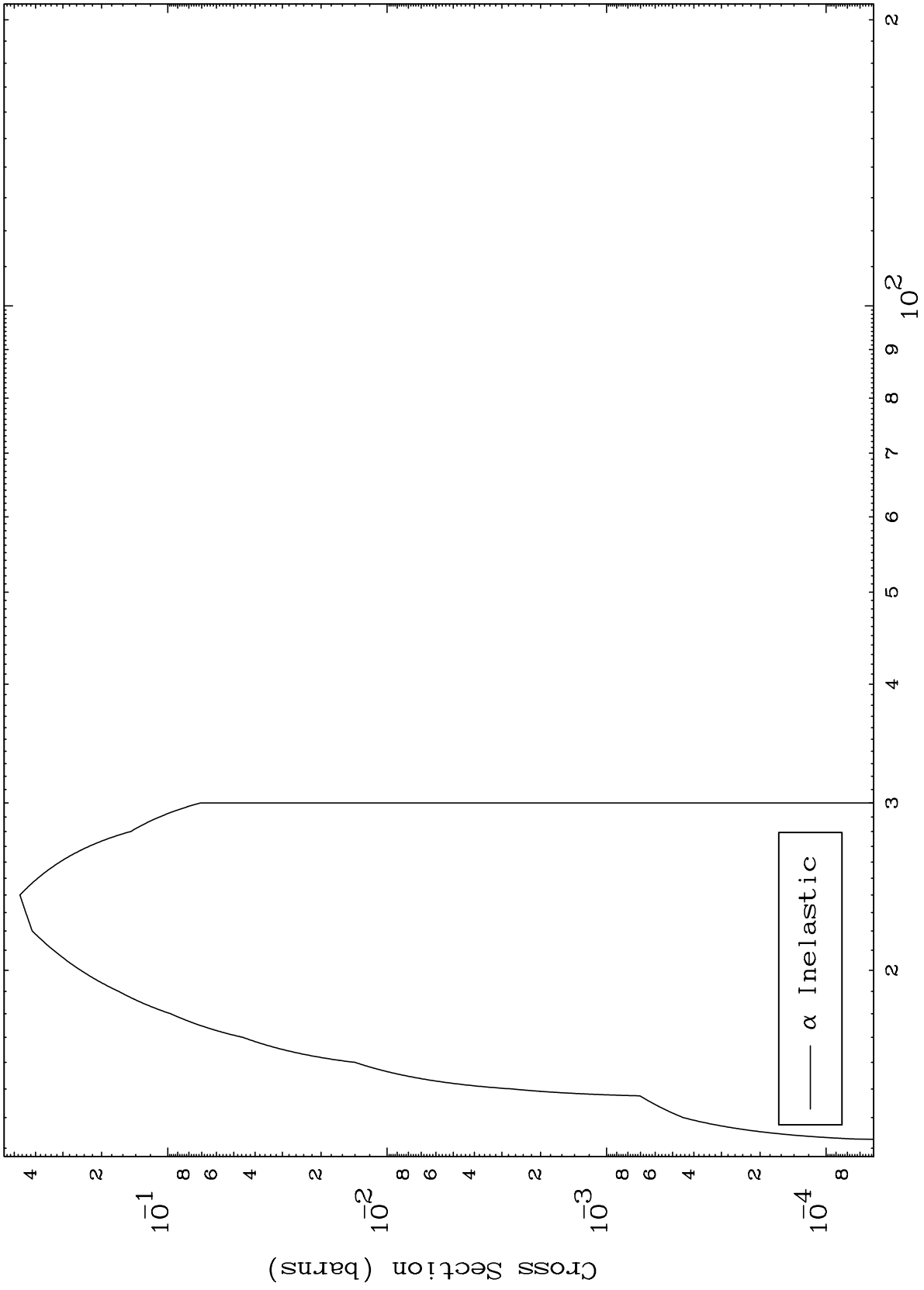
62-Sm-140



MAT 6213

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

62-Sm-140



5

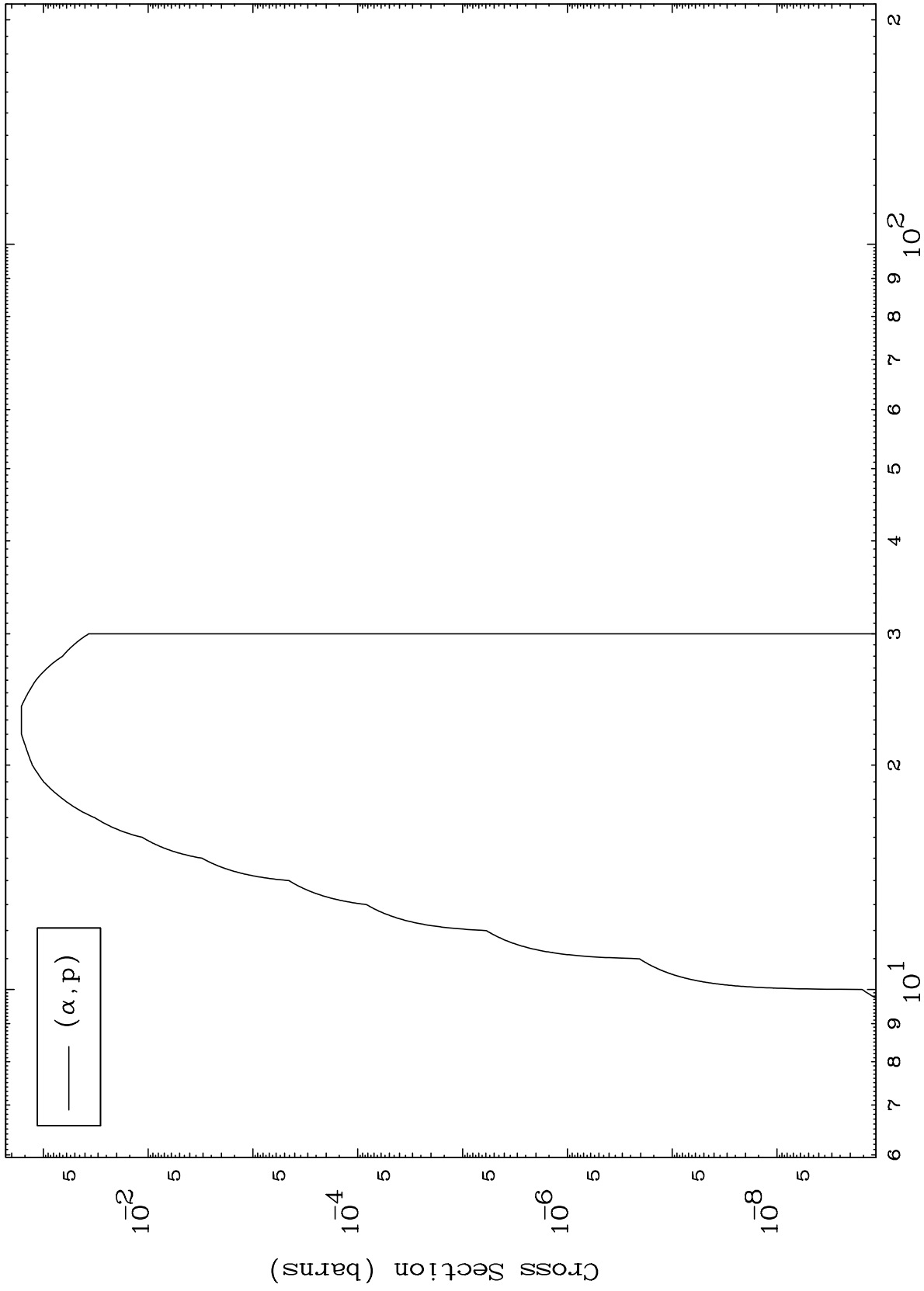
Incident Energy (MeV)

62-Sm-140

MAT 6213

62-Sm-140

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections



( $\alpha, p$ )

6

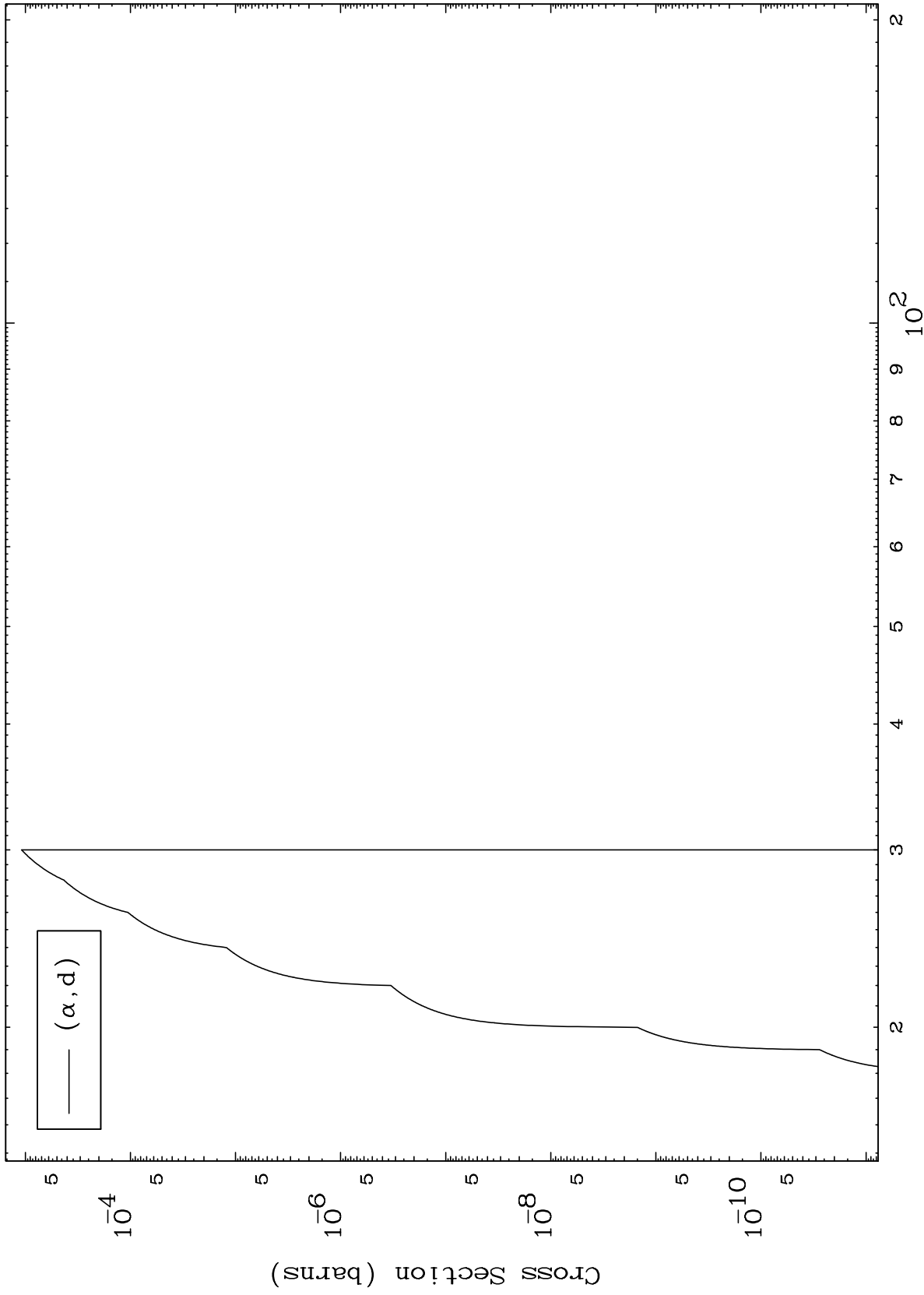
62-Sm-140

Incident Energy (MeV)

MAT 6213

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

$^{62}\text{Sm}-140$



7

Incident Energy (MeV)

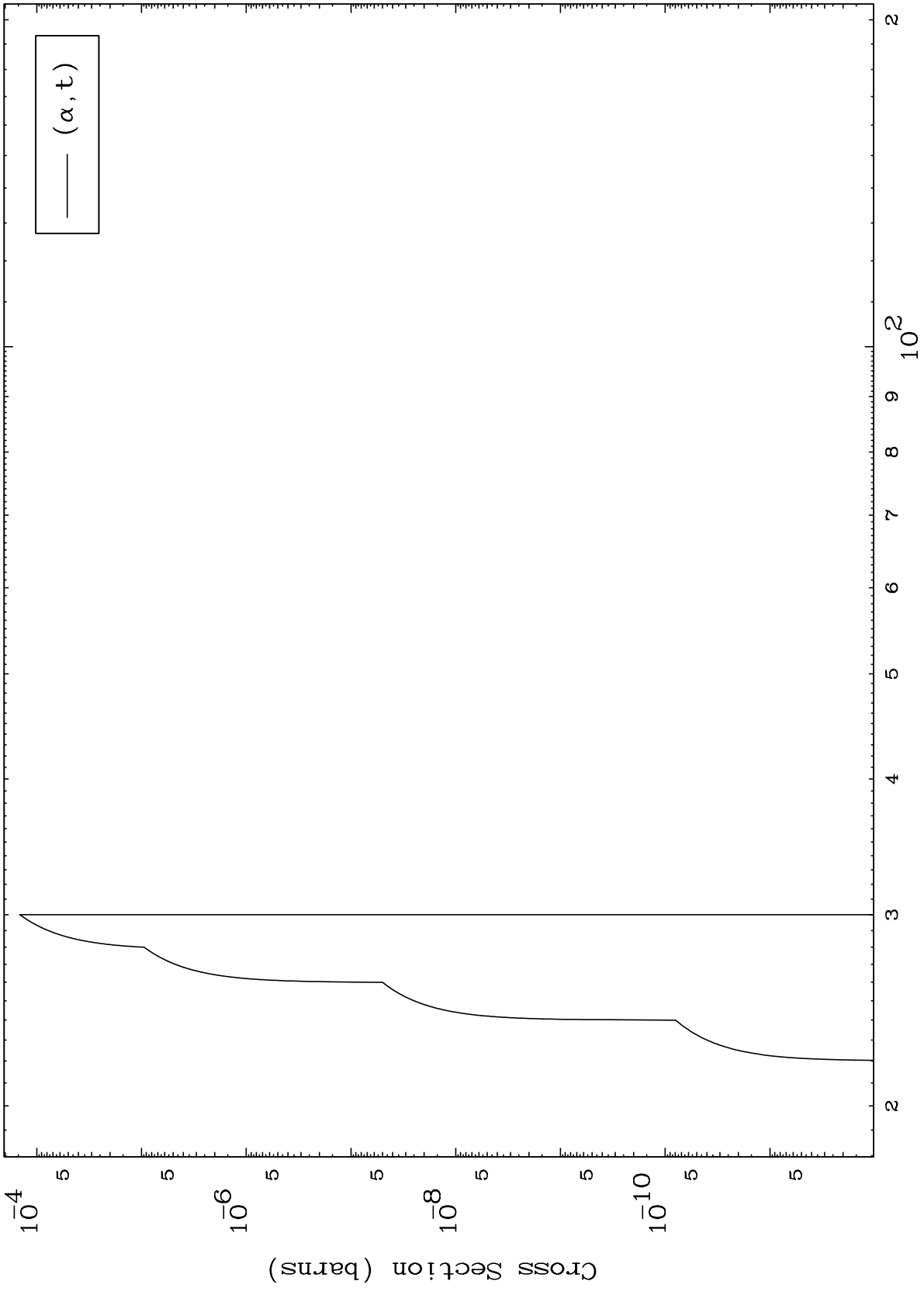
$^{62}\text{Sm}-140$



MAT 6213

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

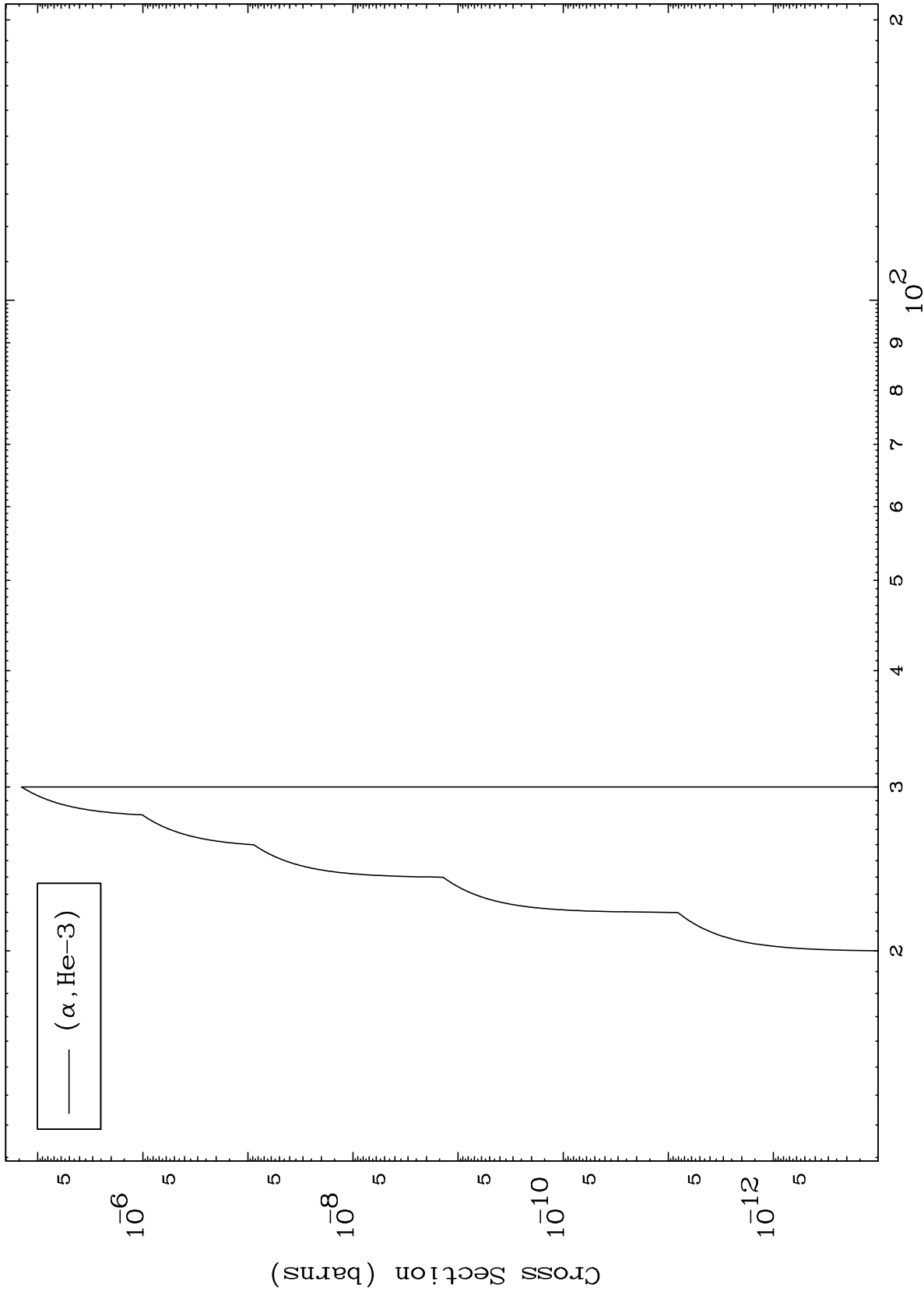
62-Sm-140



8

Incident Energy (MeV)

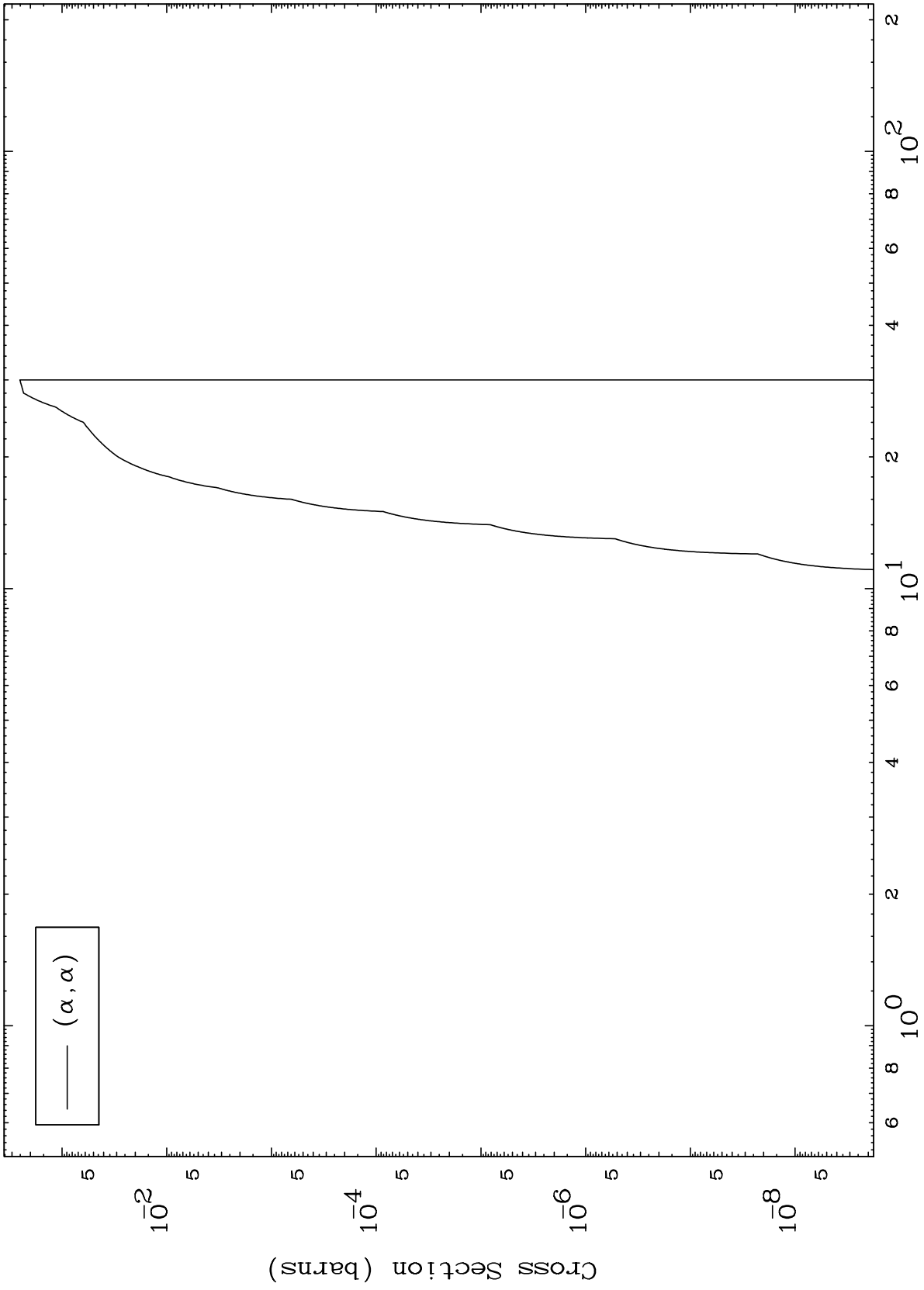
62-Sm-140



MAT 6213

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

62-Sm-140



62-Sm-140

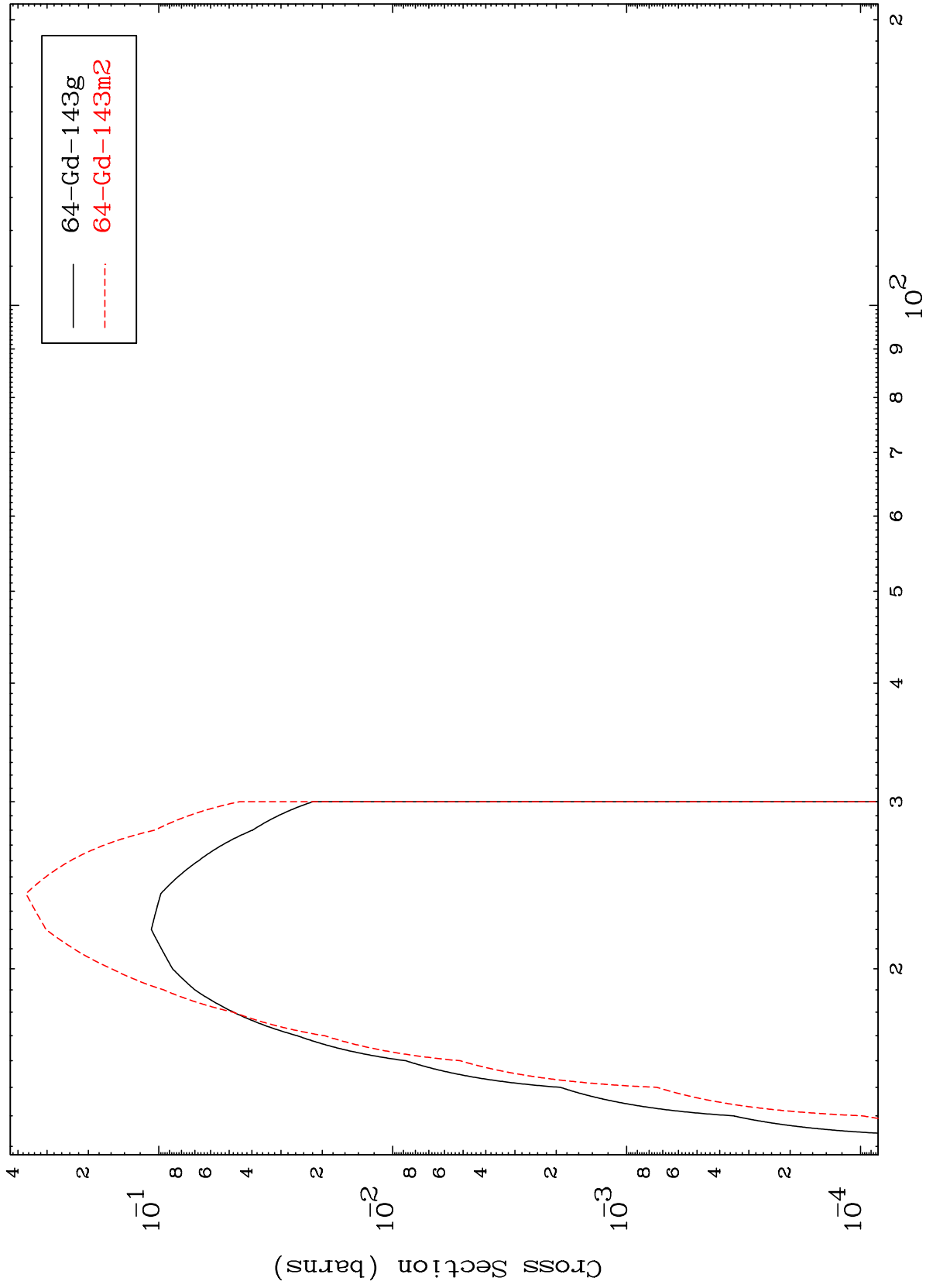
Incident Energy (MeV)

10

MAT 6213

62-Sm-140

$\alpha$  Inelastic  
Radionuclide Production Cross Section



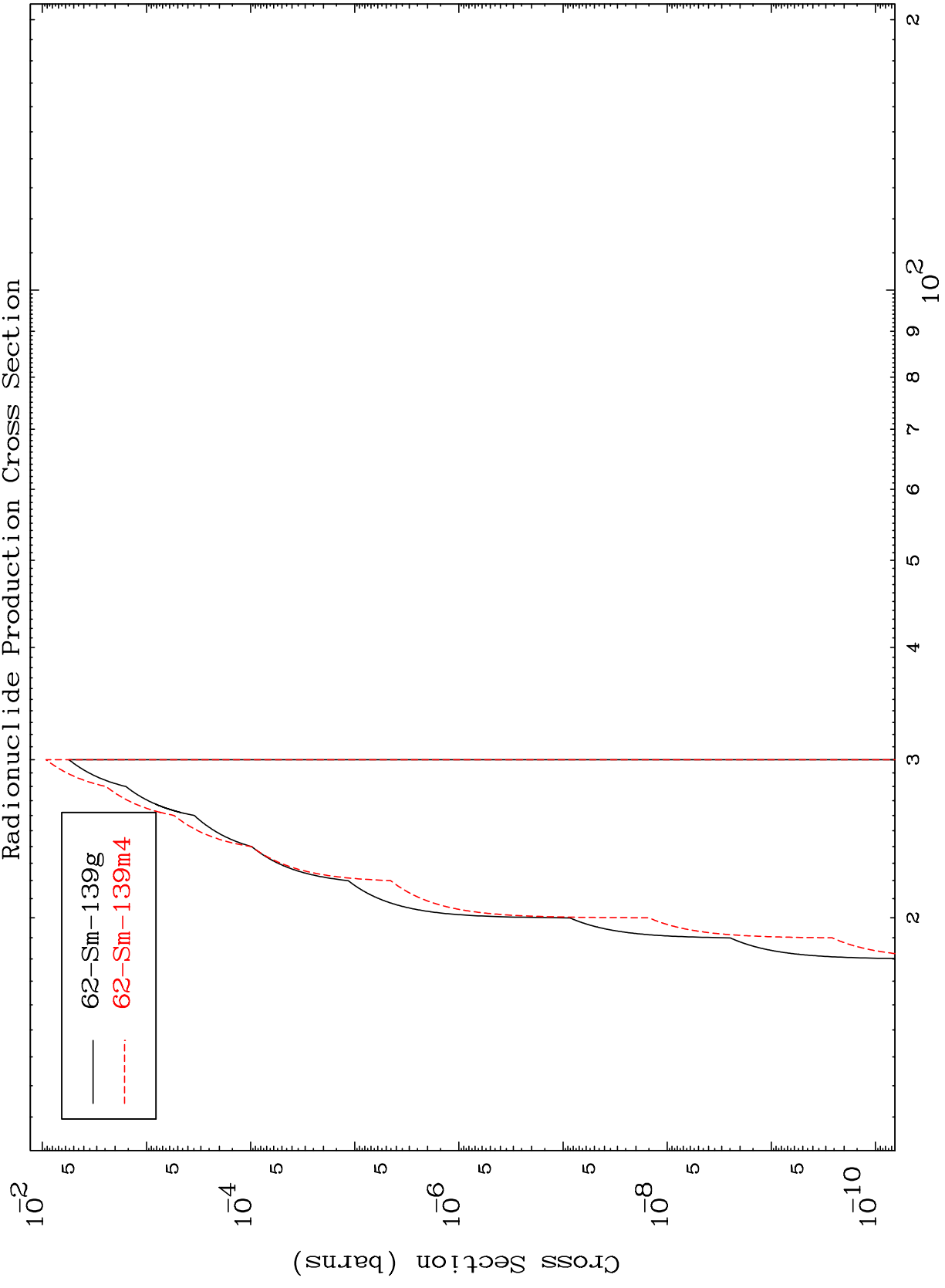
11

62-Sm-140

MAT 6213

62-Sm-140

( $\alpha, n'$ )  $\alpha$   
Radionuclide Production Cross Section



12

62-Sm-140

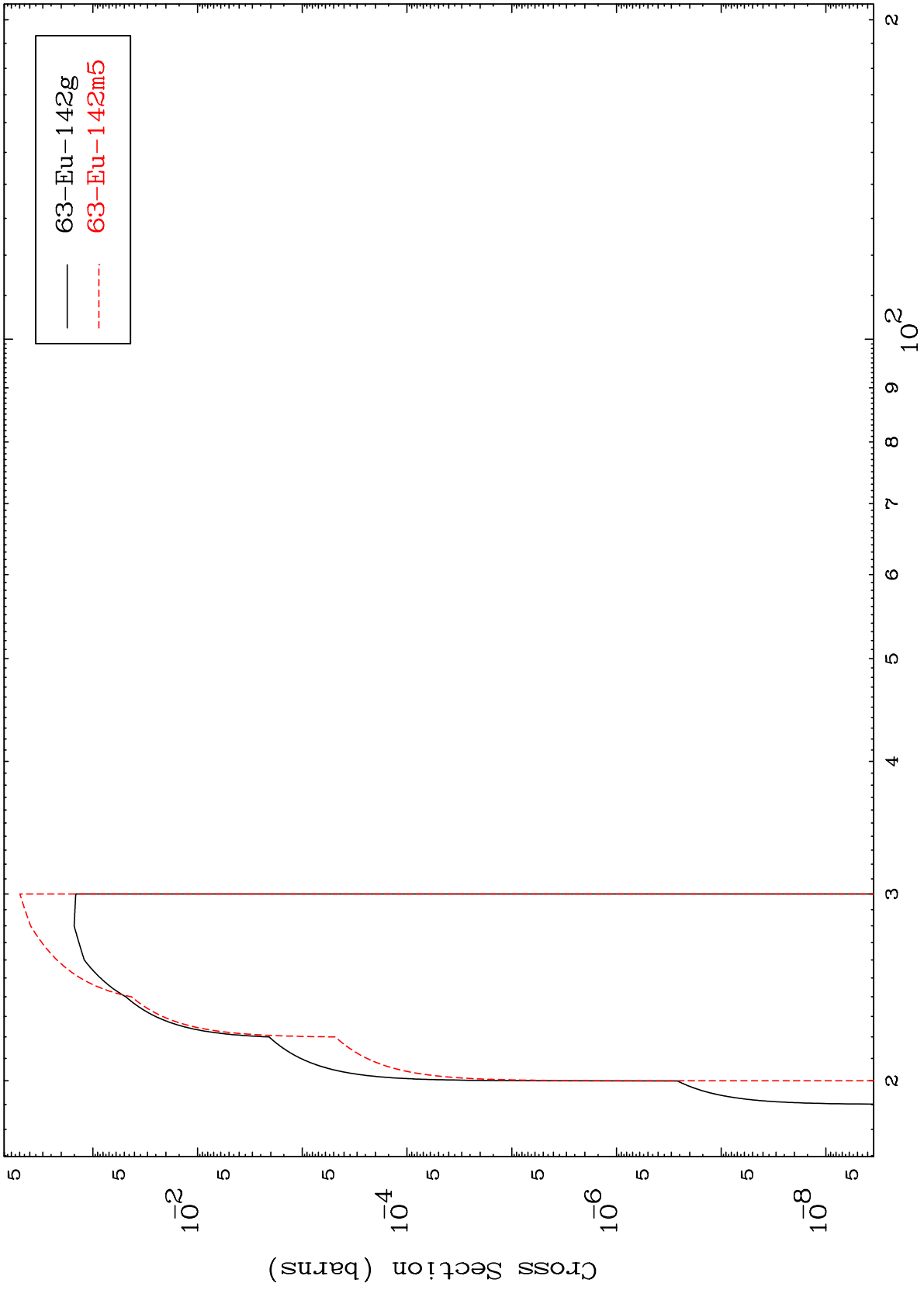
Incident Energy (MeV)

MAT 6213

( $\alpha, n'$ ) p

62-Sm-140

Radionuclide Production Cross Section



13

Incident Energy (MeV)

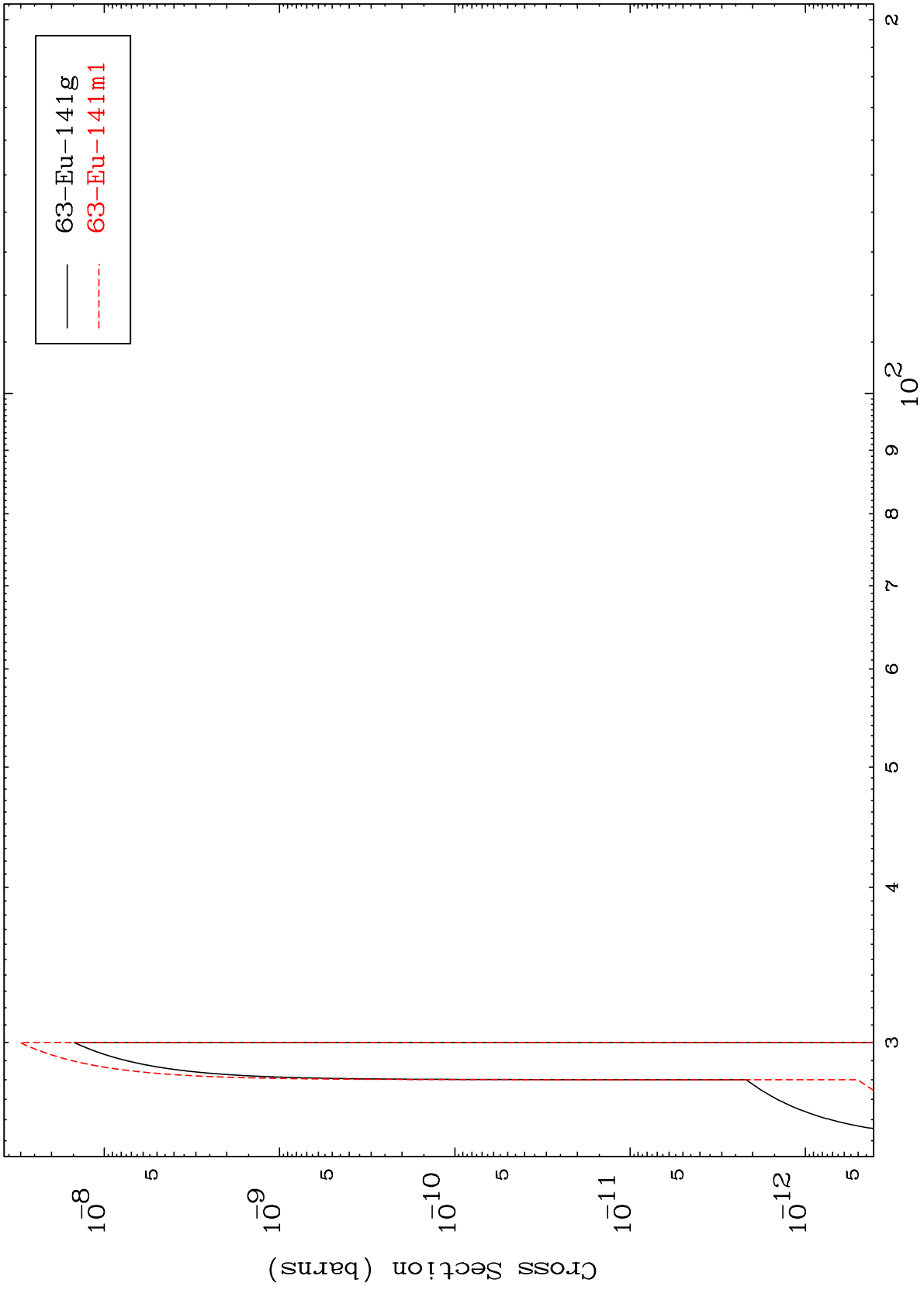
62-Sm-140

MAT 6213

( $\alpha, n'$ ) d

62-Sm-140

Radionuclide Production Cross Section



14

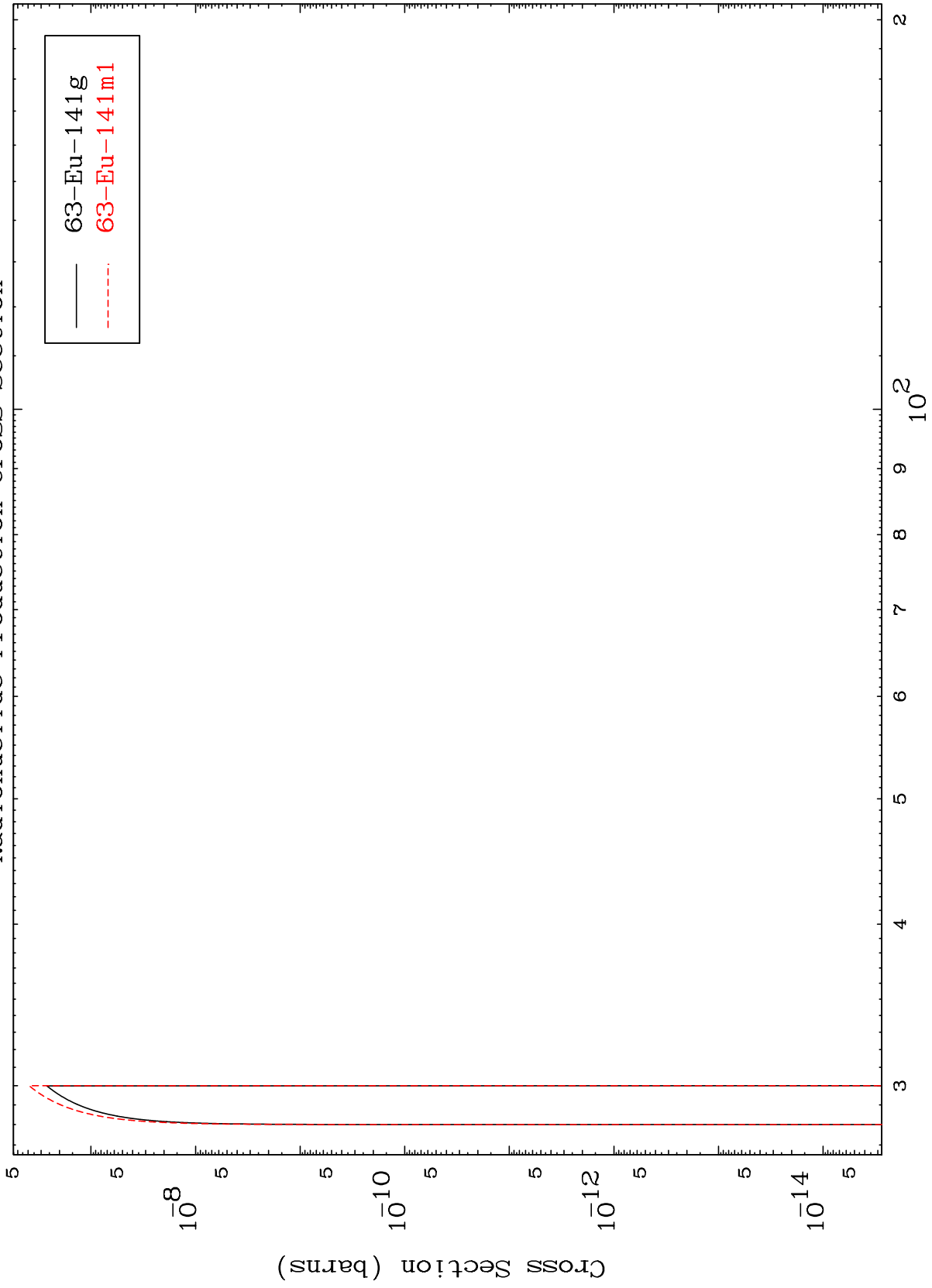
Incident Energy (MeV)

62-Sm-140

MAT 6213

62-Sm-140

( $\alpha, 2n$ ) p  
Radionuclide Production Cross Section



15

Incident Energy (MeV)

62-Sm-140

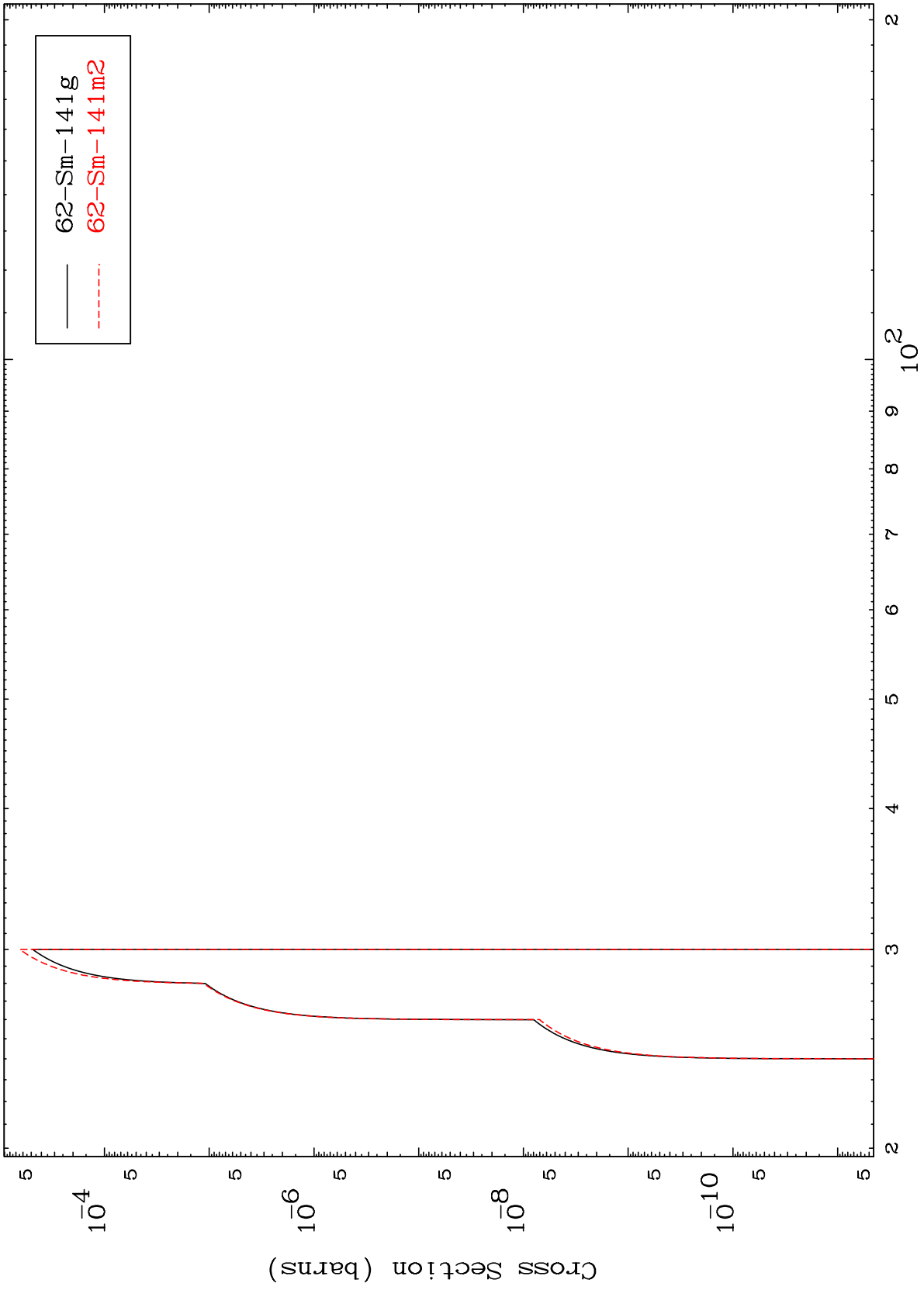


MAT 6213

$(\alpha, 2n)$  p

$^{62}\text{Sm}-140$

Radionuclide Production Cross Section



16

Incident Energy (MeV)

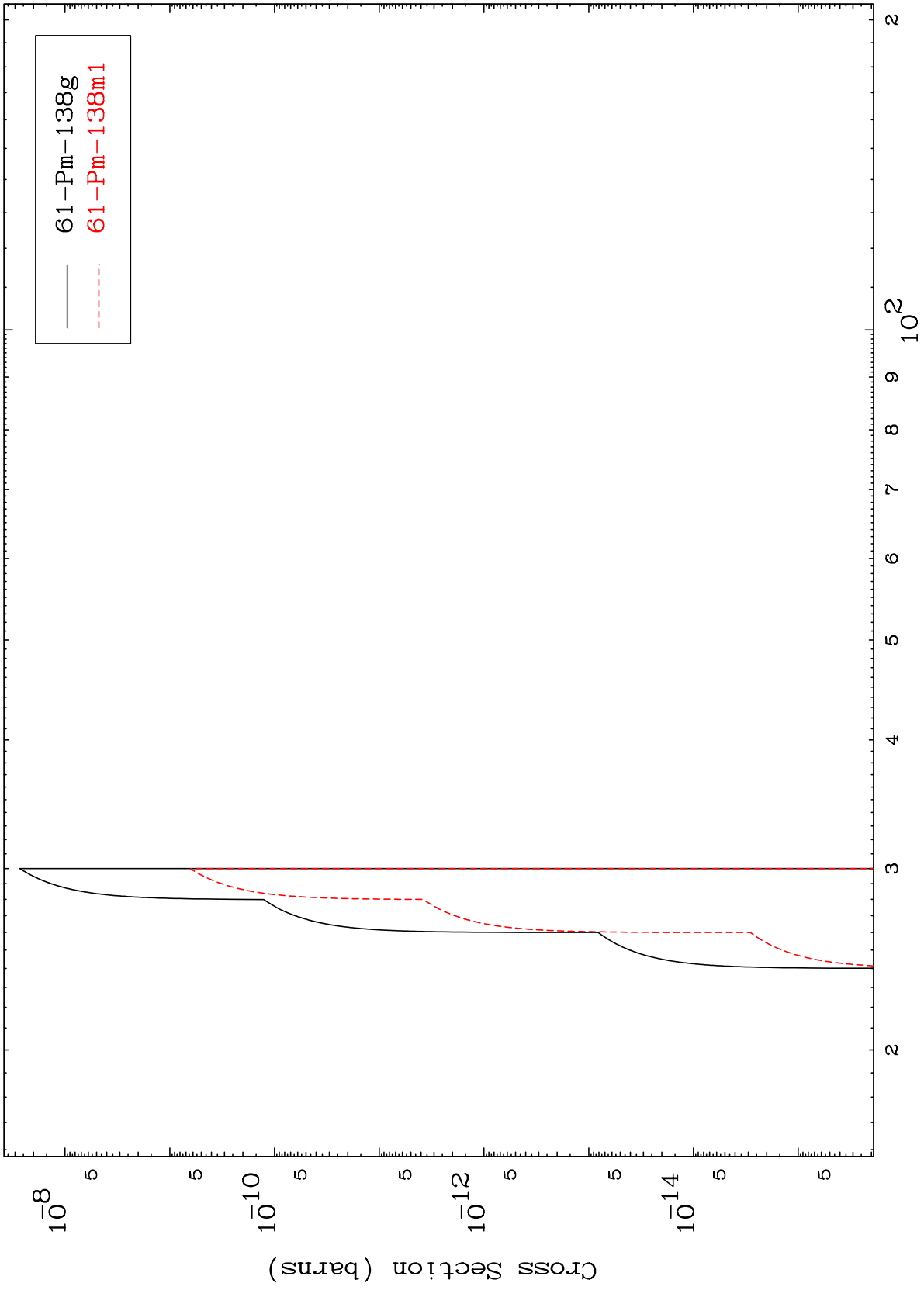
$^{62}\text{Sm}-140$

MAT 6213

$(\alpha, n')$  p  $\alpha$

62-Sm-140

Radionuclide Production Cross Section



17

Incident Energy (MeV)

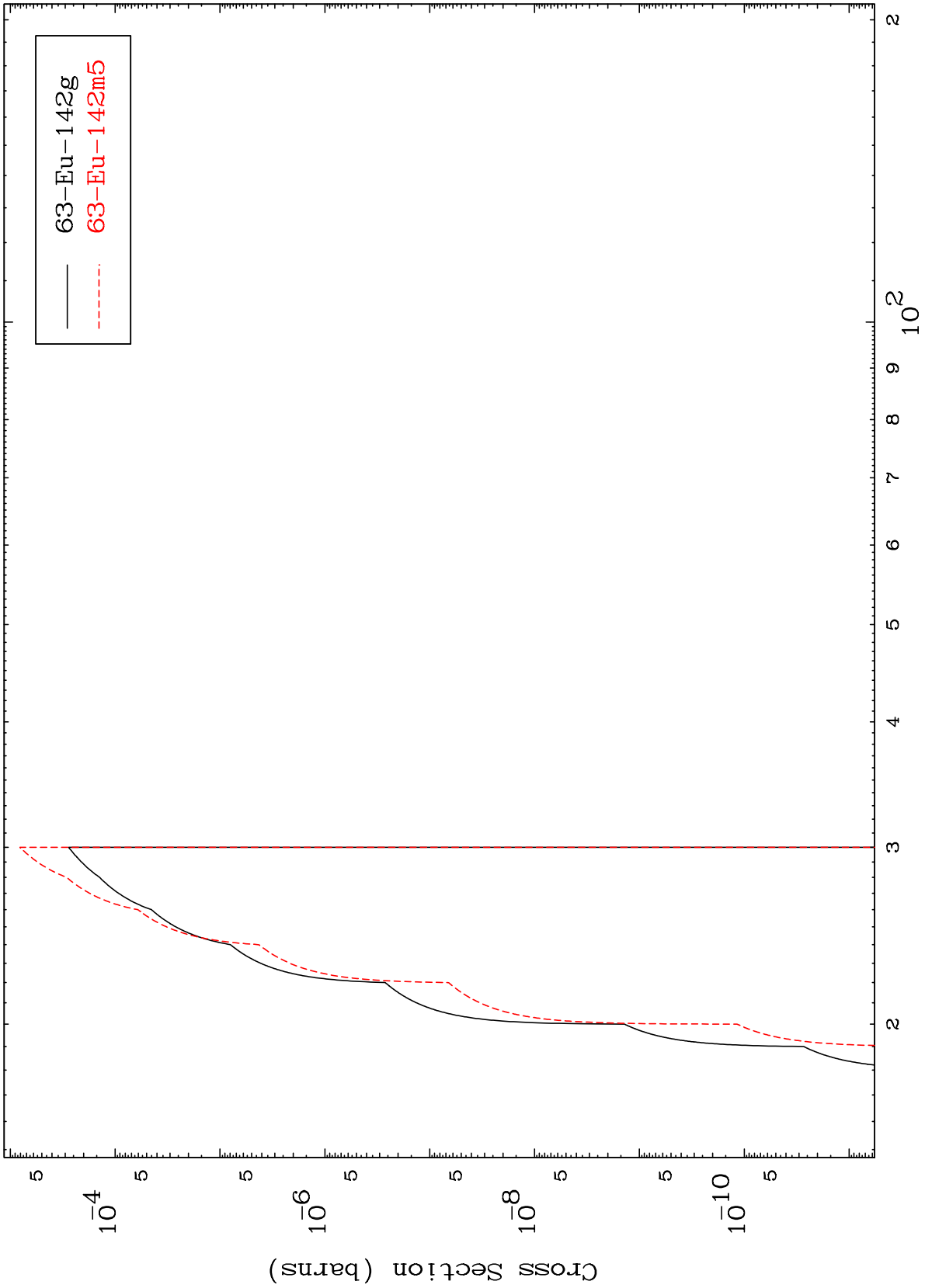
62-Sm-140

MAT 6213

( $\alpha, d$ )

62-Sm-140

Radionuclide Production Cross Section



18

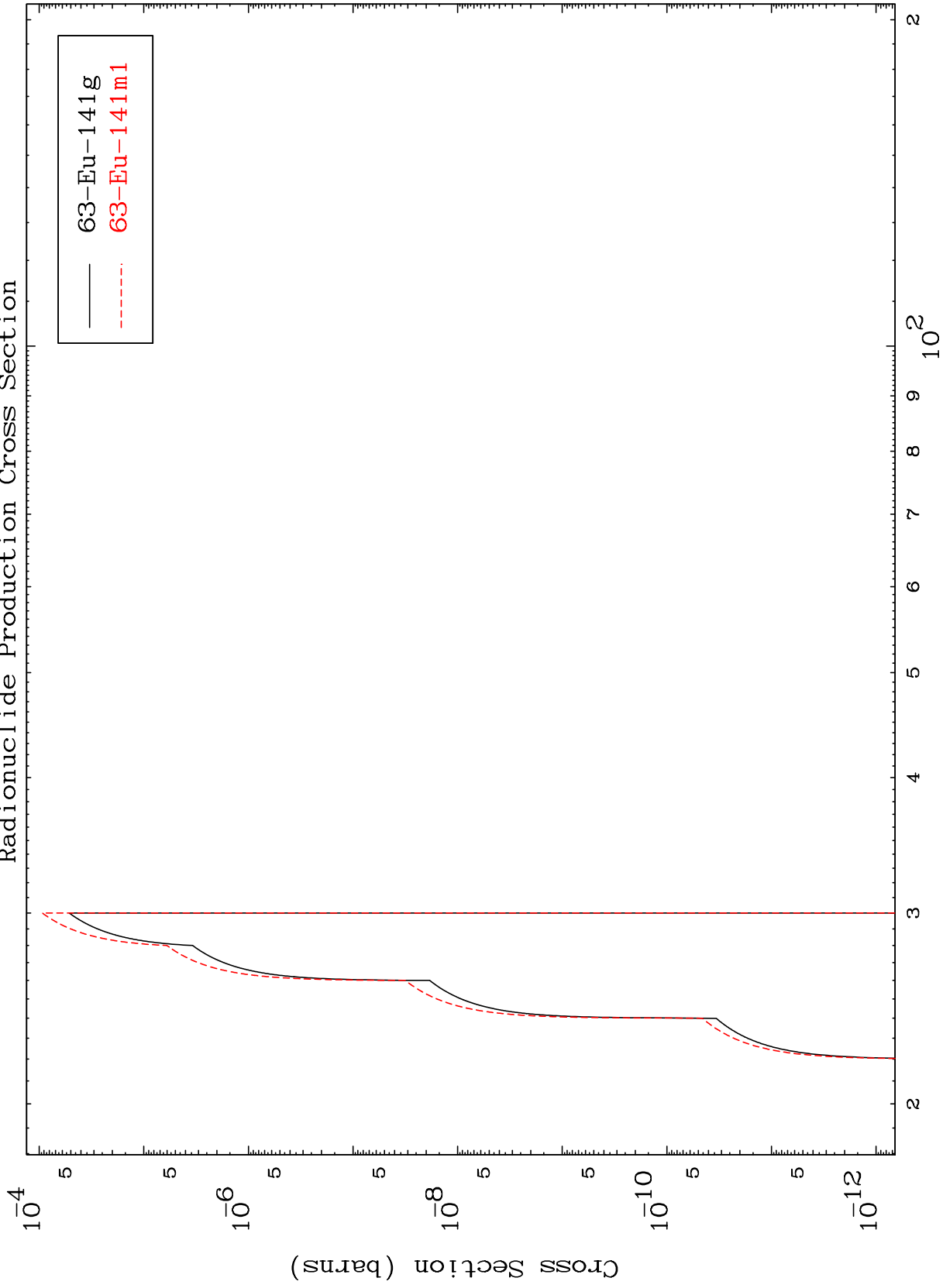
Incident Energy (MeV)

62-Sm-140

MAT 6213

62-Sm-140

$(\alpha, t)$   
Radionuclide Production Cross Section



19

Incident Energy (MeV)

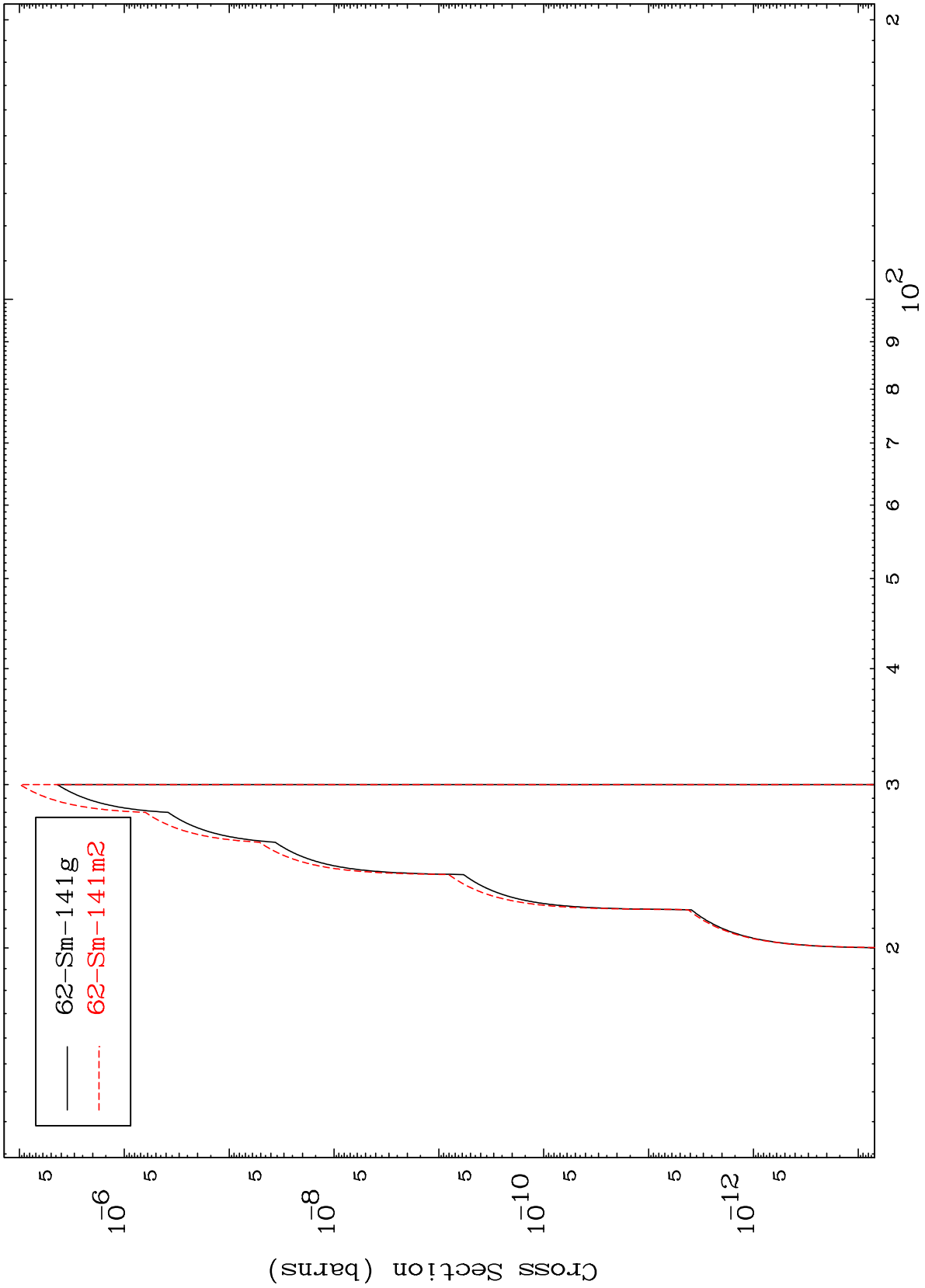
62-Sm-140

MAT 6213

( $\alpha, \text{He-3}$ )

62-Sm-140

Radionuclide Production Cross Section



20

Incident Energy (MeV)

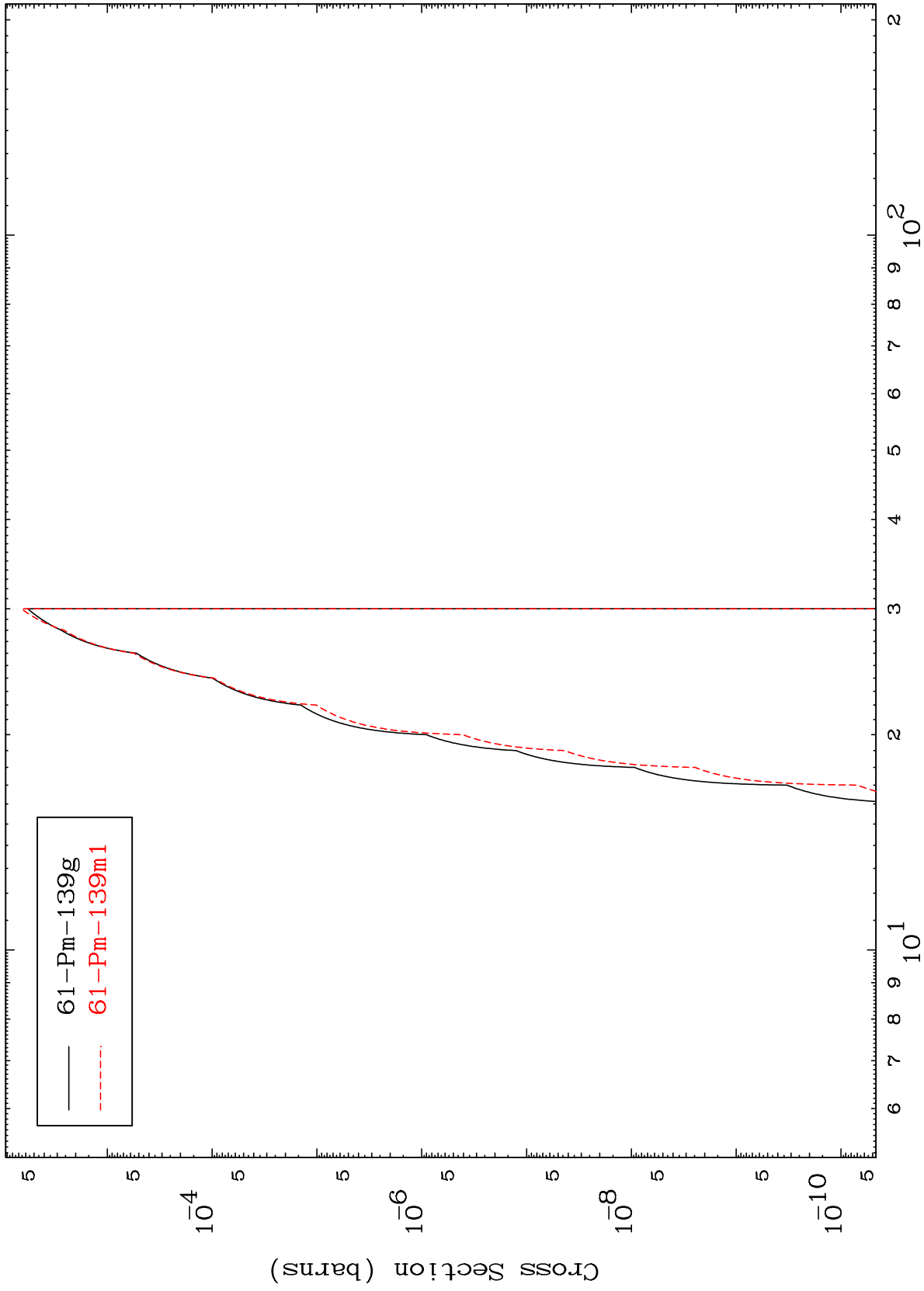
62-Sm-140

MAT 6213

( $\alpha, p$ )  $\alpha$

62-Sm-140

Radionuclide Production Cross Section



21

Incident Energy (MeV)

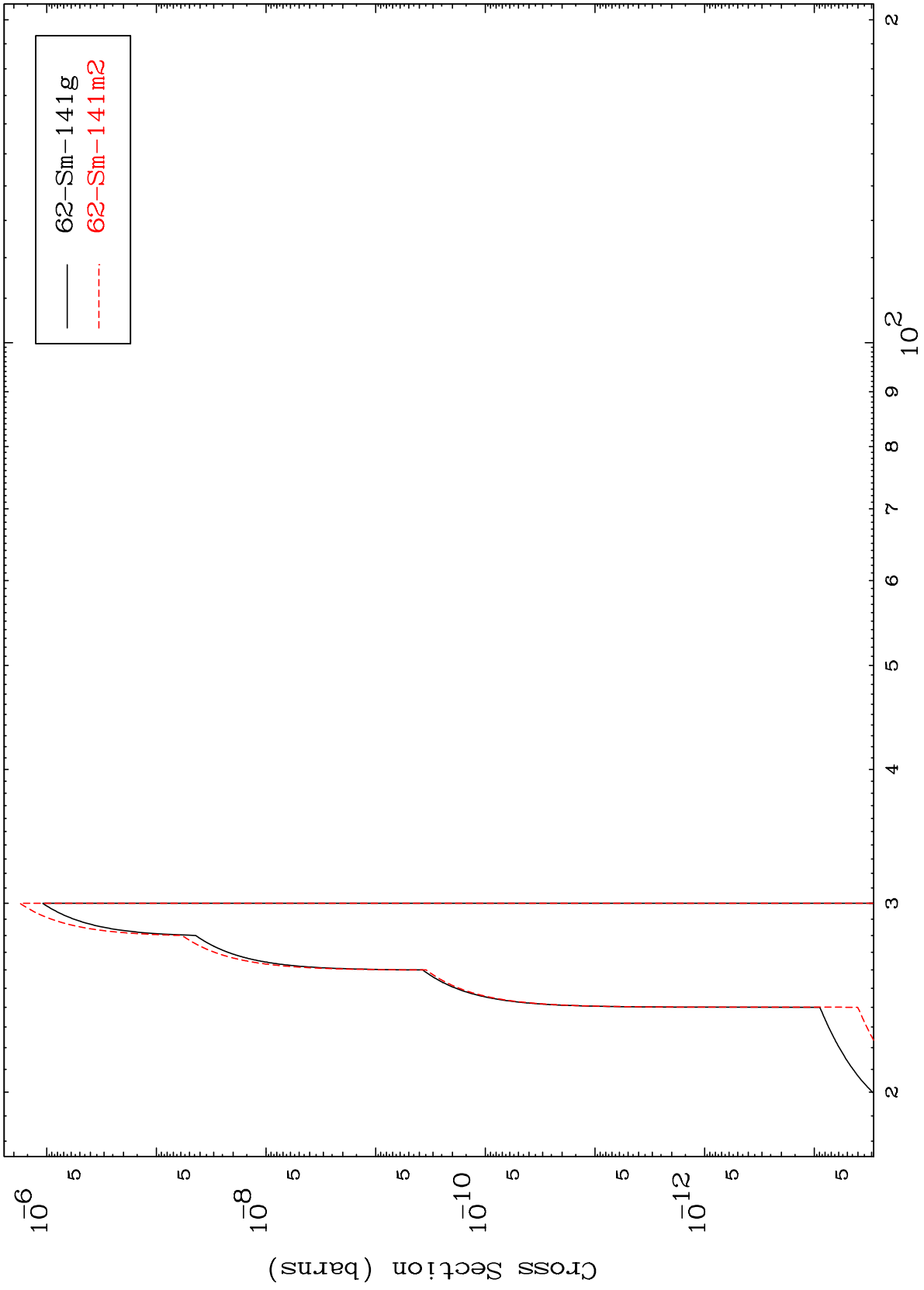
62-Sm-140

MAT 6213

( $\alpha, p$ ) d

62-Sm-140

Radionuclide Production Cross Section



22

Incident Energy (MeV)

62-Sm-140